

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL:50-263 Monticello Nuclear Generating Plant, Northern States 05000263
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 Office of Nuclear Reactor Regulation, Director

SUBJECT: Describes changes planned for fire protection safety
 evaluation sections re commitment to provide capability to
 cool RHR pump seal coolers via emergency svc water sys.
 Forwards drawing of interconnection schematic.

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N O R T H E R N S T A T E S P O W E R C O M P A N Y

Minneapolis, Minnesota 55401

June 27, 1980

Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Redundant Cooling of RHR Pump Seal Coolers Required
by Fire Protection Safety Evaluation

Sections 3.1.16, 5.14.6, 5.15.6, and 5.18.6 of the Monticello Fire Protection Safety Evaluation Report issued by the NRC Staff on August 29, 1979 contain our commitment to provide the capability to cool the RHR pump seal coolers via the emergency service water system in the event of damage to reactor building closed cooling water system cables or pumps. Section 3.1 of the report requires that design information related to this modification be reported to the NRC Staff to assure that the modification is acceptable. The purpose of this letter is to describe for the Staff the changes we are planning to make.

Cooling of RHR pump seal water is required only when these pumps operate with pumped fluid temperatures greater than 165°F. This condition is only expected to occur if a pump is operating in its shutdown cooling mode. In the event of a fire which would necessitate plant shutdown, reactor cold shutdown must be able to be achieved and maintained within 72 hours. If the fire disabled the RBCCW system, cold shutdown could be achieved by using the ESW system to provide cooling to the seal cooler for the single RHR pump required for shutdown cooling.

The attached sketch shows a schematic of the interconnection as planned. The emergency service water system is seismic class 1 up to the first valve after the RHR room coolers. The interconnection will be classified and supported as seismic class 1 up to the interconnection isolation valves. The additional load on the ESW pumps for this modification is expected to be less than 2%, 8-10 gpm. This is within capabilities of the existing ESW system.

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Please contact us if you require additional information related to this modification. To permit us to complete work on schedule, we request the NRC Staff comments related to the proposed design be conveyed to us by August 15, 1980.



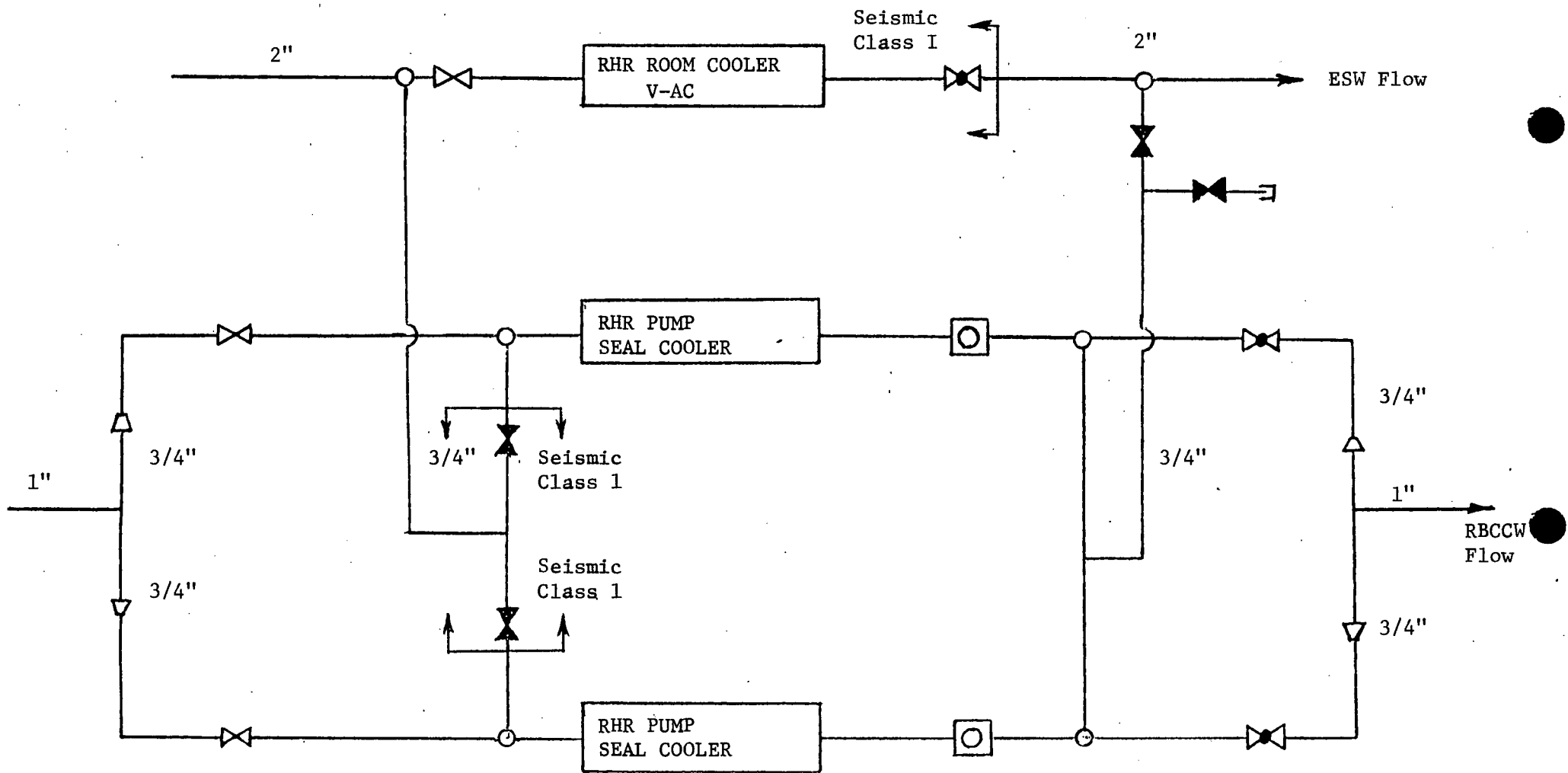
L. O. Mayer, P.E.
Manager of Nuclear Support Services

LOM/sdd

cc: J. G. Keppler
G. Charnoff

Attachment

ESW - RBCCW INTERCONNECTION*



○ Endpoints of New Pipe

* Typical of 1 loop