

50-263

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TO:  
Mr. Victor StelloFROM:  
Northern States Power Company  
Minneapolis, Minnesota  
L. O. Mayer

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DESCRIPTION

ENCLOSURE

Loss of Coolant Accident Analysis Report  
for Monticello Nuclear Generating Plant,  
NEDO 24050, September, 1977 -*See Rpts*

(1-P)

(21-P)

PLANT NAME: Monticello  
RJL 9/19/77

40 ENCL.

SAFETY

FOR ACTION/INFORMATION

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NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

September 15, 1977

REGULATORY DOCKET FILE COPY

Mr Victor Stello, Director  
Division of Operating Reactors  
c/o Distribution Services Branch, DDC, ADM  
U S Nuclear Regulatory Commission  
Washington, DC 20555



Dear Mr Stello:

MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

## Filing of ECCS Analysis

On February 18, 1977, we submitted a letter stating that we would provide a re-evaluation of the loss of coolant accident for Monticello incorporating ECCS model changes which were under discussion at that time. Our voluntary commitment was confirmed by a Nuclear Regulatory Commission Order dated March 11, 1977. The attached document entitled, "Loss of Coolant Accident Analysis Report for Monticello Nuclear Generating Plant, NEDO 24050, September, 1977" fulfills the commitments of the two above referenced documents.

The limitations resulting from the new evaluation are in general less restrictive than those in the existing technical specifications. There are, however, points where the newly calculated limits are slightly more restrictive than those in the technical specifications for the fuel types in service at Monticello. As has been the practice in such situations in the past, we will adhere to the most limiting of the present technical specifications and the newly calculated limits until our technical specifications are appropriately revised. A formal license amendment request is in preparation, and will be submitted shortly.

The ECCS model previously approved in accordance with 10 CFR 50 Appendix K, was applied to a variety of specific break sizes to determine the response for the spectrum from a small break to the DBA break size. Additional break sizes have been analyzed to better define the response over the full spectrum of break sizes using the currently approved ECCS model. This additional analysis has identified 40% of the DBA area as being slightly more severe than the full DBA area upon which limits were previously based.

Yours very truly,

*L. O. Mayer*

L O Mayer, PE  
Manager of Nuclear Support Services

cc: J G Keppler, G Charnoff, MPCA, Attn: J W Ferman

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