

AEC DISTRIBUTION FOR PART 50 DOCKET SERIAL
(TEMPORARY FORM)

CONTROL NO: 2472

FROM: Northern States Power Company Minneapolis, Minn. 55401 L. O. Mayer		DATE OF DOC: 4-13-73	DATE REC'D 4-16-73	LTR X	MEMO	RPT	OTHER
TO: Mr. O'Leary		ORIG 1 signed	CC	OTHER	SENT AEC PDR X SENT LOCAL PDR X		
CLASS: (U) PROP INFO		INPUT	NO CYS REC'D 40	DOCKET NO: 50-263			
DESCRIPTION: Ltr re their 2-20-73 submittal....furnishing suppl info regarding the first Monticello Reload..... W/Attached Fig - Cycle 2 Core			ENCLOSURES: <div style="border: 1px solid black; padding: 5px; text-align: center;"> Do Not Remove ACKNOWLEDGED </div>				
PLANT NAMES: Monticello							

FOR ACTION/INFORMATION

4-16-73

AB

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	IPPOLITO	YOUNGBLOOD	TEETS	L	DUBE
REG OPR	TEDESCO	REGAN	LEE	L	
✓ FILE & REGION(2)	LONG	PROJ LEADER	MAIGRET	L	INFO
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	VOLLMER				

EXTERNAL DISTRIBUTION

✓ 1-LOCAL PDR Minneapolis, Minn.	(1)(2)(9)-NATIONAL LAB'S	1-PDR-SAN/LA/NY
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1-ASLB-YORE/SAYRE	1- CONSULTANT'S	1-AGMED(WALTER KOESTER,
WOODWARD/H ST.	NEWMARK/BLUME/AGABIAN	RM C-427, GT)
✓ 16-CYS ACRS HOLDING SENT TO LIC ASST.	1- GERLAD ULRIKSON....ORNL	1- RD...MULLER...F-309GT
R. DIGGS ON 4-16-73		

Regulatory

File Cy.

NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

April 13, 1973

Mr. J F O'Leary, Director
Directorate of Licensing
United States Atomic Energy Commission
Washington, D C 20545



Dear Mr. O'Leary:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Supplementary Information Regarding the First Monticello Reload

On February 20, 1973 we submitted a document entitled "First Reload License Submittal." Calculations performed in support of that document were based on a target exposure of 6800 MWD/STU. The plant capacity factor exceeded the estimate, resulting in an exposure of approximately 7100 MWD/STU at the date scheduled for the outage. We, therefore, intend to modify our previously reported plans somewhat to reflect the achieved cycle 1 exposure.

The reference core discussed in the initial submittal consisted of 28 R-1 type reload fuel assemblies with 456 initial core fuel assemblies having a preoutage average exposure of 6800 MWD/STU and 64 temporary control curtains. Allowable deviations of these parameters were also discussed to establish a window for the outage. Because exposure exceeded this window, a revised core loading plan has been selected which is compatible with existing exposure conditions. Our present plan is to load a cycle 2 core consisting of 20 R-1 type reload fuel assemblies with 464 initial core assemblies and 44 temporary control curtains. The shutdown margin for this core with the rod of highest worth fully withdrawn is calculated to be 0.015 delta k which is well in excess of the 0.01 delta k design requirement stated in the FSAR. The attached figure shows the planned cycle 2 core.

The safety analysis presented earlier has been reviewed, found to be unaffected by this change in core loading and found not to involve an unreviewed safety question. The Monticello Operations Committee and Safety Audit Committee have reviewed the revised loading plan.

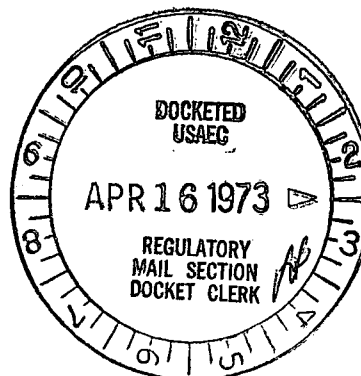
Yours very truly,

L. O. Mayer

L O Mayer, P.E.
Director of Nuclear Support Services

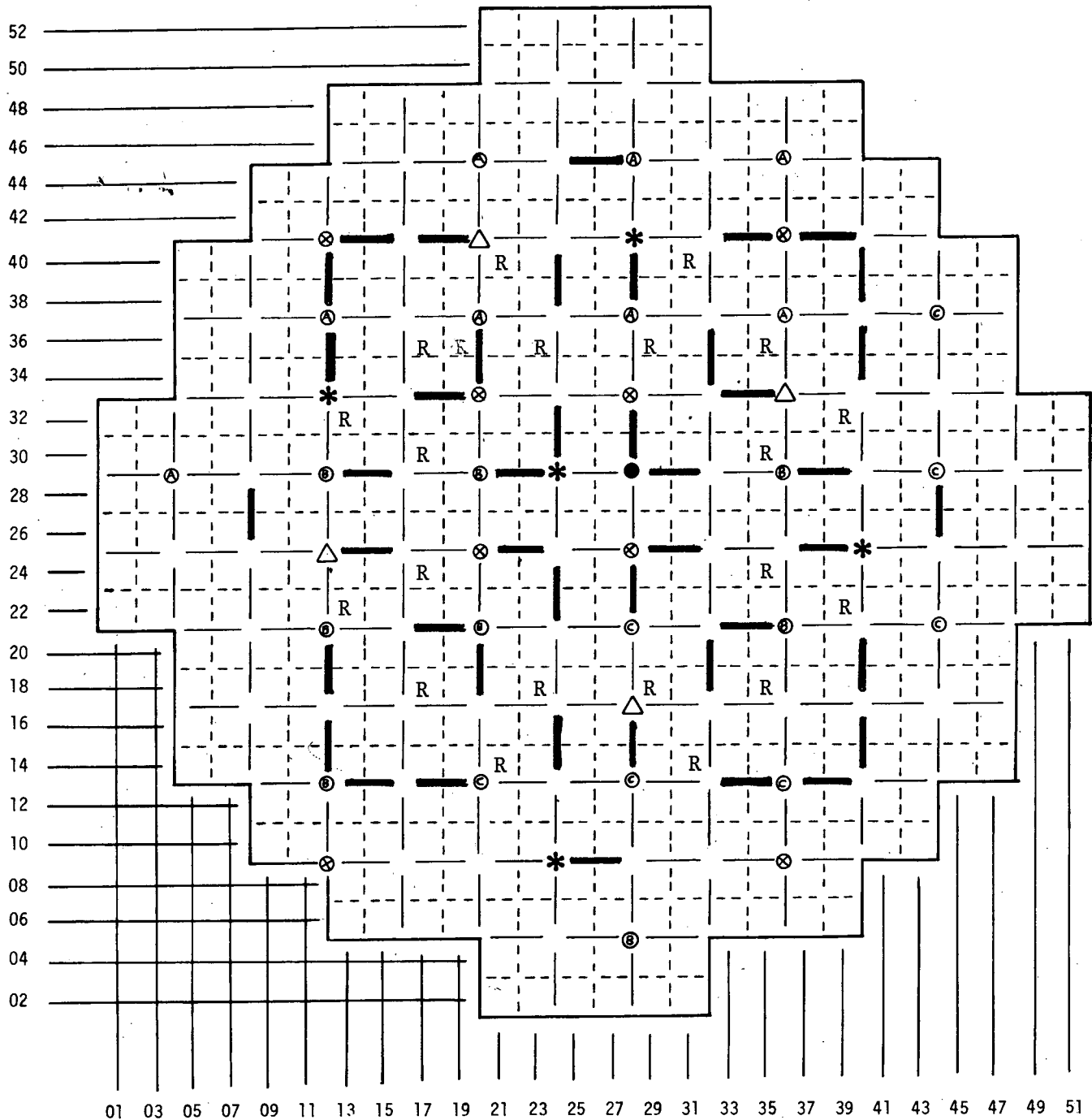
LOM/MHV/br

cc: B H Grier
G Charnoff
Minnesota Pollution Control Agency
Attn. K Dzugan



2472

CYCLE 2 CORE MONTICELLO



- ⊙ LPRM Location (Letter indicates TIP machine)
- LPRM Location (Common location for all TIP machines)
- ⊗ IRM Locations
- △ SRM Locations
- * Source Locations
- Remaining Curtain
- R - Reload Bundle

