

JUN 3 1977

Docket No. 50-263, 50-292
and 50-396

Northern States Power Company
ATTN: Mr. L. O. Hayer, Manager
Nuclear Support Services
414 Nicollet Hall - Eighth Floor
Minneapolis, Minnesota 55401

Gentlemen:

RE: MONTICELLO NUCLEAR GENERATING PLANT
PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NOS. 1 AND 2

We are currently reviewing the onsite emergency power systems of all operating nuclear power facilities to assess the susceptibility of their safety related electrical equipment with regard to (1) sustained degraded voltage conditions at the offsite power sources and (2) interaction between the offsite and onsite emergency power systems.

We have completed our review of licensed responses to our previous generic request for information relative to the electrical power distribution systems. Based on this initial review, we have prepared the Safety Evaluation and Statement of Staff Positions contained in Enclosure 1.

We request that you compare the current design of the emergency power systems at your facility(ies) with the Staff Positions stated in the enclosure and:

- (1) propose plant modifications as necessary to meet the Staff Positions, or
- (2) provide a detailed analysis which shows your facility design has equivalent capabilities and protective features.

Additionally, we require that certain technical specifications be incorporated into all facility operating licenses. Model technical specifications, consistent with the Staff Positions contained in Enclosure 1, are provided in Enclosure 2.

Accordingly, we request that you apply, within forty-five days of the receipt of this letter, for an amendment to your facility operating license(s) to incorporate comparable technical specifications to those presented in the enclosure. Additionally, you should provide a description and a schedule for the completion of any plant associated modifications.

OFFICE >						
SURNAME >						
DATE >						

JUN 3 1977

If you have any questions on this matter, please contact us.

Sincerely,

Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. Staff Positions
2. Model Technical Specifications

cc: See next page

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DATE ➤	5/31/77	5/31/77	6/12/77	6/13/77	

JUN 3 1977

cc w/enclosures:

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Minnesota-Wisconsin Boundary
Area Commission
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Hudson, Wisconsin 54016

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SURNAME ➤						
DATE ➤						



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 3, 1977

Docket No. 50-263, 50-282
and 50-306

Northern States Power Company
ATTN: Mr. L. O. Mayer, Manager
Nuclear Support Services
414 Nicollet Mall - Eighth Floor
Minneapolis, Minnesota 55401

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PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NOS. 1 AND 2

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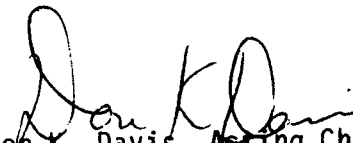
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June 3, 1977

If you have any questions on this matter, please contact us.

Sincerely,


Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. Staff Positions
2. Model Technical Specifications

cc: See next page

June 3, 1977

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ENCLOSURE 1

SAFETY EVALUATION AND STATEMENT OF STAFF POSITIONS
RELATIVE TO THE EMERGENCY POWER SYSTEMS
FOR OPERATING REACTORS

A. INTRODUCTION

The onsite emergency power systems of operating nuclear power facilities are being reviewed to assess the susceptibility of their associated redundant safety-related electrical equipment to:

- (a) Sustained degraded voltage conditions at the offsite power source; and
- (b) Interaction of the offsite and onsite emergency power systems.

We have completed our review of the responses to our generic request for additional information^{1/} relative to the electrical power distribution systems of currently operating nuclear power facilities. In response to our request, all licensees have analyzed their system designs to determine that the voltage levels at the safety-related buses have been optimized for the full load and minimum load conditions that are expected throughout the anticipated range of voltage variations for the offsite power sources. The transformer voltage tap adjustments that were necessary to optimize the voltage levels have been accomplished.

In addition to the above corrective action, we have developed the following staff positions for use in evaluation of each of the operating nuclear power plants with regard to the two items identified above. These positions were developed on the basis of our review of the licensee response to our

^{1/} Letters to all licensees, dated August 12 and 13, 1976.

requests for additional information and of other related information as cited in the text.

B. POSITIONS

1) Position 1: Second Level of Under-or-Over Voltage Protection with a Time Delay

We require that a second level of voltage protection for the onsite power system be provided and that this second level of voltage protection shall satisfy the following criteria:

- a) The selection of voltage and time set points shall be determined from an analysis of the voltage requirements of the safety-related loads at all onsite system distribution levels;
- b) The voltage protection shall include coincidence logic to preclude spurious trips of the offsite power source;
- c) The time delay selected shall be based on the following conditions:
 - (1) The allowable time delay, including margin, shall not exceed the maximum time delay that is assumed in the FSAR accident analyses;
 - (2) The time delay shall minimize the effect of short duration disturbances from reducing the availability of the offsite power source(s); and
 - (3) The allowable time duration of a degraded voltage condition at all distribution system levels shall not result in failure of safety systems or components;

- d) The voltage monitors shall automatically initiate the disconnection of offsite power sources whenever the voltage set point and time delay limits have been exceeded;
- e) The voltage monitors shall be designed to satisfy the requirements of IEEE Std. 279-1971, "Criteria for Protection Systems for Nuclear Power Generating Stations"; and
- f) The Technical Specifications shall include limiting conditions for operation, surveillance requirements, trip set points with minimum and maximum limits, and allowable values for the second-level voltage protection monitors.

General Design Criterion 17 (GDC 17) "Electric Power Systems", of Appendix A, "General Design Criteria for Nuclear Power Plants," of 10 CFR Part 50 requires: (a) two physically independent circuits from the offsite transmission network (although one of these circuits may be a delayed access circuit, one circuit must be automatically available within a few seconds following a loss-of-coolant accident); (b) redundant onsite A.C. power supplies; and (c) redundant D.C. power supplies.

GDC-17 further requires that the safety function of each a.c. system (assuming the other system is not functioning) shall be to provide sufficient capacity and capability to assure that: (a) specified acceptable fuel design limits and the design conditions for the reactor coolant pressure boundary are not exceeded as a result of anticipated operational occurrences; and (b) the core is cooled and containment integrity and other vital functions are maintained during any of the postulated accidents.

Existing undervoltage monitors automatically perform the required function of switching from offsite power, the preferred power source, to the redundant onsite power sources when the monitored voltage degrades to a level of between 50 to 70 percent of the nominal rated safety bus voltage. This is usually accomplished after a one-half to one second time delay. These undervoltage monitors are designed to function on a complete loss of the offsite power source.

The offsite power system is the common source which normally supplies power to the redundant safety-related buses. Any transient or sustained degradation of this common source will be reflected onto the onsite system's safety-related buses.

A sustained degradation of the offsite power system's voltage could result in the loss of capability of the redundant safety loads, their control circuitry, and the associated electrical components required for performing safety functions.

The operating procedures and guidelines utilized by electric utilities and their interconnected cooperative organizations minimize the probability for the above conditions to occur. However, since degradation of an offsite power system that could lead to or cause the failure of redundant safety-related electrical equipment is unacceptable, we require the additional safety margins associated with implementation of the protective measures detailed above.

2) Position 2: Interaction of Onsite Power Sources with Load

Shed Feature

We require that the current system designs automatically prevent load shedding of the emergency buses once the onsite sources are supplying power to all sequenced loads on the emergency buses. The design shall also include the capability of the load shedding feature to be automatically reinstated if the onsite source supply breakers are tripped. The automatic bypass and reinstatement feature shall be verified during the periodic testing identified in Position 3.

In the event an adequate basis can be provided for retaining the load shed feature when loads are energized by the onsite power system, we will require that the setpoint value in the Technical Specifications, which is currently specified as "...equal to or greater than..." be amended to specify a value having maximum and minimum limits. The licensees' bases for the setpoints and limits selected must be documented.

GDC 17 requires that provisions be included to minimize the probability of losing electric power from any of the remaining supplies as a result of or coincident with the loss of power generated by the nuclear power unit, the loss of power from the transmission network, or the loss of power from the onsite electric power supplies.

The functional safety requirement of the "loss-of-offsite power monitors" is to detect the loss of voltage on the offsite (preferred) power system and to initiate the necessary actions required to transfer the safety-related buses to the onsite system. The load shedding feature, which is required to function prior to connecting the onsite power sources to their respective buses can adversely interact with the onsite power sources if the load shedding feature is not bypassed after it has performed its required function. The load shed feature should also be reinstated to allow it to perform its function if the onsite sources are interrupted and are subsequently required to be reconnected to their respective buses.

3) Position 3: Onsite Power Source Testing

We require that the Technical Specifications include a test requirement to demonstrate the full functional operability and independence of the onsite power sources at least once per 18 months during shutdown. The Technical Specifications shall include a requirement for tests: (1) simulating loss of offsite power in conjunction with a safety injection actuation signal; and (2) simulating interruption and subsequent reconnection of onsite power sources to their respective buses. Proper operation shall be determined by:

- a) Verifying that on loss of offsite power the emergency buses have been de-energized and that the loads have been shed from the emergency buses in accordance with design requirements.

- b) Verifying that on loss of offsite power the diesel generators start from ambient condition on the autostart signal, the emergency buses are energized with permanently connected loads, the auto-connected emergency loads are energized through the load sequencer, and the system operates for five minutes while the generators are loaded with the emergency loads.
- c) Verifying that on interruption of the onsite sources the loads are shed from the emergency buses in accordance with design requirements and that subsequent loading of the onsite sources is through the load sequencer.

GDC 17 requires that provisions be included to minimize the probability of losing electric power from any one of the remaining supplies as a result of or coincident with the loss of power generated by the reactor power unit, the loss of power generated by the nuclear power unit, the loss of power from the transmission network, or the loss of power from the onsite electric power supplies.

The testing requirements identified in Position 3 will demonstrate the capability of the onsite power system to perform its required function. The tests will also identify undesirable interaction between the offsite and onsite emergency power systems.

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATING MODES**</u>	<u>ACTION ***</u>
LOSS OF POWER					
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)	* 4(3)/Bus	2/Bus	3(2)/Bus	1, 2, 3	A or B
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	4(3)/Bus	2/Bus	3(2)/Bus	1, 2, 3	A or B

MODEL TECHNICAL SPECIFICATIONS

ENCLOSURE 2

*(Entries in parenthesis are applicable for
2 out of 3 coincidence logic)

**Required when ESF equipment is
required to be operable

***Action A for 2 out of 4 logic
Action B for 2 out of 3 logic

TABLE 3.3-3 (Continued)

ACTION STATEMENTS

- ACTION A
- With the number of OPERABLE channels one less than the Total Number of Channels operation may proceed provided both of the following conditions are satisfied:
 - a. The inoperable channel is placed in the tripped condition within one hour.
 - b. The Minimum Channels OPERABLE requirement is met; however, one additional channel may be bypassed for up to 2 hours for surveillance testing per Specification (4.3.2.1.1).
- ACTION B
- With the number of OPERABLE Channels one less than the Total Number of Channels operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST provided the inoperable channel is placed in the tripped condition within 1 hour.

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES

<u>FUNCTIONAL UNIT</u>	<u>TRIP VALUE</u>	<u>ALLOWABLE VALUES</u>
LOSS OF POWER		
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)	($\frac{+}{-}$) volts with a ($\frac{+}{-}$) second time delay	($\frac{+}{-}$) volts with a ($\frac{+}{-}$) second time delay
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	($\frac{+}{-}$) volts with a ($\frac{+}{-}$) second time delay	($\frac{+}{-}$) volts with a ($\frac{+}{-}$) second time delay

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>OPERATING MODES IN WHICH SURVEILLANCE REQUIRED</u>
LOSS OF POWER				
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)	S	R	M	1, 2, 3
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	S	R	M	1, 2, 3

S = at least once per 12 hours

R = at least once per 18 months

M = at least once per 31 days

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS

4.8.1.1.X Each diesel generator shall be demonstrated OPERABLE:

a. At least once per 18 months during shutdown by:

1. Simulating a loss of offsite power in conjunction with a safety injection actuation test signal, and:

- a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
- b) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency busses with permanently connected loads, energizes the auto-connected emergency loads through the load sequencer and operates for ≥ 5 minutes while its generator is loaded with the emergency loads.
- c) Verifying that on diesel generator trip, the loads are shed from the emergency busses and the diesel re-starts on the auto-start signal, the emergency busses are energized with permanently connected loads, the auto-connected emergency loads are energized through the load sequencer and the diesel operates for ≥ 5 minutes while its generator is loaded with the emergency loads.