

SEP 15 1977

DOCKET FILES

Docket No. 50-263

Northern States Power Company
ATTN: Mr. L. O. Mayer, Manager
Nuclear Support Services
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Gentlemen:

RE: MONTICELLO NUCLEAR GENERATING PLANT

Boiling Water Reactor (BWR) operating history indicates that Type 304 and 316 austenitic stainless steel piping in the reactor coolant system (RCS) pressure boundary are susceptible to stress corrosion cracking. This is particularly true for "service sensitive" lines; i.e., those that are subject to high stress or that contain relatively stagnant intermittent, or low flow fluids. Although stress corrosion cracking is unlikely to cause a rapidly propagating pipe failure, the presence of such cracks constitutes a degradation of the RCS boundary, and thus is undesirable from a safety standpoint. Therefore, it is the NRC staff's position that steps should be taken to minimize stress corrosion cracking in BWR piping systems to eliminate this condition and to improve overall plant reliability. The details of the staff's position are contained in the enclosed NUREG-0313, "Technical Report on the Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping", dated July, 1977.

We request that you review all of the RCS pressure boundary piping and fitting material, including weld metal, at your facility to determine if it meets the material selection and processing guidelines set forth in the enclosed report. You should identify to us any materials that do not meet these guidelines and propose appropriate changes to your Technical Specifications to incorporate the augmented inservice inspection requirements specified in Section III of the enclosed report. In the case of "service sensitive" lines you should also provide your plans and schedule for the replacement, to the extent practicable, of non-conforming materials with those that conform to the staff's guidelines.

In addition, if you find that you have any pressure boundary piping that does not conform to the staff's guidelines, we request that you propose changes to the RCS leakage limits and surveillance requirements contained in your Technical Specifications, to bring them into conformance with the enclosed model Technical Specifications.

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DATE ➤						

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You should complete all of the above actions within 90 days. If you have any questions, please contact us.

Sincerely,



Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping"
2. Model Technical Specifications

cc w/enclosures:
See next page

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