



UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

TELEPHONE
(312) 858-2660

July 31, 1972

Northern States Power Company
ATTN: Mr. Leo Wachter, Vice President
Power Production and System
Operations
414 Nicollet Mall
Minneapolis, Minnesota 55401

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U.S. ATOMIC ENERGY COMMISSION
MAIL & RECORDS SECTION
Docket No. 50-263

Gentlemen:

This refers to the inspection conducted by Messrs. Maura and Johnson on July 12, 1972, of operations at the Monticello plant authorized by AEC Operating License No. DPR-22, and to the discussion of our findings held by the inspectors with Messrs. Scheinost, Larson, Clarity, and Dinville of your staff at the conclusion of the inspection.

Areas examined during this inspection included the reactor scram of July 10, 1972, the equipment performances during the sequence of events which followed, and corrective actions taken following the event. Within these areas, the inspection consisted of selective examination of procedures and representative records, interviews with plant personnel, and observations by the inspector.

During this inspection it was found that one of your activities appeared to be in noncompliance with AEC requirements. The item and references to the pertinent requirements are listed in the enclosure to this letter. Although we have confirmed that you have taken action to correct this item, we request that you provide us within 20 days, in writing, your comments concerning the item, any steps that have been or will be taken to prevent recurrence, and the date all preventive measures were or will be completed.

July 31, 1972

Should you have any questions concerning this inspection, we will be glad to discuss them with you.

Sincerely yours,

Boyce H. Grier
Regional Director

Enclosure:
Description of Item of Noncompliance

bcc: J. G. Keppler, RO
R. H. Engelken, RO
H. D. Thornburg, RO
P. A. Morris, RO
A. Giambusso, L
RO Files
DR Central Files
PDR
Local PDR
NSIC
DTIE

ENCLOSURE

Northern States Power Company
Monticello
Docket No. 50-263

Certain of the activities under your license appears to be in noncompliance with AEC license requirements as indicated below:

Sections 3.1A, 3.2A, 3.2B, and 3.2E.3, of the Technical Specifications require that four drywell pressure instrument channels be operable or operating when irradiated fuel is in the reactor vessel and the reactor water temperature is above 212°F. Section 1.0 of the Technical Specifications defines a component to be operable when it is capable of performing its intended function in its required manner.

Contrary to the above, two of the four drywell pressure sensing taps were found on July 11, 1972, to have been covered with tape. Although emergency core cooling system operation was properly initiated by redundant channels, failure of one of these redundant channels could have resulted in a delay in the actuation of required safeguards. This is evidenced by the fact that the pressure indicated by the drywell pressure recorder, to which one of the taped sensing taps is connected, increased to 0.8 psig prior to commencing to decrease, even though drywell pressure had increased to approximately 2 psig, as evidenced by the operation of pressure switches connected to the untaped sensing lines.