



Tennessee Valley Authority  
1101 Market Street, LP 3R  
Chattanooga, Tennessee 37402-2801

**R. M. Krich**  
Vice President  
Nuclear Licensing

September 16, 2011

10 CFR 50.36(c)(5)

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555-0001

Sequoyah Nuclear Plant, Unit 2  
Facility Operating License No. DPR-79  
NRC Docket No. 50-328

**Subject: Unit 2 Cycle 17 - 90-Day Steam Generator Report for Voltage-Based Alternate Repair Criteria and W\* Alternate Repair Criteria**

The purpose of this letter is to submit the 90-day steam generator report for the Sequoyah Nuclear Plant (SQN), Unit 2. SQN, Unit 2, entered Mode 4 from the Cycle 17 refueling outage on June 18, 2011. SQN, Unit 2, Technical Specifications (TS) 6.9.1.16.2 and 6.9.1.16.4 require the report to be submitted within 90 days after initial entry into Mode 4 following completion of inspections in accordance with the steam generator program. Therefore, this report is due September 16, 2011.

In accordance with SQN, Unit 2, TS 6.9.1.16.2, the Tennessee Valley Authority (TVA) is providing the 90-day steam generator (SG) report (Enclosure 1). The report contains results of voltage-based repair criteria that were applied during the Unit 2 Cycle 17 (U2C17) refueling outage SG inspections (end of Cycle 17 operation). The voltage-based alternate repair criteria (ARC) are for axial outside diameter stress corrosion cracking at tube support plate intersections. The report contains a condition monitoring assessment that demonstrates that the Generic Letter (GL) 95-05, "Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking," acceptance criteria are satisfied for Unit 2 at the end of Cycle 17 and an operational assessment that demonstrates that the GL 95-05 acceptance criteria is expected to continue to be satisfied through Cycle 18.

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In accordance with SQN, Unit 2, TS 6.9.1.16.4, the TVA is providing the 90-day SG report associated with the application of W\* during the U2C17 refueling outage (Enclosure 2). The cumulative leak rates as calculated for W\*, the GL 95-05 alternate repair criteria, and all other sources of primary to secondary accident-induced leakage were within the SQN, Unit 2, limit (3.7 gallons per minute) per SG.

There are no regulatory commitments associated with this submittal. Should you have any questions, please contact Dan Green at (423) 751-8423.

Respectfully,

A handwritten signature in black ink, appearing to read "R. M. Krich", is written over the typed name.

R. M. Krich

Enclosures:

1. Sequoyah 2C17 Voltage-Based ARC 90-Day Report
2. Sequoyah 2C17 W-Star 90-Day Report

cc (Enclosures):

NRC Regional Administrator – Region II  
NRC Senior Resident Inspector – Sequoyah Nuclear Plant