



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

January 31, 2012

Mr. Jon A. Franke, Vice President
Crystal River Nuclear Plant (NA2C)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

**SUBJECT: CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT – AUDIT OF THE
LICENSEE'S MANAGEMENT OF REGULATORY COMMITMENTS
(TAC NO. ME5540)**

Dear Mr. Franke:

In Regulatory Issue Summary 2000-17, "Managing Regulatory Commitments Made by Power Reactor Licensees to the NRC Staff," dated September 21, 2000, the U. S. Nuclear Regulatory Commission (NRC) informed the licensees that the Nuclear Energy Institute (NEI) document, NEI 99-04, "Guidelines for Managing NRC Commitment Changes," contains acceptable guidance for controlling regulatory commitments and encouraged licensees to use the NEI guidance or similar administrative controls to ensure that regulatory commitments are implemented and that changes to the regulatory commitments are evaluated and, when appropriate, reported to the NRC.

The NRC Office of Nuclear Reactor Regulation (NRR) has instructed its staff to perform an audit of the licensees' commitment management programs once every 3 years to determine whether the licensees' programs are consistent with the industry guidance in NEI 99-04, and regulatory commitments are being effectively implemented.

An audit of Crystal River Unit 3 Nuclear Generating Plant (CR-3) commitment management program was performed by the CR-3 project manager at the plant site during the period of June 29, 30 and 31, 2011. The Branch Chief of NRR's Division of Operating Reactor Licensing for Region II plants, which include CR-3, was also at the site during this period and participated in part of the discussions. During the audit, the NRC staff identified a missed implementation date for a regulatory commitment. The licensee's letter dated August 14, 2008, contained a regulatory commitment to revise the inservice inspection (ISI) plan regarding VT-3 examination of the reactor vessel support weld by September 30, 2008. However, the revision to the ISI plan was completed on December 15, 2008 (overdue for 2.5 months) without a notification to the NRC and without a due date extension request. The licensee entered this missed implementation date in its PassPort® database and into its nuclear condition report. Notably, on February 2009, the licensee had created a business objects reports of the open commitments for the lead licensing engineer to ensure that regulatory commitments due dates are not missed.

J. Franke

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Based on the audit, the NRC staff concluded that (1) CR-3 has generally, except as stated earlier, implemented NRC commitments on a timely basis, and (2) CR-3 has overall implemented an effective program for managing NRC commitment changes. Details of the audit are set forth in the enclosed audit report.

Sincerely,

A handwritten signature in black ink, appearing to read "Farideh E. Saba". The signature is written in a cursive, flowing style.

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: Audit Report

cc: Distribution via ListServ

AUDIT REPORT BY THE OFFICE OF NUCLEAR REACTOR REGULATION

LICENSEE'S MANAGEMENT OF REGULATORY COMMITMENTS

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

DOCKET NOS. 50-302

1.0 INTRODUCTION AND BACKGROUND

In the Regulatory Issue Summary 2000-17, "Managing Regulatory Commitments Made by Power Reactor Licensees to the NRC Staff," dated September 21, 2000, the U. S. Nuclear Regulatory Commission (NRC) informed licensees that the Nuclear Energy Institute (NEI) document NEI 99-04, "Guidelines for Managing NRC Commitment Changes," contains acceptable guidance for controlling regulatory commitments and encouraged licensees to use the NEI guidance or similar administrative controls to ensure that regulatory commitment are implemented and that changes to the regulatory commitments are evaluated and, when appropriate, reported to the NRC.

The NRC Office of Nuclear Reactor Regulation (NRR) has instructed its staff to perform an audit of licensees' commitment management programs once every 3 years to determine whether the licensees' programs are consistent with the industry guidance in NEI 99-04, and that the regulatory commitments are being effectively implemented.

The document, NEI-99-04, defines a "regulatory commitment" as an explicit statement to take a specific action agreed to, or volunteered by, a licensee and submitted in writing on the docket to the NRC. NRR guidelines direct the NRR Project Manager to audit the licensee's commitment management program by assessing the adequacy of the licensee's implementation of a sample of commitments made to the NRC in past licensing actions (i.e. amendments, reliefs, exemptions, etc.) and activities (bulletins, generic letters, etc.).

2.0 AUDIT PROCEDURE AND RESULTS

An audit of the Crystal River Nuclear Plant, Unit 3 (CR-3) commitment management program was performed at the plant site during the period of June 29, 30 and 31, 2011. This was the second audit performed at the plant. The previous audit was done in August 2008 and recorded in the audit report dated October 3, 2008. The second audit reviewed commitments made since the first audit. The audit consisted of two major parts: (1) verification of the licensee's implementation of NRC commitments that have been completed, and (2) verification of the licensee's program for managing changes to NRC commitments.

2.1 Verification of Licensee's Implementation of NRC Commitments

The primary focus of this part of the audit is to confirm that the licensee has implemented commitments made to the NRC as part of past licensing actions/activities. For commitments not yet implemented, the NRC staff determines whether they have been captured in an effective program for future implementation.

Enclosure

2.1.1 Audit Scope

The audit addressed most of the commitments made during the review period. The audit focused on regulatory commitments (as defined above) made in writing to the NRC as a result of past licensing actions (amendments, exemptions, etc.), or licensing activities (bulletins, generic letters, etc.). Commitments made in licensee event reports or in response to notices of violation may be included in the sample, but the review will be limited to verification of restoration of compliance, not the specific methods used. The audit excluded the following types of commitments that are internal to licensee processes:

- (1) Commitments made on the licensee's own initiative among internal organizational components.
- (2) Commitments that pertain to milestones of licensing actions/activities (e.g., responding to an NRC request for additional information by a certain date). Fulfillment of these commitments was indicated by the fact that the subject licensing action/activity was completed.
- (3) Commitments made as an internal reminder to take actions to comply with existing regulatory requirements such as regulations, technical specifications, and updated final safety analysis reports. Fulfillment of these commitments was indicated by the licensee having taken timely action in accordance with the subject requirements.

2.1.2 Audit Results

Before the audit, the NRC staff searched the Agencywide Documents Access and Management System for the licensee's submittals that contained a list of regulatory commitments since the last audit. In addition, during the audit the NRC staff reviewed the status of regulatory commitments that were "in progress" during the last audit. Prior to the audit, the licensee also provided a list of regulatory commitments related to licensing actions from its commitment management system and the documentation to support the NRC staff's audit. The licensee's documentation included summary sheets providing the regulatory commitment action request and assignment number, the licensee's letter and date that contained the commitment, and the status of the commitment.

The licensee's commitment management program is described in the Progress Energy Corporation (PEC), Nuclear Generation Group, Standard Procedure REG-NGGC-0110, Revision 3, "Regulatory Commitments." The PEC procedure is applicable to regulatory commitments made or modified after the effective date of this procedure.

The NRC staff reviewed the licensee's procedure, REG-NGGC-0110, Revision 3, which is an updated version of the Revision 2 that was reviewed during the last audit. The NRC staff found that REG-NGGC-0110, Revision 3 includes the following statements, which provide guidance to the licensee, is consistent with the intent of NEI 99-04, and ensures that CR-3 is appropriately implementing regulatory commitments.

- In the case where a third party (e.g., NEI, an owners group, or another organization) has been authorized to make regulatory commitments on behalf of the licensee, the affected licensee shall ensure that statements represented as regulatory commitments are appropriately documented and are subsequently managed in accordance with REG-NGGC-0110.
- Docketed correspondence containing action items shall also contain an explicit statement concerning the existence of any regulatory commitment.
- Each regulatory commitment shall be captured in a PassPort® Action Request (AR).
- The responsible licensing supervisor, or designee, shall confirm the accuracy of any implicit or explicit restatement of a regulatory commitment in docketed correspondence received from the NRC to ensure that written or oral communication has not been misconstrued.
- Except for “discretionary enforcement” situations, advise the NRC that an oral statement to take a certain action represents the intention to make a regulatory commitment but does not constitute a regulatory commitment until submitted by the licensee in writing on the docket.
- An assignment associated with a regulatory commitment must include appropriate reference to implementing documentation to provide traceability for the regulatory commitment.
- The responsible licensing supervisor, or designee, shall consider the need to incorporate the regulatory commitment into an NRC mandated licensing basis document such as the Updated Final Safety Analysis Report, Quality Assurance Program, Emergency Plan, Security Plan, Fire Protection Program, etc.
- If a regulatory commitment is not implemented as described by the due date or if noncompliance with regulatory commitment occurs, the responsible licensing supervisor, or designee, shall initiate a nuclear condition report and, if necessary, notify the NRC.
- If a regulatory commitment has yet to be implemented and implementation plans change, the revised commitment can be implemented, however, the NRC should be notified of the change as soon as practicable after approval by appropriate management, and before any committed completion date. Notification should be accomplished by supplementing or revising the docketed correspondence containing the original commitment.

The licensee enters commitments made to the NRC into a commitment database called “PassPort®.” The PassPort® database handles the commitments that are open or active. This database is used to track all commitments, including commitments to state and other agencies, and also tracks other plant activities. Each regulatory commitment should be captured in a PassPort® AR, a Nuclear Task Management (NTM) should be initiated, and appropriate actions to implement the regulatory commitment should be captured in a PassPort® Action Tracking Assignment. The assignment type is usually entered as “COMM” and is owned by the

REGCOMM group. Also, entered are due dates and the names of individuals responsible for meeting the commitment.

In addition to REG-NGGC-0110, a site-specific procedure, CP-252, "Commitment Management," Revision 8, is also being utilized at the CR-3 site. The procedure defines a process for identifying, managing, and changing ongoing regulatory commitments. An ongoing commitment is defined in this procedure as a commitment requiring ongoing and continuous implementation, whereas a one-time commitment is a commitment that is to be completed once by a specific date, event, or milestone and has no future or ongoing action. This procedure also defines the process for creating, maintaining, and changing active nuclear operations commitment system (NOCS) database entries for the ongoing regulatory commitments. A computer report from the NOCS database cross-references commitments to CR-3 implementation.

The NRC staff reviewed documentation generated by the licensee related to items listed in the attached table that are identified as regulatory commitments in response to different types of actions or communications (Category) with the NRC, in order to assess the implementation of each regulatory commitment, including the completion status. For the commitments selected for the audit, the NRC staff found that the licensee had adequately captured all of the regulatory commitments in their PassPort[®] system, which is a data management and tracking tool. For the on the ongoing commitments, the NRC staff verified that they have been correctly entered into the NOCS database, which is a commitment management tool.

During the audit, the NRC staff reviewed plant procedures that have been revised as a result of commitments made by the licensee to the NRC. The NRC staff noted that the procedures that are affected by a regulatory commitment are identified in the PassPort[®] database under the AR assigned for that commitment. As such, the licensee's procedure owner will be advised to refer to all the regulatory commitments associated with each procedure, prior to making any changes to the procedure. The review of the sample commitments in the above database reflected their status consistent with the commitment program. The attached audit summary table provides details of the audit and its results.

The NRC staff reviewed the status of the implementation of the regulatory commitments that were made since the last audit and the commitments that were in progress at that time. During the audit the NRC staff identified a missed implementation date for a regulatory commitment. The licensee's letter dated August 14, 2008, contained a regulatory commitment to revise the inservice inspection (ISI) plan regarding the VT-3 examination of the reactor vessel support weld by September 30, 2008. However the ISI plan was revised on December 15, 2008 (overdue for 2.5 months) without notifying the NRC and requesting a due date extension. The licensee entered this missed implementation date in its PassPort[®] database AR No. 474202 and into its nuclear condition report (NCR) 20110630. Notably from February 2009 and looking forward, the licensee had created a business objects reports of the open commitments to ensure that regulatory commitments due dates are not missed. This report did not look backward, therefore it did not identify the previously missed due date.

2.2 Verification of the Licensee's Program for Managing NRC Commitment Changes

The primary focus of this part of the audit is to verify that the licensee has established administrative controls for modifying or deleting commitments made to the NRC. The NRC staff compared the licensee's process for controlling regulatory commitments to the guidelines in NEI 99-04, which the NRC has found to be an acceptable guide for licensees to follow for managing and changing commitments. The process used at CR-3 is contained in the PEC's procedure, REG-NGGC-0110. This procedure along with the CR-3 procedure, CP-252, for managing ongoing regulatory commitments, are being used at CR-3 for tracking changes to regulatory commitments made to the NRC. The audit reviewed most of commitment changes that included changes that were or will be reported to the NRC, and changes that were not or will not be reported to the NRC. The audit also verified that the licensee's commitment management system includes a mechanism to ensure traceability of commitments following initial implementation. This ensures that licensee personnel are able to recognize that future proposed changes to the affected design features or operating practices require evaluation in accordance with the commitment change control process.

2.2.1 Audit Results

The NRC staff reviewed the licensee's procedure, REG-NGGC-0110 against NEI 99-04. The REG-NGGC-0110 provides guidance to the licensee that is consistent with the intent of NEI 99-04, and ensures that CR-3 is appropriately implementing regulatory commitment changes, as well as tracking changes to the commitments.

The NRC staff found that REG-NGGC-0110 adequately conforms to the guidance and intent of NEI 99-04 for commitment tracking, the commitment change process, traceability of commitments, and reporting requirements. Regulatory commitment changes are processed and tracked by the responsible licensing supervisor, or designee. The evaluation of any commitment changes is to be done by filling the "Regulatory Commitment Change Evaluation" form in Attachment 3 of the procedure. The NRC staff reviewed this form and found it consistent with the intent of the "Commitment Evaluation" form in NEI 99-04.

During the audit, the NRC staff reviewed the licensee's letter dated May 25, 2010, that includes a summary of the CR-3 regulatory commitment changes for the period of November 23, 2007 to January 5, 2010. This regulatory commitment changes report indicated that of the 15 regulatory commitments that were modified or inactivated, 7 met the NEI 99-04 criteria for NRC notification. During the audit, the NRC staff reviewed the Regulatory Commitment Change Evaluation forms for those changes that were reported to the NRC as well as for those that did not meet the NEI 99-04 criteria for reporting to the NRC. These changes were captured in the NOCS database and provided in a computer review package. A computer report from the NOCS database and the regulatory commitment change evaluations are created and maintained in a binder by the lead licensing engineer.

During the audit, the NRC staff reviewed a change to a regulatory commitment that was originated in the licensee's letter dated February 12, 2007, and was reported by the licensee in its letter dated May 25, 2010. The NRC staff noticed this is not a regulatory commitment; rather, it is the licensee's response to an NRC security order. However, the CR-3 lead licensing engineer indicated that this is consistent with the NEI proposal, as captured in the a safety

evaluation attached to the NRC's letter dated August 23, 2007, that states "the implementing details found to be acceptable means of meeting the license condition would be treated as commitments and managed in accordance with NEI 99-04." During the audit, the NRC staff found this approach acceptable, since it is endorsed by the NRC in the August 23, 2007, safety evaluation. No deficiencies were identified during this review for those changes that were reported to the NRC.

The NRC staff also reviewed the regulatory commitment changes that were made by the licensee from January 5, 2010, which is reported in the licensee's letter dated May 25, 2010, to a week prior to the audit. These regulatory commitment changes were compiled by the lead licensing engineer in a specific binder for this period and were presented to the NRC staff during the audit.

During the audit, the NRC staff identified an improper regulatory commitment change regarding the licensee's response to Notice of Violation 89-23-01 in 1989. The licensee in its letter dated November 21, 1989, committed to revise RSP-110, "Control of Auxiliary Building Rollup Door," to require closure of the sliding door in the green room in the Auxiliary building (AB) when not in use. RSP-110 was replaced by RSP-101 "Basic Radiological Safety Information and Instructions for Radiation Workers." A commitment change request, which was approved internally by the licensee on July 15, 2010, added a note to the NOCS for this commitment for including a contingency in OP-409, "Plant Ventilation System," Revision 82. Specifically, OP-409 was revised to state that OP-409 will include contingencies for maintaining negative pressure in the AB when the green room sliding door is open for reasons other than ingress and egress from the AB. Section 4.28.3 of this procedure provides steps for opening the AB sliding green room door to the environment, if AHF-11A, AHF-11B or AHF-10 are removed from service and a high differential pressure exists across any AB door. The procedure requires monitoring airflow through the AB to the environment two times per shift. Further, OP-409 states that if airflow at the room is out from the AB or AB differential pressure reads less than 0.0 inches of water gage, then close the AB green room door.

Upon reviewing the revised OP-409, the NRC staff determined that the administrative controls added to the revised procedure are not adequate to ensure continuous negative pressure in the AB. Therefore, the original adverse condition identified in the notice of violation was restored by this change to the commitment. The licensee entered this inadequacy in the CR-3 operating procedure in its PassPort® database AR No. 474433 and into NCR 20110701.

Discussions with CR-3 staff confirmed that changes to regulatory commitments are generally being handled in accordance with the guidance contained in NEI 99-04. In addition, the NRC staff reviewed documentation from the licensee related to the items that involved changes to the commitments. The NRC staff found that, except for the commitment in the licensee's letter dated November 21, 1989, the licensee properly addressed each regulatory commitment change reviewed during this audit and has implemented an effective program to manage commitment changes.

2.3 Audit Observations and Suggestions

During the audit, the NRC staff noticed that:

- (1) PassPort® database is used for all plant activities. Therefore, unless specific information (such as an AR number) is available, PassPort® is not a friendly tool for searching and identifying regulatory commitments applicable to the NRC.
- (2) In preparation for the audit, the PEC lead licensing engineer requested that a business objects report be run on PassPort® to identify all assignments that were created since last audit. The lead licensing engineer only kept those assignments that in their attribute notes referred to a letter originated by the CR-3 licensing to the NRC, which its numbering started by "3F," e.g., 3F1008-05. Although, this report captured the majority of the regulatory commitments, it did not include those commitments made by other organizations, such as the licensee's corporate office. This report also did not include commitments made in the amendment requests submittals and were considered completed upon issuance of the amendments.
- (3) No AR number was assigned to the regulatory commitments prior to the use of the PassPort® database. The responsible licensing engineer generated an AR number if the commitment required a change or an update in its status (not a regulatory commitment change).
- (4) Review of regulatory commitment changes that were not reported to the NRC shows that some of these commitments (usually, old commitments) should not have been identified as regulatory commitments because they were either codified requirements, commitments not related to the NRC, updated submittal schedules, or related to an inspection activity. As an example, the inadequacy that is discussed in Section 2.2.1 of this report is not a regulatory commitment; rather, it is a corrective action to restore compliance.

During the audit, the NRC staff and the CR-3 licensing engineer discussed ways to improve efficiency and traceability of the regulatory commitments in both the Passport® and NOCS databases, and to ensure that a regulatory commitment due date is not missed. As a result, the following items were suggested:

- (1) When creating an assignment to capture a regulatory commitment, a reference to both the letter number in which the regulatory commitment is contained and the letter date should be included.
- (2) Check the status of open regulatory commitments at least every two weeks to ensure that the regulatory commitment due date is not missed.

3.0 CONCLUSION

The NRC staff concludes that based on the above audit: (1) CR-3 has generally implemented NRC commitments on a timely basis, and (2) CR-3 has implemented an effective program for managing NRC commitment changes.

4.0 LICENSEE PERSONNEL CONTACTED FOR THIS AUDIT

Dennis Herrin, Lead Engineer- Licensing and Regulatory Programs
Dan Westcott, Supervisor - Licensing and Regulatory Programs

Principal Contributor: Farideh E. Saba

Attachment: Audit Summary

AUDIT SUMMARY

IMPLEMENTATION OF COMMITMENTS:

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
1	Relief Request	March 12, 2008 3F0208-04	Provide a report of the weld overlay examination results including a listing of indications detected. The recording criteria of the ultrasonic examination procedure to be used for the examination of the overlays requires that all indications, regardless of amplitude, be investigated to the extent necessary to provide accurate characterization, identity, and location. Additionally, the procedure requires that all indications, regardless of amplitude, that cannot be clearly attributed to the geometry of the overlay configuration be considered flaw indications.	14 days after engineering acceptance of the ultrasonic examination results.	00269743-06	Attachment 2 of Letter 3F0308-07 dated 03/26/08

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
2	Relief Request	March 12, 2008 3F0208-04	Provide a report documenting the disposition of indications using the standards of American Society of Mechanical Engineers (ASME) Section XI, IWB-3514-2 and/or IWB-3514-3 criteria and, if possible, the type and nature of the indications. The ultrasonic examination procedure requires that all suspected flaw indications are to be plotted on a cross-sectional drawing of the weld and that the plots should accurately identify the specific origin of the reflector.	14 days after engineering acceptance of the ultrasonic examination results.	00269743-07	Letter 3F0308-07 dated 03/26/08. There were no suspected flaw indications observed during the examinations. No disposition was necessary.
3	Relief Request	March 12, 2008	Provide a report discussing any repairs to the weld overlay material and/or base metal and the reason for the repairs. ⁴	14 days after engineering acceptance of the ultrasonic examination results.	00269743-08	Letter 3F0308-07 dated 03/26/08. There were no repairs necessary to the weld overlay material or base metal.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
4	Relief Request	March 12, 2008	Provide a report documenting a stress analysis summary demonstrating that the subject piping will perform its intended design functions after the weld overlay installation. The stress analysis report will include results showing that the requirements of NB-3 200 and NB-3 600 of the ASME Code, Section III, are satisfied. The stress analysis will also include results showing that the requirements of IWB-3000 of the ASME Code, Section XI, are satisfied. The results will show that the postulated crack including its growth in the nozzles will not adversely affect the integrity of the overlaid welds.	60 days after entry into Mode 4 start-up.	00269743-09	Attachment to Letter 3F0508-12 dated 05/15/08
5	Generic Letter (GL) 2008-01	May 8, 2008 3F0508-07	Complete the detailed walk-downs and ultrasonic examinations of inaccessible piping at locations potentially susceptible to gas accumulation for systems within the scope of the GL prior to startup from the next refueling outage.	Currently scheduled for 12/19/2009.	00263132-04	This commitment was completed on 10/8/10. All GL-2008-01 activities were closed by the NRC letter dated 6/24/2011.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
6	GL 2008-01	May 8, 2008 3F0508-07	Submit supplemental response to the NRC within 90 days following completion of Refueling Outage 16, which will describe any changes to the 9-month GL 2008-01 response resulting from walkdowns and ultrasonic examination of inaccessible piping.	90 days following completion of Refueling Outage 16.	00263132-05	This commitment was completed on 14/044/10. All GL-2008-01 activities were closed by the NRC letter dated 6/24/2011.
7	License Amendment Request	October 6, 2008 PE&RAS-08-037	Removal of the plant-specific technical specifications (TSs) requirements will be performed concurrently with the implementation of the Title 10 of the <i>Code of Federal Regulations</i> (10 CFR), Part 26, Subpart I requirements.	This commitment will be completed no later than October 1, 2009.	00288262-23	One TS page was updated on 06/25/09.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
8	GL 2008-01	October 13, 2008 3F1008-05	<p>Quarterly monitoring will be developed and implemented to ensure that the emergency core cooling system (ECCS), decay heat removal (DH), and containment spray system (BS) suction and discharge piping will be maintained sufficiently full of water to ensure that the systems can reliably perform their intended functions.</p> <p>The inspections will include a requirement for periodic verification (every 92 days) that the ECCS, DH, and BS piping will be maintained sufficiently full of water by a combination of ultrasonic testing (UT), and venting as deemed necessary, of locations identified to be potentially susceptible to gas intrusion. (high to low pressure interfaces)</p> <p>Additionally, should any maintenance activities breach the ECCS, DH, or BS system boundary, a UT will be performed as deemed necessary to verify the respective system(s) are sufficiently full prior to return to service.</p>	1/31/2009	00263132-08 00263132-12 00263132-13	<p>Assignment 08 was completed on 01/27/09 by establishing procedures for quarterly UT examinations.</p> <p>Assignment 12 was completed on 02/02/09 by issuing CP-1130.</p> <p>Assignment 13 was completed on 01/29/09 by Rev 19 of OPS-NGGC-130, Equipment Clearance.</p> <p>The licensee created NOCS 100549 and NOCS100550 for these ongoing RC.</p>

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
9	GL 2008-01	October 13, 2008 3F1008-05	Procedures will be enhanced to provide additional detail where needed concerning venting sequence, venting duration, dynamic venting, etc., and to UT appropriate piping locations following fill and vent as deemed necessary, to ensure piping is sufficiently full prior to return to service.	Procedure revisions will be completed by 12/31/2009.	00263132-09	This commitment was completed on 12/23/09 through completion of OP-402, OP-404 and OP-405. The licensee created NOCS 100551 for this ongoing RC.
10	GL 2008-01	October 13, 2008 3F1008-05	Florida Power Corporation (FPC) is continuing to support the industry and Nuclear Energy Institute (NEI) Gas Accumulation Management Team activities regarding resolution of generic improved technical specifications (ITS) changes via the Technical Specification Task Force (TSTF) Traveler process. Within 9 months after NRC approval of the TSTF Traveler, FPC will evaluate its applicability to Crystal River Unit 3 Nuclear Generating Plant (CR-3), and evaluate adopting the TSTF Traveler to supplement the current ITS requirements.	9 months after NRC approval of the TSTF.	00263132-06	Holding until approval of TSTF 523, which was submitted to the NRC on June 29, 2010, and is under review by the NRC staff.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
11	GL 2008-01	October 13, 2008 3F1008-05	FPC will develop an acceptance criterion for the maximum allowable void volume that will not challenge ECCS, DH, or BS System operability.	12/31/09	00263132-10	This commitment is completed on 12/23/09 by Calculation M09-0049, Revision 1 and M09-0051, Revision 1.
12	GL 2008-01	October 13, 2008 3F1008-05	FPC will install the new vent valves and relocate existing vent valves as discussed in the design evaluation section of this response.	Installation will be completed prior to breaker closure following Refueling Outage 16.	00263132-11	All on-line and outage vent valves installed by 12/29/09.
13	Relief Request	October 29, 2008 3F1008-03	The details surrounding the design analysis for the structural weld overlay are being developed to support the CR-3 16 th refueling outage.	This vendor-supplied analysis will be available for NRC review at the beginning of the CR-3 16 th refueling outage.	00282980-03	The analysis was completed by 09/25/09 and will be available at the site for NRC's review at the beginning of the 16 th refueling.
14	Relief Request	October 29, 2008 3F1008-03	After completion of the ultrasonic examination of the weld overlays, a report containing the ultrasonic examination results, a discussion of any repairs to the weld overlay material and/or base metal, and reason the repair will be submitted to the NRC.	Within 60 days of completion of the ultrasonic examination of the weld overlays performed during Refueling Outage 16.	00282980-06	The ultrasonic examination results were submitted to the NRC by letter 3F0110-05 dated 01/13/2011.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
15	Relief Request	October 29, 2008 3F1008-03	The NRC will be notified as soon as practical, but no later than Mode 4 during restart, if any cracks are the detected that exceed the preservice examination acceptance standards in ASME Code, Section XI, Table IWB-3514-2.	Prior to Mode 4 during restart from CR-3 16 th refueling outage, if required.	00282980-13	See relief request dated March 12, 2009.
16	Relief Request	March 12, 2009 ¹ 3F0309-03	NRC will be notified as soon as practical if any cracks are detected that exceed the preservice examination acceptance standards in ASME Code Section X1, Table IWB-3514-2, in accordance with Attachment 5 of the CR- 3 letter to NRC dated October 29, 2008 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML083080296).	Prior to Mode 4 during restart from Refueling Outage 16, scheduled for fall 2009, if necessary.	00282980-13	The NRC was notified by letter 3F1209-01 dated 12/15/09.4
17	Relief Request	March 12, 2009 3F0309-03	Submit preliminary analysis of the residual stresses and flaw growth of repaired weldment, including crack growth calculations.	Prior to Mode 4 during restart from Refueling Outage 16, scheduled for fall 2009.	00282980-10	Final analysis of the residual stresses and flaw growth was submitted to the NRC by letter 3F1209-02 dated 12/17/09.

¹ Also, see NRC's letter dated July 18, 2009.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
18	Relief Request	March 12, 2009 3F0309-03	After completion of the ultrasonic examination of the weld overlays performed in Refueling Outage 16, submit to the NRC the ultrasonic examination results of the weld overlays and a discussion of any repairs to the overlay material and/or base metal and reason for the repair, in accordance with Attachment 5 of the CR-3 letter to NRC dated October 29, 2008 (ADAMS Accession No. ML083080296).	60 days after completion of the ultrasonic examination of the weld overlays performed in Refueling Outage 16, scheduled for fall 2009.	00282980-06	The ultrasonic examination results were submitted to the NRC by letter 3F0110-05 dated 01/13/2011.
19	Relief Request	March 12, 2009 3F0309-03	Submit analysis of the residual stresses and flaw growth of repaired weldment, including crack growth calculations.	60 days after plant restart (output breaker closure) from Refueling Outage 16, scheduled for fall 2009.	00282980-12	Crack growth calculations were submitted to the NRC by letter 3F1209-02 dated 12/17/09.
20	License Amendment Request	August 4, 2009 3F0809-01	Proposed TS limits for frequency and voltage will be administratively enforced until the license amendment is implemented.	Procedures are in place and will be maintained until the license amendment is implemented.	00288450 (NTM 2888450)	02/25/2010- Procedures SP-354A and SP-354B were in-place and maintained prior to approval (12/10/2009) and implementation of the amendment.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
21	National Fire Protection Association (NFPA) 805	November 25, 2009 3FI 109-01	CR-3 will submit a license amendment request for transitioning to NFPA 805. The license amendment request will include a plant-wide assessment to identify fire areas and fire hazards and evaluate compliance with the existing CR-3 fire protection licensing basis. This assessment will be performed in accordance with NEI 04-02, "Guidance for Implementing a Risk- Informed, Performance-Based Fire Protection Program under 10 CFR 50.48(c)."	6 months after the date of NRC's safety evaluation report issuing the license amendment to the second NFPA 805 pilot plant.	00453160-03	This AR is completed by the licensee's letter dated June 29, 2011 (six months after approval of the 2 nd pilot plant) requesting an extension to July 1, 2014. This request was accepted by the NRC's letter dated August 8, 2011. The licensee opens a new AR to track the new due date.
22	Exemption	July 13, 2010 3F0710-03	CR-3 will ensure that the affected individuals will not work excessive overtime during the period immediately preceding the implementation of the exemption to 10 CFR 26.205(d)(3). Specifically, CR-3 will ensure covered personnel, including those temporarily assigned to outages at other units, do not work greater than an average of 54 hours per week.	For a minimum of 8 weeks preceding implementation of the exemption.		The licensee withdrew this request on April 11, 2011.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
23	License Amendment Request	December 20, 2010 ² 3F1210-05	Spent fuel loading activities using the auxiliary building (AB) overhead crane (FHCR-5) shall not commence if an approaching or potential tropical storm, an approaching or potential hurricane, or a tornado watch or warning has been declared for the site in accordance with CR-3 site procedures. If spent fuel loading activities with FHCR-5 are in progress when any of the above criteria are met, the load will be lowered to a safe location. AB overhead crane FHCR-5 will be moved to the south end of the AB, away from the spent fuel pools, and the crane secured.	Procedures will be modified and implemented, and training conducted, as needed, prior to designating FHCR-5 as a single failure proof crane.	00438875-02	In progress, this submittal was replaced by the letter dated July 20, 2011.
24	License Amendment Request	February 25, 2011 3F0211-01	Proposed TS limits for the DFT-4 fuel oil supply tank level will be administratively maintained until the license amendment is implemented.	Procedures are in place and will be maintained until the license amendment is implemented.	00448253-06 (NTM448253)	Procedures SP-306 and SP-249C and calculation M98-0118

² This amendment was supplemented by letter dated July 20, 2011.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
25	Exemption	March 3, 2011 3F0311-02	All workers beginning restart work will have a minimum 34-hour break within the 9 days that precedes the day on which the worker begins work at CR-3, and will not exceed the maximum work hour rules.	CR-3 will return to normal, online work hour controls within 60 days or when the outage activities are completed, whichever is less.		This exemption request is a supplement to the July 13, 2010 request. The licensee withdrew this request on April 11, 2011.
26	Relief Request	August 14, 2008 3F0808-03	CR-3 will revise the Inservice Inspection (ISI) Plan to assure that the performance of a VT-3 examination of the reactor vessel support skirt weld, to the extent practical, will occur when maintenance or other activities remove the insulation covering the support skirt weld.	09/30/2008	NOCS 100547	During the audit the NRC staff noticed that ISI plan is completed on 12/14/2009, which is 2.5 months later than the commitment due date without reporting to the NRC. The licensee created Action Request No. 00281127 to follow up this issue.
27	GL 2004-02	February 26, 2009 3F0209-02	FPC will respond to Request for Additional #7, and how the in-vessel downstream effects are addressed for CR-3.	Within 90 days of issuance of the final NRC staff safety evaluation.	00196070-07	In progress, completion of this item is linked to the NRC's approval of WCAP-16793.

No.	Category	Letter Date/ No.	Commitment	Scheduled Date	Action Request No. - Assignment No.	Completion Date - Method
28	Topical Report, BAW- 2374	August 25, 2009 3F0809-02	CR-3 will provide the structural limit associated with the most limiting large break loss-of-coolant accident for the replacement steam generators as part of the next once-through steam generator tube inspection report (required by the TSs) following completion of the next inspection of the tubes in the replacement steam generators.	180 days after the initial entry into Mode 4 following completion of the steam generator tube inspections in the fall 2011 refueling outage.	00344602-03	In progress, due date is unknown due to extended refueling outage of 2009.
29	Cyber Security	November 19, 2009 3F1 109-08	Enclosure 2 to request for approval of the Crystal River Unit 3 Cyber Security Plan.	Varies	00367940-02 00367940-03	Withdrawn and replaced by the corporate letter dated July 8, 2010, which was approved by the NRC on July 29, 2011.

J. Franke

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Based on the audit, the NRC staff concluded that (1) CR-3 has generally, except as stated earlier, implemented NRC commitments on a timely basis, and (2) CR-3 has overall implemented an effective program for managing NRC commitment changes. Details of the audit are set forth in the enclosed audit report.

Sincerely,

/RA/

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: Audit Report

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