

Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385



AUG 18 2011

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No. 11-411
MPS Lic/LES R0
Docket No. 50-336
License No. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 2
LICENSEE EVENT REPORT 2011-002-00
MILLSTONE POWER STATION UNIT 2 REACTOR TRIP ON LOW STEAM GENERATOR
LEVEL

This letter forwards Licensee Event Report (LER) 2011-002-00 documenting an event that occurred at Millstone Power Station Unit 2 on June 20, 2011. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B), including the Reactor Protection System and the Auxiliary Feedwater Actuation System.

If you have any questions or require additional information, please contact Mr. William D. Bartron at (860) 444-4301.

Sincerely,

A. J. Jordan
Site Vice President – Millstone

Attachments: 1

Commitments made in this letter: None

IEZZ
NRK

cc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

C. J. Sanders
Project Manager - Millstone Power Station
U.S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Mail Stop 08B3
Rockville, MD 20852-2738

NRC Senior Resident Inspector
Millstone Power Station

ATTACHMENT

LICENSEE EVENT REPORT 2011-002-00

**MILLSTONE POWER STATION UNIT 2
DOMINION NUCLEAR CONNECTICUT, INC.**

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (10/2010)		APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013 <small>Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.</small>																																					
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)																																							
1. FACILITY NAME Millstone Power Station - Unit 2		2. DOCKET NUMBER 05000336	3. PAGE 1 OF 2																																				
4. TITLE Reactor Trip on Low Steam Generator Level																																							
5. EVENT DATE <table border="1" style="width: 100%; border-collapse: collapse;"> <tr> <th style="width: 25%;">MONTH</th> <th style="width: 25%;">DAY</th> <th style="width: 25%;">YEAR</th> </tr> <tr> <td style="text-align: center;">06</td> <td style="text-align: center;">20</td> <td style="text-align: center;">2011</td> </tr> </table>		MONTH	DAY	YEAR	06	20	2011	6. LER NUMBER <table border="1" style="width: 100%; border-collapse: collapse;"> <tr> <th style="width: 25%;">YEAR</th> <th style="width: 25%;">SEQUENTIAL NUMBER</th> <th style="width: 25%;">REV NO.</th> </tr> <tr> <td colspan="3" style="text-align: center;">2011 - 002 - 00</td> </tr> </table>		YEAR	SEQUENTIAL NUMBER	REV NO.	2011 - 002 - 00																										
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10. POWER LEVEL <div style="text-align: center; font-size: 1.2em;">059</div>		Specify in Abstract below or in NRC Form 366A																																					
12. LICENSEE CONTACT FOR THIS LER																																							
FACILITY NAME William D. Bartron, Nuclear Station Licensing		TELEPHONE NUMBER (Include Area Code) 860-444-4301																																					
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																																							
CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX																														
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) <p>At 1152 on June 20, 2011, with Millstone Power Station Unit 2 operating at 59 percent power in Mode 1, the reactor tripped automatically on low steam generator water level. The decreasing water level condition was due to a low suction pressure trip on the "B" steam generator feed pump (SGFP) that occurred while placing the "A" SGFP in service. The event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B). The actuation of the Auxiliary Feedwater Actuation System also is a reportable condition under the same paragraph.</p> <p>The cause of the event was gaps in the application of operator fundamentals and some procedure quality issues associated with operations procedure OP 2204, Load Changes.</p> <p>Procedure OP 2204 has been revised to ensure the second steam generator feedwater pump is placed in service at a lower power level. Additional corrective actions to address the underlying causes of the gaps in the application of operator fundamentals are being addressed in accordance with the station's corrective action program.</p>																																							

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
Millstone Power Station - Unit 2	05000336	YEAR	SEQUENTIAL NUMBER	REV NO.	2 OF 2
		2011	-- 002 --	00	

NARRATIVE

1. Event Description

At 1152 on June 20, 2011, with Millstone Power Station Unit 2 operating at 59 percent power in Mode 1, the reactor tripped automatically on low steam generator (SG) water level 300 microseconds prior to a manual reactor trip initiated from the control room. The decreasing SG water level condition was due to a low suction pressure trip on the operating "B" steam generator main feedwater pump (SGFP) [SJ]. The "B" SGFP low suction pressure trip occurred while attempting to place the "A" steam generator main feedwater pump in service. Established operating procedures had recently been changed to permit starting the second SGFP at a higher power level than previously performed.

Auxiliary Feedwater (AFW) [BA] initiated as expected following loss of the operating SGFP. Standard post trip actions were carried out, and all other safety systems responded as expected.

The event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B), including the Reactor Protection System. The actuation of the Auxiliary Feedwater Actuation System also is a reportable condition under the same paragraph.

2. Cause

The cause of the event was gaps in the application of operator fundamentals and some procedure quality issues associated with operations procedure OP 2204, Load Changes.

3. Assessment of Safety Consequences

The safety consequences of this event were low. The reactor automatically tripped on low steam generator level. Steam generator levels remained in the visible range on narrow range SG level instrumentation. All other safety systems responded as expected.

There was no loss of decay heat removal capability, because main and auxiliary feedwater pumps were available to feed the steam generators. Neither departure from nucleate boiling nor fuel centerline melt design limits were challenged. As such, there were no challenges to the fuel, reactor coolant system or containment fission product barriers.

4. Corrective Action

Procedure OP 2204 has been revised to ensure the second steam generator feedwater pump is placed in service at a lower power level. Additional corrective actions to address the underlying causes of the gaps in the application of operator fundamentals are being addressed in accordance with the station's corrective action program.

5. Previous Occurrences

No previous similar events/conditions were identified.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].