

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

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TO: Mr. George Lear

FROM: Iowa Elec. Light & Power Co.
Cedar Rapids, Iowa
Lee LiuDATE OF DOCUMENT
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2-17-76☒ LETTER
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1

DESCRIPTION Ltr re our 12-19-75 ltr...furn info
on proposed program for the verification of
sensor response time for the reactor protection
system by 2-15-76.....

ENCLOSURE

PLANT NAME: Duane Arnold Plant

Do Not Remove

ACKNOWLEDGED

SAFETY

FOR ACTION/INFORMATION

ENVIRO

DHL 2-19-76

✓	ASSIGNED AD :		ASSIGNED AD :
✓	BRANCH CHIEF :	LEAR (6)	BRANCH CHIEF :
✓	PROJECT MANAGER:	PAULSON	PROJECT MANAGER :
✓	LIC. ASST. :	PARRISH	LIC. ASST. :

INTERNAL DISTRIBUTION

✓	REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	ENVIRO TECH
✓	NRC PDR	HEINEMAN	TEDESCO	ERNST
✓	I & E (2)	SCHROEDER	BENAROYA	BALLARD
✓	OELD		LAINAS	SPANGLER
	GOSSICK & STAFF	ENGINEERING	IPPOLITO	
	MIPC	MACCARY		SITE TECH
	CASE	KNIGHT	OPERATING REACTORS	GAMMILL
	HANAUER	SIHWEIL	STELLO	STEPP
	HARLESS	PAWLICKI		HULMAN
			OPERATING TECH	
	PROJECT MANAGEMENT	REACTOR SAFETY	✓ EISENHUT	SITE ANALYSIS
	BOYD	ROSS	✓ SHAO	VOLLMER
	P. COLLINS	NOVAK	✓ BAER	BUNCH
	HOUSTON	ROSZTOCZY	✓ SCHWENCER	J. COLLINS
	PETERSON	CHECK	✓ GRIMES	KREGER
	MELTZ			
	HELTEMES	AT & I	SITE SAFETY & ENVIRO	
	SKOVHOLT	SALTZMAN	ANALYSIS	
		RUTBERG	DENTON & MULLER	

EXTERNAL DISTRIBUTION

CONTROL NUMBER

✓	LPDR: Cedar Rapids, Iowa	NATL. LAB	BROOKHAVEN NATL LAB
✓	TIC	REG. V-IE	ULRIKSON(ORNL)
✓	NSIC	LA PDR	
✓	ASLB	CONSULTANTS	
✓	ACRS 16 HOLDING/SENT	To L.A.	

1525

IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office

CEDAR RAPIDS, IOWA

February 12, 1976

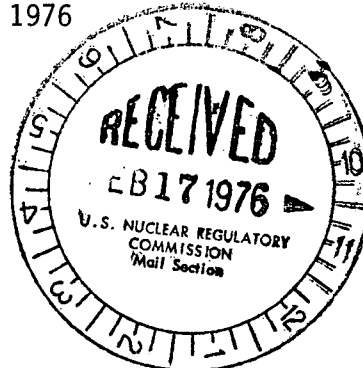
IE-76-246

50-331

LEE LIU

VICE PRESIDENT - ENGINEERING

Mr. George Lear, Chief
Operating Reactors Branch 3
Division of Operating Reactors
Nuclear Regulatory Commission
Washington, D.C. 20555



Dear Mr. Lear:

On December 19, 1975 you requested that we submit our proposed program for the verification of sensor response time for the reactor protection system by February 15, 1976.

We have been following the recent work done by EPRI and feel that it has now progressed sufficiently to define a program for pressure and differential pressure instrumentation. This program would include:

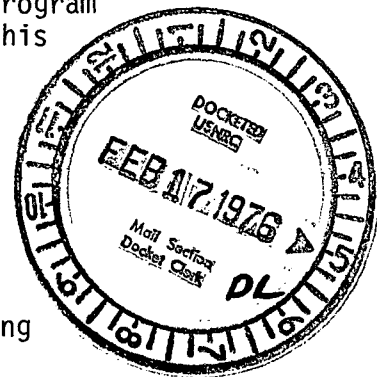
- a) Reactor Vessel Steam Dome Pressure - High
- b) Reactor Vessel Water Level - Low
- c) Turbine Control Valve Fast Closure.

We also believe that a position switch response time testing program can be defined shortly, which would include:

- a) Main Steam Line Isolation Valve Closure
- b) Turbine Stop Valve Closure.

We propose to submit this program to you no later than September 1, 1976.

At the present time active discussion between the NRC and the BWR Standardized Technical Specification group is proceeding to define the scope of the instrument response time testing program.



Mr. George Lear
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We are actively following these discussions. Further work through EPRI is progressing which we are also following. Over the next few months the scope of response time testing program will be further defined and further successful testing will occur. Iowa Electric will expand their program to include testing which has been adequately demonstrated and has been defined as a necessary portion of this program.

Very truly yours,



Lee Liu
Vice President, Engineering

LL/KAM/ms

cc: D. Arnold
W. Paulson (NRC)
J. Keppler (NRC)
J. Newman