



Monticello Nuclear Generating Plant
2807 W County Road 75
Monticello, MN 55362

August 23, 2011

L-MT-11-046
10 CFR 50.55a

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Monticello Nuclear Generating Plant
Docket 50-263
Renewed Facility Operating License No. DPR-22

Cycle 25 Inservice Inspection Summary Report

Pursuant to 10 CFR 50.55a, the Northern States Power Company – a Minnesota corporation (NSPM), d/b/a as Xcel Energy, is providing the enclosed Inservice Inspection (ISI) Summary Report (Owner's Activity Report, Form OAR-1) for the Monticello Nuclear Generating Plant (MNGP). Also enclosed within this submittal is a summary of the comprehensive in-vessel visual inspections (IVVI) of the reactor vessel internal components performed in accordance with the Boiling Water Reactor Vessel Internals Projects (BWRVIP), BWRVIP-94, "BWR Vessel and Internals Project Program Implementation Guide".

The Owner's Activity Report has been prepared in accordance with the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI Code Case N-532-4, "Repair / Replacement Activity Documentation Requirements and Inservice Summary Report Preparation and Submission, Section XI, Division 1". This code case is applied as an alternative to the requirements of ASME Boiler and Pressure Vessel Code, Section XI, Article IWA-4000 and IWA-6000, as approved by Regulatory Guide 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1, Revision 16".

This Owner's Activity Report is contained in Enclosure 1. It provides an overview of the inservice examination results performed during Cycle 25 (including the 2011 Refueling Outage) for the fourth 10-year ISI interval. The report also includes a summary of repair and replacement activities during the same time period. The applicable provisions of the ASME Code Case require that this report be submitted within 90 days from the end of the outage.

Additionally, the ISI Summary Report also provides amended information to the previously submitted Cycle 24 ISI Summary Report, including a supplemental NIS-1 Form and a supplemental NIS-2 Form.

The summary of IVVI inspections conducted in accordance with BWRVIP-94 is provided in Enclosure 2.

Should you have questions regarding this letter, please contact Mr. Richard Loeffler at (763) 295-1247.

Summary of Commitments

This letter proposes no new commitments and does not revise any existing commitments.



Timothy J. O'Connor
Site Vice President, Monticello Nuclear Generating Plant
Northern States Power Company – Minnesota

Enclosures: (2)

cc: Administrator, Region III, USNRC
Resident Inspector, Monticello, USNRC
Project Manager, Monticello, USNRC
BWRVIP Project Manager, USNRC

ENCLOSURE 1

**MONTICELLO NUCLEAR GENERATING PLANT
INSERVICE INSPECTION SUMMARY REPORT**

(11 pages follow)



Xcel Energy, Inc.
Northern States Power Co. - Minnesota
414 Nicollet Mall
Minneapolis, MN 55401

INSERVICE INSPECTION

SUMMARY REPORT

Monticello Nuclear Generating Plant Unit 1
2807 West County Road 75
Monticello, MN 55362

Commercial Service Date: June 30, 1971

Cycle No. 25 Refueling Outage

Outage Dates: March 5, 2011 to May 25, 2011

Inservice Inspection Interval 4, Period 3

Prepared By:

Richard Deopere
Program Engineer, ISI Program Owner

Reviewed By:

Thomas Jones
Program Engineer, NDE Level III

Approved By:

Paul Young
Supervisor, Materials Inspection & Repair

Approved By:

Gary Sherwood
Manager, Program Engineering

ANII:

Kurt Suleski
Hartford Steam Boiler - CT

Report Date: August 17, 2011

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SECTION 1.

SUMMARY

Page 1 of 1

In accordance with the ASME Section XI Code and alternative requirements of Code Case N-532-4, this report provides the following:

- Form OAR-1 , Owner's Activity Report
- Table 1, Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service
- Table 2, Abstract of Repair/Replacement Activities Required for Continued Service

This information is applicable to Inservice Inspection (ISI) activities performed during operating Cycle 25 at Monticello Nuclear Generating Plant (MNGP) Unit 1, including the 25th refueling outage. Cycle 25 began May 10, 2009, immediately following completion of the 24th refueling outage.

All examinations and tests for Cycle 25 were scheduled and conducted in accordance with the 4th Interval ISI Plan. Exams and tests were credited to the 2nd and 3rd Period of the 4th ISI Interval, as applicable.

This report also provides supplemental Repair/Replacement (R/R) information for the Cycle 24 ISI Summary Report submitted in August 2009. The 24th refueling outage began March 14, 2009 and concluded on May 10, 2009.

Monticello Nuclear Generating Plant
Inservice Inspection

Cycle 25 Summary Report
2011 Refueling Outage

SECTION 2.

FORM OAR-1

Page 1 of 1

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number MNGP 1R25 OAR-1

Plant Monticello Nuclear Generating Plant, 2807 West County Road 75, Monticello, Minnesota 55362

Unit No. 1 Commercial service date June 30, 1971 Refueling outage no. 1R25
(if applicable)

Current inspection interval 4th Inspection Interval
(1st, 2nd, 3rd, 4th, other)

Current inspection period 3rd Inspection Period
(1st, 2nd, 3rd)

Edition and addenda of Section XI applicable to the inspection plans 1995 Edition with 1996 Addenda

Date and revision of inspection plans Revision 4, dated 6/5/2010

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans _____
2001 Edition, No Addenda

Code Cases used: N-307-3, N-323-1, N-460, N-491-2, N-498-4, N-513-3, N-532-4, N-546, N-552, N-598, N-623, N-661,
N-686, N-702, N-705
(if applicable)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of 1R25 conform to the requirements of Section XI.
(refueling outage number)

Signed: [Signature] ISI Program Owner Date 8/17/2011
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Minnesota and employed by Hartford Steam Boiler Insp. & Insp. Company of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all the activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB11863 A.N.I. MN 004028-CO
Inspector's Signature National Board, State, Province, and Endorsements

Date August 17, 2011

SECTION 3.

Page 1 of 1

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED
EVALUATION FOR CONTINUED SERVICE

Examination Category and Item Number	Item Description	Evaluation Description
F-A/F1.20a	ISO#: ISI-13142-62 Comp. ID: H- 2, FPWH-111 Restraint Hanger	Engineering evaluation performed in conjunction with CAP AR# 1281071. Bent items, lack of thread engagement Determined to be acceptable "as-is".
F-A/F1.20c	ISO#: ISI-13142-42-A Comp. ID: H- 6, PSH-104 Spring / Clamp	Engineering evaluation performed in conjunction with CAP AR# 1262741, Bent rod and improper setting on hanger PSH-104. Determined to be acceptable "as-is".
F-A/F1.40a	ISO#: ISI-97005-C Comp. ID: H-10, RCAKR-22A Restraint	Engineering evaluation performed in conjunction with CAP AR# 1274943, Arc strike on restraint. Determined to be acceptable "as-is".
B-N-2/B13.40	Comp. ID: C-7x Shroud Support Legs H-10 Weld	Engineering evaluation performed in conjunction with CAP AR# s 1177677, 1282530, 1283709 and 1283715. Linear indications in shroud support leg welds. Determined to be acceptable "as-is".
B-N-2/B13.30	Comp. ID: C-3C, C-3D Shroud Shelf H-9 Weld	Engineering evaluation performed in conjunction with CAP AR# 1280490, H-8/H9 Weld Inspection, linear indications. Determined to be acceptable "as-is".
B-N-2/B13.40	Comp. ID: C-3E, C-3F Shroud Shelf H-8 Weld	Engineering evaluation performed in conjunction with CAP AR# 1280490, H-8/H9 Weld Inspection, linear indications. Determined to be acceptable "as-is".
C-H/C7.10	Comp. ID: T-200 Standby Liquid Control Tank	Engineering evaluation performed in conjunction with CAP AR# 1223696, Leak discovered on SLC Tank T-200 Determined to be acceptable for operation until the next scheduled refueling outage per Code Case N-705 and Relief Request 18. (repaired in 1R25)

CAP AR# - acronyms for "Corrective Action Program - Action Request Number"

SECTION 4.

Page 1 of 1

TABLE 2
ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
3	Division1 RHR Service Water Piping	Replaced piping and fittings in the Intake Structure and Tunnel due to internal corrosion. Previous use of Code Case N-661.	4/1/2011	2010-25-016
3	Division 2 Emergency Service Water Piping	Replaced piping and fittings in the Intake Structure due to internal corrosion. Previously repaired for through-wall leakage at weld. (Reference R/R Plan 2010-25-024)	3/27/2011	2010-25-017
3	Division 2 Emergency Service Water Piping	Repaired through-wall weld defect.	7/29/2010	2010-25-024
2	Standby Liquid Control Storage Tank T-200	Replaced bottom due to through-thickness cracking. Previous use of Code Case N-705 and Relief Request 18.	5/8/2011	2011-25-042
3	Division 2 RHR Service Water Piping	Linear surface indication identified near replacement weld, mechanically removed	3/21/2011	2011-25-043
2/MC	Primary Containment Penetration X-101A	Rounded surface indications near replacement weld, mechanically removed.	4/7/2011	2011-25-050, Rev.1
1	Main Steam D Piping	Linear surface indication identified at strain gauge removal area, buffed out.	4/22/2011	2011-25-057

SECTION 5. FORM NIS-1 INFORMATION - SUPPLEMENT

Page 1 of 2

Form NIS-1 Owner's Data Report for Inservice Inspections
(Supplement to Submittal for Cycle 24, 2009 RFO**)**
As required by the Provisions of the ASME Code Rules

-
-
1. **Owner:** Xcel Energy, Inc. / Northern States Power Company - Minnesota
Address: 414 Nicollet Mall, Mpls, MN 55401
2. **Plant:** Monticello Nuclear Generating Plant
Address: Monticello, MN 55362
3. **Plant Unit:** 1 4. **Owner (Certificate of Authorization):** N/A
5. **Commercial Service Date:** 6/30/71 6. **National Board No. for Unit:** N/A
7. **Components Inspected:**
- | <u>Component or Appurtenance</u> | <u>Manufacturer or Installer</u> | <u>Manufacturer or Installer Serial No.</u> | <u>State or Province No.</u> | <u>National Board No.</u> |
|----------------------------------|----------------------------------|---|------------------------------|---------------------------|
| --- | --- | --- | --- | --- |
8. **Examination Dates:** 04/28/2007 to 05/10/2009 .
9. **Inspection Interval:** 05/01/2003 to 05/31/2012 .
10. **Applicable Edition of Section XI:** 1995 Addenda: 1996 .
11. **Abstract of Examinations.**
- See Report Sections 5 and 6 for supplemental Cycle 24 Form NIS-2
12. **Abstract of Conditions Noted.**
- See Report Sections 5 and 6 for supplemental Cycle 24 Form NIS-2
13. **Abstract of Corrective Measures Recommended and Taken.**
- See Report Sections 5 and 6 for supplemental Cycle 24 Form NIS-2

SECTION 5. FORM NIS-1 INFORMATION - SUPPLEMENT

Page 2 of 2

(**Supplement to Submittal for Cycle 24, 2009 RFO**)

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Certification of Authorization No. (if applicable) N/A Expiration Date N/A

Date 8/17 20 11 Signed NORTHERN STATES POWER CO-MINNESOTA
dba. XCELENERGY, INC. By [Signature]
Owner Section XI ISI Coordinator
RICHARD A. DEOPERE

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of MINNESOTA and employed by HARTFORD STEAM BOILER INT OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 4/28/07 to 8/17/11, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 11863 A, N, I 1411004028-00
Inspector's Signature National Board, State, Province, Endorsements

Date AUGUST 17 2011

SECTION 6. FORM NIS-2 INFORMATION - SUPPLEMENT Page 1 of 3

(Supplement to Submittal for Cycle 24, 2009 RFO**)**

Cycle 24 NIS-2 Summary Addendum
and
Form NIS-2 for R/R 2009-24-057

R/R ITEM NUMBER	WORK DOCUMENT	COMPONENT OR SYSTEM	WORK DESCRIPTION
2009-24-057	WO-359192	AO-2387	Replaced 4 fasteners per EC-14182

(2 pages to follow)

Monticello Nuclear Generating Plant
Inservice Inspection

Cycle 25 Summary Report
2011 Refueling Outage

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY	Revision 5
		Page 1 of 2

Repair/Replacement Plan No.: 2009-24-057

- Owner NSP-Minnesota, an Xcel Energy Company Date 8/15/2011
Name
414 Nicollet Mall, Minneapolis, MN 55401 Sheet 1 of 2
Address
- Plant Monticello Nuclear Generating Plant Unit 1
Name
2807 West County Road 75, Monticello, MN 55362 WO-359192, EC-14182
Address Repair Organization P.O. No., Job No. etc.
- Work Performed By NSPM / Xcel Energy Type Code Symbol Stamp n/a
Company Name
2807 West County Rd. 75, Monticello, MN 55362 Authorization No. n/a
Address Expiration Date n/a
- Identification of System Primary Containment (PCT), AO-2387, Class 2
- (a) Applicable Construction Code B31.1, 1967E/No Add Edition / Addenda
(b) Applicable Edition of Section XI Used for Repair/Replacement Activity 2001 Edition, No Addenda
(c) Applicable Section XI Code Case(s) n/a
- Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed, or Installed	ASME Code Stamped	
							YES	NO
AO-2387 Studs	Fisher	n/a	n/a	n/a	1969	removed	<input type="checkbox"/>	<input checked="" type="checkbox"/>
AO-2387 Studs	Nova	n/a	n/a	n/a	2009	installed	<input type="checkbox"/>	<input checked="" type="checkbox"/>
AO-2387 Nuts	Fisher	n/a	n/a	n/a	1969	removed	<input type="checkbox"/>	<input checked="" type="checkbox"/>
AO-2387 Nuts	Nova	n/a	n/a	n/a	2009	installed	<input type="checkbox"/>	<input checked="" type="checkbox"/>
							<input type="checkbox"/>	<input type="checkbox"/>

- Description of Work Like for like replacement of (4) studs and (8) nuts
- Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐
Exempt ☐ Other ☐ Pressure _____ psi Test Temp _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8-1/2 in x 11 in, (2) information in Items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form. (See 4 AWI-09.04.03 [ASME Section XI Repair/Replacement Program] for further documentation guidance.)

Approval: PCR 01148046

M/asc

MONTICELLO NUCLEAR GENERATING PLANT		3613
TITLE:	FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY	Revision 5
		Page 2 of 2

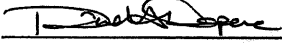
Repair/Replacement Plan No.: 2009-24-057

9. Remarks (Applicable Manufacturer's Data Reports to be Attached)

*Like for like replacement of studs and nuts in support of EC-14182 during a valve PM.
No PMT or PSI required for bolting replacement.*

CERTIFICATE OF COMPLIANCE

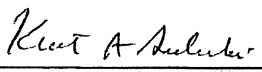
We certify that the statements made in the report are correct and that this conforms to the requirements of the ASME Code, Section XI.

Signed , ISI PROGRAM OWNER Date 8/16/2011
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Minnesota and employed by Hartford Steam Boiler Insp & Ins Company of Connecticut of Hartford, Connecticut have inspected the components described in this Owner's Report during the period 3/30/2010 to 8/16/2011, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer **SHALL** be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 11863 A.W. I MW 004028-CO
Inspector's Signature National Board, Province and Endorsements
Date 8/16/11

ENCLOSURE 2

**MONTICELLO NUCLEAR GENERATING PLANT
BWRVIP INSPECTION SUMMARY REPORT**

(9 pages follow)



Xcel Energy, Inc.
Northern States Power Co. - Minnesota
414 Nicollet Mall
Minneapolis, MN 55401

BWRVIP RFO25 IVVI

SUMMARY REPORT

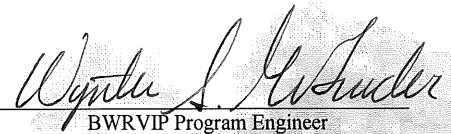
Monticello Nuclear Generating Plant Unit 1
2807 West County Road 75
Monticello, MN 55362

Commercial Service Date: June 30, 1971

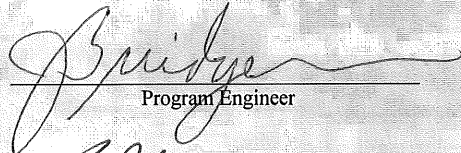
Cycle No. 25 Refueling Outage

Outage Dates: March 5, 2011 to May 25, 2011

Prepared By:


BWRVIP Program Engineer

Reviewed By:


Program Engineer

Approved By:


Supervisor, MIR

Approved By:


Manager, Program Engineering

Report Date: August 17, 2011

RFO25 IVVI BWRVIP Summary Report

Comprehensive in-vessel visual inspections (IVVI) of reactor vessel internal components were conducted to ensure continued integrity of the Reactor Pressure Vessel (RPV) internals. Additionally, enhanced visual inspections (EVT-1) were performed per recommendations set forth in Boiling Water Reactor Vessel Internals Projects (BWRVIP) guideline BWRVIP-03, "Reactor Pressure Vessel and Internals Examination Guidelines." The IVVI was performed in accordance with MNGP procedure PEI-02.05.05, "Visual Examination of Monticello Reactor Vessel Inspection," Revision 4, as directed by the MNGP ISI Program Owner and BWRVIP Program Owner. Examinations were performed by teams of Westinghouse camera handlers and IVVI Level II/III personnel performing the examinations from the refueling bridge.

The bottom head drain line inspection was performed by EPRI to meet BWRVIP requirements outlined in BWRVIP-205, "Bottom Head Drain Line Inspection and Evaluation Guidelines." The bottom head drain line UT was performed in accordance with FP-PE-NDE-427, "Ultrasonic testing of the Monticello Bottom Head Drain Line Using Remote Tooling."

Westinghouse Level III personnel performed independent review of all of the recorded IVVI examinations. Owner review of the recorded IVVI examinations was performed by MNGP ISI/IVVI personnel.

Examinations performed during RFO25 are listed in the following pages. The new indications and their resolutions can be found in the Table BWRVIP-1 below. The BWRVIP required inspections were performed in accordance with BWRVIP guidelines.

Table BWRVIP-1: RFO25 Indications

<u>Component</u>	<u>Indication/CAP</u>	<u>Resolution</u>
Guide Rod	Indication on the guide rod at 175 degree location. (CAP 01280462)	Indication evaluated and accepted for continued service through the next cycle in Condition Evaluation for CAP 01280462. Indication will be re-examined in next outage in 2013.
Shroud Support Leg	Indication discovered on H10 weld at 90 degree location. (CAP 01282530)	Indication evaluated and accepted for continued service through the next cycle in Engineering Change (EC) 18051. Indication will be re-examined in next outage in 2013.
Shroud Support Leg	Indication discovered on H10 weld at 330 degree location. (CAP 01283709)	Indication evaluated and accepted for continued service through the next cycle in EC 18051. Indication will be re-examined in next outage in 2013.
Shroud Support Leg	Indication discovered on H10 weld at 350 degree location. (CAP 01283715)	Indication evaluated and accepted for continued service through the next cycle in EC 18051. Indication will be re-examined in next outage in 2013.
Shroud Support Plate/Baffle Plate	Indications discovered on H8 and H9 welds on lower plenum side. (CAP 01280490)	Indications evaluated and accepted for continued service in EC 18051, EC 18095 and EC 18067. Indications will be re-examined in next outage in 2013.
N/A	Legacy FME/CAP 01280284	All FME removed from vessel.

IVVI Inspection Summary

Core Spray Piping

- EVT-1 examination of the CSP-P1-90, Thermal Sleeve to T-box (hidden weld).
- EVT-1 examination of the CSP-P1-270, Thermal Sleeve to T-box (hidden weld).
- EVT-1 examination of the CSP-P2-90, T-box to Cover Plate.
- EVT-1 examination of the CSP-P2-270, T-box to Cover Plate.
- EVT-1 examination of the P3a, b, c, d, T-box to Pipe welds at 80°, 100°, 260°, and 280°.
- EVT-1 examination of the P5, Downcomer to Sliding Sleeve welds at 13°, 166°, 193°, and 346°.
- EVT-1 examination of the P6, Sliding Sleeve to Outer Sleeve welds at 13°, 166°, 193°, and 346°.
- EVT-1 examination of the P7, Downcomer to Sliding Sleeve welds at 13°, 166°, 193°, and 346°.
- EVT-1 examination of the P8a, Pipe to Collar welds at 13°, 166°, 193°, and 346°.
- EVT-1 examination of the P8b, Collar to Shroud welds at 13°, 166°, 193°, and 346°.
- EVT-1 examination of the previous indication on P3A.
- VT-3 examination of the Piping Clamp Assemblies at 90° and 270°.

Core Spray Sparger

- EVT-1 examination of the CSS-S1-A and D, Sparger T-box Cover Plate at 13° and 346°.
- EVT-1 examination of the CSS-S2-A-1 and 2, Sparger T-box to Sparger Pipe at 11° and 15°.

Core Spray Sparger (Continued)

- EVT-1 examination of the CSS-S2-D-1 and 2, Sparger T-box to Sparger Pipe at 344° and 348°.
- VT-1 examination of the CSS-AS3A and B, Sparger Nozzles from 272° to 87°.
- VT-1 examination of the CSS-DS3A and B, Sparger Nozzles from 272° to 87°.
- VT-1 examination of the CSS-AS3C-1 and 2, Sparger Drain Plug welds.
- EVT-1 examination of the CSS-S4-A-1 and 2, Sparger End Cap welds at 87° and 272°.
- EVT-1 examination of the CSS-S4-D-1 and 2, Sparger End Cap welds at 87° and 272°.
- VT-1 examination of the CSS-SB-xxx, upper and lower Sparger Brackets at 13°, 54°, 87°, 272°, 306°, and 346°.

Top Guide Assembly

- VT-1 examination of the Top Guide Aligner Pins and Sockets at 72° and 162°.
- EVT-1 and VT-3 examination of Top Guide Cells 38-07, 34-35, 34-19, 30-39, 18-35, 18-19, 14-11, and 14-07.
- VT-3 examination of the Top Guide accessible areas.

Shroud Support Inspection

- EVT-1 examination of the H8 and H9 at 0° and 180°.
- VT-3 examination of previous indications on H10 at 10°, 30°, 60°, 120°, 150°, 170°, 190°, 210°, 240°, 270°, and 300°.
- VT-3 examination of accessible areas of the H8 and H9 above and below the Shroud Support Plate.
- VT-3 examination of accessible areas of the Shroud Support Plate.

Shroud Support Inspection (Continued)

- VT-3 examination of the Shroud Support Legs at 10°, 30°, 60°, 90°, 120°, 150°, 170°, 190°, 210°, 240°, 270°, 300°, 330°, and 350°.

Control Rod Guide Tubes

- VT-3 examination of Guide Tubes in cell locations 38-07, 34-35, 34-19, 30-39, 18-35, 18-19, 14-11, and 14-07.

Core Plate

- VT-3 examination of accessible Core Plate areas in cell locations 38-07, 34-35, 34-19, 30-39, 18-35, 18-19, 14-11, and 14-07.

Jet Pumps

- VT-1 examination of the WD-1 wedges on Jet Pumps 1 through 20.
- EVT-1 examination of the RS-1, RS-2, and RS-3, Riser welds on Jet Pumps 6 through 10.
- EVT-1 examinations of the RS-4 and RS-5 Riser Sleeve to Riser welds on Jet Pumps 14 through 20.
- EVT-1 examination of the RS-8, RS-9, RS-10, and RS-11 Riser Brace Yokes to Riser welds on Jet Pumps 1 through 20.
- EVT-1 examination of the MX-1, MX-2, and MX-4, Inlet Mixer welds on Jet Pumps 15 through 20.
- EVT-1 examination of the DF-1 and DF-2 Diffuser welds on Jet Pumps 16 through 20.
- EVT-1 examination of the AD-2, AD-3a, and AD-3b Diffuser Adapter welds on Jet Pumps 16 through 20.
- VT-1 examination of the Riser Brace Pad to Vessel attachment welds on Jet Pumps 1 through 20.

Dry Tubes

- VT-1 examination of Intermediate Range Monitors (IRMs) at core locations 20-33, 28-23, 20-41, 12-41, 36-41, 12-09, 28-25 and 20-25.
- VT-1 examination of Source Range Monitors (SRMs) at core locations 12-25, 28-17, 36-09 and 36-33.

RPV Attachments

- VT-3 examination of the Feedwater Sparger End Brackets and attachment welds at 0, 90°, 180°, and 270°.
- VT-3 examination of Surveillance Sample Holder Upper Brackets at 30°, 120°, and 300°.
- VT-3 examination of Surveillance Sample Holder 120°.
- VT-1 examination of Surveillance Sample Holder Lower Brackets at 30°, 120°, and 300°.
- VT-1 examination of the Guide Rod Bracket Vessel attachment welds at 175° and 355°.
- VT-3 examination of the Guide Rod Brackets at 175° and 355°.
- VT-3 examination of the Steam Dryer Support Bracket and weld at 35°, 145°, 215°, and 325°.

Bottom Head Drain Line

- UT of Bottom Head Drain Line REW31-2-ED.

RF025 Inspection Results

Core Spray Piping

- Examination of previously recorded indication on Core Spray Piping weld P3A at 90° showed no change.
- No new relevant indications.

Core Spray Sparger

- Examination of the Core Spray Spargers revealed no relevant indications.

Top Guide Assembly

- Examination of the Top Guide Assembly revealed no relevant indications.

Shroud Support Inspection

- Examination of previously recorded indications on H10 at 10°, 30°, 60°, 120°, 150°, 170°, 190°, 210°, 240°, 270°, and 300° showed no change.
- Examinations of previously recorded indications on H10 revealed new relevant indications at 90°.
- VT-3 examination of the Shroud Support Legs revealed new indications at 330° and 350°.
- During examinations of previously recorded indications on H10, new relevant indications were reported on the H-8 and H-9 welds on the underside of the Shroud Support Plate.

Control Rod Guide Tubes

- Examination of accessible Control Rod Guide Tubes revealed no relevant indications.

Core Plate

- Examination of accessible areas of the Core Plate revealed no relevant indications.

Jet Pumps

- Examination of previously recorded indication on Jet Pump 8 Restrainer Bracket showed no change.
- No new relevant indications.

Dry Tubes

- Examination of accessible SRM's and IRM's revealed no relevant indications.

RPV Attachments

- Examination of the 175° Guide Rod revealed a relevant indication.

Bottom Head Drain Line

- Examination of Bottom Head Drain Line REW31-2-ED revealed no relevant indications, results were acceptable.