

ENCLOSURE 1

**GLE LICENSE APPLICATION FRONT
MATTER, CHAPTER 1, AND CHAPTER 3,
PUBLIC VERSIONS**

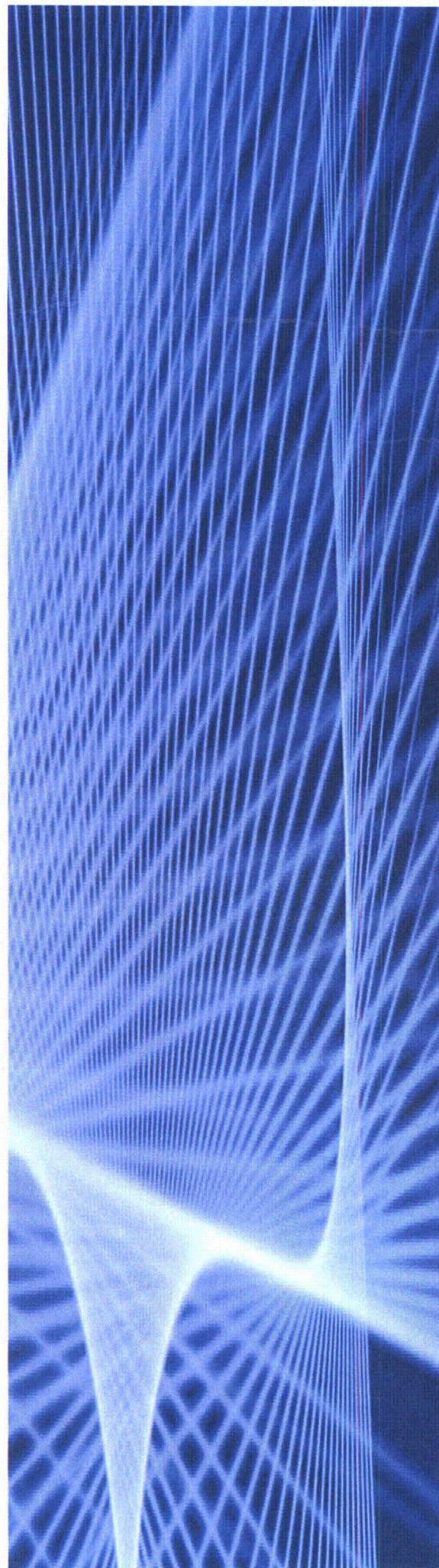
**GE-HITACHI GLOBAL
LASER ENRICHMENT LLC
COMMERCIAL FACILITY**

**WILMINGTON,
NORTH CAROLINA**

LICENSE APPLICATION

August 2011

Docket No. 70-7016



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LIST OF EFFECTIVE PAGES

Chapter	Revision No	Date of Revision	Revision Description
Front Matter	3	08/12/2011	Changes to Chapters 1, 3, and Definitions. Updated revision information for Chapters 2, 5, 8, and 11 to reflect current revisions of those chapters.
Chapter 1	5	08/12/2011	Revised Section 1.2.5.6 to update the definitions of Basic Component, Critical Characteristics, Dedication Process, Dedicating Entity to be consistent with the GLE QAPD. Revised Section 1.2.6 to clarify that the Facility Clearance has been granted to GLE (previously granted to GNFA). Revised portions of Section 1.3 to be consistent with the GLE ISA Summary.
Chapter 2	4	03/30/2011	Added bullet to Fire Safety Manager's responsibilities.
Chapter 3	5	08/12/2011	Revised use terminology consistent with the ISA Summary
Chapter 4	2	10/29/2010	Incorporate RAI from NRC letter dated Oct. 5, 2010, added commitments in Section 4.7.1 related to transportation requirements.
Chapter 5	5	08/01/2011	Added detail regarding likelihood of cylinder damage due to forklift breach followed by rain accumulation. Changed "activities with the potential to result in inadvertent nuclear criticality" to "fissile materials operations". Replaced minimum margin of subcriticality (MMS) with margin of subcriticality (MoS) throughout chapter to align with supporting validation report terminology and ANSI/ANS-8.24 (2007) national consensus standard

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Chapter	Revision No	Date of Revision	Revision Description
			guidance. Revised Section 5.4.4.2 to add clarity about crediting neutron absorbers. Revised Section 5.4.4.8 to add clarity about crediting neutron absorbers. Updated revision of reference 5-12.
Chapter 6	0	04/30/2009	Initial Application Submittal
Chapter 7	2	06/30/2010	Revised International Building Code Occupancy Classifications and added Solid Waste Storage Buildings discussion.
Chapter 8	1	03/30/2011	Revised to include commitment to 10 CFR 70.32 for changes to the RC&EP.
Chapter 9	1	10/29/2010	Incorporated RAI responses. Added description of cylinder pad stormwater monitoring. Added text and table describing mitigation measures.
Chapter 10	1	03/31/2010	Incorporate RAIs responses submitted to the NRC via MFN-09-802 dated 12/28/2009 and MFN-10-056 dated 02/10/2010.
Chapter 11	4	03/30/2011	Added language to clarify the application of the graded approach to applying management measures to IROFS. Replaced reference to deleted section of this chapter with GLE QAPD. Revised Section 11.1.4 to indicate that procedures control changes during design, construction and operations. Replaced the term "as built" with "as constructed".

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ACRONYMS

A/E	Architect/Engineering
ACP	American Centrifuge Plant
AEC	Active Engineered Control
AEGL	Acute Exposure Guideline Levels
AEP	Annual Exceedance Probability
ALARA	As Low As Reasonably Achievable
ALI	Annual Limit on Intake
ANS	American Nuclear Society
ANSI	American National Standards Institute
APF	Assigned Protection Factor
ASCE	American Society of Civil Engineers
AST	Autoclave Surge Tank
ASTM	American Society for Testing and Materials
BDC	Baseline Design Criteria
BOD	Biochemical Oxygen Demand
CAA	Controlled Access Area
CAAS	Criticality Accident Alarm System
CAP	Corrective Action Plan
CBA	Cost-Benefit Analysis
CDE	Committed Dose Equivalent
CEDE	Committed Effective Dose Equivalent
CEO	Chief Executive Officer
CFCM	Commercial Facility Project Manager
CFR	Code of Federal Regulations
CJHA	Chemical Job Hazards Analysis
CM	Configuration Management
CSA	Criticality Safety Analysis
CTPS	Cold Trap Purification System
CY	Calendar Year
DAC	Derived Air Concentration
DFP	Decommissioning Funding Plan
DOE	U.S. Department of Energy
DOT	U.S. Department of Transportation
DP	Decommissioning Plan
ECC	Emergency Control Center
ECF	Entry Control Facility
EDMS	Electronic Document Management System
EHS	Environmental, Health, and Safety
EMT	Emergency Medical Technician
EPA	U.S. Environmental Protection Agency
EPCRA	Emergency Planning and Community Right-to-Know Act
ER	Environmental Report

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ACRONYMS

ERO	Emergency Response Organization
ERT	Emergency Response Team
ETA	Event Tree Analysis
FCS	Facility Control System
FHA	Fire Hazards Analysis
FMO	Fuel Manufacturing Operation
FNMCP	Fundamental Nuclear Material Control Plan
FOCI	Foreign Ownership, Control, Influence
FPLTF	Final Process Lagoon Treatment Facility
FSRC	Facility Safety Review Committee
FTA	Fault Tree Analysis
FVC	Feed Vaporization Chamber
GE	General Electric Company
GEH	GE-Hitachi Nuclear Energy Americans LLC
GEMER	Geometry Enhanced MERIT
GET	General Employee Training
GLE	GE-Hitachi Global Laser Enrichment LLC
GNF-A	Global Nuclear Fuel – Americas, LLC
HAZOP	Hazards and Operability Analysis
HEGA	High-Efficiency Gas Absorption
HEPA	High-Efficiency Particulate Air
HEU	High-Enriched Uranium
HFCVB	Heated Flow Control Valve Box
HVAC	Heating, Ventilation, and Air Conditioning
IBC	International Building Code
ICEA	Industry Cabling Engineers Association, Inc.
ICRP	International Commission on Radiological Protection
IEEE	Institute of Electrical and Electronics Engineers
IFC	International Fire Code
IROFS	Items Relied on for Safety
ISA	Integrated Safety Analysis
ISAS	Integrated Safety Analysis Summary
ITM	Inspection, Testing, and Maintenance
JHA	Job Hazards Analysis
LA	License Application
LEL	Lower Explosive Limit
LES	Louisiana Energy Services, L.P.
LEU	Low Enriched Uranium
LLMW	Low-Level Mixed Waste
LLRW	Low-Level Radioactive Waste

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ACRONYMS

LTTS	Low Temperature Take-off Station
M&TE	Measuring and Test Equipment
MC&A	Material Control and Accounting
MCA	Moderation Controlled Area
MDC	Minimum Detectable Concentration
MCES	Monitored Central Exhaust System
MMS	Minimum Margin of Subcriticality
MOU	Memorandum of Understanding
MRA	Moderation Restricted Area
MSDS	Material Safety Data Sheet
MSW	Municipal Solid Waste
NC DAQ	North Carolina Division of Air Quality
NC DWQ	North Carolina Division of Water Quality
NCS	Nuclear Criticality Safety
NEF	National Enrichment Facility
NELAC	National Environmental Laboratory Accreditation Conference
NEPA	National Environmental Policy Act
NFPA	National Fire Protection Association
NIOSH	National Institute for Occupational Safety and Health
NIST	National Institute of Standards and Technology
NMSS	Nuclear Material Safety and Safeguards
NPDES	National Pollutant Discharge Elimination System
NPH	Natural Phenomena Hazard
NRC	U.S. Nuclear Regulatory Commission
NSI	Nuclear Safety Instruction
NSSL	National Severe Storms Laboratory
NUREG	NRC Publication
NVLAP	National Voluntary Laboratory Accreditation Program
OJT	On-the-Job Training
OSHA	Occupational Safety and Health Administration
OSTV	Onsite Transfer Vehicle
P&ID	Piping and Instrumentation Diagram
PHA	Process Hazards Analysis
PLC	Programmable Logic Controllers
PM	Preventive Maintenance
PMT	Post-Maintenance Testing
PNC	Potential Noncompliance
PPE	Personal Protective Equipment
PRA	Probabilistic Risk Assessment
PSP	Physical Security Plan

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QA	Quality Assurance
QAPD	Quality Assurance Program Description
QL	Quality Level
QRA	Quantitative Risk Assessment
RA	Response Agreements
RASCAL	Radiological Assessment System for Consequence Analysis
RC&EP	Radiological Contingency and Emergency Plan
RCA	Radiological Controlled Area
RCRA	Resource Conservation and Recovery Act
RD	Restricted Data
RLETS	Radiological Liquid Effluent Treatment System
RM	Records Management
RP	Radiation Protection
RSA	Radiological Safety Assessments
RSC	Radiation Safety Committee
RWP	Radiation Work Permit
SCA	Sample Containment Autoclave
SEM	Standard Error of Measurements
SFS	Solid Feed Station
SNM	Special Nuclear Material
SPPP	Standard Practice Procedures Plan
SRD	Secret Restricted Data
SSC	System, Structure, and Component
SSLCB	Single-Sided Lower Confidence Band
SSLTB	Single-Sided Lower Tolerance Band
SSLTL	Single-Sided Lower Tolerance Limit
SWU	Separative Work Unit
TEDE	Total Effective Dose Equivalent
TLD	Thermo Luminescent Dosimeters
TSDF	Treatment, Storage, and Disposal Facility
UBC	Uniform Building Code
UIR	Unusual Incident Report
UL	Underwriters Laboratory
UNC-W	University of North Carolina – Wilmington
U.S.	United States
USEC	United States Enrichment Corporation, Inc.
USGS	U.S. Geological Survey
USL	Upper Subcritical Limit
VRCT	Volume Reducing Compressor Train

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ACRONYMS

WFPP
WWTF

Wilmington Fire Protection Program
Waste Water Treatment Facility

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CHEMICALS AND UNITS OF MEASURE

⁴⁰ K	potassium-40
⁹⁹ Tc	technetium-99
²²² Rn	radon-222
²²⁶ Ra	radium-226
²³² Th	thorium-232
²³⁵ U	uranium-235
²³⁸ U	uranium-238 (depleted ²³⁵ U)
°F	Fahrenheit
ADU	ammonium diuranate
bgs	below ground surface
Bq	Becquerel
cc	cubic centimeters
CFC	chlorofluorocarbon
Ci	curie
cm	centimeter
cm ²	square centimeters
CO	carbon monoxide
CO ₂	carbon dioxide
cP	continental polar
dBa	a-weighted decibels
DCE	cis-1,2 dichloroethylene
dpm	disintegrations per minute
ft	foot
ft ²	square foot
g	gram
gal	gallon
gpd	gallons per day
gpm	gallons per minute
GWe	gigawatt electrical
ha	hectare
HF	hydrogen fluoride

CHEMICALS AND UNITS OF MEASURE

hz	hertz
in	inches
kg	kilogram
km	kilometers
kts	knots
lb	pound
L _{DN}	day-night average sound levels
Lpd	liters per day
m	meter
m ²	square meter
Mg	megagram
mg	milligram
mm	millimeter
mph	miles per hour
mrem	millirem
mrem/yr	millirem per year
msl	mean sea level
mSv	millisievert
mSv/yr	millisievert per year
mT	maritime tropical
MWe	megawatt electrical
NO ₂	nitrous oxide
O ₃	ozone
Pb	lead
pCi	picocurie
PM	particulate matter
PM ₁₀	particulate matter with aerodynamic diameter of 10 µm or less
PM ₂₅	particulate matter with aerodynamic diameter of 2.5 µm or less
ppm	parts per million
psi	pound per square inch
PU	Plutonium

CHEMICALS AND UNITS OF MEASURE

scfph	standard cubic feet per hour
sL/m	standard liters per minute
SO ₂	sulfur dioxide
Sv	sieverts
TCE	trichloroethylene
TSP	total suspended particulates
TSS	total suspended solids
U ₃ O ₈	triuranium octaoxide
UF ₄	uranium tetrafluoride
UF ₆	uranium hexafluoride
UO ₂	uranium dioxide
UO ₂ F ₂	uranyl fluoride
μCi	micocuries
μm	micrometer
VC	vinyl chloride
wt	weight
yd ³	cubic yard
yr	year

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100-Year Flood – A flood elevation (for a given area) that has a 1 percent chance of being equaled or exceeded each year. Thus, the 100-year flood could occur more than once in a relatively short period of time. The 100-year flood, which is the standard used by most federal and state agencies, is used by the National Flood Insurance Program (NFIP) as the standard for floodplain management and to determine the need for flood insurance. The term 100-year flood is synonymous with the one percent annual chance flood. [FEMA]

500-Year Flood – Refers to the flood elevation for a given area that has a 0.2 percent chance of being equaled or exceeded each year. This term is synonymous with the 0.2 percent annual chance of flood. [FEMA]

Absorbed Dose – The energy imparted by ionizing radiation per unit mass of irradiated material. [10 CFR 20.1003]

Accident Sequence – An unintended sequence of events that, given the failure of certain items relied on for safety (IROFS) identified in the sequence, would result in environmental contamination, radiation exposure, release of radioactive material, inadvertent nuclear criticality, or exposure to hazardous chemicals (provided that the chemicals are produced from licensed radioactive material). The term “accident” may be used interchangeably with “accident sequence.” [NUREG-1520]

Act – The Atomic Energy Act of 1954 (68 Stat 919), including any amendments thereto. [10 CFR 70.4]

Active Engineered Control (AEC) – A physical device that uses active sensors, electrical components, or moving parts to maintain safe process conditions without any required human action. [NUREG-1520]

Administrative Control – Either an augmented administrative control or a simple administrative control. [NUREG-1520]

Airborne Radioactive Material – Radioactive material dispersed in the air in the form of dusts, fumes, particulates, mists, vapors, or gases. [10 CFR 20.1003]

Airborne Radioactivity Area – A room, enclosure, or area in which airborne radioactive materials, composed wholly or partly of licensed material, exist in concentrations in excess of the derived air concentrations (DACs) specified in 10 CFR 20.1001 through 20.2401, Appendix B; or to such a degree that an individual present in the area without respiratory protective equipment could exceed, during the hours an individual is present in a week, an intake of 0.6 percent of the annual limit on intake (ALI) or 12 DAC-hours. [10 CFR 20.1003]

Alert – Events may occur, are in progress, or have occurred that could lead to a release of radioactive material(s) but that the release is not expected to require a response by an offsite response organization to protect persons offsite. [10 CFR 70.4]

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Annual Limit on Intake (ALI) – The derived limit for the amount of radioactive material taken into the body of an adult worker by inhalation or ingestion in a year. ALI is the smaller value of intake of a given radionuclide in a year by the reference man that would result in a committed effective dose equivalent of five rems (0.05 Sv) or a committed dose equivalent of 50 rems (0.5 Sv) to any individual organ or tissue. (ALI values for intake by ingestion or inhalation of selected radionuclides are given in 10 CFR 20.1001 through 10 CFR 20-2401, Appendix B, Table 1, Columns 1 and 2. [10 CFR 20.1003]

Area Manager – Individual responsible for implementation of nuclear safety requirements in an assigned area. The generic title “Area Manager” does not necessarily refer to the title of any specific position in the GLE organization or position nomenclature.

Area of Environmental Concern – Designated by the North Costal Resources Commission within 20 North Carolina counties as areas of natural importance that may be easily destroyed by erosion or floodwater or may have environmental, social, economic, or aesthetic values to the state. [GLE ER]

As Low As Reasonably Achievable (ALARA) – Making every reasonable effort to maintain exposures to radiation as far below the dose limits in 10 CFR 20 as is practical, consistent with the purpose for which the licensed activity is undertaken, taking into account the state of technology, the economics of improvements in relation to state of technology, the economics of improvements in relation to benefits to the public health and safety, and other societal and socioeconomic considerations, and in relation to utilization of nuclear energy and licensed materials in the public interest. [10 CFR 20.1003]

Assessments – An assessment is used to determine the effectiveness of activities in achieving applicant-specified objectives that provide reasonable assurance of the continued availability and reliability of IROFS. [NUREG-1520]

Assigned Protection Factor (APF) – The expected workplace level of respiratory protection that would be provided by a properly functioning respirator or a class of respirators to properly fitted and trained users. Operationally, the inhaled concentration can be estimated by dividing the ambient airborne concentration by the APF. [10 CFR 20.1003]

Audits – An audit is used to monitor compliance with regulatory requirements and license commitments. [NUREG-1520] A planned and documented activity performed to determine by investigation, examination, or evaluation of objective evidence the adequacy of and compliance with established procedures, instructions, drawings, and other applicable documents, and the effectiveness of implementation. An audit should not be confused with self-assessment, surveillance, and inspection activities performed for the purpose of process control or product acceptance. [ANSI NQA-1-1989]

Augmented Administrative Control – A procedurally required or prohibited human action, combined with a physical device that alerts the operator that the action is needed to maintain safe process conditions, or otherwise adds substantial assurance of the required human performance. [NUREG-1520]

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Available and Reliable to Perform Their Function When Needed – Based on the analyzed, credible conditions in the integrated safety analysis (ISA), items relied on for safety (IROFS) will perform their intended safety function when needed, and management measures will be implemented that ensure compliance with the performance requirements of 10 CFR 70.61, considering factors such as necessary maintenance, operating limits, common-cause failures, and the likelihood and consequences of failure or degradation of the times and measures. [10 CFR 70.4]

Background Radiation – Radiation from cosmic sources; naturally occurring radioactive material, including radon (except as a decay product of source or special nuclear material; and global fallout as it exists in the environment from the testing of nuclear explosive devices or from past nuclear accidents such as Chernobyl, that contribute to background radiation and are not under the control of the licensee. “Background Radiation” does not include radiation from source, byproduct, or special nuclear materials regulated by the U.S. Nuclear Regulatory Commission. [10 CFR 20.1003]

Baseline Design Criteria – A set of criteria specifying design features and management measures that are required and acceptable under certain conditions for new processes or facilities specified in 10 CFR 70.64. In general, these criteria are the acceptance criteria that apply to safety design for new facilities and new processes. [NUREG-1520]

Basic Component – A structure, system, or component (SSC), or part thereof, designated as an item relied on for safety (IROFS) identified as QL-1 or QL-2, that affects the IROFS function, that is directly procured by the licensee of a facility or activity subject to the regulations in 10 CFR 70 and in which a defect or failure to comply with any applicable regulation in 10 CFR 70, order, or license issued by the U.S. Nuclear Regulatory Commission (NRC) could create a substantial safety hazard (i.e., exceed the performance requirements of 10 CFR 70.61). Basic components include QL-1 and QL-2 identified IROFS-related design, analysis, inspection, testing, fabrication, replacement of parts, or consulting services that are associated with the component hardware, whether these services are performed by the component supplier or others.

When applied to IROFS identified as QL-NFPA, a basic component is a SSC, or part thereof, that affects the safety function of the IROFS that is directly procured by the licensee or a facility or activity subject to the requirements of the National Fire Protection Administration (NFPA) Code of Record, and in which a defect or failure to comply with requirements of the NFPA Code of Record could create a substantial safety hazard that cannot be addressed by compensatory measures as allowed by the NFPA Code of Record. Basic component includes QL-NFPA identified IROFS-related design, analysis, inspection, testing, fabrication, replacement of parts, or consulting services that are associated with the component hardware, whether these services are performed by the component supplier or others, to the extent required by the NFPA Code of Record.

Bias – The systematic difference between calculated results and experimentally measured values of k_{eff} for a fissile system.

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Bias Uncertainty – The integrated uncertainty in experimental data, calculational methods, and models, estimated by a valid statistical analysis of calculated k_{eff} values for critical experiments.

Bioassay (Radiobioassay) – The determination of kinds, quantities or concentrations, and in some cases, the locations of radioactive material in the human body, whether by direct measurement (in vivo counting) or by analysis and evaluation of materials excreted or removed from the human body. [10 CFR 20.1003]

Closed Security Area – Designated Controlled Access Areas that are established to safeguard classified material. Typically the classified material in these areas, because of its size, nature, and operational necessity, cannot be adequately protected during work hours by normal safeguards or stored during non-working hours.

Collective Dose – The sum of the individual doses received in a given period of time by a specified population from exposure to a specified source of radiation. [10 CFR 20.1003]

Commencement of Construction – Any clearing of land, excavation, or other substantial action that would adversely affect the natural environment of a site but does not include changes desirable for the temporary use of the land for public recreational uses, necessary borings to determine site characteristics or other preconstruction monitoring to establish background information related to the suitability of a site or to the protection of environmental values. [10 CFR 70.4]

Commercial-Grade Item – A structure, system, or component (SSC), or part thereof that affects its **QL-1 and/or QL-2 identified** IROFS function, which **is** not designed and manufactured as a **Basic Component**. Commercial-grade items do not include items where the design and manufacturing process require in-process inspections and verifications to ensure that defect or failures to comply are identified and corrected (i.e., one or more critical characteristics of the item cannot be verified.)

When applied to items identified as **QL-NFPA** (being items in facilities and activities licensed pursuant to 10 CFR 70), commercial grade item means an item that is (1) not subject to design or specification requirements that are unique to facilities or activities; (2) used in applications other than those facilities and activities; and (3) to be ordered from the manufacturer/supplier on the basis of specifications set forth in the manufacture's published product description.

Configuration Management (CM) – A management measure that provides oversight and control of design information, safety information, and records of modifications (both temporary and permanent) that might impact the ability of items relied on for safety to perform their functions when needed. [10 CFR 70.4]

Consequence – Any result of interest caused by an event or sequence of events. In this context, "adverse consequence" refers to adverse health or safety effects on either workers, the public, or the environment. [NUREG-1520]

Constraint – A value above which specified licensee actions are required. [10 CFR 20.1003]

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Contractor Personnel – All persons who are not GLE/GEH/GNF employees, active pensioners, or variable workers. Contract Workers have been contracted to provide a service or activity for GE.

Controlled Area – An area, outside of a restricted area but inside the site boundary, access to which can be limited by the licensee for any reason. [10 CFR 20.1003]

Controlled Parameter – A measurable parameter that is maintained within a specified range by one or more specific controls to ensure the safety of an operation. [NUREG-1520]

Corrective Action – A measure taken to rectify significant conditions adverse to quality and to preclude repetition. [ANSI/ASME NQA-1]

Critical Characteristics – Those important to design, material, and performance characteristics of a commercial-grade item that, once verified, will provide reasonable assurance that the item will perform its intended **QL-1 and/or QL-2 identified** IROFS function.

When applied to items identified as QL-NFPA, critical characteristics are those important to design, material, and performance characteristics of a commercial grade item that will provide reasonable assurance that the item will perform its intended QL-NFPA identified IROFS function.

Critical Mass of Special Nuclear Material – Special nuclear material in a quantity exceeding 700 grams of contained ^{235}U ; 520 grams of ^{233}U ; 450 grams of plutonium; 1500 grams of contained ^{235}U ; if no uranium enriched to more than four percent by weight of ^{235}U is present; 450 grams of any combination thereof; or one-half such quantities if massive moderators or reflectors made of graphite, heavy water, or beryllium may be present. [10 CFR 70.4]

Declared Pregnant Woman – A woman who has voluntarily informed the licensee, in writing, of her pregnancy and the estimated date of conception. The declaration remains in effect until the declared pregnant woman withdraws the declaration in writing or is no longer pregnant. [10 CFR 20.1003]

Decommission – To remove a facility or site safety from service and reduce residual radioactivity to a level that permits: (1) release of the property for unrestricted use and termination of the license; or (2) release of the property under restricted conditions and termination of the license. [10 CFR 70.4]

Dedication Process – An acceptance process undertaken to provide reasonable assurance that a commercial-grade item to be used as a basic component will perform its intended **QL-1 and/or QL-2** item relied on for safety (IROFS) function and, in this respect, is deemed equivalent to an item designed and manufactured under **QL-1 or QL-2 requirements in accordance with the GLE QAPD**. This assurance is achieved by identifying the critical characteristics of the item and verifying their acceptability by inspections, tests, or analyses performed by the purchaser or third-party dedicating entity after delivery, supplemented as necessary by one or more of the following: commercial grade surveys; product inspections or witness at holdpoints at the

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manufacturer's facility, and analysis of historical records for acceptable performance. In all cases, the dedication process must be conducted in accordance with the applicable provisions of **the GLE QAPD**. The process is considered complete when the item is designated for use as a basic component **applicable to QL-1 and/or QL-2 IROFS**.

When applied to items identified as QL-NFPA (being items in facilities and activities licensed pursuant to 10 CFR 70), the dedication process is applied to commercial-grade items to be used as basic components to provide reasonable assurance that they will perform their intended QL-NFPA identified IROFS function and are deemed equivalent to an item designed and manufactured under QL-NFPA requirements in accordance with the GLE QAPD. This assurance is achieved by confirming that the commercial-grade item is manufactured to established, acceptable national codes or standards that include one or more independent product endorsement based on qualification testing or periodic testing of selected characteristics of the item except in cases where such listing/approval is not required by codes and standards. In all cases, the applicable provisions of the GLE QAPD will be used to conduct the dedication process. The process is considered complete when the commercial-grade item is designated as a basic component.

Dedicating Entity – The organization that performs the dedication process **for QL-1 and QL-2 identified IROFS**. Dedication may be performed by the manufacturer of the item, a third-party dedicating entity, or the licensee itself. The dedicating entity, pursuant to 10 CFR 21.21(c), *Notification of Failure to Comply or Existence of a Defect and its Evaluation*, is responsible for identifying and evaluating deviations, reporting defects and failure to comply for the dedicated item, and maintaining auditable records of the dedication process. In cases where the Licensee applies the commercial-grade item procurement strategy and performs the dedication process, the Licensee would assume full responsibility as the dedicating entity.

When applied to items identified as QL-NFPA (being items in facilities and activities licensed pursuant to 10 CFR 70), the dedicating entity is the licensee. The licensee, pursuant to 10 CFR 21.21(c), is responsible for reporting defects and failure to comply for the dedicated item, maintaining auditable records of the dedication process, and assumes full responsibility as the dedicating entity.

Derived Air Concentration (DAC) – The concentration of a given radionuclide in air which, if breathed by the reference man for a working year of 2,000 hours under conditions of light work (inhalation Rate 1.2 cubic meters of air per hour), results in an intake of one ALI. DAC values are given in 10 CFR 20.1001 through 20.2401, Appendix B, Table 1, Column 3. [10 CFR 20.1003]

Derived Air Concentration-Hour (DAC-Hour) – The product of the concentration of radioactive material in air (expressed as a fraction or multiple of the DAC for each radionuclide) and the time of exposure to that radionuclide, in hours. A licensee may take 2,000 DAC-hours to represent one ALI, equivalent to a committed effective dose equivalent of five rems (0.05 Sv). [10 CFR 20.1003]

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Double Contingency Principle – Process designs should incorporate sufficient factors of safety to require at least two unlikely, independent, and concurrent changes in process conditions before a criticality accident is possible. [10 CFR 70.4]

Double Contingency Protection – A characteristic or attribute of a process that has incorporated sufficient safety factors to that at least two unlikely, independent, and concurrent changes in process conditions are required before a nuclear criticality accident is possible. [NUREG-1520]

Effective Dose Equivalent – The sum of the products of the dose equivalent to the body organ or tissue and the weighting factors applicable to each of the body organs or tissues that are irradiated. Weighting factors are: 0.25 for gonads; 0.15 for breast; 0.12 for red bone marrow; 0.12 for lungs, 0.03 for thyroid; 0.03 for bone surface, and 0.06 for each of the other five organs receiving the highest dose equivalent. [10 CFR 70.4]

Effective Kilograms of Special Nuclear Material – (1) For plutonium and ^{233}U , their weight in kilograms; (2) For uranium with an enrichment in the isotope ^{235}U of 0.01 (one percent) and above, its element weight in kilograms multiplied by the square of its enrichment expressed as a decimal weight fraction; and (3) For uranium with an enrichment in the isotope ^{235}U below 0.01 (one percent), by its element weight in kilograms multiplied by 0.0001. [10 CFR 70.4]

Engineered Control – See active engineered control or a passive engineered control. [NUREG-1520]

Entrance or Access Point – Any location through which an individual could gain access to radiation areas or to radioactive materials. This includes entry or exit portals of sufficient size to permit human entry, irrespective of their intended use. [10 CFR 20.1003]

Exposure – Being exposed to ionizing radiation or to radioactive materials. [10 CFR 20.1003]

External Dose – The portion of the dose equivalent received from radiation sources outside the body. [10 CFR 20.1003]

External Event – An event for which the likelihood cannot be altered by changes to the regulated facility or its operation. This would include all natural phenomena events, plus airplane crashes, explosions, toxic releases, fires, etc., occurring near or on the plant site. [NUREG-1520]

GLE Commercial Facility – The structures, systems, and components that comprise the GLE Site infrastructure established to support the enrichment processing and support operations. The GLE Commercial Facility includes the Operations Building, multiple administrative and support buildings or areas, a parking lot, retention basins, cylinder storage pads, and connecting roadways. A cleared security buffer surrounds the entire GLE Commercial Facility and defines both the Restricted Area and the Protected Area of the facility.

GLE Site – The approximate 100 acres of land upon which the GLE Commercial Facility is built.

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GLE Study Area – The area of the Wilmington Site evaluated in the GLE Environmental Report which includes the GLE Site as well as additional land surrounding the GLE Site.

Hazardous Chemicals Produced from Licensed Materials – Substances having licensed material as precursor compound(s) or substances that physically or chemically interact with licensed materials; and that are toxic, explosive, flammable, corrosive, or reactive to the extent that they can endanger life or health if not adequately controlled. These include substances commingled with licensed material, and include substances such as hydrogen fluoride that is produced by the reactor of uranium hexafluoride and water, but do not include substances prior to process addition to licensed material or after process separation from licensed material. [10 CFR 70.4]

High Radiation Area – An area, accessible to individuals, in which radiation levels from radiation sources external to the body could result in an individual receiving a dose equivalent in excess of 0.1 rem (1 mSv) in one hour at 30 centimeters from the radiation source or 30 centimeters from any surface that the radiation penetrates. [10 CFR 20.1003]

Individual Monitoring – (1) The assessment of dose equivalent by the use of devices designed to be worn by an individual; (2) The assessment of committed effective dose equivalent by bioassay or by determination of the time-weighted air concentrations to which an individual has been exposed; or (3) The assessment of dose equivalent by the use of survey data. [10 CFR 20.1003]

Individual Monitoring Devices – Devices designed to be worn by a single individual for the assessment of dose equivalent such as film badges, thermo luminescence dosimeters (TLDs), pocket ionization chambers, and personal ("lapel") air sampling devices. [10 CFR 20.1003]

Integrated Safety Analysis (ISA) – A systematic analysis to identify facility and external hazards and their potential for initiating accident sequences, the potential accident sequences, their likelihood and consequences, and the IROFS. As used here, integrated means joint consideration of, and protection from, all relevant hazards, including radiological, nuclear criticality, fire, and chemical. However, with respect to compliance with the regulations of 10 CFR 70, the NRC requirement is limited to consideration of the effects of all relevant hazards on radiological safety, prevention of nuclear criticality accidents, or chemical hazards directly associated with NRC licensed radioactive material. An ISA can be performed process by process, but all processes must be integrated, and process interactions considered. [10 CFR 70.4]

Integrated Safety Analysis (ISA) Summary – A document or documents submitted with the license application, license amendment application, license renewal application, or pursuant to 10 CFR 70.62(c)(3)(ii) that provides a synopsis of the results of the integrated safety analysis and contains the information specified in 10 CFR 70.65(b). The ISA Summary can be submitted as one document for the entire facility, or as multiple documents that cover all portions and processes of the facility. [10 CFR 70.4]

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Internal Dose – The portion of the dose equivalent received from radioactive material taken into the body. [10 CFR 20.1003]

Items Relied on for Safety (IROFS) – Structures, systems, equipment, components, and activities of personnel that are relied on to prevent potential accidents at a facility that could exceed the performance requirements in 10 CFR 70.61 or to mitigate their potential consequences. This does not limit the licensee from identifying additional structures, systems, equipment, components, or activities of personnel (i.e., beyond those in the minimum set necessary for compliance with the performance requirements) as items relied on for safety. [10 CFR 70.4]

IROFS Boundary Definition Package – IROFS boundary definition packages are documents that contain the physical descriptions and parameters of SSCs which are used to meet the performance requirements of 10 CFR 70.61. IROFS Boundary Definition Packages are also prepared for administrative procedures or workers actions which are defined as IROFS. The boundary packages also identify the facility areas in which the IROFS is used, design and functional attributes, management measures, any open items, and supporting documentation (i.e. P&IDs, schematics, etc.). Open items that affect the reliability and/or effectiveness of the IROFS should be closed prior to the NRC Operational Readiness Review (ORR). The open items section should identify open items associated with the IROFS during the NRC License Review and describe how the open items were resolved. [NUREG-1520, Rev 1]

ISA Baseline Documents – Includes technical reports, Process Hazard Analyses, Quantitative Risk Analyses (QRAs), calculations, drawings, white papers, IROFS Boundary Definition Packages, and memos or notes to file that capture the ISA.

Licensed Material – Source material, special nuclear material, or byproduct material received, possessed, used, transferred, or disposed of under a general or specific license issued by the U.S. Nuclear Regulatory Commission. [10 CFR 20.1003]

Licensee – Holder of a license from the U.S. Nuclear Regulatory Commission. [10 CFR 20.1003]

Limits – The permissible upper bounds of radiation doses. [10 CFR 20.1003]

Line Management – Managers who are charged with the administration of a group of people having a common organizational function. Line Managers are responsible for the assigned organization's output.

Management Measures – The functions performed by the licensee, generally on a continuing basis, that are applied to items relied on for safety, to ensure the items are available and reliable to perform their functions when needed. Management measures include Configuration Management, Maintenance, Training and Qualifications, Procedures, Audits and Assessments, Incident Investigations, Records Management, and other Quality Assurance elements. [10 CFR 70.4]

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Member of the Public – Any individual except when that individual is receiving an occupational dose. [10 CFR 20.1003]

Minimum Margin of Subcriticality (MMS) – An allowance for any unknown (or difficult to identify or quantify) errors or uncertainties in the method of calculating k_{eff} , that may exist beyond those which have been accounted for explicitly in calculating bias and bias uncertainty.

Mitigative Control – A control intended to reduce the consequence of an accident sequence, not to prevent it. When a mitigative control works as intended, the results of the sequence are called the mitigated consequences. [NUREG-1520]

Monitoring (Radiation Monitoring) – The measurement of radiation levels, concentrations, surface area concentrations or quantities of radioactive material and the use of the results of these measurements to evaluate potential exposures and doses. [10 CFR 20.1003]

Natural Phenomena Event – Earthquakes, floods, tornadoes, tsunamis, hurricanes, and other events that occur in the natural environment and could adversely affect safety. Natural phenomena events may be credible or incredible, depending on their likelihood of occurrence. [NUREG-1520]

Nuclear Criticality Safety (NCS) Control – A fixed physical design feature, active device, or procedure that is implemented to maintain safe process conditions. NCS controls are preventive and may be passive engineered, active engineered, or administrative (procedural). The NCS controls that are necessary to maintain the system subcritical under normal and credible abnormal conditions and achieve an overall likelihood of less than or equal to 10^{-5} per year (per event), are declared as IROFS in the ISA Summary.

New Processes at Existing Facilities – Systems-level or facility-level design changes to processes equipment, process technology, facility layout, or types of licensed material possessed or used. Generally, this definition does not include component-level design changes or equipment replacement. [NUREG-1520]

Occupational Dose – The dose received by an individual in the course of employment in which the individual's assigned duties involve exposure to radiation or to radioactive material from licensed and unlicensed sources of radiation, whether in the possession of the licensee or other person. Occupational dose does not include doses received from background radiation, from any medical administration the individual has received, from exposure to the individuals administered radioactive material and released under 10 CFR 35.75, from voluntary participation in medical research programs, or as a member of the public. [10 CFR 20.1003]

Out-of-Specification Cylinder – A cylinder that contains material that is outside of a design specification parameter or cylinder design parameters (capacity, volume, enrichment, wall thickness, etc.)

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Over-Filled Cylinder – A cylinder that contains more than the design capacity/volume of material.

Passive Engineered Control – A device that uses only fixed physical design features to maintain safe process conditions without any required human action. Assurance is maintained through specific periodic inspections or verification measurement(s), as appropriate. [NUREG-1520]

Plutonium Processing and Fuel Fabrication Plant – A plant in which the following operations or activities are conducted: (1) Operations for manufacture of reactor fuel containing plutonium including any of the following: (i) preparation of fuel material; (ii) formation of fuel material into desired shapes; (iii) application of protective cladding; (iv) recovery of scrap material; and (v) storage associated with such operations; or (2) Research and development activities involving any of the operations described in Paragraph (1) of this definition except for research and development activities utilizing unsubstantial amounts of plutonium. [10 CFR 70.4]

Preventive Control – A control intended to prevent an accident (such as, any of the radiological or chemical consequences described in 10 CFR 70.61. [NUREG-1520]

Procedure – A document that specifies or describes how an activity is to be performed.

Protected Area – An area encompassed by physical barriers and to which access is controlled. For GLE, this includes the GLE Site surrounded by the vehicle barrier, physical barrier, and fencing systems with controlled access points at Entry Control Facilities. [10 CFR 73.2]

Public Dose – The dose received by a member of the public from exposure to radiation or to radioactive material released by a licensee, or to any other source of radiation under the control of a licensee. Public dose does not include occupational dose or doses received from background radiation, from any medical administration the individual has received, from exposure to individuals administered radioactive material and released under 10 CFR 35.75, or from voluntary participation in medical research programs. [10 CFR 20.1003]

Qualitative Fit Test (QLFT) – A pass/fail fit test to assess the adequacy of respirator fit that relies on the individual's response to the test agent. [10 CFR 20.1003]

Quantitative Fit Test (QNFT) – An assessment of the adequacy of respirator fit by numerically measuring the amount of leakage into the respirator. [10 CFR 20.1003]

Radiation (Ionizing Radiation) – Alpha particles, beta particles, gamma rays, x-rays, neutrons, high-speed electrons, high-speed protons, and other particles capable of producing ions. Radiation does not include non-ionizing radiation, such as radio-waves or microwaves, or visible, infrared, or ultraviolet light. [10 CFR 20.1003]

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Radiation Area – An area, accessible to individuals, in which radiation levels could result in an individual receiving a dose equivalent in excess of 0.005 rem (0.05 mSv) in one hour at 30 centimeters from the radiation source or from any surface that the radiation penetrates. [10 CFR 20.1003]

Radiological Controlled Area (RCA) – An area to which access is limited by the licensee for the purpose of protecting individuals against undue risks from exposure to radiation and radioactive materials. For regulatory purposes, a radiological controlled area is equivalent to a restricted area, as defined in 10 CFR 20.1003.

Research and Development – (1) Theoretical analysis, exploration, or experimentation; or (2) The extension of investigative finding and theories of a scientific or technical nature into practical application for experimental and demonstration purposes, including the experimental production and testing of models, devices, equipment, materials, and processes. [10 CFR 70.4]

Residual Radioactivity – Radioactivity in structure, materials, soils, groundwater, and other media at a site resulting from activities under the licensee's control. This includes radioactivity from all licensed and unlicensed sources used by the licensee, but excludes background radiation. It also includes radioactive materials remaining at the site as a result of routine or accident releases of radioactive material at the site and previous burials at the site, even if those burials were made in accordance with the provisions of 10 CFR 20. [10 CFR 20.1003]

Respiratory Protective Device – An apparatus, such as a respirator, used to reduce the individual's intake of airborne radioactive materials. [10 CFR 20.1003]

Restricted Area – An area, access to which is limited by the licensee for the purpose of protecting individuals against undue risks from exposure to radiation and radioactive materials. Restricted area does not include areas used as residential quarters, but separate rooms in a residential building may be set apart as a restricted area. [10 CFR 20.1003]

Restricted Data – All data concerning (1) design, manufacture, or utilization of atomic weapons; (2) the production of special nuclear material; or (3) the use of special nuclear material in the production of energy, but shall not include data declassified or removed from the Restricted Data category pursuant to Section 142 of the Act. [10 CFR 70.4]

Restricted Security Area – Designated Controlled Access Areas that are established to safeguard classified material. Typically classified material in these areas, which due to its size or nature, cannot be adequately protected during working hours by usual safeguards measures but is capable of being stored during non-working hours in an approved repository or secured by other approved methods.

Safety Control (Safeguard) – A system, device, or procedure that is intended to regulate a device, process, or human activity to maintain a safe state. Controls may be engineered controls or administrative (procedural) controls, and may be either preventive or mitigative.

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Sanitary Sewerage – A system of public sewers for carrying off waste water and refuse, but excluding sewage treatment facilities, septic tanks, and leach fields owned or operated by the licensee. [10 CFR 20.1003]

Sealed Source – Any special nuclear material that is encased in a capsule designed to prevent leakage or escape of the special nuclear material. [10 CFR 70.4]

Simple Administrative Control – A procedural human action that is prohibited or required to maintain safe process conditions. [NUREG-1520]

Single-Sided Lower Confidence Band (SSLCB): Estimates bias uncertainty to ensure, at a 95% level of confidence, a future calculation of k_{eff} for a critical system or process is actually above the lower confidence limit. The SSLCB may be used when there is a clear trend in the calculated critical benchmark results.

Single-Sided Lower Tolerance Band (SSLTB): Estimates the bias uncertainty to ensure, at a 95% level of confidence, at least 95% of future calculations of k_{eff} for critical systems or processes are actually above the lower tolerance limit. The SSLTB may be used when there is a clear trend in the calculated critical benchmark results.

Single-Sided Lower Tolerance Limit (SSLTL): Estimates the bias uncertainty to ensure, at a 95% level of confidence, at least 95% of future calculations of k_{eff} for critical systems or processes are actually above the lower tolerance limit. The SSLTL is used when there are no trends apparent in the calculated critical benchmark results.

Site Area Emergency – Events may occur, are in progress, or have occurred that could lead to a significant release of radioactive material and that could require a response by offsite response organization to protect persons offsite. [10 CFR 70.4]

Site Boundary – The line beyond which the land or property is not owned, leased, or otherwise controlled by the licensee. [10 CFR 20.1003] For the GLE Commercial Facility, the Site Boundary is coincident with the Wilmington Site boundary.

Source Material – (1) Uranium or thorium or any combination of uranium and thorium in any physical or chemical form; or (2) Ores that contain, by weight, one-twentieth of one percent (0.05 percent), or more, of uranium, thorium, or any combination of uranium and thorium. Source material does not include special nuclear material. [10 CFR 20.1003]

Special Nuclear Material (SNM) – (1) Plutonium, ^{233}U , uranium enriched in the Isotope 233 or in the Isotope 235, and any other material which the Commission, pursuant to the provisions of Section 51 of the Atomic Energy Act, determines to be special nuclear material, but does not include source material; or (2) Any material artificially enriched by any of the foregoing but does not include source material. [10 CFR 70.4]

Special Nuclear Material of Low Strategic Significance – (1) Less than an amount of special nuclear material of moderate strategic significance as defined in Paragraph (1) of the definition

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of strategic nuclear material of moderate strategic significance, but more than 15 grams of ^{235}U (contained in uranium enriched to 20 percent or more in ^{235}U isotope) or 15 grams of ^{233}U or 15 grams of plutonium or the combination of 15 grams when computed by the equation, grams = (grams contained ^{235}U) + (grams plutonium) + (grams ^{233}U); or (2) Less than 10,000 grams but more than 1,000 grams of ^{235}U (contained in uranium enriched to 10 percent or more but less than 20 percent in the ^{235}U isotope); or (3) 10,000 grams or more of ^{235}U (contained in uranium enriched above natural but less than 10 percent in the ^{235}U isotope). This class of material is sometimes referred to as a Category III quantity of material. [10 CFR 70.4]

Special Nuclear Material of Moderate Strategic Significance – (1) Less than a formula quantity of strategic special nuclear material but more than 1,000 grams of ^{235}U (contained in uranium enriched to 20 percent or more in the ^{235}U isotope) or more than 500 grams of ^{233}U or plutonium, or in a combined quantity of more than 1,000 grams when computed by the equation, grams = (grams contained ^{235}U) + 2 (grams ^{233}U + grams plutonium); or (2) 10,000 grams or more of ^{235}U (contained in uranium enriched to 20 percent or more in the ^{235}U isotope), ^{233}U , or plutonium. This class of material is sometimes referred to as a Category II quantity of material. [10 CFR 70.4]

Special Nuclear Material Scrap – The various forms of special nuclear material generated during chemical and mechanical processing, other than recycle material and normal process intermediates, which are unsuitable for use in their present form, but all or part of which will be used after further processing. [10 CFR 70.4]

Strategic Special Nuclear Material – ^{235}U (contained in uranium enriched to 20 percent or more in ^{235}U isotope), ^{233}U , or plutonium. [10 CFR 70.4]

Survey (Radiological) – An evaluation of the radiological conditions and potential hazards incident to the production, use, transfer, release, disposal, or presence of radioactive material or their sources of radiation. When appropriate, such an evaluation includes a physical survey of the location of radioactive material and measurements or calculations of levels of radiation, or concentrations or quantities of radioactive material present. [10 CFR 20.1003]

Tail Cylinder – A 48-inch, UF_6 cylinder that contains less than 0.72 percent weight ^{235}U material.

Total Effective Dose Equivalent – Means the sum of the effective dose equivalent (for external exposures) and the committed effective dose equivalent (for internal exposures). [10 CFR 20.1003]

Unacceptable Performance Deficiencies – Deficiencies in the items relied on for safety or the management measures that need to be corrected to ensure an adequate level of protection as defined in 10 CFR 70.61(b), (c), or (d). [10 CFR 70.4]

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Unmitigated Initial Condition – The unmitigated initial conditions are those initial conditions of the designed process or SSC that define how the process works, the physical conditions/constraints under which the material is handled, confined, or processed, and/or specifications of the SSC that define the normal operating characteristics of the process from which a deviation is postulated and its impacts are judged. For SSCs these initial conditions are defined based on application of the applicable Codes and Standards used in good engineering practices as they would be applied to non-regulated material (for example, in processing an HF gas stream in the chemical industry). These conditions do not include reliance on safety devices or abnormal or emergency procedures that mitigate abnormal deviations from the normal conditions (even when required by Codes and Standards). (For example, the unmitigated initial condition of a pressure vessel includes the vessel's location; materials normally processed; reaction rates, flow, pressure, and temperature ratings; materials of manufacture; etc. During the PHA process, deviations to these initial conditions are postulated and unmitigated event sequences are postulated based on the response of the process, system, structure, or component as defined by these parameters without relying on such items as pressure relief devices or abnormal operating procedures.)

Unmitigated Event Sequences – Unmitigated event sequences are those unintentional sequences of events that occur as a response of a process or SSC to a process deviation. The response is postulated based on the magnitude or effect of the process deviation on the unmitigated initial condition(s) defined by the designed process or SSC. The unmitigated initial conditions are defined by the process description, procedures, design documents, codes and standards, etc., that are used for defining the process or SSC. These unmitigated event sequences describe the process upset without the application of IROFS, and present the unmitigated likelihood of their occurrences, the unmitigated consequences that result, and an indication of the unmitigated risk. When the unmitigated risks are identified, they can be determined to be either acceptable or unacceptable. When unacceptable risks are identified for an unmitigated event sequence (that is, the risks exceed the performance requirements of 10 CFR 70.61), it is required that IROFS be identified to either prevent and/or mitigate the risk to the point that it becomes acceptable. The new sequences, including IROFS, are defined as accident sequences. (For example, a credible path for a reactant not normally used in a pressure vessel functioning as a reaction vessel is introduced as a process deviation. If the resulting pressure of the reaction can credibly result in exceeding the pressure or temperature ratings of the vessel then the vessel is assumed to rupture. The likelihood of the deviation is estimated and, based on the materials involved, the severity of the release is identified. The unmitigated event sequence includes the introduction of the reactant - the deviation-, the response to the deviation - the rupture and release of the contents at the respective rate consistent with the deviation-, and does not include the presence of a relief device that may prevent the rupture and direct the released pressure to a safe location. The unmitigated event sequence becomes an accident sequence of concern if the likelihood category and the severity category result in unacceptable risk.)

Unmitigated Consequence/Likelihood - Unmitigated consequence/likelihood is the consequence/likelihood of the unmitigated event sequence before IROFS are identified and applied.

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Unmitigated Risk - Unmitigated risk is the product of the unmitigated likelihood category (a number from 1 to 3) and the unmitigated consequence category (a number from 1 to 3). Unacceptable unmitigated risk carries a value of 6 or 9 (exceeds a value of 4).

Unrestricted Area – An area, access to which is neither limited nor controlled by the licensee. [10 CFR 20.1003]

Uranium Enrichment Facility – (1) Any facility used for separating the isotopes of uranium or enriching uranium in the Isotope 235, except laboratory scale facilities designed or used for experimental or analytical purposes only; (2) Any equipment or device, or important component part especially designed for such equipment or device, capable of separating the isotopes or uranium or enriching uranium in the Isotope 235. [10 CFR 70.4]

Very High Radiation Area – An area, accessible to individuals, in which radiation levels from radiation sources external to the body could result in an individual receiving an absorbed dose in excess of 500 rads (five grays) in one hour at one meter from a radiation source or one meter from any surface that the radiation penetrates. [10 CFR 20.1003]

Waste – Those low-level radioactive wastes containing source, special nuclear, or byproduct material that are acceptable for disposal in a land disposal facility. For the purposes of this definition, low-level radioactive waste means radioactive waste not classified as high-level radioactive waste, transuranic waste, spent nuclear fuel, or byproduct material. [10 CFR 20.1003]

Wilmington Site – The approximately 1600 acre GE property located in Wilmington, NC, where various nuclear and non-nuclear industrial facilities are located, including the GLE Commercial Facility.

Worker – An individual who receives an occupational dose as defined in 10 CFR 20.1003. [10 CFR 70.4]

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