August 16, 2011

Mr. E. Kurt Hackmann
Director Decommissioning Project
Westinghouse Electric Company
Nuclear Fuels
3300 State Road P
Festus, MO 63028

SUBJECT: NRC INSPECTION REPORT 070-00036/10-05(DNMS) – WESTINGHOUSE ELECTRIC COMPANY (HEMATITE)

Dear Mr. Hackmann:

On June 21, 2011, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at the Westinghouse Hematite facility located near Festus, Missouri. The purpose of the inspection was to determine whether decommissioning activities were conducted safely and in accordance with NRC requirements. Specifically, the inspection focused on the implementation of your radiation protection program during building demolition and associated preparatory work, liquid effluent monitoring, demolition contractor oversight, and radioactive waste transportation activities. The enclosed report presents the results of this inspection, which were discussed with you and members of your staff during a telephonic exit meeting on June 21, 2011.

The inspection consisted of an examination of decommissioning activities at the Westinghouse Hematite facility as they relate to safety and compliance with the Commission's rules and regulations. Areas examined during the inspection are identified in the enclosed report. Within these areas, the inspection consisted of a selective examination of procedures and representative records, and interviews with personnel.

Based on the results of the inspection, no violations were identified.

In accordance with Title 10 Code of Federal Regulations (CFR) 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the NRC's Agencywide Documents Access and Management System (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html.

We will gladly discuss any questions you may have regarding this inspection. If you have questions, please feel free to contact Michael LaFranzo or Jeremy Tapp of my staff at (630) 829-9865 or (630) 829-9862.

Sincerely,

/RA/

Christine A. Lipa, Chief
Materials Control, ISFSI
and Decommissioning Branch
Division of Nuclear Materials Safety

Docket No. 070-00036
License No. SNM-00033

Enclosure:
Inspection Report No. 070-00036/10-05(DNMS)

cc w/encl: Hematite Distribution Service List

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cc w/encl: Hematite Distribution Service List

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DATE	08/11/11		08/15/11		08/15/11		08/16/11	

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No.:	070-00036
License No.:	SNM-00033
Report No.:	070-00036/10-05(DNMS)
Licensee:	Westinghouse Electric Company, LLC
Facility:	Former Hematite Fuel Manufacturing Facility
Location:	3300 State Road P Festus, Missouri
Dates:	December 14 -17, 2010; January 11-12, April 27- 29, May 3 - 5, & June 14 -17, 2011
Inspectors:	Mike McCann, Senior Health Physicist Wayne Slawinski, Senior Health Physicist Michael LaFranzo, Senior Health Physicist Jeremy Tapp, Health Physicist
Approved by:	Christine A. Lipa, Chief Materials Control, ISFSI, and Decommissioning Branch Division of Nuclear Materials Safety

Enclosure

EXECUTIVE SUMMARY

Westinghouse Electric Company, LLC
Hematite Fuel Manufacturing Facility (Decommissioning)
NRC Inspection Report 070-00036/10-05(DNMS)

This routine decommissioning inspection evaluated the Westinghouse Electric Company's (WEC) on-going decommissioning activities at its Hematite facility, Festus, Missouri. This routine decommissioning inspection focused on the licensee's management controls during decommissioning activities, the occupational radiation safety program during building demolition and associated preparations, the monitoring of liquid effluents with a new water sampler, and radioactive waste transportation activities.

Management Organization and Controls

- Management was adequately engaged in the U.S. Nuclear Regulatory Commission (NRC) licensed activities and provided appropriate oversight to ensure work was executed consistent with procedures (Section 1.0).

Radiation Protection

- Radiological work instructions and procedures for process building demolition activities were adequately developed and implemented to meet regulatory requirements (Section 2.1).

- Radiological instrumentation was used consistent with licensee procedures to ensure that both area and worker radiological hazards were identified (Section 2.2).

- Decontamination activities were performed in accordance with applicable radiation work permits (RWPs) and work plans. Respiratory protection equipment, implementing procedures, and training and certification of workers wearing respiratory protection met procedural requirements and those required in Subpart H of Title 10 of the Code of Federal Regulations (CFR) Part 20. The licensee adequately performed radiological surveys, and controlled contamination and dust during demolition work (Section 2.3).

- An as low as reasonably achievable (ALARA) program was implemented during process building preparatory work and subsequent building demolition activities as provided in work plans and RWP documents (Section 2.4).

Transportation Activities

- Radioactive waste from process building demolition was adequately classified and characterized as required by 10 CFR 61.55 and 61.56 (Section 3.1).

- Process building waste was prepared and manifested as provided by the licensee's procedures to meet NRC and Department of Transportation requirements (Section 3.2).

- The In-Situ Object Counting System was used for process building waste shipment gamma spectroscopy measurements as provided by licensee procedure (Section 3.3).

Effluent Control and Environmental Protection

- The licensee implemented adequate radiological and industrial controls for dust reduction, water runoff and for the control of contamination during process building demolition and preparatory work (Section 4.1).

- The licensee developed evironmental monitoring program associated sediment, soil and vegetation sampling procedures and the licensee's staff implemented those procedures adequately to ensure compliance with NRC requirements. Also, the licensee's actions to address NRC concerns regarding discrepancies in water collection procedures, which were documented in unresolved item (URI) 070-00036/09-02-01, were adequate. The NRC considers URI 070-00036/09-02-01 closed (Section 4.2).

Report Details

1.0 Management Organization and Controls (88005)

a. Inspection Scope

The inspectors reviewed the licensee's procedure approval process to determine the adequacy of management involvement. Management oversight of the radiological safety program was also reviewed to determine its effectiveness. The inspectors observed various decommissioning activities to assess management involvement, management support and the level of engagement in NRC licensed activities.

b. Observations and Findings

The inspectors determined that adequate processes were developed for procedure review and management approval. The inspectors selectively verified that radiological safety procedures including those associated with building demolition were approved consistent with the licensee's process. Through observations and interviews, the inspectors determined that licensee management was actively involved in NRC licensed activities and appropriately monitored ongoing building demolition work.

No findings of significance were identified.

c. Conclusions

Management was adequately engaged in NRC licensed activities and provided appropriate oversight to ensure work was executed consistent with procedures.

2.0 Radiation Protection (83822)

2.1 Radiation Protection Procedures

a. Inspection Scope

The inspectors reviewed changes to site operations (building demolition activities) to determine if the radiological safety hazards to onsite workers had been evaluated and procedures were revised accordingly so as to address the hazard. The inspectors determined whether the licensee had evaluated the impact of these changes and had implemented periodic monitoring, as appropriate, to detect and quantify the radiological hazard.

Radiation protection program procedures were reviewed to determine if they were consistent with regulatory requirements and included appropriate limits, precautions and controls.

b. Observations and Findings

The inspectors determined that radiological hazards associated with building demolition work was properly identified and evaluated. The inspectors determined that procedures were executed adequately to ensure worker safety from radiation.

No findings of significance were identified.

c. Conclusions

Radiological work instructions and procedures for process building demolition activities were adequately developed and implemented to meet regulatory requirements.

2.2 Instruments and Equipment

a. Inspection Scope

The inspectors reviewed the instrumentation utilized by the licensee to monitor onsite and offsite radiological hazards including portable survey instruments and air sampling equipment. The inspectors determined if appropriate instrumentation was used, had been source checked prior to use and whether instrument alarm set points were established, if applicable.

b. Observations and Findings

The inspectors determined that appropriate radiological instrumentation was used by licensee staff to identify the potential hazards associated with building demolition work. Worker breathing zone (lapel) air samplers and air samplers used to monitor area conditions were properly positioned and utilized in a manner so as to provide meaningful assessments of airborne radioactivity.

The inspectors observed that workers used instrumentation appropriately, consistent with the methods and for applications described in the licensee's procedures.

No findings of significance were identified.

c. Conclusions

Radiological instrumentation was used consistent with licensee procedures to ensure that both area and worker radiological hazards were identified.

2.3 Internal Exposure Control

a. Inspection Scope

The inspectors observed licensee staff perform radiological surveys and determined how decontamination of floor scabbling equipment was conducted through tours and interviews. The inspectors assessed compliance with the work plan, focusing on worker and site safety performance. The inspectors reviewed the applicable Radiation Work Permits (RWPs), toured areas where work was being performed and evaluated the use

of respiratory protection equipment. The inspectors reviewed aspects of the respiratory protection program to determine if it met the required elements of to 10 CFR 20, Subpart H.

The inspectors reviewed radiation survey procedures, observed a radiation protection technician perform routine smear surveys for the Process Buildings, and reviewed the procedure used to analyze contamination smears using a bench top counter. These reviews were performed to determine if the licensee conducted routine surveys in accordance with approved procedures and used acceptable health physics practices.

The inspectors observed radiation protection (RP) technician and radiation worker performance during preparatory work in the process building and subsequent building demolition activities and evaluated radiological controls and RWP adherence. During these activities, the inspectors determined whether air samplers were representative of the breathing zone air and/or area, as applicable.

During job performance observations, the inspectors evaluated radiation worker performance with respect to RWP requirements. The inspectors assessed whether workers were aware of the radiological conditions in their workplace and if their performance reflected the level of radiological hazards present. During building demolition work, the inspectors evaluated the measures implemented for dust reduction and for contamination control as provided in the licensee's work plan instruction and RWP.

b. Observations and Findings

Prior to the inspection, the licensee performed concrete shaving of the Process Building floor slab and had concluded the work. The inspectors toured the area where the equipment and tools were being cleaned and decontaminated. The inspectors noted the area was generally clean, free of dirt and debris and tools and equipment were put in a safe condition while decontamination work had stopped. In addition, the licensee had quarantined areas where decontamination work was being conducted with plastic sheeting in order to minimize the potential airborne contaminants. Workers used respiratory protection equipment as required and air samples were collected using appropriate air flows.

The inspectors verified through a review of the training records of four employees who used respiratory protection that they were trained and medically certified to safely wear the equipment. The inspectors also verified that respiratory protection procedures satisfied 10 CFR 20.1703 requirements. Through record review, the inspectors determined that only National Institute for Occupational Safety and Health (NIOSH) tested and certified respiratory protection equipment was used.

Radiation worker and RP technician performance satisfied RWP requirements and was consistent with good health physics practices. Contamination smears were taken and stored as necessary to minimize the potential for cross contamination. The bench top counter that analyzed smears was calibrated as required by procedure, and appropriate calibration sources were used for the radionuclides of concern at WEC Hematite.

No findings of significance were identified.

c. Conclusions

The licensee conducted decontamination activities in accordance with the applicable RWP, consistent with its work plan. Respiratory protection equipment, implementing procedures, and training and certification of workers wearing respiratory protection met procedural requirements and the requirements of Subpart H of 10 CFR Part 20. The licensee adequately performed radiological surveys, and controlled contamination and dust consistent with procedure and work plan instructions.

2.4 As-Low-As-Is-Reasonably-Achievable Program

a. Inspection Scope

The inspectors reviewed ALARA and work plan documents, total effective dose equivalent ALARA evaluations (i.e., respiratory protection evaluations), and exposure mitigation techniques. The inspectors evaluated the integration of ALARA requirements into work plans and RWP documents for adequacy. The inspectors determined whether the licensee considered the ALARA philosophy during process building equipment removal and demolition work.

b. Observations and Findings

Work plans and RWPs adequately incorporated ALARA initiatives. Radiological engineering controls were employed, as applicable, given the hazards present. Radiation protection staff understood ALARA philosophies and were provided the authority to implement ALARA policies. Work plans were executed during building demolition work and preparatory work in the process buildings consistent with ALARA protocols and procedures.

No findings of significance were identified.

c. Conclusions

An ALARA program was implemented during process building preparatory work and subsequent building demolition activities as provided in work plan instructions and RWP documents.

3.0 Inspection of Transportation Activities (86740)

3.1 Radioactive Waste Characterization and Classification

a. Inspection Scope

The inspectors reviewed the results of the licensee's sample (contamination smear survey) analyses used to identify the site waste stream isotopic mix. The inspectors reviewed the licensee's development and use of scaling factors to quantify difficult-to-measure (DTM) radionuclides (e.g., pure alpha or beta emitting radionuclies) for the classification of process building waste. The review was conducted to determine if the licensee assured compliance with 10 CFR 61.55 and 10 CFR 61.56, as required by Appendix G of 10 CFR Part 20.

b. Observations and Findings

The licensee developed a technical basis document to correlate detectable gamma emitting isotopes with DTM nuclides for purposes of waste classification, as provided in 10 CFR 61.55. The inspectors determined that the licensee's waste classification document was technically sound and provided an appropriate means of classifying process building waste to meet regulatory requirements. The inspectors observed the licensee load process building rubble into waste transport vehicles to determine if suitable means were implemented to ensure the waste characteristics of 10 CFR 61.56 were satisfied.

No findings of significance were identified.

c. Conclusions

Radioactive waste from process building demolition was adequately classified and characterized as required by 10 CFR 61.55 and 61.56.

3.2 Shipment Preparation and Manifests

a. Inspection Scope

The inspectors reviewed the documentation of shipment packaging, radiation surveys, package marking, vehicle inspections and surveys, emergency instructions, determination of waste classification/isotopic identification, and licensee verification of shipment readiness for several shipments of process building rubble that were made during the inspection.

For each of these shipments, the inspectors determined if the requirements of 10 CFR Parts 20 and 61 and those of the Department of Transportation (DOT) in 49 CFR Parts 170-189 were met. Specifically, the inspectors reviewed shipment records, interviewed staff and observed waste packaging, transport vehicle surveys and overall shipment preparatory activities. The inspectors also determined if specific limitations of the waste recipient (waste processor) were met, and if the quantity and type of radionuclides in each shipment was determined accurately.

b. Observations and Findings

Although process building rubble shipped during the inspection was DOT exempt as provided in 49 CFR 173.436, the licensee completed manifesting and surveys consistent with those for non-exempt shipments. The inspectors verified that vehicle and package readiness activities, loading of building rubble, radiological surveys and completing of manifests were performed consistent with licensee procedures. The inspectors determined that the licensee had implemented adequate processes to ensure building waste met the waste recipient's bulk survey release criteria.

No findings of significance were identified.

c. Conclusions

Process building waste was prepared and manifested as provided by licensee procedure to meet NRC and DOT requirements.

3.3 Use of the In-Situ Object Counting System

a. Inspection Scope

The inspectors reviewed use of the licensee's In-Situ Object Counting System (ISOCS) for building waste shipment gamma spectroscopy measurements. The inspectors discussed system setup and quality control (QC) testing with the ISOCS operator, and observed the measurements taken for several waste shipments made during the inspection. The review was performed to determine whether system use satisfied the licensee's procedure and to assess operator knowledge of the system.

The inspectors also reviewed measurement results to determine if they were applied properly to classify the waste stream.

b. Observations and Findings

The ISOCS operator performed system QC tests prior to daily use with a mixed gamma source as provided in the licensee's procedure. Quality control reports were reviewed by the operator to verify that parameters met control limits prior to system operation. Upper and lower bound control limits for full width half maximum and peak energy parameters for detectable gamma emitters were adequately established and used. Technical review of sample measurements was performed to meet the licensee's procedure before waste manifesting was completed.

No findings of significance were identified.

c. Conclusions

The ISOCS system was used for process building waste shipment gamma spectroscopy measurements as provided by licensee procedure.

4.0 Effluent Control and Environmental Protection (88045)

4.1 Airborne and Liquid Effluent Control

a. Inspection Scope

The inspectors reviewed the licensee's technical basis document that linked airborne dust concentrations to radiological thresholds to determine if an adequate technical basis was provided. Similarly, the inspectors reviewed activity hazard analyses for process building demolition and the associated work plan to determine if the licensee's planning adequately assessed the hazard. The inspectors determined whether the licensee established action levels commensurate with the hazard and whether an adequate means for hazard reduction was established through measures such as fog/water spray.

The inspectors assessed the licensee's radiological and industrial controls for dust reduction, water runoff and for the control of contamination during process building demolition and preparatory work. The inspectors evaluated the control mechanisms and discussed their implementation with licensee and contractor staff. Air sample results were reviewed to determine if they were timely analyzed, reviewed by licensee supervision, and if the results demonstrated the effectiveness of the controls.

b. Observations and Findings

The inspectors determined that real-time aerosol monitoring, gravimetric monitoring and air sampling for radioactive particulates was performed during building demolition as provided in the licensee's procedure and as defined in the licensee's hazard analysis. Monitors were properly positioned in locations to provide representative measurements of work activities as dictated by wind conditions. Real-time dust monitoring data was reviewed by licensee staff throughout a given day to identify the need for adjustment. Water misting was used successfully to reduce dust and water runoff was adequately controlled. No industrial dust or airborne radioactivity thresholds were exceeded during building demolition activities.

No findings of significance were identified.

c. Conclusions

The licensee implemented adequate radiological and industrial controls for dust reduction, water runoff and for the control of contamination during process building demolition and preparatory work.

4.2 Environmental Monitoring

a. Inspection Scope

The inspectors reviewed the licensee's environmental monitoring program associated sediment, soil and vegetation sampling procedures and the licensee's staff implementing those procedures. The inspectors also reviewed the circumstances and the licensee's response concerning an Unresolved Item (URI 070-00036/09-02-01) documented in Inspection Report No. 070-00036/09-02.

b. Observations and Findings

The inspectors reviewed procedures associated with sediment, soil and vegetation sampling and noted that the licensee was in compliance with analysis and documentation of the samples taken for calendar years (CY) 2009, 2010 and 2011 (up to the time of the inspection). The inspectors interviewed selected licensee staff involved in the implementation of those procedures. Each individual interviewed had adequate knowledge to ensure the procedures were implemented as required. The inspectors determined that the licensee had adequate quantities of collection equipment to ensure environmental samples were taken in accordance with the procedures reviewed. The inspectors reviewed the licensee's sediment, soil and vegetation analysis, including trending of data; the inspectors noted that the licensee had adequate documentation to

indicate no significant adverse trending had occurred in CY 2009, 2010 and 2011 (up to the time of the inspection).

As documented in Inspection Report No. 070-00036/09-02, the NRC identified three issues during the collection of groundwater samples and review of the procedures being used for sampling of groundwater. These were: 1) an inconsistency between the two procedures used to decontaminate non-dedicated equipment between sampling locations; 2) ambiguous verbiage in procedure HDP-PR-EM-011, in regard to purging of the well prior to sampling; and 3) wells were not sampled in order of least contaminated to most contaminated. The issues could not be resolved during the inspection and, subsequently, were documented within the report as URI 070-00036/09-02-01. During this inspection, the inspectors reviewed the licensee's procedure HDP-PR-EM-011, Rev. 1 titled "Low Flow Well Sampling" and identified that each issue above was resolved to ensure that proper sampling and analysis was performed. The NRC considers URI 070-00036/09-02-01 closed.

No findings of significance were identified.

c. Conclusions

The inspectors determined that the licensee's environmental monitoring program associated sediment, soil and vegetation sampling procedures and licensee's staff implementing those procedures were adequate to ensure compliance with NRC requirements. Also, the inspectors determined that the licensee's actions to address NRC concerns concerning discrepancies in water collection procedures, which were documented in URI 070-00036/09-02-01, were adequate and URI 070-00036/09-02-01 is closed.

5.0 Exit Meeting Summary

The NRC inspectors presented inspection results to members of the facility management team telephonically following the on-site inspections on June 21, 2011. The licensee acknowledged the results presented.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

PARTIAL LIST OF PERSONS CONTACTED

Westinghouse Electric Company

E. Kurt Hackmann, Director, Hematite Decommissioning Project
G. Rood, Radiation Safety Officer
M. Michelson, Manager, Licensing
K. Harris, Environmental, Health & Safety Manager
C. Cummin, Waste Management/Transportation Specialist
D. Atchison, Training Supervisor

INSPECTION PROCEDURES USED

IP 88005	Management Organization and Controls
IP 83822	Radiation Protection
IP 86740	Inspection of Transportation Activities
IP 88045	Effluent Control and Environmental Protection

ITEMS OPENED, CLOSED, AND DISCUSSED

Closed	Type	Summary
URI 070-00036/09-02-01	URI	Discrepancies in water collection procedures

Opened

None

Discussed

None

LIST OF ACRONYMS USED

ADAMS	Agencywide Documents Access and Management System
ALARA	As Low As Reasonably Achievable
CFR	Code of Federal Regulations
CY	Calendar Year
DNMS	Division of Nuclear Materials Safety
DOT	Department of Transportation
DTM	Difficult to Measure
ISOCS	In-Situ Object Counting System
NIOSH	National Institute for Occupational Safety and Health
NRC	U.S. Nuclear Regulatory Commission
QC	Quality Control

LIST OF ACRONYMS USED (continued)

RP	Radiation Protection
RWP	Radiation Work Permit
URI	Unresolved Item
WEC	Westinghouse Electric Company

DOCUMENTS REVIEWED

HDP-PO-HP-800, Respiratory Protection Plan; Revision 1

HDP-EHS-LP-RPT, Respiratory Protection Training Lesson Plan; Revision 0

Form HDP-PR-HP-311-1, Weekly Process Bldg contamination survey; dated January 7, 2011

HDP-PR-HP-801, Appendix A, Respiratory Issue Log; dated January 11, 2011

HDP-PR-HP-501-6, Pre-Job ALARA Evaluation; performed for RWP No. RP-11-S008; dated December 14, 2010

Form HDP-PR-HP-501-2, Radiation Work Permit; RWP No. RP-11-S008; dated January 1, 2011

Form HDP-PR-HP-501-2, Radiation Work Permit; RWP No. RP-11-S006; dated January 1, 2011

HDP-PR-HP-501-6, Pre-Job ALARA Evaluation; performed for RWPs No. RP-11-S005 and S006; both dated December 14, 2010

HDP-PR-HP-414, Tennelec LB5100 Calibration and Operation; Revision 3

HDP-PR-HP-311, Radiological Surveys; Revision 0

HDP-PR-HP-314, Unrestricted Release of Materials and Equipment; Revision 0

HDP-PR-HP-312, Routine Surveillances; Revision 0

HDP-PR-HP-515, Personnel and Equipment Decontamination; Revision 0

HDP-PR-HP-411, Radiological Instrumentation; Revision 0

Shipment Manifest and Associated Documentation (Shipment No. HDP-0322), Building Rubble, May 2, 2011

HDP-PR-WM-907, Radiological Surveys for Shipment and Receipt of Radioactive Material; Revision 0

HDP-PR-EHS-036, Ambient Dust Monitoring; Revision 0

HDP-TBD-HP-504, Assessment of Conditions During Process Building Demolition; Revision 0

HDP-PR-HP-301-2, Effluent & Environmental Air Sample Report; Sample Results for April 18 – 26, 2011

HDP-PR-HP-413; ISOCS operation and Data Verification; Revision 3

HDP-PO-GM-002; Training Plan; Revision 3

DOCUMENTS REVIEWED (continued)

HDP-PR-HP-302, DAC-Hour Calculation and Recording; Revision 0

HDP-PR-HP-501, Radiation Work Permits; Revision 1

HDP-Pr-EM-011, Rev. 1, Low Flow Well Sampling

HDP-PR-EM-005, Sediment Sampling

HDP-PR-EM-007, Vegetation Sampling

HDP-PR-EM-004, Soil Sampling

HDP-PO-EM-001, Effluent and Environmental Monitoring Plan

HDP-PR-MCA-002, Rev. 2, Scale Calibration

Westinghouse - Hematite Service List

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