

DISTRIBUTION AFTER ISSUANCE OF OPERATING LICENSE

NRC FORM 195
(2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER

50-331

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO: Mr. Edson G. Case

FROM: Iowa Elect Light & Power Co.
Cedar Rapids, Iowa
Lee LiuDATE OF DOCUMENT
12/27/77DATE RECEIVED
01/03/78☒ LETTER
☒ ORIGINAL
☐ COPY☒ NOTORIZED
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

35162ED

DESCRIPTION

Notorized 12/27/77...Trans The
Following:

ENCLOSURE

License No. DPR-49 Appl for Amend:
tech specs proposed change concerning proposed
change RTS-94, RTS-98 and RTS-99 to DAEC Tech
Specs...

1p

6p

PLANT NAME: DUANE ARNOLD
• jcm 01/04/78

40 ENCL.

SAFETY

FOR ACTION/INFORMATION

BRANCH CHIEF: (7)

LEAR

INTERNAL DISTRIBUTION

☒ REG FILE

NRC PDR

I & E (2)

OELD

HANAUER

CHECK

EISENHUT

SHAO

BAER

BUTLER

ERIMES

J. COLLINS

J. MCGOUGH

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: CEDAR RAPIDS 1A

TIC

NSIC

ACRS 16 CYS SENT CATEGORY B

780040012

Ap 2
GD

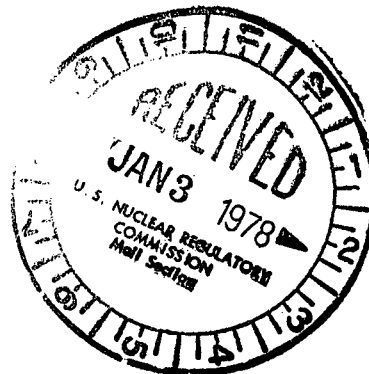
IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office
CEDAR RAPIDS, IOWA

December 27, 1977
IE-77-2317

LEE LIU
VICE PRESIDENT - ENGINEERING

50-331



Mr. Edson G. Case
Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Case:

Transmitted herewith, in accordance with the requirements of 10 CFR 50.59 and 50.90, is an application for amendment of DPR-49 (Appendix A to License) for the Duane Arnold Energy Center.

This application, consisting of proposed Technical Specification changes RTS-94, RTS-98 and RTS-99, has been reviewed and approved by the DAEC Operations Committee and the DAEC Safety Committee. This application does not involve a significant hazards consideration.

Three signed and notarized originals and 37 additional copies of this application are transmitted herewith. This application, consisting of the foregoing letter and enclosures hereto, is true and accurate to the best of my knowledge and belief.

IOWA ELECTRIC LIGHT AND POWER COMPANY

By

Lee Liu

Vice President-Engineering

LL/OCS/A
Encs.

c.c. Mr. D. Arnold
Mr. K. Meyer
Mr. R. Lowenstein
Mr. J. Keppler (NRC)
Mr. R. Clark (NRC)
Mr. L. Root
File: A-117

Subscribed and sworn to before me this
27th day of December, 1977.

Notary Public in and for the
State of Iowa

Jean R. Smith
NOTARY PUBLIC
STATE OF IOWA
Commission Expires
September 30, 1978

780040012

PROPOSED CHANGE RTS-94 TO DAEC TECHNICAL SPECIFICATIONS

I. Affected Technical Specifications

Appendix A of the Technical Specifications for the DAEC (DPR-49) provides as follows:

Specification 6.10.2 lists records which must be retained for the duration of the Facility Operating License. Specification 6.10.2.7 states: "Records of training and qualification for members of the plant staff." and Specification 6.10.2.9 states: "Records of Quality Assurance activities."

II. Proposed Changes in Technical Specifications

The licensees of DPR-49 propose the following changes in the Technical Specifications set forth in I above:

Change Specification 6.10.2.7 to "Records of training and qualification for current members of the plant staff."

Change Specification 6.10.2.9 to "Records of Quality Assurance activities required by the QA Manual."

III. Justification for Proposed Change

This Technical Specification change is proposed in order to clarify the intent and to bring them into agreement with the Standardized Technical Specifications.

IV. Review Procedures

This proposed change has been reviewed by the DAEC Operations Committee and Safety Committee which have found that this proposed change does not involve a significant hazards consideration.

7. Records of training and qualification for current members of the plant staff.

8. Records of in-service inspections performed pursuant to these Technical Specifications.

9. Records of Quality Assurance activities required by the QA Manual.

10. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

11. Records of meetings of the Operations Committee and the Safety Committee.

PROPOSED CHANGE RTS-98 TO DAEC TECHNICAL SPECIFICATIONS

I. Affected Technical Specifications

Appendix A of the Technical Specifications for the DAEC (DPR-49) provides as follows:

Table 4.6-4, Snubbers Inaccessible During Normal Operation, lists, among others, snubbers SSA-10 and SSB-10 as being in the Recirculation System in the Drywell.

II. Proposed Changes in Technical Specifications

The licensees of DPR-49 propose the following changes in the Technical Specifications set forth in I above:

After SSA-10 and SSB-10, add "(2 ea.)".

III. Justification for Proposed Change

There are two snubbers with the same identification number installed at the subject location.

IV. Review Procedure

This proposed change has been reviewed by the DAEC Operations Committee and Safety Committee which have found that this proposed change does not involve a significant hazards consideration.

DAEC-1
TABLE 4.6-4 (cont.)

<u>Identification No.</u>	<u>System</u>	<u>Location</u>	<u>Vendor Dwg. No.</u>
GBC-9-SS-43	Main Stm. Relief Valve Discharge	Drywell	6043
GBC-10-SS-24	" " "	Drywell	6024
GBC-10-SS-25	" " "	Drywell	6025
GBC-10-SS-39	" " "	Drywell	6039
GBC-10-SS-40 (2 ea.)	" " "	Drywell	6040
GBC-11-SS-26	" " "	Drywell	6026
GBC-11-SS-27	" " "	Drywell	6027
GBC-11-SS-32	" " "	Drywell	6032
GBC-11-SS-33	" " "	Drywell	6033
SSB-1-MS	Main Steam	Drywell	GE-BP 405 Rev 2
SSB-2-MS	Main Steam	Drywell	GE-BP 406 Rev 3
SSC-1-MS	Main Steam	Drywell	GE-BP 407 Rev 2
SSC-2-MS	Main Steam	Drywell	GE-BP 408 Rev 2
SSA-1-MS	Main Steam	Drywell	GE-BP 401 Rev 1
SSA-2-MS	Main Steam	Drywell	GE-BP 402 Rev 1
SSD-1-MS	Main Steam	Drywell	GE-BP 403 Rev 1
SSD-2-MS	Main Steam	Drywell	GE-BP 404 Rev 1
SSA-1	Recirc	Drywell	GE-BP 201 Rev 2
SSB-1	Recirc	Drywell	GE-BP 202 Rev 2
SSA-2	Recirc	Drywell	GE-BP 203 Rev 1
SSB-2	Recirc	Drywell	GE-BP 204 Rev 1
SSA-3	Recirc	Drywell	GE-BP 205 Rev 1
SSB-3	Recirc	Drywell	GE-BP 206 Rev 1
SSA-4	Recirc	Drywell	GE-BP 207 Rev 1
SSB-4	Recirc	Drywell	GE-BP 208 Rev 1
SSA-5	Recirc	Drywell	GE-BP 209 Rev 1
SSB-5	Recirc	Drywell	GE-BP 210 Rev 1
SSA-6	Recirc	Drywell	GE-BP 211 Rev 1
SSB-6	Recirc	Drywell	GE-BP 212 Rev 1
SSA-7	Recirc	Drywell	GE-BP 213 Rev 1
SSB-7	Recirc	Drywell	GE-BP 214 Rev 1
SSA-8	Recirc	Drywell	GE-BP 215 Rev 1
SSB-8	Recirc	Drywell	GE-BP 216 Rev 1
SSA-9	Recirc	Drywell	GE-BP 217 Rev 1
SSB-9	Recirc	Drywell	GE-BP 218 Rev 1
SSA-10 (2 ea.)	Recirc	Drywell	GE-BP 219 Rev 1
SSB-10 (2 ea.)	Recirc	Drywell	GE-BP 220 Rev 1
SSA-11	Recirc	Drywell	GE-BP 221 Rev 1
SSB-11	Recirc	Drywell	GE-BP 222 Rev 1

(2 ea.) - Indicates there are 2 snubbers with that number.

Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

PROPOSED CHANGE RTS-99 TO DAEC TECHNICAL SPECIFICATIONS

I. Affected Technical Specifications

Appendix A of the Technical Specifications for the DAEC (DPR-49) provides as follows:

Note 1 to Table 3.2-C (page 3.2-17) states, in part, as follows: "For the startup and run positions of the Reactor Mode Selector Switch, there shall be two operable or tripped trip systems for each function. The SRM and IRM blocks need not be operable in "Run" mode, and the APRM and RBM rod blocks need not be operable in "Startup" mode."

II. Proposed Changes in Technical Specifications

The licensees of DPR-49 propose the following changes in the Technical Specifications set forth in I above:

In the second sentence after the word "APRM" add "[except for APRM Upscale (Not in Run Mode)]".

III. Justification for Proposed Change

This change is proposed in order to bring the notes into agreement with the intent of Table 3.2-C, Instrumentation That Initiates Control Rod Blocks.

Item 2 in Table 3.2-C, APRM Upscale (Not in Run Mode) indicates a trip level setting of " ≤ 12 indicated on scale" so it must be operable in "Startup" mode.

IV. Review Procedure

This proposed change has been reviewed by the DAEC Operations Committee and Safety Committee which have found that this proposed change does not involve a significant hazards consideration.

NOTES FOR TABLE 3.2-C

1. For the startup and run positions of the Reactor Mode Selector Switch, there shall be two operable or tripped trip systems for each function. The SRM and IRM blocks need not be operable in "Run" mode, and the APRM [except for APRM Upscale (Not in Run Mode)] and RBM rod blocks need not be operable in "Startup" mode. If the first column cannot be met for one of the two trip systems, this condition may exist for up to seven days provided that during that time the operable system is functionally tested immediately and daily thereafter; if this condition lasts longer than seven days, the system shall be tripped. If the first column cannot be met for both trip systems, the systems shall be tripped.
2. W is the recirculation loop flow in percent of design. Trip level setting is in percent of rated power (1593 MWt). Refer to Limiting Safety System Settings for variation with peaking factors. Peaking factor is applicable only when it exceeds 2.61 (7 x 7 array) or 2.43 (8 x 8 array).
3. IRM downscale is bypassed when it is on its lowest range.
4. This function is bypassed when the count rate is >100 cps.