

50-331

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

TO: Iowa Elec Light  
Mr KepplerFROM: Iowa Elec Light & Pwr  
Cedar Rapids, Ia  
G G HuntDATE OF DOCUMENT  
11-10-76

DATE RECEIVED 11-16-76

☒ LETTER  
☐ ORIGINAL  
☒ COPY☐ NOTORIZED  
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1cc

## DESCRIPTION

Ltr trans the following:

PLANT NAME:

Duane Arnold

## ENCLOSURE

Licensee Event Report (RO# 76-71) on 10-24-76  
concerning failure of IRM F to overlap by the  
time SR M indication was 100000 CPS.....

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

11-16-76

ehf

BRANCH CHIEF:

hear

W/3 CYS FOR ACTION

LIC. ASST.:

Parrish

W/1 CYS

ACRS 16 CYS HOLDING/SENT TO LA AS CAT B 11-16-76

## INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I &amp; E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

CASE

BUTLER

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/HUNCH

KREGER/J. COLLINS

## EXTERNAL DISTRIBUTION

LPDR: Cedar Rapids, Ia

TIC:

NSIC:

## CONTROL NUMBER

11696

apj

IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER  
P. O. Box 351  
Cedar Rapids, Iowa 52406  
November 10, 1976  
DAEC-76-360



Mr. James G. Keppler, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission - Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 76-71  
(30 day)

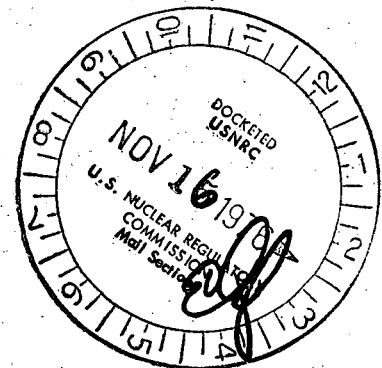
File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

*G. G. Hunt* ELH  
G. G. Hunt  
Chief Engineer  
Duane Arnold Energy Center



Docket 50-331

attachment

DLW/GGH/mg

cc: Director, Office of Inspection and Enforcement (30)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director, Management Information and Program Control (3)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

[PLEASE PRINT ALL REQUIRED INFORMATION]

EVENT DESCRIPTION

02	During reactor startup IRM F did not demonstrate overlap by the time SR	7	8	9	80
03	M indication was 100000 CPS. All other IRM's indicated properly. A quic	7	8	9	80
04	k check showed that the high voltage cable was not connected. The cable	7	8	9	80
05	was connected and the IRM was checked out satisfactorily. (R0 76-71)	7	8	9	80
06		7	8	9	80

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

SYSTEM CODE CAUSE CODE COMPONENT CODE PRIME COMPONENT SUPPLIER COMPONENT MANUFACTURER VIOLATION

07 I B F I N S T R U N G 0 8 0 N

## CAUSE DESCRIPTION

08	Unknown. Test jacks were installed in December, 1975 to eliminate need f	
7 8 9		
09	or disconnecting this cable during tests. Reactor was shutdown two days	80
7 8 9		
10	before this event and IRM F responded properly. No work was done on IRM	80
7 8 9		

FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	C	0	0	0	NA	A	Observation during startup			
7	8	9	10	11	12	13	44	45	46	80
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE				
12	Z	Z	NA			NA				
7	8	9	10	11	44	45	80			

## PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	Z NA

## PERSONNEL INJURIES

NUMBER				DESCRIPTION
14	0	0	0	NA

## OFFSITE CONSEQUENCES

15	NA
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LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION
1	6	Z	NA

PUBLICITY

17	NA
7 8 9	80

### ADDITIONAL FACTORS

18 Cause Desc. Cont. - during the shutdown.

19 7 8 9

PHONE: 319-851-5611

# IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER

P. O. Box 351

Cedar Rapids, Iowa 52406

November 10, 1976

DAEC-76-360

Mr. James G. Keppler, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission - Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 76-71  
(30 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

*G. G. Hunt* ELH

G. G. Hunt

Chief Engineer

Duane Arnold Energy Center

Docket 50-331

attachment

DLW/GGH/mg

cc: Director, Office of Inspection and Enforcement (30)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director, Management Information and Program Control (3)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

NOV 15 1976

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME			LICENSE NUMBER										LICENSE TYPE					EVENT TYPE									
01	I	A	D	A	C	1	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3				
7	8	9				14	15										25	26				30	31	32			
CATEGORY			REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE					
01	CONT				L	L	0	5	0	-	0	3	3	1	1	0	2	4	7	6	1	1	1	0	7	6	
7	8	9	57	58	59	60	61							66	69						74	75					80

## EVENT DESCRIPTION

02	During reactor startup IRM F did not demonstrate overlap by the time SR																								7	8	9	60
03	M indication was 100000 CPS. All other IRM's indicated properly. A quic																								7	8	9	80
04	k check showed that the high voltage cable was not connected. The cable																								7	8	9	80
05	was connected and the IRM was checked out satisfactorily. (RO 76-71)																								7	8	9	80
06																									7	8	9	80

SYSTEM CODE			CAUSE CODE		COMPONENT CODE										PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION				
07	I	B	F	I	N	S	T	R	U	N	G	O	8	O	N	7	8	9	10	11	12	17	43	44	47	48

## CAUSE DESCRIPTION

08	Unknown. Test jacks were installed in December, 1975 to eliminate need f																								7	8	9	80
09	or disconnecting this cable during tests. Reactor was shutdown two days																								7	8	9	80
10	before this event and IRM F responded properly. No work was done on IRM																								7	8	9	80

FACILITY STATUS			% POWER			OTHER STATUS							METHOD OF DISCOVERY		DISCOVERY DESCRIPTION												
11	C	0	0	0	NA	A	Observation during startup																				
7	8	9	10	12	13									44	45	46											80
FORM OF ACTIVITY RELEASED			CONTENT OF RELEASE			AMOUNT OF ACTIVITY										LOCATION OF RELEASE											
12	Z	Z	NA	NA	NA	NA																					
7	8	9	10	11										44	45												80

## PERSONNEL EXPOSURES

NUMBER			TYPE		DESCRIPTION																					
13	0	0	0	Z	NA																					
7	8	9	11	12	13																					80

## PERSONNEL INJURIES

NUMBER			DESCRIPTION																							
14	0	0	0	NA																						
7	8	9	11	12																						80

## OFFSITE CONSEQUENCES

15	NA																								7	8	9	60
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## LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION																								
16	Z	NA																									
7	8	9	10																								80

## PUBLICITY

17	NA																								7	8	9	80
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## ADDITIONAL FACTORS

18	Cause Desc. Cont. - during the shutdown.																								7	8	9	80
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19																									7	8	9	80
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NAME: M. Schwartz

PHONE: 319-851-5611