

Enclosure 4

Generic Communications - Revision 6 Changes

GENERIC COMMUNICATIONS: REVISION 6 CHANGES

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 74-003	Failure of Structural or Seismic Support Bolts on Class I Components	CI	TVA: memo dated January 22, 1985
		06	NRC: IR 390/391 85-08
			Approach accepted in IR 50-390/85-08 and 50-391/85-08 (March 29, 1985).
			Unit 2 Action: Implement per NUREG-0577 as was done for Unit 1.
		REVISION 06 UPDATE:	
			Corrective action for this item consisted of a bolting reheat treatment program for both units; it has been completed.
B 74-015	Misapplication of Cutler-Hammer Three Position Maintained Switch Model No. 10250T	C	TVA: letter dated May 5, 1975
		06	NRC: IR 390/391 75-5
			Unit 2 Action: Install modified A3 Cutler-Hammer 10250T switches.
			REVISION 06 UPDATE:
			It has been confirmed that WBN Unit 2 never had the faulty switches.
			NRC Inspection Report 391/2010-605 closed B 74-015.
B 75-006	Defective Westinghouse Type OT-2 Control Switches	CI	TVA: letter dated July 31, 1975
		06	NRC: IR 390/85-25 and 391/85-20
			Unit 2 Action: Inspect Westinghouse Type OT-2 control switches.
			[WAS "NOTE 3."]
		REVISION 06 UPDATE:	
			All Unit 2 Type OT-2 switches procured or refurbished are inspected and tested.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 79-009	Failure of GE Type AK-2 Circuit Breaker in Safety Related Systems	CI 06	<p>TVA: letter dated June 20, 1979</p> <p>-----</p> <p>Unit 2 Action:</p> <p>Complete preservice preventive maintenance on AK-2 Circuit Breakers.</p> <p>[WAS "NOTE 3."]</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>It has been confirmed that AK-2 Circuit Breakers are not used on Unit 2.</p>
B 79-021	Temperature Effects on Level Measurements	C 06	<p>Reviewed in 7.2.5 of both the original 1982 SER and SSER14.</p> <p>Unit 2 Action: Update accident calculation.</p> <p>-----</p> <p>CONFIRMATORY ISSUE - address IEB 79-21 to alleviate temperature dependence problem associated with measuring SG water level</p> <p>In SSER14, NRC concurred with TVA's assessment to not insulate the steam generator water level instrument reference leg.</p> <p>Unit 2 Action: Update accident calculation.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>The calculations were updated.</p> <p>-----</p> <p>NRC Inspection Report 391/2010-605 closed B 79-021.</p>
B 80-004	Analysis of a PWR Main Steam Line Break with Continued Feedwater Addition	CI 06	<p>IR 50-390/85-60 and 50-391/85-49 (December 6, 1985) required completion of actions that included determination of temperature profiles inside and outside of containment following a MSLB for Unit 1.</p> <p>Unit 2 Action: Complete analysis for Unit 2.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>The analysis for Unit 2 was completed.</p>
B 80-010	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	CI 06	<p>Unit 2 Actions: 1) Correct deficiencies involving monitoring of systems.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			Chemistry procedure CM-3.01 (System Chemistry Specification) includes a radiation monitoring system for non-radioactive systems and provides appropriate surveillance limits. Additionally, it provides required actions if the surveillance limits are not met.
B 80-010	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	CI 06	Unit 2 Actions: 2) Include proper monitoring of non-radioactive systems in procedures.
			REVISION 06 UPDATE: Chemistry procedure CM-3.01 (System Chemistry Specification) includes a radiation monitoring system for non-radioactive systems and provides appropriate surveillance limits. Additionally, it provides required actions if the surveillance limits are not met.
B 80-018	Maintenance of Adequate Minimum Flow Thru Centrifugal Charging Pumps Following Secondary Side High Energy Rupture	CO 06	IR 50-390/85-60 and 50-391/85-49 (Unit 1) Unit 2 Action: Implement design and procedure changes.
			REVISION 06 UPDATE: NRC Inspection Report 391/2011-604 closed B 80-018.
B 80-020	Failure of Westinghouse Type W-2 Spring Return to Neutral Control Switches	CI 06	Unit 2 Action: Modify switches.
			REVISION 06 UPDATE: The switches were modified.
			NRC Inspection Report 391/2011-604 closed B 80-020.
B 80-024	Prevention of Damage Due to Water Leakage Inside Containment (10/17/80 Indian Point 2 Event)	CI 06	Unit 2 Action: Confirm that the reactor cavity can not be flooded, resulting in the partial or total submergence of the reactor vessel unnoticed by the reactor operators.
			REVISION 06 UPDATE: It was confirmed that the reactor cavity can not be flooded, resulting in the partial or total submergence of the reactor vessel unnoticed by the reactor operators.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 82-002	Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants	CI 06	TVA: memo dated February 6, 1985 NRC: IR 390/391 85-08 ----- Approach accepted in IR 50-390/85-08 and 50-391/85-08 (March 29, 1985). Unit 2 Action: Implement same approach as Unit 1. ----- ----- REVISION 06 UPDATE: The boric acid corrosion program applies to both units.
B 83-004	Failure of the Undervoltage Trip Function of Reactor Trip Breakers	C 06	NRC: IR 390/391 85-08 ----- Unit 2 Action: Install new undervoltage attachment with wider grooves on the reactor trip breakers. ----- ----- REVISION 06 UPDATE: New breakers have been installed on Unit 2. ----- NRC Inspection Report 391/2011-602 closed B 83-004.
B 85-002	Undervoltage Trip Attachment of Westinghouse DB-50 Type Reactor Trip Breakers	C 06	Unit 2 Action: Install automatic shunt trip on the Westinghouse DS-416 reactor trip breakers on Unit 2. ----- ----- REVISION 06 UPDATE: New breakers (including an automatic shunt trip) have been installed on Unit 2. ----- NRC Inspection Report 391/2011-602 closed B 85-002.
B 88-009	Thimble Tube Thinning in Westinghouse Reactors	CI 06	Reviewed in Appendix EE of SSER16. Unit 2 Action: TVA letter dated March 11, 1994, for both units committed to establish a program and inspect the thimble tubes during the first refueling outage.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 06 UPDATE:</p> <p>Unit 2 is installing the Westinghouse In-core, Information, Surveillance, and Engineering (WINCISE) system. Westinghouse has analyzed WINCISE to exhibit essentially no wear due to vibrations, and should there be a breach of the thimble tube there would not be a loss of into the seal table room. Therefore, the thimble tubes for WINCISE do not need eddy current testing.</p>
B 89-001	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	C 06	<p>NRC acceptance letter dated September 26, 1991 for both units.</p> <p>Unit 2 Action: Remove SG tube plugs.</p> <p>REVISION 06 UPDATE:</p> <p>The SG tube plugs were removed.</p> <p>NRC Inspection Report 391/2011-602 closed B 89-001.</p>
B 89-002	Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valves of Similar Nature	CI 06	<p>NRC reviewed in Appendix EE of SSER16.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Replace the flapper assembly hold-down bolts fabricated on the 14 (12 valves are installed) Atwood and Morrell Mark No. 47W450-53 check valves. * Replacement bolts are to be fabricated from ASTM F593 Alloy 630. * A review of the remaining Unit 2 safety related swing check valves will be performed. <p>REVISION 06 UPDATE:</p> <ul style="list-style-type: none"> * Bolts fabricated from ASTM F593 Alloy 630 have been procured. * The review of the remaining Unit 2 safety related swing check valves was completed. Needed corrective actions were initiated.
B 90-001	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	CO 06	<p>Unit 2 Action:</p> <p>Implement applicable recommendations from this Bulletin including identification of potentially defective transmitters and an enhanced surveillance program which monitors transmitters for loss of fill oil.</p> <p>REVISION 06 UPDATE:</p> <p>NRC Inspection Report 391/2011-603 closed B 90-001.</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 96-001, last part	Control Rod Insertion Problems (PWR)	CI 06	NRC acceptance letter for Unit 1 dated July 22, 1996 – Initial response for Unit 2 on September 7, 2007.
			Unit 2 Action: and provide core map.
			REVISION 03 UPDATE:
			NRC issued the Safety Evaluation (corrected) for Bulletin 1996-001 on May 3, 2010.
			REVISION 04 UPDATE:
			Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.
			REVISION 06 UPDATE:
			SSER22 contained the following for NRC Action:
			"Closed. NRC letter dated May 3, 2010 (ADAMS Accession No. ML101200035) required Confirmatory Action (See Appendix HH)"
			The applicable item from SER22, Appendix HH for this item is Open Item 5, "Verify timely submittal of pre-startup core map and perform technical review. (TVA letter dated September 7, 2007, ADAMS Accession No. ML072570676)."
			TVA to NRC letter dated April 6, 2011 provided the following response to Open Item 5:
			"Attachment 1 provides the requested core map."
B 96-002	Movement of Heavy Loads over Spent Fuel, Over Fuel in the Reactor, or Over Safety-Related Equipment	CI 06	NRC closure letter dated May 20, 1998.
			Unit 2 Action:
			Unit 2 Heavy Loads Program will be in compliance with NUREG-0612.
			REVISION 02 UPDATE:
			NRC issued the Safety Evaluation for Bulletin 1996-002 on March 4, 2010.
			REVISION 06 UPDATE:

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC letter dated March 4, 2010 (ADAMS Accession No. ML100480062)"</p>
B 01-001	Circumferential Cracking of Reactor Pressure Vessel (RPV) Head Penetration Nozzles	C 06	<p>NRC acceptance letter dated November 20, 2001 (Unit 1) – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <p>REVISION 02 UPDATE:</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Perform baseline inspection. * Evaluate or repair as necessary. <p>REVISION 03 UPDATE:</p> <p>NRC issued the Safety Evaluation for Bulletin 2001-001 on June 30, 2010.</p> <p>REVISION 04 UPDATE:</p> <p>Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.</p> <p>REVISION 06 UPDATE:</p> <p>The baseline inspection was performed with evaluations and repairs as necessary.</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)"</p> <p>NRC Inspection Report 391/2011-602 closed B 01-001.</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 02-001	RPV Head Degradation and Reactor Coolant Pressure Boundary Integrity	C 06	<p>NRC review of Unit 1's 15 day response in letter dated May 20, 2002 – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <p>REVISION 02 UPDATE:</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Perform baseline inspection. * Evaluate or repair as necessary. <p>REVISION 03 UPDATE:</p> <p>NRC issued the Safety Evaluation for Bulletin 2002-001 on June 30, 2010.</p> <p>REVISION 04 UPDATE:</p> <p>Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.</p> <p>REVISION 06 UPDATE:</p> <p>The baseline inspection was performed with evaluations and repairs as necessary.</p> <p>SSSER22 contained the following for NRC Action:</p> <p>"Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)"</p> <p>NRC Inspection Report 391/2011-602 closed B 02-001.</p>
B 02-002	RPV Head and Vessel Head Penetration Nozzle Inspection Programs	C 06	<p>NRC acceptance letter dated December 20, 2002 (Unit 1) – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <p>REVISION 02 UPDATE:</p> <p>Unit 2 Actions:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>* Perform baseline inspection.</p> <p>* Evaluate or repair as necessary.</p> <p>-----</p> <p>REVISION 03 UPDATE:</p> <p>NRC issued the Safety Evaluation for Bulletin 2002-002 on June 30, 2010.</p> <p>-----</p> <p>REVISION 04 UPDATE:</p> <p>Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>The baseline inspection was performed with evaluations and repairs as necessary.</p> <p>-----</p> <p>SSSER22 contained the following for NRC Action:</p> <p>"Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)"</p> <p>-----</p> <p>NRC Inspection Report 391/2011-602 closed B 02-002.</p>
B 03-002	Leakage from RPV Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity (PWRs)	CI 06	<p>NRC acceptance letter dated October 6, 2004 (Unit 1) – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <p>-----</p> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Bulletin 2003-002 on January 21, 2010.</p> <p>Unit 2 Actions:</p> <p>* Perform baseline inspection.</p> <p>* Evaluate or repair as necessary.</p> <p>-----</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
B 04-001	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at PWRs	CI 06	<p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"</p> <hr/> <p>Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Provide details of pressurizer and penetrations, and * apply Material Stress Improvement Process. <hr/> <p>REVISION 02 UPDATE:</p> <p>TVA provided details of the pressurizer and penetrations on September 29, 2008. This letter committed to:</p> <p>Prior to placing the pressurizer in service, TVA will apply the Material Stress Improvement Process (MSIP) to the Pressurizer Power Operated Relief Valve connections, the safety relief valve connections, the spray line nozzle and surge line nozzle connections.</p> <p>TVA will perform a bare metal visual (BMV) inspection of the upper pressurizer Alloy 600 locations at the first refueling outage.</p> <hr/> <p>REVISION 03 UPDATE:</p> <p>April 1, 2010, letter committed to:</p> <p>TVA will perform NDE prior to and after performance of the MSIP. If circumferential cracking is observed in either pressure boundary or non-pressure boundary portions of any locations covered under the scope of the bulletin, TVA will develop plans to perform an adequate extent-of-condition evaluation, and TVA will discuss those plans with cognizant NRC technical staff prior to starting Unit 2.</p> <p>After performing the BMV inspection during the first refueling outage, if any evidence of apparent reactor coolant pressure boundary leakage is discovered, then NDE capable of determining crack orientation will be performed in order to accurately characterize the flaw, the orientation, and extent. TVA will develop plans to perform an adequate extent of condition evaluation, and plans to possibly expand the scope of NDE to other components in the pressurizer will be discussed with NRC technical staff prior to restarting of Unit 2.</p> <hr/> <p>REVISION 04 UPDATE:</p> <p>NRC issued the Safety Evaluation for Bulletin 2004-001 on August 4, 2010.</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated August 4, 2010 (ADAMS Accession No. ML102080017)"</p>
B 07-001	Security Officer Attentiveness	C 06	Item concerns a multi-unit issue that was completed for both units.
			<p>REVISION 05 UPDATE:</p> <p>The NRC closed this bulletin via letter dated March 25, 2010 (ADAMS Accession No. ML100770549).</p>
			<p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated March 25, 2010 (ADAMS Accession No. ML 100770549)"</p>
GL 82-028	Inadequate Core Cooling Instrumentation System	CO 06	<p>LICENSE CONDITION - Detectors for Inadequate core cooling (II.F.2)</p> <p>In the original SER, the review of the ICC instrumentation was incomplete. The January 24, 1992, letter superseded the previous responses on this issue. TVA letter for Units 1 and 2 dated January 24, 1992, committed to install Westinghouse ICCM-86 and associated hardware. NRC completed the review for Units 1 and 2 in SSER10. For Unit 2 due to obsolescence of the ICCM-86 system, TVA intends to install the Westinghouse Common Q Post-Accident Monitoring System.</p> <p>Unit 2 Action: Install Westinghouse Common Q PAM system.</p>
			<p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. Subsumed as part of NRC staff review of Instrumentation and Controls submitted April 8, 2010."</p>
GL 83-028	<p>"Required Actions Based on Generic Implications of Salem ATWS Events:</p> <p>2.1 – Equipment Classification and Vendor Interface (Reactor Trip System Components)</p>	CI 06	<p>TVA: letters dated November 7, 1983 and August 24, 1990</p> <p>NRC: letters dated October 20, 1986 and June 18, 1990</p> <p>Unit 2 Action:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>Ensure that required information on Critical Structures and Components is properly incorporated into procedures.</p> <p>[WAS "NOTE 3."]</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>Confirmed that required information on Critical Structures and Components is properly incorporated into procedures.</p>
GL 83-028	<p>"Required Actions Based on Generic Implications of Salem ATWS Events:</p> <p>3.2 – Post-Maintenance Testing (All SR Components)</p>	<p>S ----- 06</p>	<p>TVA: letters dated November 7, 1983, January 17, 1986 and November 1, 1993</p> <p>NRC: letters dated December 10, 1985, October 27, 1986, and July 2, 1990; IR 390, 391/86-04</p> <p>-----</p> <p>Unit 2 Action:</p> <p>Test and maintenance procedures and Technical Specifications will include post-maintenance operability testing of other (than reactor trip system) safety-related components.</p> <p>-----</p> <p>REVISION 02 UPDATE:</p> <p>Developmental Revision A of the Unit 2 TS (including the TS Bases) was submitted on March 4, 2009.</p> <p>The Bases for TS Surveillance Requirement 3.0.1 states, in part, "Upon completion of maintenance, appropriate post maintenance testing is required to declare equipment OPERABLE. This includes ensuring applicable Surveillances are not failed and their most recent performance is in accordance with SR 3.0.2."</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>Watts Bar's Preventative Maintenance Program is not unit specific; no further action is required for Unit 2.</p>
GL 83-028	<p>"Required Actions Based on Generic Implications of Salem ATWS Events:</p> <p>4.1 – Reactor Trip System Reliability (Vendor Related Modifications)</p>	<p>CO ----- 06</p>	<p>TVA: letter dated May 19, 1986</p> <p>-----</p> <p>Unit 2 Action:</p> <p>Confirm vendor-recommended DS416 breaker modifications are implemented.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			NRC Inspection Report 391/2011-602 closed GL 83-028, Item 4.1.
GL 88-005	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR plants	CI 06	NRC acceptance letter dated August 8, 1990 for both units. Unit 2 Action: Implement program.
			REVISION 06 UPDATE: The program has been implemented on Unit 2.
GL 88-020	Individual Plant Examination for Severe Accident Vulnerabilities	S 06	Unit 2 Action: Complete evaluation for Unit 2.
			REVISION 02 UPDATE: The Probabilistic Risk Assessment Individual Plant Examination Summary Report was submitted on February 9, 2010.
			REVISION 04 UPDATE: The Individual Plant Examination of External Events Design Report was submitted on April 30, 2010.
			REVISION 06 UPDATE: The NRC issued Requests for Additional Information (RAIs) on November 12, 2010. TVA responded to the RAIs on December 17, 2010, and April 1, 2011.
GL 89-013	Service Water System Problems Affecting Safety-Related Equipment	CI 06	NRC letters dated July 9, 1990 and June 13, 1997, accepting approach. Unit 2 Actions: 1) Implement initial performance testing of the heat exchangers; and 2) Establish eddy current baseline data for the Containment Spray heat exchangers.
			REVISION 06 UPDATE: NRC Inspection Report 391/2011-602 closed GL 89-013.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
GL 89-021	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	S 06	<p>TVA responded to GL 89-21 with the status of USIs for both units on November 29, 1989. NRC provided an assessment of WBN USI status on May 1, 1990. The NRC assessment included a list of incomplete USIs for WBN. USIs were initially reviewed for WBN in the SER Appendix C. USIs were subsequently reviewed in SSER 15 Appendix C (June 1995) and SSER 16 (September 1995).</p> <p>Unit 2 actions:</p> <ul style="list-style-type: none"> * Provide a status of WBN Unit 2 USIs. * Complete implementation of USIs. <hr/> <p>REVISION 02 UPDATE:</p> <p>Status of USIs was provided by Enclosure 2 of TVA letter dated September 26, 2008.</p> <p>The applicable USIs are either closed, deleted, or captured in either the SER Framework or the Generic Communications Framework, or they are part of the CAPs and SPs.</p> <hr/> <p>REVISION 06 UPDATE:</p> <p>Updated status of USIs was provided on January 25, 2011.</p>
GL 93-004	Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies, 10 CFR 50.54(f)	CO 06	<p>NRC letter dated December 9, 1994, accepted TVA commitments for both units.</p> <p>Unit 2 Action: Implement modifications and testing.</p> <hr/> <p>REVISION 06 UPDATE:</p> <p>NRC Inspection Report 391/2011-604 closed GL 93-004.</p>
GL 95-003	Circumferential Cracking of Steam Generator Tubes	CI 06	<p>NRC acceptance letter dated May 16, 1997 (Unit 1) – Initial response for Unit 2 on September 7, 2007. TVA responded to a request for additional information on December 17, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <hr/> <p>REVISION 02 UPDATE:</p> <p>Unit 2 Action:</p> <ul style="list-style-type: none"> * Perform baseline inspection. * Evaluate or repair as necessary. <hr/>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)."</p> <p>-----</p> <p>100% of the steam generator tubes have been inspected.</p>
GL 95-005	Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking	C ----- 06	<p>No specific action or response required by the GL; TVA responded on September 7, 2007.</p> <p>-----</p> <p>REVISION 02 UPDATE:</p> <p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)."</p>
GL 95-007	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	CI ----- 06	<p>Unit 1 SER for GL 95-07 dated Sept 15, 1999</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Perform evaluation for pressure locking and thermal binding of safety related power-operated gate valves, and * take corrective actions for those valves identified as being susceptible. <p>-----</p> <p>REVISION 03 UPDATE:</p> <p>April 1, 2010, letter committed to evaluate missing GL 89-10 motor-operated valves for susceptibility to pressure locking and thermal binding.</p> <p>-----</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 04 UPDATE:</p> <p>NRC letter dated July 29, 2010, provided RAIs on the GL.</p> <p>TVA letter dated July 30, 2010, answered the RAIs and provided the following commitments:</p> <ul style="list-style-type: none"> * EDCRs 53292 and 53287 shall be implemented to eliminate the potential for pressure locking prior to startup. * Valves 2-FCV-63-25 and -26 will be evaluated for impact due to new parameters from the JOG Topical Report MPR 2524A prior to startup. <p>-----</p> <p>NRC issued the Safety Evaluation for GL 1995-007 on August 12, 2010.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>TVA letter to NRC dated July 30, 2010, documented that none of the missing Watts Bar Unit 2 GL 89-10 valves are GL 95-07 valves.</p> <p>-----</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated August 12, 2010 (ADAMS Accession No. ML100190443)"</p>
GL 96-003	Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits	CI 06	<p>No response required</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Submit Pressure Temperature limits, and * similar to Unit 1, upon approval, incorporate into licensee-controlled document. <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>The Pressure and Temperature Limits Report (PTLR) was submitted via TVA to NRC letter dated February 2, 2010.</p> <p>The PTLR was incorporated in the system description for the Reactor Coolant System (WBN2-68-4001).</p>
GL 96-006	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	C 06	<p>NRC letter dated April 6, 1999, accepting TVA response for Unit 1.</p> <p>Unit 2 Action:</p> <p>Implement modification to provide containment penetration relief.</p> <p>-----</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 1996-006 on January 21, 2010.</p> <hr/> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML100130227)."</p> <hr/> <p>Modification to provide containment penetration relief was implemented.</p> <hr/> <p>NRC Inspection Report 391/2011-603 closed GL 96-006.</p>
GL 97-001	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	CI 06	<p>NRC acceptance letter dated November 4, 1999 (Unit 1).</p> <p>Unit 2 Action: Provide a report to address the inspection program.</p> <hr/> <p>REVISION 03 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 97-001 on June 30, 2010.</p> <hr/> <p>REVISION 04 UPDATE:</p> <p>Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.</p> <hr/> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated June 30, 2010 (ADAMS Accession No. ML100539515)"</p>
GL 97-004	Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps	CI 06	<p>NRC acceptance letter dated June 17, 1998 (Unit 1) – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Install new sump strainers, and * perform other modification-related activities identical to Unit 1.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 1997-004 on February 18, 2010.</p> <p>REVISION 06 UPDATE:</p> <p>See the REVISION 06 UPDATE for GL 04-002 for new commitments.</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated February 18, 2010 (ADAMS Accession No. ML100200375)"</p>
GL 97-005	Steam Generator Tube Inspection Techniques	CI 06	<p>NRC acceptance letter dated September 22, 1998 (Unit 1) - Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action:</p> <p>Employ the same approach used on the original Unit 1 SGs. TVA responded to a request for additional information on December 17, 2007.</p> <p>REVISION 02 UPDATE:</p> <p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01.</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"</p>
GL 97-006	Degradation of Steam Generator Internals	CI 06	<p>NRC acceptance letter dated October 19, 1999 (Unit 1) - Initial response for Unit 2 on September 7, 2007. TVA responded to a request for additional information on December 17, 2007.</p> <p>Unit 2 Action: Perform SG inspections during each refueling outage.</p> <p>REVISION 02 UPDATE:</p> <p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>01.</p> <hr/> <hr/> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"</p>
GL 98-002	Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	CI <hr/> 06	<p>Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Actions:</p> <ol style="list-style-type: none"> 1) Review the ECCS designs to ensure they do not contain design features which can render them susceptible to common-cause failures; and 2) document the results. <hr/> <hr/> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 1998-002 on March 3, 2010.</p> <hr/> <hr/> <p>REVISION 03 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 98-002 on May 11, 2010. This letter noted that it superseded the SE issued by NRC on March 3, 2010.</p> <hr/> <p>April 1, 2010, letter committed to ensure that the guidance added to the Unit 1 procedure as a result of the review of NRC GL 98-02 is incorporated into the Unit 2 procedures. Specifically, when decreasing power, valve HCV-74-34, Refueling Water Return (normally locked closed valve) has a hold order placed with specific release criteria before entry into Mode 4 and to remove the hold order before entry into Mode 3 when returning to power.</p> <hr/> <hr/> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated May 11, 2010 (ADAMS Accession No. ML101200155)"</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
GL 98-004	Potential for Degradation of the ECCS and the Containment Spray System After a LOCA Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	CI 06	<p>NRC closure letter dated November 24, 1999 (Unit 1). – Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Install new sump strainers, and * perform other modification-related activities identical to Unit 1. <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 1998-004 on February 1, 2010.</p> <p>REVISION 06 UPDATE:</p> <p>See the REVISION 06 UPDATE for GL 04-002 for new commitments.</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated February 1, 2010 (ADAMS Accession No. ML100260594)"</p>
GL 03-001	Control Room Habitability	S 06	<p>Initial response for Unit 2 on September 7, 2007</p> <p>Unit 2 Action: Incorporate TSTF-448 into Technical Specifications.</p> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 2003-01 on February 1, 2010.</p> <p>Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010.</p> <p>TS Surveillance Requirement 3.7.10.4 requires performance of a Control Room Envelope (CRE) unfiltered air inleakage test in accordance with the CRE Habitability Program.</p> <p>TS 5.7.2.20 provides for the CRE Habitability Program.</p> <p>These portions of the Unit 2 TS were based on the Unit 1 TS which incorporated TSTF-448 per Amendment 70 (NRC approved A70 on 10/08/2008).</p> <p>REVISION 06 UPDATE:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated February 1, 2010 (ADAMS Accession No. ML100270076)"</p>
GL 04-001	Requirements for Steam Generator Tube Inspection	CI 06	<p>NRC acceptance letter dated April 8, 2005 (Unit 1) - Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Perform baseline inspection.</p> <p>REVISION 02 UPDATE:</p> <p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01.</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"</p> <p>100% of the steam generator tubes have been inspected.</p>
GL 04-002	Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at PWRs	OV 06	<p>NRC Audit Report dated February 7, 2007 (Unit 1) - Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Install new sump strainers, and * perform other modification-related activities identical to Unit 1. <p>REVISION 06 UPDATE:</p> <p>Additional TVA letters concerning GL 2004-02 were sent to the NRC on the following dates:</p> <ul style="list-style-type: none"> - January 29, 2008, - May 19, 2008, - September 10, 2010, - March 4, 2011, and - April 29, 2011. <p>The March 4, 2011, letter provided a response that superseded previous responses and commitments. It provided the following new commitments:</p> <ul style="list-style-type: none"> - Unit 2 will install sump modifications per the requirements of Generic Letter (GL) 2004-02 prior to Unit 2 fuel load.

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>- A confirmatory walkdown for loose debris will be performed on Unit 2 after containment work is completed and the containment has been cleaned. This walkdown will be completed prior to startup.</p> <p>- New throttle valves will be installed in the CVCS and SI injection lines to the RCS. The new valves will be opened sufficiently to preclude downstream blockage.</p> <p>- The current Unit 1 TVA protective coating program contains requirements for conducting periodic visual examinations of Coating Service Level I and Level II protective coatings. The Unit 2 program will be the same.</p> <p>- Procedural controls will be put in place at WBN Unit 2 to ensure that potential quantities of post-accident debris are maintained within the bounds of the analyses and design bases that support ECCS and CSS recirculation functions.</p> <p>- TVA will complete the WBN in-vessel downstream effects evaluation discussed in the supplemental response to Generic Letter 2004-02 following issuance of the final NRC Safety Evaluation Report (SER) for Topical Report No. WCAP-16793-NP, "Evaluation of Long-Term Cooling Considering Particulate, Fibrous, and Chemical Debris in the Recirculating Fluid."</p> <p>- The design basis of the modified emergency sump strainer has been incorporated into the plant's current licensing basis. The WBN Unit 2 FSAR will be amended to include this information.</p> <p>-----</p> <p>- Unit 1 and Unit 2 share a common protective coatings program.</p> <p>- Amendment 103 to the Unit 2 FSAR was submitted to the NRC on March 15, 2010. This amendment included the design basis of the modified emergency sump strainer.</p>
GL 06-001	Steam Generator Tube Integrity and Associated Technical Specifications	S ----- 06	<p>Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Incorporate TSTF-449 into Technical Specifications.</p> <p>-----</p> <p>REVISION 02 UPDATE:</p> <p>On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01.</p> <p>-----</p> <p>Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010.</p> <p>TS 5.7.2.12 is the Steam Generator (SG) Program. This program is implemented to ensure that SG tube integrity is maintained.</p> <p>Unit 2 TS 5.7.2.12 was based on Unit 1 TS 5.7.2.12. Unit 1 TS 5.7.2.1.12 was based on TSTF-449 (NRC approved Unit 1 TS A65 on 1/03/2006).</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061) (See Appendix HH)"</p> <p>-----</p> <p>The applicable item from SER22, Appendix HH for this item is Open item 6, "Verify implementation of TSTF-449. (TVA letter dated September 7, 2007, ADAMS Accession No. ML072570676)."</p> <p>TVA to NRC letter dated April 6, 2011 provided the following response to Open Item 6:</p> <p>"Amendment 65 to the Unit 1 TS revised the existing steam generator tube surveillance program and was modeled after TSTF-449, Rev. 4. The NRC approved Amendment 65 via letter dated November 3, 2006, 'Watts Bar Nuclear Plant, Unit 1 - Issuance of Amendment Regarding Steam Generator Tube Integrity (TS-05-10) (TAC No. MC9271).' Revision 82 made the associated changes to the Unit 1 TS Bases.</p> <p>Developmental Revision A to the Unit 2 TS and TS Bases made the equivalent changes to the Unit 2 TS / TS Bases. Affected TS sections include the following: LEAKAGE definition in 1.1, LCO 3.4.13 (RCS Operational LEAKAGE), LCO 3.4.17 (SG Tube Integrity), 5.7.2.12 (Steam Generator (SG) Program), and 5.9.9 (Steam Generator Tube Inspection Report).</p> <p>Developmental Revision A of the Unit 2 TS was submitted to the NRC via letter dated March 4, 2009, 'Watts Bar Nuclear Plant (WBN) Unit 2 - Operating License Application Update,' (ADAMS Accession number ML090700378)."</p>
GL 06-002	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	CI 06	<p>Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action:</p> <p>Complete the two unit baseline electrical calculations and implementing procedures.</p> <p>-----</p> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 2006-002 on January 20, 2010.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061) (See Appendix HH)"</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			Note that the correct date and ADAMS Accession No. are January 20, 2010, and ML100080768, respectively.
GL 06-003	Potentially Nonconforming Hemyc and MT Fire Barrier Configurations	CI 06	<p>TVA does not rely on Hemyc or MT materials to protect electrical and instrumentation cables or equipment that provide safe shutdown capability during a postulated fire.</p> <p>Unit 2 Action:</p> <p>Addressed in CAP/SP.</p> <p>The Fire Protection Corrective Action Program will ensure Unit 2 conforms with NRC requirements and applicable guidelines.</p> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 2006-003 on February 25, 2010.</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed. NRC Letter dated February 25, 2010 (ADAMS Accession No. ML100470398)"</p>
GL 07-001	Inaccessible or Underground Power Cable Failures That Disable Accident Mitigation Systems or Cause Plant Transients	CI 06	<p>Initial response for Unit 2 on September 7, 2007.</p> <p>Unit 2 Action: Complete testing of four additional cables.</p> <p>REVISION 02 UPDATE:</p> <p>NRC issued the Safety Evaluation for Generic Letter 2007-001 on January 26, 2010.</p> <p>REVISION 04 UPDATE:</p> <p>NRC Inspection Report 391/2010-603 closed GL 2007-001.</p> <p>REVISION 06 UPDATE:</p> <p>The four additional cables passed the testing.</p> <p>SSER22 contained the following for NRC Action:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			"Closed. NRC Letter dated January 26, 2010 (ADAMS Accession No. ML100120052)"
GL 08-001	Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray Systems	O 06	<p>Initial response for Unit 2 on October 1, 2008.</p> <hr/> <p>REVISION 02 UPDATE:</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> - TVA will provide a submittal within 45 days of completion of the engineering for the ECCS, RHR, and CSS systems. - WBN Unit 2 will complete the required modifications and provide a submittal consistent with the information requested in the GL 90 days prior to fuel load. <hr/> <p>REVISION 06 UPDATE:</p> <p>The submittal was provided in TVA to NRC letter dated March 11, 2011. This submittal satisfied the above Unit 2 actions and generated the following new commitments:</p> <ul style="list-style-type: none"> - TVA will evaluate adopting the revised ISTS SR 3.5.2.3 (NUREG 1431) at WBN within 6 months of NRC approval of the Traveler. - Complete evaluation of CS pump 2A-A pipe chase horizontal suction piping for venting. Add a vent valve to this location or conduct periodic UT examinations if necessary. (90 days prior to fuel load.) - Add vent valves to selected locations in the ECCS and RHRS piping to enhance filling and venting. (90 days prior to fuel load.) - Complete walk down survey of ECCS and RHRS piping and evaluate the piping for latent voids that could exceed 5% of the pipe cross sectional area. (90 days prior to fuel load.) - Operating procedures are being revised to improve instructions for filling and venting portions of the ECCS discharge pipe. (90 days prior to fuel load.) - Complete Preoperational tests on ECCS and RHRS systems to confirm Unit 1 operating experience showing no gas intrusion/accumulation issues. (90 days prior to fuel load.) - Periodic venting procedures used to meet SR 3.5.2.3 are being revised to require that, for an extended gas release, a report is entered into the Corrective Action Program. (90 days prior to fuel load.)
NUREG-0737, I.B.1.2	Independent Safety Engineering Group	OV 06	<p>LICENSE CONDITION - Independent Safety Engineering Group (ISEG) (NUREG-0737, I.B.1.2)</p> <p>Resolved for Unit 1 only in SSER8.</p> <p>Unit 2 action:</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
			<p>Implement the alternate ISEG that was approved for the rest of the TVA units including WBN Unit 1 by NRC on August 26, 1999. The function will be performed by the site engineering organizations.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>By letter of March 2, 1999, TVA proposed to eliminate the ISEG function from the fleet-wide nuclear organization.</p> <p>NRC safety evaluation of August 26, 1999 shows that the NRC accepted the elimination of the ISEG with alternate organizational responsibilities provided in TVA-NQA-PLN89A and TVA-NPOD89-A.</p> <p>By letter of August 26, 1999, TVA revised Topical Report TVA-NPOD89-A, Rev 8 to describe the alternate organizations responsible for the management and operation of TVA's nuclear projects that replaced the ISEG function.</p> <p>The developmental Unit 2 TS were modeled after the Unit 1 TS. There is no reference to the ISEG.</p> <p>The current revision of TVA-NQA-PLN89-A (24A1) was written to include Unit 2.</p> <p>The current revision of TVA-NPOD89-A (18) was written to include Unit 2.</p>
NUREG-0737, I.D.1	Control Room Design Review	CI ----- 06	<p>NRC reviewed in SSER5, SSER6, SSER15, and Appendix EE of SSER16.</p> <p>Unit 2 Actions:</p> <ul style="list-style-type: none"> * Complete the CRDR process. * Perform rewiring in accordance with ECN 5982. * Take advantage of the completed Human Engineering reviews to ensure appropriate configuration for Unit 2 control panels. <p>See CRDR Special Program.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed in SSER22, Section 18.2"</p>
NUREG-0737, II.F.1.2.d.	Accident-Monitoring Instrumentation - Containment Pressure	CO ----- 06	<p>Reviewed in SSER9.</p> <p>Unit 2 Action: Verify installation of containment pressure indication.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>NRC Inspection Report 391/2011-604 closed NUREG-0737, II.F.1.2.d.</p>

ITEM	TITLE	REV	ADDITIONAL INFORMATION
NUREG-0737, II.F.1.2.f.	Accident-Monitoring Instrumentation - Containment Hydrogen	CO	Reviewed in SSER9.
		06	Unit 2 Action: Verify installation of containment hydrogen accident monitoring instrumentation.
			REVISION 06 UPDATE:
			NRC Inspection Report 391/2011-604 closed NUREG-0737, II.F.1.2.F.
NUREG-0737, II.G.1	Power Supplies For Pressurizer Relief Valves, Block Valves and Level Indicators	CI	Reviewed in original 1982 SER and 8.3.3 of SSER7.
		06	Unit 2 Action: Implement modifications such that PORVS and associated Block Valves are powered from same train but different buses.
			REVISION 06 UPDATE:
			Modifications were implemented such that PORVS and associated Block Valves are powered from same train but different buses.
NUREG-0737, II.K.3.3	Reporting SV/RV Failures/Challenges	C	(Action from GL 82-16) – NRC reviewed in Appendix EE of SSER16.
		06	Unit 2 Action: Include, as necessary, in Technical Specifications submittal.
			REVISION 02 UPDATE:
			Developmental Revision A of the Unit 2 Technical Specifications (TS) was submitted on March 04, 2009.
			Rev. 0 of the Unit 1 TS contained 5.9.4 (Monthly Operating Reports) which implemented the above commitment for Unit 1.
			Amendment 57 to the Unit 1 TS (approved by the NRC on March 21, 2005) deleted this section of the TS.
			The markup for Unit 2 Developmental Revision A noted that Unit 2 will apply this change, and the Unit 2 TS will contain no requirement for Monthly Operating Reports.
			REVISION 06 UPDATE:
			SSER22 contained the following for NRC Action:
			"Closed in SSER22, Section 13.5.3."

ITEM	TITLE	REV	ADDITIONAL INFORMATION
NUREG-0737, II.K.3.9	PID Controller	CI	Reviewed in original 1982 SER.
		06	Unit 2 Action: Set the derivative time constant to zero.
REVISION 06 UPDATE:			
The derivative time constant was set to zero.			
NUREG-0737, II.K.3.25	Power On Pump Seals	C	NRC reviewed and closed in IR 390/84-35 based on Diesel Generator (DG) power to pump sealing cooling system.
		06	Unit 2 Action:
Ensure DG power is provided to pump sealing cooling system.			
REVISION 06 UPDATE:			
It was confirmed that DG power is provided to pump sealing cooling system.			
NRC Inspection Report 391/2010-605 closed NUREG-0737, II.K.3.25.			
NUREG-0737, II.K.3.30	Small Break LOCA Methods	C	TVA: letter dated October 29, 1981
		06	NRC: letters dated March 29, 1985, and July 24, 1986; SSER 16
The staff determined in SSER4 that their review of Items II.K.3.30 and II.K.3.31 did not have to be completed to support the full-power license and considered this LICENSE CONDITION resolved in SSER4. In SSER5, the staff further reviewed responses to these items, and concluded that the Units 1 and 2 FSAR methods and analysis met the requirements of II.K.3.30 and II.K.3.31. This item was further reviewed in Appendix EE of SSER16.			
Unit 2 Action: Complete analysis for Unit 2.			
REVISION 06 UPDATE:			
The analysis has been completed for Unit 2.			
NRC Inspection Report 391/2011-603 closed NUREG-0737, II.K.3.30.			

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ITEM	TITLE	REV	ADDITIONAL INFORMATION
NUREG-0737, II.K.3.31	Plant Specific Analysis	C	<p>The staff determined in SSER4 that their review of Items II.K.3.30 and II.K.3.31 did not have to be completed to support the full-power license and considered this LICENSE CONDITION resolved in SSER4. In SSER5, the staff further reviewed responses to these items, and concluded that the Units 1 and 2 FSAR methods and analysis met the requirements of II.K.3.30 and II.K.3.31. This item was further reviewed in Appendix EE of SSER16.</p> <p>Unit 2 Action: Complete analysis for Unit 2.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>The analysis has been completed for Unit 2.</p> <p>-----</p> <p>NRC Inspection Report 391/2011-603 closed NUREG-0737, II.K.3.31.</p>
		06	
NUREG-0737, III.D.3.4	Control-Room Habitability	CI	<p>TVA: letter dated October 29, 1981</p> <p>-----</p> <p>NRC: SSER 16</p> <p>-----</p> <p>NRC reviewed in SER and in Appendix EE of SSER16.</p> <p>Unit 2 Action: Complete with CRDR completion.</p> <p>-----</p> <p>REVISION 06 UPDATE:</p> <p>SSER22 contained the following for NRC Action:</p> <p>"Closed in SSER22, Section 6.4"</p>
		06	

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
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STATUS CODE DEFINITIONS

- C:** CLOSED: Previous staff review of NUREG-0847 and/or supplements has closed the item either for both units at WBN or explicitly for WBN Unit 2.
- CI:** CLOSED/IMPLEMENTATION: Staff has approved either for both units at WBN or explicitly for WBN Unit 2; there is no change to the approved design; and implementation is recommended through Regional Inspection.
- CO:** CLOSED - OPEN: Staff has approved closure of the item; however, TVA actions remain to be completed.
- CT:** CLOSED/TECHNICAL SPECIFICATIONS: Item has been approved either for both units at WBN or explicitly for WBN Unit 2; however, a change to the original approval requires submittal of the Technical Specifications and staff review.
- NA:** NOT APPLICABLE: Justification as to why a section / subsection is not applicable is provided in the ADDITIONAL INFORMATION column.
- O:** OPEN: No action or documentation is provided that shows the staff has reviewed the item for WBN Unit 2.
- OT:** OPEN/TECHNICAL SPECIFICATIONS: No action or documentation is provided that shows the staff has reviewed the item for WBN Unit 2, and the resolution is through submittal of a Technical Specification.
- OV:** OPEN/VALIDATION: The proposed approach has been approved for Watts Bar Unit 1; the same approach is proposed for use on WBN Unit 2 without change.
- S:** SUBMITTED: Information has been submitted, and is under review by NRC staff.