



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 25, 2011

Mr. Thomas Joyce  
President and Chief Nuclear Officer  
PSEG Nuclear LLC  
P.O. Box 236, N09  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE  
OF AMENDMENTS RE: TECHNICAL SPECIFICATION REQUIREMENTS FOR  
SNUBBERS (TAC NOS. ME4796 AND ME4797)

Dear Mr. Joyce:

The Commission has issued the enclosed Amendment Nos. 301 and 284 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, in response to your application dated October 4, 2010, as supplemented by letters dated April 7, 2011, and May 23, 2011. The amendments modify the Technical Specification requirements for snubbers due to revisions to the inservice inspection program.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "R B Ennis", is positioned below the word "Sincerely".

Richard B. Ennis, Senior Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. Amendment No. 301 to  
Renewed License No. DPR-70
2. Amendment No. 284 to  
Renewed License No. DPR-75
3. Safety Evaluation

cc w/encs: Distribution via ListServ



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 301  
Renewed License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated October 4, 2010, as supplemented by letters dated April 7, 2011, and May 23, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 301, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Eddie Miller FOR HKC".

Harold K. Chernoff, Chief  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License  
and the Technical Specifications

Date of Issuance: August 25, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 301

RENEWED FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove  
Page 3

Insert  
Page 3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove  
3/4 7-28  
3/4 7-29  
3/4 7-30  
3/4 7-31  
3/4 7-32  
6-19f

Insert  
3/4 7-28  
3/4 7-29  
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6-19f

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (1) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (2) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 301, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety

## PLANT SYSTEMS

### 3/4.7.9 SNUBBERS

#### LIMITING CONDITION FOR OPERATION

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3.7.9 All snubbers shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

#### ACTION:

With one or more snubbers inoperable, within 72 hours, replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9 on the supported component or declare the supported system inoperable and follow appropriate ACTION statement for that system.

#### SURVEILLANCE REQUIREMENTS

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4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the Snubber Testing Program for examination, testing and service life monitoring of snubbers.

## PLANT SYSTEMS

Pages 3/4 7-30 through 3/4 7-32 have been deleted.

## ADMINISTRATIVE CONTROLS

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### 6.8.4.l Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies. The program shall ensure that Surveillance Requirements specified in the Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies of those Surveillance Requirements for which the Frequency is controlled by the program.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1.
- c. The provisions of Surveillance Requirements 4.0.2 and 4.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program.

### 6.8.4.m Snubber Testing Program

This program conforms to the examination, testing and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

- (i) This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.
- (ii) The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (BPV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b) subject to limitations and modifications listed in 10 CFR 50.55a(b) and subject to Commission approval.
- (iii) The program shall, as allowed by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants" in lieu of Section XI of the ASME BPV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(a)(3).
- (iv) The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(b)(3)(v)) subject to the limitations and modifications listed therein.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 284  
Renewed License No. DPR-75

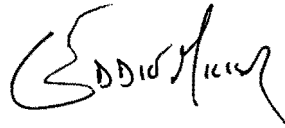
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated October 4, 2010, as supplemented by letters dated April 7, 2011, and May 23, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 284, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

 FOR HKC

Harold K. Chernoff, Chief  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License  
and the Technical Specifications

Date of Issuance: August 25, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 284

RENEWED FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove  
Page 3

Insert  
Page 3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove  
3/4 7-23  
3/4 7-24  
3/4 7-25  
3/4 7-26  
3/4 7-27  
6-19g

Insert  
3/4 7-23  
3/4 7-24  
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6-19g

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
  - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
  - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 284, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

## PLANT SYSTEMS

### 3/4.7.9 SNUBBERS

#### LIMITING CONDITION FOR OPERATION

---

3.7.9 All snubbers shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

#### ACTION:

With one or more snubbers inoperable, within 72 hours, replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9 on the supported component or declare the supported system inoperable and follow appropriate ACTION statement for that system.

#### SURVEILLANCE REQUIREMENTS

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4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the Snubber Testing Program for examination, testing and service life monitoring of snubbers.

Pages 3/4 7-25 through 3/4 7-27 have been deleted.

## ADMINISTRATIVE CONTROLS

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### 6.8.4.l Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies. The program shall ensure that Surveillance Requirements specified in the Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies of those Surveillance Requirements for which the Frequency is controlled by the program.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1.
- c. The provisions of Surveillance Requirements 4.0.2 and 4.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program.

### 6.8.4.m Snubber Testing Program

This program conforms to the examination, testing and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

- (i) This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.
- (ii) The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (BPV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b) subject to limitations and modifications listed in 10 CFR 50.55a(b) and subject to Commission approval.
- (iii) The program shall, as allowed by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants" in lieu of Section XI of the ASME BPV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(a)(3).
- (iv) The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(b)(3)(v)) subject to the limitations and modifications listed therein.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 301 AND 284

TO RENEWED FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75

PSEG NUCLEAR LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By application dated October 4, 2010, as supplemented by letters dated April 7, 2011, and May 23, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML102780066, ML110980107, and ML111430636, respectively), PSEG Nuclear LLC (PSEG or the licensee) submitted license amendment requests for Salem Nuclear Generating Station (Salem) Unit Nos. 1 and 2.

The proposed amendments would revise Technical Specification (TS) 3/4.7.9, "Snubbers," due to planned revisions to the snubber inservice inspection (ISI) and testing program. Specifically, the specific requirements in TS Surveillance Requirement (SR) 4.7.9 would be deleted and replaced with a reference to "the Snubber Testing Program for examination, testing and service life monitoring of snubbers." Due to the changes to SR 4.7.9, Limiting Condition for Operation (LCO) 3.7.9 would revise its existing reference to "Specification 4.7.9c" with a reference to "Specification 4.7.9." In addition, TS Section 6.0, "Administrative Controls," would be revised to add new TS 6.8.4.m, "Snubber Testing Program."

The supplements dated April 7, 2011, and May 23, 2011, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* (FR) on December 28, 2010 (75 FR 81672).

2.0 REGULATORY EVALUATION

Licensees are required to perform the ISI and testing of snubbers in accordance with Section XI of the American Society of Mechanical Engineers (ASME) *Boiler and Pressure Vessel Code* (Code) or the ASME *Code for Operation and Maintenance of Nuclear Power Plants* (OM Code) and the applicable addenda, as required by Section 50.55a(g) of Title 10 of the *Code of Federal Regulations* (10 CFR) or 10 CFR 50.55a(b)(3)(v), except where the NRC has granted specific

Enclosure



relief, pursuant to 10 CFR 50.55a(g)(6)(i), or authorized alternatives pursuant to 10 CFR 50.55a(a)(3).

As discussed in NRC Regulatory Issue Summary 2010-06, "Inservice Inspection and Testing Requirements of Dynamic Restraints (Snubbers)," dated June 1, 2010 (ADAMS Accession No. ML101310338), licensees have the option to control the ASME Code-required ISI and testing of snubbers through their TSs or through licensee-controlled documents (e.g., the Technical Requirements Manual).

In a letter dated July 28, 2003 (ADAMS Accession No. ML032090200), the NRC staff authorized the use of TS 3/4.7.9 for Salem Unit No. 1 for testing and examination of snubbers as an alternative to the requirements contained in Article IWF-5000 of Section XI of the ASME Code. A similar alternative was authorized for Salem Unit No. 2 as documented in the NRC's letter dated May 4, 2005 (ADAMS Accession No. ML051180344). The alternatives were authorized for the respective third ISI interval at each of the units. For Salem Unit No. 1, the third ISI interval began on May 19, 2001, and ended on May 19, 2011. For Salem Unit No. 2, the third ISI interval began on November 27, 2003, and will end on November 27, 2013.

Section 50.55a(g)(4)(ii) of 10 CFR requires that inservice examination of components conducted during successive 120-month inspection intervals must comply with the requirements of the latest edition and addenda of the ASME Code incorporated by reference in 10 CFR 50.55a(b), 12 months before the start of the inspection interval. For the Salem Unit No. 1 fourth 10-year ISI interval, which began on May 20, 2011, the licensee intends to adopt Subsection ISTD of the ASME OM Code, 2004 Edition, in place of the requirements for snubbers in ASME Code, Section XI, Articles IWF-5200(a) and (b) and IWF-5300(a) and (b), as permitted by 10 CFR 50.55a(b)(3)(v). The licensee also intends to adopt Subsection ISTD of the ASME OM Code for the remainder of the Salem Unit No. 2 third 10-year ISI interval which ends on November 27, 2013.

In accordance with 10 CFR 50.55a(g)(5)(ii), if a revised ISI program for a facility conflicts with the TSs for the facility, licensees are required to apply to the Commission for amendment of the TSs to conform the TSs to the revised program. The licensee shall submit this application at least 6 months before the start of the period during which the provisions become applicable. The licensee's application dated October 4, 2010, stated that the proposed TS changes are required to conform the Salem Unit Nos. 1 and 2 TSs to the planned revisions to the ISI program for snubbers.

The Commission's regulatory requirements related to the content of the TSs are set forth in 10 CFR 50.36, "Technical specifications." This regulation requires that the TSs include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) LCOs; (3) SRs; (4) design features; and (5) administrative controls. Paragraph (c)(3) of 10 CFR 50.36 states that SRs are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met.

### 3.0 TECHNICAL EVALUATION

Currently, SR 4.7.9 requires that each snubber be demonstrated OPERABLE by performance of an augmented ISI program in addition to the regular ISI program requirements. The augmented ISI program requirements are specifically delineated in SRs 4.7.9a through 4.7.9f. The specific requirements relate to the following areas:

SR 4.7.9a - Visual Inspection

SR 4.7.9b - Visual Inspection Acceptance Criteria

SR 4.7.9c - Functional Tests

SR 4.7.9d - Hydraulic Snubbers Functional Test Acceptance Criteria

SR 4.7.9e - Mechanical Snubbers Functional Acceptance Criteria

SR 4.7.9f - Snubber Service Life Monitoring

The proposed amendments would delete all the specific requirements in SR 4.7.9a through 4.7.9f. In lieu of these requirements, SR 4.7.9 would be revised to reference a program for the examination, testing and service life monitoring of snubbers. Specifically SR 4.7.9 would read as follows:

Each snubber shall be demonstrated OPERABLE by performance of the Snubber Testing Program for examination, testing and service life monitoring for snubbers.

The amendments would add the following description of the Snubber Testing Program to the Administrative Controls section (Section 6.0) of the TSs:

#### 6.8.4.m Snubber Testing Program

This program conforms to the examination, testing and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

- (i) This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.
- (ii) The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (BPV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b) subject to limitations and modifications listed in 10 CFR 50.55a(b) and subject to Commission approval.
- (iii) The program shall, as allowed by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination

and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants" in lieu of Section XI of the ASME BPV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(a)(3).

- (iv) The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(b)(3)(v)) subject to the limitations and modifications listed therein.

Due to the changes to SR 4.7.9, LCO 3.7.9 would revise its existing reference to "Specification 4.7.9c" with a reference to "Specification 4.7.9."

The NRC staff has reviewed the requirements for the Snubber Testing Program as described in proposed new TS 6.8.4.m. The staff also reviewed the planned revised snubber program requirements as described in the license's supplement dated April 7, 2011. The staff finds that the program requirements are consistent with the ISI and testing requirements for snubbers as required by 10 CFR 50.55a. Therefore, the NRC staff concludes that the proposed new TS 6.8.4.m is acceptable.

Based on the above finding that the Snubber Testing Program is consistent with 10 CFR 50.55a, the NRC staff further finds that the examination, testing and service life monitoring for snubbers in accordance with the Snubber Testing Program, as required by SR 4.7.9, is sufficient to demonstrate whether snubbers are OPERABLE in accordance with LCO 3.7.9. Therefore, the NRC staff concludes that the proposed changes to SR 4.7.9 are consistent with the requirements in 10 CFR 50.36(c)(3) and, therefore, are acceptable.

The NRC staff finds that the proposed change to LCO 3.7.9 to revise its existing reference to "Specification 4.7.9c" with a reference to "Specification 4.7.9" is administrative in nature. Therefore, the staff concludes that this change is acceptable.

Based on the above considerations, the NRC staff concludes that the proposed amendments are acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State Official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such

finding (75 FR 81672). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Bedi  
R. Ennis

Date: August 25, 2011

August 25, 2011

Mr. Thomas Joyce  
President and Chief Nuclear Officer  
PSEG Nuclear LLC  
P.O. Box 236, N09  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE  
OF AMENDMENTS RE: TECHNICAL SPECIFICATION REQUIREMENTS FOR  
SNUBBERS (TAC NOS. ME4796 AND ME4797)

Dear Mr. Joyce:

The Commission has issued the enclosed Amendment Nos. 301 and 284 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, in response to your application dated October 4, 2010, as supplemented by letters dated April 7, 2011, and May 23, 2011. The amendments modify the Technical Specification requirements for snubbers due to revisions to the inservice inspection program.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/ra/

Richard B. Ennis, Senior Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

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Renewed License No. DPR-70
2. Amendment No. 284 to  
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cc w/encls: Distribution via ListServ

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RidsNrrDorIDpr Resource

RidsOgcRp Resource  
GHill, OIS  
RidsNrrPMSalem Resource  
GBedi, CPTB

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OFFICE	LPL1-2/PM	LPL2-1/LA	ITSB/BC	CPTB/BC	OGC	LPL1-2/BC
NAME	REnnis	ABaxter	RElliott (Waig for)	AMcMurtray	MMarsh	HChernoff (GMiller for)
DATE	7/21/11	8/3/11	8/5/11	8/17/11	8/25/11	8/25/11

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