

COMBINED LICENSE

VIRGIL C. SUMMER NUCLEAR STATION UNIT 2

SOUTH CAROLINA ELECTRIC AND GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

Docket No. 52-027

License No. NPF-[XXX]

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for a combined license (COL) for Virgil C. Summer Nuclear Station (VCSNS) Unit 2 filed by South Carolina Electric & Gas Company (SCE&G), acting on behalf of itself and South Carolina Public Service Authority (Santee Cooper), herein referred to as "the VCSNS owners," which incorporates by reference Appendix D to 10 CFR Part 52, complies with the applicable standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
 - B. There is reasonable assurance that the facility will be constructed and will operate in conformity with the application, as amended, the provisions of the Act, and the Commission regulations set forth in 10 CFR Chapter I, except as exempted from compliance in Section 2.F below;
 - C. There is reasonable assurance (i) that the activities authorized by this COL can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission regulations set forth in 10 CFR Chapter I, except as exempted from compliance in Section 2.F below;
 - D. SCE&G¹ is technically qualified to engage in the activities authorized by this license in accordance with the Commission regulations set forth in 10 CFR Chapter I. The VCSNS owners are financially qualified to engage in the activities authorized by this COL in accordance with the Commission regulations set forth in 10 CFR Chapter I;
 - E. The VCSNS owners have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements;"
 - F. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;
 - G. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering reasonable

¹ SCE&G is authorized by Santee Cooper to exercise responsibility and control over the physical construction, operation, and maintenance of the facility.

available alternatives, the issuance of this license subject to the conditions for protection of the environment set forth herein is in accordance with Subpart A of 10 CFR Part 51 and all applicable requirements have been satisfied; and

- H. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the applicable regulations in 10 CFR Parts 30, 40, and 70.
2. On the basis of the foregoing findings regarding this facility, COL No. NPF-[XXX] is hereby issued to SCE&G and Santee Cooper (the licensees), to read as follows:
- A. This license applies to the VCSNS Unit 2, a light-water nuclear reactor and associated equipment (the facility), owned by the VCSNS owners. The facility would be located approximately 1 mile from the center of VCSNS Unit 1 in western Fairfield County, approximately 15 miles west of Winnsboro, and 26 miles northwest of Columbia, SC and is described in the licensee's final safety analysis report (FSAR), as supplemented and amended.
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
 - (1)
 - (a) SCE&G, pursuant to Sections 103 and 185b. of the Act and 10 CFR Part 52, to construct, possess, use, and operate the facility at the designated location in accordance with the procedures and limitations set forth in this license;
 - (b) Santee Cooper pursuant to the Act and 10 CFR Part 52, to possess but not operate the facility at the designated location in Fairfield County, South Carolina, in accordance with the procedures and limitations set forth in this license;
 - (2)
 - (a) SCE&G, pursuant to the Act and 10 CFR Part 70, to receive and possess at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and in amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;
 - (b) SCE&G, pursuant to the Act and 10 CFR Part 70, to use special nuclear material as reactor fuel, after a Commission finding under 10 CFR 52.103(g) has been made, in accordance with the limitations for storage and in amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;
 - (3)
 - (a) SCE&G, pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, at any time before a Commission finding under 10 CFR 52.103(g), such byproduct and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts, as necessary;

- (b) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as necessary;
- (4)
 - (a) SCE&G, pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, before a Commission finding under 10 CFR 52.103(g), in amounts not exceeding those specified in 10 CFR 30.72, any byproduct or special nuclear material that is (1) in unsealed form; (2) on foils or plated surfaces, or (3) sealed in glass, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components;
 - (b) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), in amounts as necessary, any byproduct, source, or special nuclear material without restriction as to chemical or physical form, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components but not uranium hexafluoride; and
- (5) SCE&G, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. The license is subject to, and the licensees shall comply with, all applicable provisions of the Act and the rules, regulations, and orders of the Commission, including the conditions set forth in 10 CFR Chapter I, now or hereafter in effect.
- D. The license is subject to, and SCE&G shall comply with, the conditions specified and incorporated below:
 - (1) Changes during Construction
 - (a) SCE&G may request use of a preliminary acceptability review (PAR) process, for license amendments, at any time before a Commission finding under 10 CFR 52.103(g). To use the PAR process, SCE&G shall submit a written request to the Office of New Reactors (NRO) in accordance with COL-ISG-025, "Changes during Construction under Part 52."
 - (b) Before NRO's issuance of a written PAR notification, SCE&G shall submit the license amendment request (LAR). Thereafter, NRO will issue a written PAR notification, setting forth whether SCE&G may proceed in accordance with the PAR, LAR, and COL-ISG-025. If SCE&G elects to proceed and the LAR is subsequently denied, SCE&G shall return the facility to its current licensing basis.

(2) Pre-operational Testing

- (a) SCE&G shall perform the design-specific pre-operational tests identified below:
1. In-Containment Refueling Water Storage Tank (IRWST) Heatup Test (first plant test as identified in AP1000 Design Control Document (DCD), Rev. 19, Section 14.2.9.1.3 Item (h));
 2. Pressurizer Surge Line Stratification Evaluation (first plant test as identified in AP1000 DCD, Rev. 19, Section 14.2.9.1.7 Item (d));
 3. Reactor Vessel Internals Vibration Testing (first plant test as identified in AP1000 DCD, Rev. 19, Section 14.2.9.1.9);
 4. Core Makeup Tank Heated Recirculation Tests (first three plants test as identified in AP1000 DCD, Rev. 19, Section 14.2.9.1.3 Items (k) and (w)); and
 5. Automatic Depressurization System Blowdown Test (first three plants test as identified in AP1000 DCD, Rev. 19, Section 14.2.9.1.3 Item (s)).
- (b) SCE&G shall review and evaluate the results of the tests identified in Section 2.D.(2)(a) of this license and confirm that these test results are within the range of acceptable values predicted or otherwise confirm that the tested systems perform their specified functions in accordance with AP1000 DCD Rev. 19, Section 14.2.9.
- (c) SCE&G shall notify the Director of NRO, or the Director's designee, in writing, upon successful completion of the design-specific pre-operational tests identified in Section 2.D.(2)(a) of this license; and
- (d) SCE&G shall notify the Director of NRO, or the Director's designee, in writing, upon the successful completion of all the ITAAC included in Appendix C to this license.

(3) Nuclear Fuel Loading and Pre-critical Testing

- (a) Until the submission of the notification required by Section 2.D.(2)(c) of this license, SCE&G shall not load fuel into the reactor vessel;
- (b) Upon submission of the notification required by Section 2.D.(2)(c) of this license and upon a Commission finding in accordance with 10 CFR 52.103(g) that all the acceptance criteria in the ITAAC in Appendix C to this license are met, SCE&G is authorized to

perform pre-critical tests in accordance with the conditions specified herein;

- (c) SCE&G shall perform the pre-critical tests identified in AP1000 DCD Rev. 19, Section 14.2.10.1;
- (d) SCE&G shall review and evaluate the results of the tests identified in Section 2.D.(3)(c) of this license and confirm that these test results are within the range of acceptable values predicted or otherwise confirm that the tested systems perform their specified functions in accordance with AP1000 DCD Rev. 19, Section 14.2.10; and
- (e) SCE&G shall notify the Director of NRO, or the Director's designee, in writing, upon successful completion of the pre-critical tests identified in Section 2.D.(3)(c) of this license.

(4) Initial Criticality and Low-Power Testing

- (a) Upon submission of the notification required by Section 2.D.(3)(e) of this license, SCE&G is authorized to operate the facility at reactor steady-state core power levels not to exceed 5-percent thermal power in accordance with the conditions specified herein;
- (b) SCE&G shall perform the initial criticality and low-power tests identified in AP1000 DCD Rev. 19, Sections 14.2.10.2 and 14.2.10.3, respectively, the Natural Circulation (first plant test) identified in AP1000 DCD Rev. 19, Section 14.2.10.3.6, and the Passive Residual Heat Removal Heat Exchanger (first plant test) identified in AP1000 DCD Rev. 19, Section 14.2.10.3.7;
- (c) SCE&G shall review and evaluate the results of the tests identified in Section 2.D.(4)(b) of this license and confirm that these test results are within the range of acceptable values predicted or otherwise confirm that the tested systems perform their specified functions in accordance with AP1000 DCD Rev. 19, Section 14.2.10.2 and 14.2.10.3; and
- (d) SCE&G shall notify the Director of NRO, or the Director's designee, in writing, upon successful completion of initial criticality and low-power tests identified in Section 2.D.(4)(b) of this license, including the design-specific tests identified therein.

(5) Power Ascension Testing

- (a) Upon submission of the notification required by Section 2.D.(4)(d) of this license, SCE&G is authorized to operate the facility at reactor steady-state core power levels not to exceed 100-percent thermal power in accordance with the conditions specified herein, but only for the purpose of performing power ascension testing;

- (b) SCE&G shall perform the power ascension tests identified in the AP1000 DCD Rev. 19, Section 14.2.10.4, the Rod Cluster Control Assembly Out of Bank Measurements (first plant test) identified in AP1000 DCD, Rev. 19, Section 14.2.10.4.6, and the Load Follow Demonstration (first plant test) identified in AP1000 DCD, Rev. 19, Section 14.2.10.4.22;
- (c) SCE&G shall review and evaluate the results of the tests identified in Section 2.D.(5)(b) of this license and confirm that these test results are within the range of acceptable values predicted or otherwise confirm that the tested systems perform their specified functions in accordance with AP1000 DCD Rev.19, Section 14.2.10.4; and
- (d) SCE&G shall notify the Director of NRO, or the Director's designee, in writing, upon successful completion of power ascension tests identified in Section 2.D.(5)(b) of this license, including the design-specific tests identified therein.

(6) Maximum Power Level

Upon submission of the notification required by Section 2.D.(5)(d) of this license, SCE&G is authorized to operate the facility at steady state reactor core power levels not to exceed 3400 MW thermal (100-percent thermal power), as described in the FSAR, in accordance with the conditions specified herein.

(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in FSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SCE&G shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SCE&G shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively, of this license are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SCE&G shall implement the programs or portions of programs identified below, on or before the date SCE&G achieves the following milestones.

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt of byproduct or special nuclear materials that are not fuel (excluding exempt quantities as described in 10 CFR 30.18);
 - 2. The fire protection measures in accordance with RG 1.189 for new fuel storage area (including adjacent fire areas that could affect the new fuel storage area) implemented before receipt of fuel onsite;
 - 3. All fire protection program features implemented before initial fuel load;
- (f) Standard Radiological Effluent Controls implemented before initial fuel load;
- (g) Offsite Dose Calculation Manual implemented before initial fuel load;
- (h) Radiological Environmental Monitoring Program implemented before initial fuel load;
- (i) Process Control Program implemented before initial fuel load;

- (j) Radiation Protection Program (RPP) (including the ALARA principle) or applicable portions as identified in FSAR Section 12.5 thereof:
 - 1. RPP features applicable to receipt of by-product, source, or special nuclear materials (excluding exempt quantities as described in 10 CFR 30.18) implemented before initial receipt of such materials;
 - 2. RPP features (including the ALARA principle) applicable to new fuel implemented before receipt of initial fuel on site;
 - 3. All other RPP features (including the ALARA principle) except for those applicable to control radioactive waste shipment implemented before initial fuel load;
 - 4. RPP features (including the ALARA principle) applicable to radioactive waste shipment implemented before first shipment of radioactive waste;
- (k) Reactor Operator Training Program implemented 18 months before the scheduled date of initial fuel load;
- (l) Motor-Operated Valve Testing Program implemented before initial fuel load;
- (m) Initial Test Program
 - 1. Construction Test Program implemented before the first construction test;
 - 2. Preoperational Test Program implemented before the first preoperational test; and
 - 3. Startup Test Program implemented before initial fuel load;
- (n) Special Nuclear Material Control and Accounting Program implemented before initial receipt of special nuclear material; and
- (o) Special Nuclear Material Physical Protection Plan implemented before initial receipt of special nuclear material on site.

(11) Operational Program Implementation Schedule

No later than 12 months after issuance of the COL, SCE&G shall submit to the Director of NRO, or the Director's designee, a schedule for implementation of the operational programs listed in FSAR Table 13.4-201, including the associated estimated date for initial loading of fuel. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until all the operational programs listed in FSAR Table 13.4-201 have been fully implemented.

(12) Site- and Unit-specific Conditions

- (a) Before commencing installation of individual piping segments and connected components in their final locations, SCE&G shall complete the as-designed pipe rupture hazards analysis for compartments (rooms) containing those segments in accordance with the criteria outlined in the AP1000 DCD, Rev. 19, Sections 3.6.1.3.2 and 3.6.2.5, and shall inform the Director of NRO, or the Director's designee, in writing, upon the completion of this analysis and the availability of the as-designed pipe rupture hazards analysis reports.
- (b) Before commencing installation of individual piping segments identified in AP1000 DCD, Rev. 19, Section 3.9.8.7, and connected components in their final locations in the facility, SCE&G shall complete the analysis of the as-designed individual piping segments and shall inform the Director of NRO, or the Director's designee, in writing, upon the completion of these analyses and the availability of the design reports for the selected piping packages.
- (c) No later than 180 days before initial fuel load, SCE&G shall submit to the Director of NRO, or the Director's designee, in writing, a fully developed set of plant-specific emergency action levels (EALs) for VCSNS Unit 2 in accordance with Nuclear Energy Institute (NEI) 07-01, "Methodology for Development of Emergency Action Levels Advanced Passive Light Water Reactors," Revision 0, with no deviations. The EALs shall have been discussed and agreed upon with State and local officials.
- (d) SCE&G shall not revise or modify the provisions of Sections 5.3, 5.4, 5.6, 5.9, and 5.10 of the Special Nuclear Material (SNM) Physical Protection Plan until the requirements of 10 CFR 73.55 are implemented.
- (e) No later than 12 months after issuance of the COL, SCE&G shall submit to the Director of NRO, or the Director's designee, a schedule for implementation of the following license conditions. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until

each license condition has been fully implemented. The schedule shall identify the completion of or implementation of the following:

1. The construction and inspection procedures for steel concrete composite (SC) construction activities for seismic Category I nuclear island modules (including shield building SC modules) described in AP1000 DCD Rev. 19, Section 3.8.4.8;
2. The spent fuel rack Metamic Coupon monitoring program (before initial fuel load);
3. Implementation of the flow accelerated corrosion (FAC) program including construction phase activities (before initial fuel load);
4. A turbine maintenance and inspection program, which must be consistent with the maintenance and inspection program plan activities and inspection intervals identified in FSAR Section 10.2.3.6 (before initial fuel load);
5. The availability of documented instrumentation uncertainties to calculate a power calorimetric uncertainty (before initial fuel load);
6. The availability of administrative controls to implement maintenance and contingency activities related to the power calorimetric uncertainty instrumentation (before initial fuel load);
7. The site-specific severe accident management guidelines (before startup testing);
8. The operational and programmatic elements of the mitigative strategies for responding to circumstances associated with loss of large areas of the plant due to explosions or fire developed in accordance with 10 CFR 50.54(hh)(2) (before initial fuel load); and
9. The pre-operational and startup procedures (including the site-specific startup administration manual) identified in FSAR Section 14.2.3 (before initiating the initial test program).

(f) Before initial fuel load, SCE&G shall:

1. Update the seismic interaction analysis in AP1000 DCD, Rev. 19, Section 3.7.3.5 to reflect as-built information, which must be based on as-procured data, as well as the as-constructed condition;

2. Reconcile the seismic analyses described in Section 3.7.2 of the AP1000 DCD, Rev. 19, to account for detailed design changes, including, but not limited to, those due to as-procured or as-built changes in component mass, center of gravity, and support configuration based on as-procured equipment information;
3. Calculate the instrumentation uncertainties of the actual plant operating instrumentation to confirm that either the design limit departure from nucleate boiling ratio (DNBR) values remain valid or that the safety analysis minimum DNBR bounds the new design limit DNBR values plus DNBR penalties;
4. Update the pressure-temperature (P-T) limits using the pressure temperature limits report (PTLR) methodologies approved in AP1000 DCD, Rev. 19, using the plant-specific material properties or confirm that the reactor vessel material properties meet the specifications of and use the Westinghouse generic PTLR curves;
5. Verify that plant-specific belt line material properties are consistent with the properties given in AP1000 DCD Rev. 19, Section 5.3.3.1 and Tables 5.3-1 and 5.3-3. The verification must include a pressurized thermal shock (PTS) evaluation based on as-procured reactor vessel material data and the projected neutron fluence for the plant design objective. Submit this PTS evaluation report to the Director of NRO, or the Director's designee, in writing, at least 18 months before initial fuel load;
6. Review differences between the as-built plant and the design used as the basis for the AP1000 seismic margin analysis. SCE&G shall perform a verification walkdown to identify differences between the as-built plant and the design. SCE&G shall evaluate any differences and must modify the seismic margin analysis as necessary to account for the plant-specific design and any design changes or departures from the certified design. SCE&G shall compare the as-built structures, systems, and components (SSC) high confidence, low probability of failures (HCLPFs) with those assumed in the AP1000 seismic margin evaluation, before initial fuel load. SCE&G shall evaluate deviations from the HCLPF values or assumptions in the seismic margin evaluation due to the as-built configuration and final analysis to determine if vulnerabilities have been introduced;
7. Review differences between the as-built plant and the design used as the basis for the AP1000 probabilistic risk assessment (PRA) and the AP1000 DCD, Rev. 19,

Table 19.59-18. SCE&G shall evaluate the plant-specific PRA-based insight differences and shall modify the plant-specific PRA model as necessary to account for the plant-specific design and any design changes or departure from the PRA certified in Rev. 19 of the AP1000 DCD;

8. Review differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analysis. SCE&G shall evaluate the plant-specific internal fire and internal flood analyses and shall modify the analyses as necessary to account for the plant-specific design and any design changes or departures from the design certified in Rev. 19 of the AP1000 DCD; and
 9. Perform a thermal lag assessment of the as-built equipment listed in Tables 6b and 6c in Attachment A of APP-GW-GLR-069, "Equipment Survivability Assessment," to provide additional assurance that this equipment can perform its severe accident functions during environmental conditions resulting from hydrogen burns associated with severe accidents. SCE&G shall perform this assessment for equipment used for severe accident mitigation that has not been tested at severe accident conditions. SCE&G shall assess the ability of the as-built equipment to perform during accident hydrogen burns using the environment enveloping method or the test based thermal analysis method described in Electric Power Research Institute (EPRI) NP-4354, "Large Scale Hydrogen Burn Equipment Experiments."
- (g) SCE&G shall perform geologic mapping of excavations for safety related structures; examine and evaluate geologic features discovered in these excavations; and shall inform the Director of NRO, or the Director's designee, in writing, once excavations for these safety related structures are open for examination.
- E. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- F. Exemptions
- (1) The following exemptions from any part of the referenced design certification rule meet the requirements of 10 CFR 52.7 and Section VIII.A.4, VIII.B.4, or VIII.C.4 of Appendix D to 10 CFR Part 52, are authorized by law, will not present an undue risk to the public health or safety, and are consistent with the common defense and security. Special circumstances are present in that the application of the regulation in these particular circumstance are not necessary to achieve the

underlying purpose of the rule (10 CFR 50.12(a)(2)(ii)) as described in the application and the staff SER dated XXX. In addition, for exemption 2.G.(1)(b) the exemption will not result in a significant decrease in the level of safety otherwise provided by the design, and the special circumstances outweigh any decrease in the safety that may result from the reduction in standardization caused by the exemption.

- (a) The licensees are exempt from the requirement of 10 CFR Part 52, Appendix D, Section IV.A.2a to include a plant-specific DCD containing the same type of information and using the same organization and numbering as the generic DCD for the AP1000 certified design. This exemption is specific to the organization and numbering scheme in the FSAR and is related to departure number STD DEP 1.1-1 and VCS DEP 2.0-1.
 - (b) The licensees are exempt from the requirement of 10 CFR Part 52, Appendix D, Section IV.A.2d to include information demonstrating compliance with the site parameters and interface requirements. This exemption is specific to the maximum safety wet bulb (noncoincident) air temperature in the FSAR and is related to departure number VCS DEP 2.0-2.
- (2) The following exemptions from regulations were granted in the rulemaking for the design certification rule that is referenced in the application. In accordance with 10 CFR Part 52, Appendix D, Section V, Applicable Regulations, Subsection B, and pursuant to 10 CFR 52.63(a)(5), SCE&G is exempt from portions of the following regulations:
- (a) Paragraph (f)(2)(iv) of 10 CFR 50.34—Plant Safety Parameter Display Console;
 - (b) Paragraph (c)(1) of 10 CFR 50.62—Auxiliary (or emergency) feedwater system; and
 - (c) Appendix A to 10 CFR Part 50, GDC 17—Second offsite power supply circuit.
- (3) For the reasons set forth below, the following specific exemptions which are outside the scope of the design certification rule referenced in the application are granted:
- (a) SCE&G is exempt from the requirements of 10 CFR 70.22(b), 10 CFR 70.32(c), 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51 because SCE&G meets the requirements of 10 CFR 70.17 and 74.7 as follows. The exemption is authorized by law, will not present an undue risk to the public health or safety, and is consistent with the common defense and security. Additionally, special circumstances are present in that the application of the regulations in this particular circumstance is not necessary to achieve the underlying purpose of the rule (10 CFR

50.12(a)(2)(ii)) as described in the FSAR and the staff SER dated **XXX**.

- (b) SCE&G are exempt from the requirements of 10 CFR 52.93(a)(1) as it relates to the exemption granted in Section 2.F.(1)(a) of this license because the exemption meets the requirements of 10 CFR 52.7 because the exemption is authorized by law, will not present an undue risk to the public health or safety, and is consistent with the common defense and security. Additionally, special circumstances are present in that the application of the regulation in this particular circumstance is not necessary to achieve the underlying purpose of the rule (10 CFR 50.12(a)(2)(ii)) as described in the staff SER dated **XXX**.
- H. Following SCE&G's ITAAC closure notifications under paragraph (c)(1) of 10 CFR 52.99 until the Commission makes the finding under 10 CFR 52.103(g), SCE&G shall notify the NRC, in a timely manner, of new information that materially alters the bases for determining that either inspections, tests, or analyses were performed as required, or that acceptance criteria are met. The notification must contain sufficient information to demonstrate that, notwithstanding the new information, the prescribed inspections, tests, or analyses have been performed as required, and the prescribed acceptance criteria are met.
- I. SCE&G shall maintain the guidance and strategies developed in accordance with 10 CFR 50.54(hh)(2).
- J. This license is effective as of **[insert actual date of license issuance]** and shall expire at midnight on the date 40 years from the date that the Commission finds that the acceptance criteria in the combined license are met in accordance with 10 CFR 52.103(g).

FOR THE NUCLEAR REGULATORY
COMMISSION

Michael R. Johnson, Director
Office of New Reactors

Appendices:

Appendix A – Technical Specifications

Appendix B – Environmental Protection Plan

Appendix C – Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC)

APPENDIX A

VIRGIL C. SUMMER NUCLEAR STATION UNIT 2

TECHNICAL SPECIFICATIONS

The unit-specific technical specifications from the Summer COL Application, Part 4, Section B, will be included in Appendix A of the Summer Unit 2 combined license. The applicant has committed to providing the NRC with unit-specific versions of the Unit 2/Unit 3 technical specifications that were submitted in the COL application in a suitable timeframe to support issuance of each individual unit's combined license. These unit-specific technical specifications will exceed 800 pages. Therefore, for ease of handling, the technical specifications are not included in this draft combined license, but can be viewed on the NRC's website at <http://www.nrc.gov/reactors/new-reactors/col/summer/documents.html#application>.

APPENDIX B

VIRGIL C. SUMMER NUCLEAR STATION UNIT 2

ENVIRONMENTAL PROTECTION PLAN

(NONRADIOLOGICAL)

1.0 Objectives of the Environmental Protection Plan

The Environmental Protection Plan (EPP) objectives are to ensure compliance with Biological Opinions issued pursuant to the Endangered Species Act of 1973, as amended (ESA), and to ensure that the U.S. Nuclear Regulatory Commission (NRC) is kept informed of other environmental matters. The EPP is intended to be consistent with Federal, State, and local requirements for environmental protection.

2.0 Environmental Protection Issues

In the Final Environmental Impact Statement (FEIS) dated April 2011, the staff considered the environmental impacts associated with the construction and operation of Virgil C. Summer Nuclear Station Unit Nos. 2 and 3. This EPP applies to the licensees' actions affecting the protected environmental resources evaluated in the FEIS and the licensees' actions that may affect any newly discovered protected environmental resources.

2.1 Aquatic Resources Issues

Federal agencies other than the NRC, such as the U.S. Environmental Protection Agency and the U.S. Army Corps of Engineers, have jurisdiction to regulate aquatic resources under the Federal Water Pollution Control Act (Clean Water Act or CWA) and the Rivers and Harbors Appropriation Act of 1899 (RHA). Certain water quality environmental considerations identified in the FEIS, including effluent limitations, monitoring requirements, and mitigation measures, are regulated under the licensees' CWA permits, such as National Pollutant Discharge Elimination System and Section 404 permits, and RHA Section 10 permit. Nothing within this EPP shall be construed to place additional requirements on the regulation of aquatic resources except the imposition of the requirements in a Biological Opinion under the ESA (see Section 2.3). The licensees are required to inform the NRC of events or situations concerning aquatic resources consistent with the provisions of 10 CFR 50.72(b)(2)(xi), and this EPP does not expand any reporting requirement required by that regulation.

2.2 Terrestrial Resources Issues

Several statutes govern the regulation of terrestrial resources. For example, the U.S. Fish and Wildlife Service (FWS) regulates matters involving migratory birds and their nests in accordance with the Migratory Bird Treaty Act. Activities affecting migratory birds or their nests may require permits under the Migratory Bird Treaty Act. The FWS also regulates matters involving the protection and taking of bald and golden eagles in accordance with the Bald and Golden Eagle Protection Act. The licensees shall inform NRC of any events or situations concerning

terrestrial resources consistent with the provisions of 10 CFR 50.72(b)(2)(xi), and this EPP does not expand any reporting requirement required by that regulation.

2.3 Endangered Species Act of 1973

The NRC may be required to protect some aquatic resources and terrestrial resources in accordance with the ESA. If a Biological Opinion is issued to the NRC in accordance with ESA Section 7 prior to the issuance of the combined license, the licensees shall comply with the terms and conditions set forth in the Incidental Take Statement of the Biological Opinion. If any Federally listed species or critical habitat occurs in an area affected by construction or operation of the plant that was not previously identified as occurring in such areas, including species and critical habitat that were not previously Federally listed, the licensees shall inform the NRC within four hours of discovery. The time of discovery is identified as the specific time when a decision is made to notify another agency or to issue a press release. Similarly, the licensees shall inform the NRC within four hours of discovery of any take, as defined in the ESA, of a Federally listed species or destruction or adverse modification of critical habitat. The four-hour discovery notifications shall be made to the NRC Operations Center via the Emergency Notification System. The licensees shall provide any necessary information to the NRC if the NRC initiates or reinitiates consultation under the ESA.

Unusual Event - The licensees shall inform the NRC of any onsite mortality, injury, or unusual occurrence of any species protected by the ESA within four hours of discovery, followed by a written report in accordance with Section 4.1. The time of discovery is identified as the specific time when a decision is made to notify another agency or to issue a press release. Such incidents shall be reported regardless of the licensees' assessment of causal relation to plant construction or operation.

3.0 Consistency Requirements

The licensees shall notify the NRC of proposed changes to permits or certifications concerning aquatic or terrestrial resources by providing the NRC with a copy of the proposed change(s) at the same time it is submitted to the permitting agency. The licensees shall provide the NRC with a copy of the application for renewal of permits or certifications at the same time the application is submitted to the permitting agency.

Changes to or renewals of such permits or certifications shall be reported to the NRC within 30 days following the later of the date the change or renewal is approved or the date the change becomes effective. If a permit or certification, in part or in its entirety, is appealed and stayed, the NRC shall be notified within 30 days following the date the stay is granted.

4.0 Administrative Procedures

4.1 Plant Reporting Requirements: Non-routine Reports

A written report shall be submitted to the NRC within 30 days of occurrence of any unusual event described in Section 2.3 of this EPP. The report shall: (a) describe, analyze, and evaluate the event, including extent and magnitude of the impact and plant operating characteristics at the time of the event, (b) describe the probable cause of the event, (c) indicate the action taken to correct the reported event, (d) indicate the corrective action taken to

preclude repetition of the event and to prevent similar occurrences involving similar components or systems, and (e) indicate the agencies notified and their preliminary responses.

Events reportable under this subsection, which also require reports to other Federal, State, or local agencies, shall be reported in accordance with those reporting requirements in lieu of the requirements of this subsection. The NRC shall be provided a copy of such report at the same time it is submitted to the other agency.

4.2 Review and Audit

The licensees shall provide for review and audit of compliance with Section 2.3 of this EPP. The audits shall be conducted independently of the individual or groups responsible for performing the specific activity. A description of the organizational structure utilized to achieve the independent review and audit function and results of the audit activities shall be maintained and made available for inspection.

4.3 Records Retention

Records required by this EPP shall be made and retained in a manner convenient for review and inspection. These records shall be made available to the NRC on request. The records, data, and logs relating to this EPP shall be retained for five years or, where applicable, in accordance with the requirements of other agencies.

4.4 Changes in Environmental Protection Plan

A request for a change in the EPP shall include an assessment of the environmental impact of the proposed change and a supporting justification. Implementation of such changes in the EPP shall not commence prior to NRC approval of the proposed changes in the form of a license amendment incorporating the appropriate revision to the EPP.

The licensees shall request a license amendment to incorporate the requirements of any Terms and Conditions set forth in the Incidental Take Statement of applicable Biological Opinions issued subsequent to the effective date of this EPP

APPENDIX C

VIRGIL C. SUMMER NUCLEAR STATION UNIT 2

INSPECTIONS, TESTS, ANALYSES, AND ACCEPTANCE CRITERIA (ITAAC)

The ITAAC Master List is a table of unit-specific ITAAC, which are from the DCD and the Summer COL. The consolidated set of unit-specific ITAAC will be included in Appendix C of the Summer Unit 2 combined license. These unit-specific ITAAC details will exceed 170 pages. Therefore, for ease of handling, the ITAAC details are not included in this draft combined license, but can be viewed on the NRC's website at the following URLs:

- COL Application Part 10, Rev. 5 – <http://www.nrc.gov/reactors/new-reactors/col/summer/documents.html#application>
- Westinghouse AP1000 DCD, Rev. 19, document files 5 through 19 – <http://pbadupws.nrc.gov/docs/ML1117/ML11171A500.html>

	Section No.	Tier 1	Source
	1.0	Introduction	DCD19
	1.1	Definitions	DCD19
	1.2	General Provisions	DCD19
	1.3	Figure Legend	DCD19
	1.4	List of Acronyms and Abbreviations	DCD19

No.	ITAAC No.	Plant System ITAAC	Source
1	2.1.01.01	Fuel Handling and Refueling System	DCD19
2	2.1.01.02		DCD19
3	2.1.01.03		DCD19
4	2.1.01.04		DCD19
5	2.1.01.05		DCD19
6	2.1.01.06.i		DCD19
7	2.1.01.06.ii		DCD19
8	2.1.01.07.i		DCD19
9	2.1.01.07.ii		DCD19
10	2.1.01.07.iii		DCD19
11	2.1.01.07.iv		DCD19
12	2.1.02.01	Reactor Coolant System	DCD19
13	2.1.02.02a		DCD19
14	2.1.02.02b		DCD19
15	2.1.02.03a		DCD19
16	2.1.02.03b		DCD19
17	2.1.02.04a		DCD19
18	2.1.02.04b		DCD19
19	2.1.02.05a.i		DCD19
20	2.1.02.05a.ii		DCD19
21	2.1.02.05a.iii		DCD19
22	2.1.02.05b		DCD19
23	2.1.02.06		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
24	2.1.02.07a.i	Reactor Coolant System (cont'd)	DCD19
25	2.1.02.07a.ii		DCD19
26	2.1.02.07b		DCD19
27	2.1.02.07c		DCD19
28	2.1.02.08a.i		DCD19
29	2.1.02.08a.ii		DCD19
30	2.1.02.08b		DCD19
31	2.1.02.08c		DCD19
32	2.1.02.08d.i		DCD19
33	2.1.02.08d.ii		DCD19
34	2.1.02.08d.iii		DCD19
35	2.1.02.08d.iv		DCD19
36	2.1.02.08d.v		DCD19
37	2.1.02.08d.vi		DCD19
38	2.1.02.08d.vii		DCD19
39	2.1.02.08d.viii		DCD19
40	2.1.02.08e		DCD19
41	2.1.02.09a		DCD19
42	2.1.02.09b.i		DCD19
43	2.1.02.09b.ii		DCD19
44	2.1.02.09c		DCD19
45	2.1.02.10		DCD19
46	2.1.02.11a.i		DCD19
47	2.1.02.11a.ii		DCD19
48	2.1.02.11b.i		DCD19
49	2.1.02.11b.ii		DCD19
50	2.1.02.11b.iii		DCD19
51	2.1.02.11c.i		DCD19
52	2.1.02.11c.ii		DCD19
53	2.1.02.12a.i		DCD19
54	2.1.02.12a.ii		DCD19
55	2.1.02.12a.iii		DCD19
56	2.1.02.12a.iv		DCD19
57	2.1.02.12a.v		DCD19
58	2.1.02.12a.vi		DCD19
59	2.1.02.12a.vii		DCD19
60	2.1.02.12a.viii		DCD19
61	2.1.02.12a.ix		DCD19
62	2.1.02.12b		DCD19
63	2.1.02.13a		DCD19
64	2.1.02.13b		DCD19
65	2.1.02.13c		DCD19
66	2.1.02.14		DCD19
67	2.1.02.15		DCD19
68	2.1.03.01	Reactor System	DCD19
69	2.1.03.02a		DCD19
70	2.1.03.02b		DCD19
71	2.1.03.02c		DCD19
72	2.1.03.03		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
73	2.1.03.04	Reactor System (cont'd)	DCD19
74	2.1.03.05		DCD19
75	2.1.03.06.i		DCD19
76	2.1.03.06.ii		DCD19
77	2.1.03.06.iii		DCD19
78	2.1.03.07.i		DCD19
79	2.1.03.07.ii		DCD19
80	2.1.03.08		DCD19
81	2.1.03.09a.i		DCD19
82	2.1.03.09a.ii		DCD19
83	2.1.03.09b		DCD19
84	2.1.03.09c		DCD19
85	2.1.03.10		DCD19
86	2.1.03.11		DCD19
87	2.1.03.12		DCD19
88	2.1.03.13		DCD19
89	2.1.03.14		DCD19
90	2.2.01.01	Containment System	DCD19
91	2.2.01.02a		DCD19
92	2.2.01.02b		DCD19
93	2.2.01.03a		DCD19
94	2.2.01.03b		DCD19
95	2.2.01.04a.i		DCD19
96	2.2.01.04a.ii		DCD19
97	2.2.01.04b		DCD19
98	2.2.01.05.i		DCD19
99	2.2.01.05.ii		DCD19
100	2.2.01.05.iii		DCD19
101	2.2.01.06a.i		DCD19
102	2.2.01.06a.ii		DCD19
103	2.2.01.06b		DCD19
104	2.2.01.06c		DCD19
105	2.2.01.06d.i		DCD19
106	2.2.01.06d.ii		DCD19
107	2.2.01.07.i		DCD19
108	2.2.01.07.ii		DCD19
109	2.2.01.08		DCD19
110	2.2.01.09		DCD19
111	2.2.01.10a		DCD19
112	2.2.01.10b		DCD19
113	2.2.01.10c		DCD19
114	2.2.01.11a.i		DCD19
115	2.2.01.11a.ii		DCD19
116	2.2.01.11a.iii		DCD19
117	2.2.01.11a.iv		DCD19
118	2.2.01.11b		DCD19
119	2.2.02.01	Passive Containment Cooling System	DCD19
120	2.2.02.02a		DCD19
121	2.2.02.02b		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
122	2.2.02.03a	Passive Containment Cooling System (cont'd)	DCD19
123	2.2.02.03b		DCD19
124	2.2.02.04a		DCD19
125	2.2.02.04b		DCD19
126	2.2.02.05a.i		DCD19
127	2.2.02.05a.ii		DCD19
128	2.2.02.05a.iii		DCD19
129	2.2.02.05b		DCD19
130	2.2.02.05c		DCD19
131	2.2.02.06a.i		DCD19
132	2.2.02.06a.ii		DCD19
133	2.2.02.06b		DCD19
134	2.2.02.06c		DCD19
135	2.2.02.07a.i		DCD19
136	2.2.02.07a.ii		DCD19
137	2.2.02.07a.iii		DCD19
138	2.2.02.07b.i		DCD19
139	2.2.02.07b.ii		DCD19
140	2.2.02.07b.iii		DCD19
141	2.2.02.07c		DCD19
142	2.2.02.07d		DCD19
143	2.2.02.07e.i		DCD19
144	2.2.02.07e.ii		DCD19
145	2.2.02.07f.i		DCD19
146	2.2.02.07f.ii		DCD19
147	2.2.02.08a		DCD19
148	2.2.02.08b		DCD19
149	2.2.02.08c		DCD19
150	2.2.02.09		DCD19
151	2.2.02.10a		DCD19
152	2.2.02.10b		DCD19
153	2.2.02.10c		DCD19
154	2.2.02.11a.i		DCD19
155	2.2.02.11a.ii		DCD19
156	2.2.02.11a.iii		DCD19
157	2.2.02.11b		DCD19
158	2.2.03.01	Passive Core Cooling System	DCD19
159	2.2.03.02a		DCD19
160	2.2.03.02b		DCD19
161	2.2.03.03a		DCD19
162	2.2.03.03b		DCD19
163	2.2.03.04a		DCD19
164	2.2.03.04b		DCD19
165	2.2.03.05a.i		DCD19
166	2.2.03.05a.ii		DCD19
167	2.2.03.05a.iii		DCD19
168	2.2.03.05b		DCD19
169	2.2.03.06		DCD19
170	2.2.03.07a.i		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
171	2.2.03.07a.ii	Passive Core Cooling System (cont'd)	DCD19
172	2.2.03.07b		DCD19
173	2.2.03.07c		DCD19
174	2.2.03.08a		DCD19
175	2.2.03.08b.01		DCD19
176	2.2.03.08b.02		DCD19
177	2.2.03.08c.i.01		DCD19
178	2.2.03.08c.i.02		DCD19
179	2.2.03.08c.i.03		DCD19
180	2.2.03.08c.i.04		DCD19
181	2.2.03.08c.ii		DCD19
182	2.2.03.08c.iii		DCD19
183	2.2.03.08c.iv.01		DCD19
184	2.2.03.08c.iv.02		DCD19
185	2.2.03.08c.iv.03		DCD19
186	2.2.03.08c.iv.04		DCD19
187	2.2.03.08c.v.01		DCD19
188	2.2.03.08c.v.02		DCD19
189	2.2.03.08c.vi.01	Steam Generator System	DCD19
190	2.2.03.08c.vi.02		DCD19
191	2.2.03.08c.vi.03		DCD19
192	2.2.03.08c.vii		DCD19
193	2.2.03.08c.viii		DCD19
194	2.2.03.08c.ix		DCD19
195	2.2.03.08c.x		DCD19
196	2.2.03.08c.xi		DCD19
197	2.2.03.08c.xii		DCD19
198	2.2.03.08c.xiii		DCD19
199	2.2.03.08c.xiv		DCD19
200	2.2.03.08d		DCD19
201	2.2.03.09a.i		DCD19
202	2.2.03.09a.ii		DCD19
203	2.2.03.09a.iii		DCD19
204	2.2.03.09b		DCD19
205	2.2.03.09c		DCD19
206	2.2.03.10		DCD19
207	2.2.03.11a.i	DCD19	DCD19
208	2.2.03.11a.ii		DCD19
209	2.2.03.11b.i		DCD19
210	2.2.03.11b.ii		DCD19
211	2.2.03.11b.iii		DCD19
212	2.2.03.11c.i		DCD19
213	2.2.03.11c.ii		DCD19
214	2.2.03.12a.i		DCD19
215	2.2.03.12a.ii		DCD19
216	2.2.03.12a.iv		DCD19
217	2.2.03.12b		DCD19
218	2.2.03.13		DCD19
219	2.2.04.01	Steam Generator System	DCD19

No.	ITAAC No.	Plant System ITAAC	Source
220	2.2.04.02a	Steam Generator System (cont'd)	DCD19
221	2.2.04.02b		DCD19
222	2.2.04.03a		DCD19
223	2.2.04.03b		DCD19
224	2.2.04.04a		DCD19
225	2.2.04.04b		DCD19
226	2.2.04.05a.i		DCD19
227	2.2.04.05a.ii		DCD19
228	2.2.04.05a.iii		DCD19
229	2.2.04.05b		DCD19
230	2.2.04.06		DCD19
231	2.2.04.07a.i		DCD19
232	2.2.04.07a.ii		DCD19
233	2.2.04.07b		DCD19
234	2.2.04.07c		DCD19
235	2.2.04.08a.i		DCD19
236	2.2.04.08a.ii		DCD19
237	2.2.04.08b.i		DCD19
238	2.2.04.08b.ii		DCD19
239	2.2.04.08c		DCD19
240	2.2.04.09a.i		DCD19
241	2.2.04.09a.ii		DCD19
242	2.2.04.09b.i		DCD19
243	2.2.04.09b.ii		DCD19
244	2.2.04.10		DCD19
245	2.2.04.11a		DCD19
246	2.2.04.11b.i		DCD19
247	2.2.04.11b.ii		DCD19
248	2.2.04.12a.i		DCD19
249	2.2.04.12a.ii		DCD19
250	2.2.04.12a.iii		DCD19
251	2.2.04.12b		DCD19
252	2.2.05.01	Main Control Room Emergency Habitability System	DCD19
253	2.2.05.02a		DCD19
254	2.2.05.02b		DCD19
255	2.2.05.03a		DCD19
256	2.2.05.03b		DCD19
257	2.2.05.04a		DCD19
258	2.2.05.04b		DCD19
259	2.2.05.05a.i		DCD19
260	2.2.05.05a.ii		DCD19
261	2.2.05.05a.iii		DCD19
262	2.2.05.05b		DCD19
263	2.2.05.06a		DCD19
264	2.2.05.06b		DCD19
265	2.2.05.07a.i		DCD19
266	2.2.05.07a.ii		DCD19
267	2.2.05.07a.iii		DCD19
268	2.2.05.07b.i		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
269	2.2.05.07b.ii	Main Control Room Emergency Habitability System (cont'd)	DCD19
270	2.2.05.07c		DCD19
271	2.2.05.07d		DCD19
272	2.2.05.08		DCD19
273	2.2.05.09a		DCD19
274	2.2.05.09b		DCD19
275	2.2.05.10		DCD19
276	2.2.05.11		DCD19
277	2.2.05.12		DCD19
278	2.3.01.01	Component Cooling Water System	DCD19
279	2.3.01.02		DCD19
280	2.3.01.03.i		DCD19
281	2.3.01.03.ii		DCD19
282	2.3.01.04		DCD19
283	2.3.01.05		DCD19
284	2.3.02.01	Chemical and Volume Control System	DCD19
285	2.3.02.02a		DCD19
286	2.3.02.02b		DCD19
287	2.3.02.03a		DCD19
288	2.3.02.03b		DCD19
289	2.3.02.04a		DCD19
290	2.3.02.04b		DCD19
291	2.3.02.05.i		DCD19
292	2.3.02.05.ii		DCD19
293	2.3.02.05.iii		DCD19
294	2.3.02.06a.i		DCD19
295	2.3.02.06a.ii		DCD19
296	2.3.02.06b		DCD19
297	2.3.02.06c		DCD19
298	2.3.02.07a		DCD19
299	2.3.02.07b		DCD19
300	2.3.02.07c		DCD19
301	2.3.02.08a.i		DCD19
302	2.3.02.08a.ii		DCD19
303	2.3.02.08a.iii		DCD19
304	2.3.02.08b		DCD19
305	2.3.02.09		DCD19
306	2.3.02.10a		DCD19
307	2.3.02.10b.i		DCD19
308	2.3.02.10b.ii		DCD19
309	2.3.02.11a.i		DCD19
310	2.3.02.11a.ii		DCD19
311	2.3.02.11a.iii		DCD19
312	2.3.02.11a.iv		DCD19
313	2.3.02.11b		DCD19
314	2.3.02.12a		DCD19
315	2.3.02.12b		DCD19
316	2.3.02.13		DCD19
317	2.3.02.14		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
318	2.3.03.01	Standby Diesel Fuel Oil System	DCD19
319	2.3.03.02		DCD19
320	2.3.03.03a		DCD19
321	2.3.03.03b		DCD19
322	2.3.03.03c		DCD19
323	2.3.03.03d		DCD19
324	2.3.03.04		DCD19
325	2.3.03.05		DCD19
326	2.3.04.01	Fire Protection System	DCD19
327	2.3.04.02.i		DCD19
328	2.3.04.02.ii		DCD19
329	2.3.04.03		DCD19
330	2.3.04.04.i		DCD19
331	2.3.04.04.ii		DCD19
332	2.3.04.05		DCD19
333	2.3.04.06		DCD19
334	2.3.04.07		DCD19
335	2.3.04.08		DCD19
336	2.3.04.09		DCD19
337	2.3.04.10		DCD19
338	2.3.04.11		DCD19
339	2.3.05.01	Mechanical Handling System	DCD19
340	2.3.05.02.i		DCD19
341	2.3.05.02.ii		DCD19
342	2.3.05.02.iii		DCD19
343	2.3.05.03a.i		DCD19
344	2.3.05.03a.ii		DCD19
345	2.3.05.03a.iii		DCD19
346	2.3.05.03b.i		DCD19
347	2.3.05.03b.ii		DCD19
348	2.3.05.03b.iii		DCD19
349	2.3.05.03c.i		DCD19
350	2.3.05.03c.ii		DCD19
351	2.3.05.03d.i		DCD19
352	2.3.05.03d.ii		DCD19
353	2.3.05.04		DCD19
354	2.3.06.01	Normal Residual Heat Removal System	DCD19
355	2.3.06.02a		DCD19
356	2.3.06.02b		DCD19
357	2.3.06.03a		DCD19
358	2.3.06.03b		DCD19
359	2.3.06.04a		DCD19
360	2.3.06.04b		DCD19
361	2.3.06.05a.i		DCD19
362	2.3.06.05a.ii		DCD19
363	2.3.06.05a.iii		DCD19
364	2.3.06.05b		DCD19
365	2.3.06.06		DCD19
366	2.3.06.07a.i		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
367	2.3.06.07a.ii	Normal Residual Heat Removal System (cont'd)	DCD19
368	2.3.06.07b		DCD19
369	2.3.06.07c		DCD19
370	2.3.06.08a		DCD19
371	2.3.06.08b		DCD19
372	2.3.06.09a.i		DCD19
373	2.3.06.09a.ii		DCD19
374	2.3.06.09b.i		DCD19
375	2.3.06.09b.ii		DCD19
376	2.3.06.09b.iii		DCD19
377	2.3.06.09b.iv		DCD19
378	2.3.06.09b.v		DCD19
379	2.3.06.09c		DCD19
380	2.3.06.09d		DCD19
381	2.3.06.10		DCD19
382	2.3.06.11a		DCD19
383	2.3.06.11b		DCD19
384	2.3.06.12a.i		DCD19
385	2.3.06.12a.ii		DCD19
386	2.3.06.12a.iii		DCD19
387	2.3.06.12a.iv		DCD19
388	2.3.06.12b		DCD19
389	2.3.06.13		DCD19
390	2.3.06.14		DCD19
391	2.3.07.01	Spent Fuel Pool Cooling System	DCD19
392	2.3.07.02a		DCD19
393	2.3.07.02b		DCD19
394	2.3.07.03		DCD19
395	2.3.07.04		DCD19
396	2.3.07.05.i		DCD19
397	2.3.07.05.ii		DCD19
398	2.3.07.05.iii		DCD19
399	2.3.07.06a		DCD19
400	2.3.07.06b		DCD19
401	2.3.07.07a		DCD19
402	2.3.07.07b.i		DCD19
403	2.3.07.07b.ii		DCD19
404	2.3.07.07b.iii		DCD19
405	2.3.07.07b.iv		DCD19
406	2.3.07.07b.v		DCD19
407	2.3.07.07b.vi		DCD19
408	2.3.07.07c		DCD19
409	2.3.07.08.i		DCD19
410	2.3.07.08.ii		DCD19
411	2.3.07.09		DCD19
412	2.3.07.10		DCD19
413	2.3.07.11		DCD19
414	2.3.08.01	Service Water System	DCD19
415	2.3.08.02.i		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
416	2.3.08.02.ii	Service Water System (cont'd)	DCD19
417	2.3.08.02.iii		DCD19
418	2.3.08.03		DCD19
419	2.3.08.04		DCD19
420	2.3.09.01	Containment Hydrogen Control System	DCD19
421	2.3.09.02a		DCD19
422	2.3.09.02b		DCD19
423	2.3.09.03.i		DCD19
424	2.3.09.03.ii		DCD19
425	2.3.09.03.iii		DCD19
426	2.3.09.03.iv		DCD19
427	2.3.09.04a		DCD19
428	2.3.09.04b		DCD19
429	2.3.09.05		DCD19
430	2.3.10.01	Liquid Radwaste System	DCD19
431	2.3.10.02a		DCD19
432	2.3.10.02b		DCD19
433	2.3.10.03a		DCD19
434	2.3.10.03b		DCD19
435	2.3.10.04a		DCD19
436	2.3.10.04b		DCD19
437	2.3.10.05a.i		DCD19
438	2.3.10.05a.ii		DCD19
439	2.3.10.05a.iii		DCD19
440	2.3.10.05b		DCD19
441	2.3.10.06a		DCD19
442	2.3.10.06b		DCD19
443	2.3.10.07a.i		DCD19
444	2.3.10.07a.ii		DCD19
445	2.3.10.07b		DCD19
446	2.3.10.08		DCD19
447	2.3.10.09		DCD19
448	2.3.10.10		DCD19
449	2.3.11.01	Gaseous Radwaste System	DCD19
450	2.3.11.02.i		DCD19
451	2.3.11.02.ii		DCD19
452	2.3.11.02.iii		DCD19
453	2.3.11.03a		DCD19
454	2.3.11.03b		DCD19
455	2.3.11.03c		DCD19
456	2.3.12.01	Solid Radwaste System	DCD19
457	2.3.12.02		DCD19
458	2.3.13.01	Primary Sampling System	DCD19
459	2.3.13.02		DCD19
460	2.3.13.03		DCD19
461	2.3.13.04		DCD19
462	2.3.13.05.i		DCD19
463	2.3.13.05.ii		DCD19
464	2.3.13.05.iii		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
465	2.3.13.06a.i	Primary Sampling System (cont'd)	DCD19
466	2.3.13.06a.ii		DCD19
467	2.3.13.06b		DCD19
468	2.3.13.06c		DCD19
469	2.3.13.07		DCD19
470	2.3.13.08		DCD19
471	2.3.13.09		DCD19
472	2.3.13.10a		DCD19
473	2.3.13.10b		DCD19
474	2.3.13.11a		DCD19
475	2.3.13.11b		DCD19
476	2.3.13.12		DCD19
477	2.3.14.01	Demineralized Water Transfer and Storage System	DCD19
478	2.3.14.02		DCD19
479	2.3.14.03		DCD19
480	2.3.14.04		DCD19
481	2.3.15.01	Compressed and Instrument Air System	DCD19
482	2.3.15.02		DCD19
483	2.3.15.03		DCD19
484	2.3.19.01a	Communication System	DCD19
485	2.3.19.01b		DCD19
486	2.3.19.02a		DCD19
487	2.3.19.02b		DCD19
488	2.3.29.01	Radioactive Waste Drain System	DCD19
489	2.3.29.02		DCD19
490	2.3.29.03		DCD19
491	2.3.29.04		DCD19
492	2.4.01.01	Main and Startup Feedwater System	DCD19
493	2.4.01.02		DCD19
494	2.4.01.03		DCD19
495	2.4.01.04		DCD19
496	2.4.02.01	Main Turbine System	DCD19
497	2.4.02.02a		DCD19
498	2.4.02.02b		DCD19
499	2.4.02.02c		DCD19
500	2.4.02.03.i		DCD19
501	2.4.02.03.ii		DCD19
502	2.4.02.03.iii		DCD19
503	2.4.06.01	Condensate System	DCD19
504	2.4.06.02		DCD19
505	2.5.01.01	Diverse Actuation System	DCD19
506	2.5.01.02a		DCD19
507	2.5.01.02b		DCD19
508	2.5.01.02c.i		DCD19
509	2.5.01.02c.ii		DCD19
510	2.5.01.02d		DCD19
511	2.5.01.03a		DCD19
512	2.5.01.03b		DCD19
513	2.5.01.03c		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
514	2.5.01.03d	Diverse Actuation System (cont'd)	DCD19
515	2.5.01.03e		DCD19
516	2.5.01.03f		DCD19
517	2.5.01.03g		DCD19
518	2.5.01.03h		DCD19
519	2.5.01.04		DCD19
520	2.5.01.05		DCD19
521	2.5.02.01	Protection and Safety Monitoring System	DCD19
522	2.5.02.02.i		DCD19
523	2.5.02.02.ii		DCD19
524	2.5.02.02.iii		DCD19
525	2.5.02.03		DCD19
526	2.5.02.04		DCD19
527	2.5.02.05a		DCD19
528	2.5.02.05b		DCD19
529	2.5.02.06a.i		DCD19
530	2.5.02.06a.ii		DCD19
531	2.5.02.06b		DCD19
532	2.5.02.06c.i		DCD19
533	2.5.02.06c.ii		DCD19
534	2.5.02.07a		DCD19
535	2.5.02.07b		DCD19
536	2.5.02.07c		DCD19
537	2.5.02.07d		DCD19
538	2.5.02.07e		DCD19
539	2.5.02.08a.i		DCD19
540	2.5.02.08a.ii		DCD19
541	2.5.02.08a.iii		DCD19
542	2.5.02.08b.i		DCD19
543	2.5.02.08b.ii		DCD19
544	2.5.02.08c		DCD19
545	2.5.02.09a		DCD19
546	2.5.02.09b		DCD19
547	2.5.02.09c		DCD19
548	2.5.02.09d		DCD19
549	2.5.02.10		DCD19
550	2.5.02.11		DCD19
551	2.5.02.12		DCD19
552	2.5.02.13		DCD19
553	2.5.02.14		DCD19
554	2.5.03.01	Plant Control System	DCD19
555	2.5.03.02		DCD19
556	2.5.04.01	Data Display and Processing System	DCD19 & COL 5
557	2.5.04.02.i		DCD19
558	2.5.04.02.ii		DCD19
559	2.5.04.02.iii		DCD19
560	2.5.04.03		DCD19
561	C.2.5.04.04a		COL 5
562	C.2.5.04.04b		COL 5

No.	ITAAC No.	Plant System ITAAC	Source
563	C.2.5.04.04c	Data Display and Processing System (cont'd)	COL 5
564	2.5.05.01	In-Core Instrumentation System	DCD19
565	2.5.05.02.i		DCD19
566	2.5.05.02.ii		DCD19
567	2.5.05.02.iii		DCD19
568	2.5.05.03a.i		DCD19
569	2.5.05.03a.ii		DCD19
570	2.5.05.03b		DCD19
571	2.5.05.03c		DCD19
572	2.5.05.04		DCD19
573	2.5.06.01	Special Monitoring System	DCD19
574	2.5.06.02		DCD19
575	2.5.09.01	Seismic Monitoring System	DCD19
576	2.5.09.02		DCD19
577	2.5.09.03		DCD19
578	2.6.01.01	Main AC Power System	DCD19
579	2.6.01.02.i		DCD19
580	2.6.01.02.ii		DCD19
581	2.6.01.02.iii		DCD19
582	2.6.01.03a		DCD19
583	2.6.01.03b		DCD19
584	2.6.01.04a		DCD19
585	2.6.01.04b		DCD19
586	2.6.01.04c		DCD19
587	2.6.01.04d		DCD19
588	2.6.01.04e		DCD19
589	2.6.01.04f		DCD19
590	2.6.01.05		DCD19
591	2.6.01.06		DCD19
592	2.6.02.01	Non-Class 1E DC & Uninterruptible Power Supply System	DCD19
593	2.6.02.02a		DCD19
594	2.6.02.02b		DCD19
595	2.6.02.02c		DCD19
596	2.6.03.01	Class 1E DC & Uninterruptible Power Supply System	DCD19
597	2.6.03.02.i		DCD19
598	2.6.03.02.ii		DCD19
599	2.6.03.02.iii		DCD19
600	2.6.03.03		DCD19
601	2.6.03.04a		DCD19
602	2.6.03.04b		DCD19
603	2.6.03.04c		DCD19
604	2.6.03.04d		DCD19
605	2.6.03.04e		DCD19
606	2.6.03.04f		DCD19
607	2.6.03.04g		DCD19
608	2.6.03.04h		DCD19
609	2.6.03.04i		DCD19
610	2.6.03.05a		DCD19
611	2.6.03.05b		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
612	2.6.03.05c	Class 1E DC & Uninterruptible Power Supply System (cont'd)	DCD19
613	2.6.03.05d.i		DCD19
614	2.6.03.05d.ii		DCD19
615	2.6.03.06		DCD19
616	2.6.03.07		DCD19
617	2.6.03.08		DCD19
618	2.6.03.09		DCD19
619	2.6.03.10		DCD19
620	2.6.03.11		DCD19
621	2.6.04.01	Onsite Standby Power System	DCD19
622	2.6.04.02a		DCD19
623	2.6.04.02b		DCD19
624	2.6.04.02c		DCD19
625	2.6.04.03		DCD19
626	2.6.04.04		DCD19
627	2.6.05.01	Lighting System	DCD19
628	2.6.05.02.i		DCD19
629	2.6.05.02.ii		DCD19
630	2.6.05.03.i		DCD19
631	2.6.05.03.ii		DCD19
632	2.6.05.04		DCD19
633	2.6.05.05.i		DCD19
634	2.6.05.05.ii		DCD19
635	2.6.05.06.i		DCD19
636	2.6.05.06.ii		DCD19
637	2.6.06.01.i	Grounding and Lightning Protection System	DCD19
638	2.6.06.01.ii		DCD19
639	2.6.06.01.iii		DCD19
640	2.6.06.01.iv		DCD19
641	2.6.09.01	Plant Security System	DCD19
642	2.6.09.03		DCD19
643	2.6.09.04		DCD19
644	2.6.09.05a		DCD19
645	2.6.09.05b		DCD19
646	2.6.09.05c		DCD19
647	2.6.09.06		DCD19
648	2.6.09.07a		DCD19
649	2.6.09.07b		DCD19
650	2.6.09.08		DCD19
651	2.6.09.09		DCD19
652	2.6.09.13a		DCD19
653	2.6.09.13b		DCD19
654	2.6.09.13c		DCD19
655	2.6.09.15a		DCD19
656	2.6.09.15b		DCD19
657	2.6.09.16		DCD19
658	C.2.6.09.01	Physical Security	COL 5
659	C.2.6.09.02		COL 5
660	C.2.6.09.03a		COL 5

No.	ITAAC No.	Plant System ITAAC	Source
661	C.2.6.09.03b	Physical Security (cont'd)	COL 5
662	C.2.6.09.04a		COL 5
663	C.2.6.09.04b		COL 5
664	C.2.6.09.05a		COL 5
665	C.2.6.09.05b		COL 5
666	C.2.6.09.06		COL 5
667	C.2.6.09.07		COL 5
668	C.2.6.09.08a		COL 5
669	C.2.6.09.08b		COL 5
670	C.2.6.09.09		COL 5
671	C.2.6.12.01	Offsite Power System	COL 5
672	C.2.6.12.02		COL 5
673	C.2.6.12.03		COL 5
674	C.2.6.12.04		COL 5
675	C.2.6.12.05		COL 5
676	C.2.6.12.06		COL 5
677	2.7.01.01	Nuclear Island Nonradioactive Ventilation System	DCD19
678	2.7.01.02a		DCD19
679	2.7.01.02b		DCD19
680	2.7.01.03a		DCD19
681	2.7.01.03b		DCD19
682	2.7.01.04a		DCD19
683	2.7.01.04b		DCD19
684	2.7.01.05.i		DCD19
685	2.7.01.05.ii		DCD19
686	2.7.01.05.iii		DCD19
687	2.7.01.06a		DCD19
688	2.7.01.06b		DCD19
689	2.7.01.07		DCD19
690	2.7.01.08a		DCD19
691	2.7.01.08b		DCD19
692	2.7.01.08c		DCD19
693	2.7.01.08d		DCD19
694	2.7.01.09		DCD19
695	2.7.01.10a		DCD19
696	2.7.01.10b		DCD19
697	2.7.01.11		DCD19
698	2.7.01.12		DCD19
699	2.7.01.13		DCD19
700	2.7.01.14		DCD19
701	2.7.02.01	Central Chilled Water System	DCD19
702	2.7.02.02		DCD19
703	2.7.02.03a		DCD19
704	2.7.02.03b		DCD19
705	2.7.02.04		DCD19
706	2.7.02.05		DCD19
707	2.7.03.01	Annex/Auxiliary Building Nonradioactive Ventilation System	DCD19
708	2.7.03.02a		DCD19
709	2.7.03.02b		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
710	2.7.03.03	Annex/Auxiliary Building Nonradioactive Ventilation System (cont'd)	DCD19
711	2.7.03.04		DCD19
712	2.7.04.01	Diesel Generator Building Ventilation System	DCD19
713	2.7.04.02a		DCD19
714	2.7.04.02b		DCD19
715	2.7.04.02c		DCD19
716	2.7.04.03		DCD19
717	2.7.04.04		DCD19
718	2.7.05.01	Radiologically Controlled Area Ventilation System	DCD19
719	2.7.05.02.i		DCD19
720	2.7.05.02.ii		DCD19
721	2.7.05.02.iii		DCD19
722	2.7.05.03		DCD19
723	2.7.06.01	Containment Air Filtration System	DCD19
724	2.7.06.02.i		DCD19
725	2.7.06.02.ii		DCD19
726	2.7.06.03.i		DCD19
727	2.7.06.03.ii		DCD19
728	2.7.06.03.iii		DCD19
729	2.7.06.04		DCD19
730	2.7.06.05		DCD19
731	2.7.07.01	Containment Recirculation Cooling System	DCD19
732	2.7.07.02		DCD19
733	3.1.00.01	Emergency Response Facilities	DCD19
734	3.1.00.02		DCD19
735	3.1.00.03		DCD19
736	3.1.00.04		DCD19
737	3.1.00.05		DCD19
738	3.1.00.06		DCD19
739	C.3.8.01.01.01	Emergency Planning - Emergency Classification System	COL5
740	C.3.8.01.02.01	Emergency Planning - Notification Methods and Procedures	COL5
741	C.3.8.01.02.02		COL5
742	C.3.8.01.02.03		COL5
743	C.3.8.01.03.01	Emergency Planning - Emergency Communication	COL5
744	C.3.8.01.03.02		COL5
745	C.3.8.01.04.01	Emergency Planning - Public Education and Information	COL5
746	C.3.8.01.05.01.01	Emergency Planning - Emergency Facilities and Equipment	COL5
747	C.3.8.01.05.01.02		COL5
748	C.3.8.01.05.01.03		COL5
749	C.3.8.01.05.01.04		COL5
750	C.3.8.01.05.01.05		COL5
751	C.3.8.01.05.01.06		COL5
752	C.3.8.01.05.01.07		COL5
753	C.3.8.01.05.01.08		COL5
754	C.3.8.01.05.02.01		COL5
755	C.3.8.01.05.02.02		COL5
756	C.3.8.01.05.02.03		COL5
757	C.3.8.01.05.02.04		COL5
758	C.3.8.01.06.01	Emergency Planning - Accident Assessment	COL5

No.	ITAAC No.	Plant System ITAAC	Source
759	C.3.8.01.06.02	Emergency Planning - Accident Assessment (cont'd)	COL5
760	C.3.8.01.06.03		COL5
761	C.3.8.01.06.04		COL5
762	C.3.8.01.06.05		COL5
763	C.3.8.01.06.06		COL5
764	C.3.8.01.06.07		COL5
765	C.3.8.01.07.01	Emergency Planning - Protective Response	COL5
766	C.3.8.01.08.01.01	Emergency Planning - Exercise and Drill	COL5
767	C.3.8.01.08.01.02		COL5
768	C.3.8.01.08.01.03		COL5
769	C.3.8.01.09.01	Emergency Planning - Emergency Classification System	COL5
770	3.2.00.01a	Human Factors Engineering	DCD19
771	3.2.00.01b		DCD19
772	3.2.00.01c.i		DCD19
773	3.2.00.01c.ii		DCD19
774	3.2.00.01d		DCD19
775	3.2.00.01e		DCD19
776	3.2.00.02		DCD19
777	3.2.00.03.i		DCD19
778	3.2.00.03.ii		DCD19
779	3.2.00.03.iii		DCD19
780	3.2.00.03.iv		DCD19
781	3.2.00.03.v		DCD19
782	3.2.00.04		DCD19
783	3.2.00.05		DCD19
784	3.2.00.06.i		DCD19
785	3.2.00.06.ii		DCD19
786	3.2.00.06.iii		DCD19
787	3.2.00.07		DCD19
788	3.2.00.08		DCD19
789	3.2.00.09		DCD19
790	3.3.00.01	Buildings	DCD19
791	3.3.00.02a.i.a		DCD19
792	3.3.00.02a.i.b		DCD19
793	3.3.00.02a.i.c		DCD19
794	3.3.00.02a.i.d		DCD19
795	3.3.00.02a.ii.a		DCD19
796	3.3.00.02a.ii.b		DCD19
797	3.3.00.02a.ii.c		DCD19
798	3.3.00.02a.ii.d		DCD19
799	3.3.00.02a.ii.e		DCD19
800	3.3.00.02a.ii.f		DCD19
801	3.3.00.02b		DCD19
802	3.3.00.02c		DCD19
803	3.3.00.02d		DCD19
804	3.3.00.02e		DCD19
805	3.3.00.02f		DCD19
806	3.3.00.02g		DCD19
807	3.3.00.02h		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
808	3.3.00.03a	Buildings (cont'd)	DCD19
809	3.3.00.03b		DCD19
810	3.3.00.03c		DCD19
811	3.3.00.03d		DCD19
812	3.3.00.04a		DCD19
813	3.3.00.04b		DCD19
814	3.3.00.04c		DCD19
815	3.3.00.05a		DCD19
816	3.3.00.05b		DCD19
817	3.3.00.05c		DCD19
818	3.3.00.06a		DCD19
819	3.3.00.06b		DCD19
820	3.3.00.07aa		DCD19
821	3.3.00.07ab		DCD19
822	3.3.00.07ac		DCD19
823	3.3.00.07ba		DCD19
824	3.3.00.07bb		DCD19
825	3.3.00.07bc		DCD19
826	3.3.00.07c.i.a		DCD19
827	3.3.00.07c.i.b		DCD19
828	3.3.00.07c.ii.a		DCD19
829	3.3.00.07c.ii.b		DCD19
830	3.3.00.07d.i		DCD19
831	3.3.00.07d.ii.a		DCD19
832	3.3.00.07d.ii.b		DCD19
833	3.3.00.07d.ii.c		DCD19
834	3.3.00.07d.iii.a		DCD19
835	3.3.00.07d.iii.b		DCD19
836	3.3.00.07d.iii.c		DCD19
837	3.3.00.07d.iv.a		DCD19
838	3.3.00.07d.iv.b		DCD19
839	3.3.00.07d.iv.c		DCD19
840	3.3.00.07d.v.a		DCD19
841	3.3.00.07d.v.b		DCD19
842	3.3.00.07d.v.c		DCD19
843	3.3.00.07e		DCD19
844	3.3.00.08		DCD19
845	3.3.00.09		DCD19
846	3.3.00.10.i		DCD19
847	3.3.00.10.ii		DCD19
848	3.3.00.10.iii		DCD19
849	3.3.00.12		DCD19
850	3.3.00.13		DCD19
851	3.3.00.14		DCD19
852	3.3.00.16		DCD19
853	3.3.00.17		DCD19
854	3.5.00.01.i	Radiation Monitoring System	DCD19
855	3.5.00.01.ii		DCD19
856	3.5.00.01.iii		DCD19

No.	ITAAC No.	Plant System ITAAC	Source
857	3.5.00.02.i	Radiation Monitoring System (cont'd)	DCD19
858	3.5.00.02.ii		DCD19
859	3.5.00.03		DCD19
860	3.5.00.04		DCD19
861	3.5.00.05		DCD19
862	3.5.00.06		DCD19
863	3.5.00.07		DCD19
864	3.5.00.08		DCD19
865	3.6.00.01.i	Reactor Coolant System Leak Detection System	DCD19
866	3.6.00.01.ii		DCD19
867	3.6.00.01.iii		DCD19
868	3.6.00.01.iv		DCD19
869	3.6.00.01.v		DCD19
870	3.6.00.01.vi		DCD19
871	3.6.00.01.vii		DCD19
872	3.7.00.01	Design Reliability Assurance Program	DCD19
873	C.3.8.02.01	Pipe Rupture Hazard Analysis	COL 5
874	C.3.8.03.01	Piping Design	COL 5