

**NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)**

CONTROL NO: 4993

FILE: INCIDENT REPORT 101

FROM: Wisconsin Public Service Green Bay, Wis. 54305 E. W. James			DATE OF DOC 5-2-75	DATE REC'D 5-6-75	LTR XX	TWX	RPT	OTHER
TO: Mr. Bernard Rusche			ORIG 1 signed	CC 1	OTHER	SENT AEC PDR XX SENT LOCAL PDR XX		
CLASS	UNCLASS XX	PROP INFO	INPUT	NO CYS REC'D 1		DOCKET NO: 50-305		

DESCRIPTION: Ltr trans the following:

**NOT RECORDED**  
**Do Not Remove**

ENCLOSURES: Abnormal Occurrence Report  
NO. AO 75-8 on 4-24-75 concerning valve S1-2A  
failure to open when the manual control switch  
was placed in the open position...

1 copy encl rec'd

PLANT NAME: **KEWAUNEE**

FOR ACTION/INFORMATION

WTM 5-8-75

BUTLER (L) W/ Copies	SCHWENGER (L) W/ Copies	ZIEMANN (L) W/ Copies	REGAN (E) W/ Copies
CLARK (L) W/ Copies	STOLZ (L) W/ Copies	DICKER (E) W/ Copies	LEAR (L) W/ Copies
DAIR (L) W/ Copies	VASSALLO (L) W/ Copies	KNIGHTON (E) W/ Copies	SPOLS W/ Copies
KNIEL (L) W/ Copies	PURPLE (L) W/ Copies	YOUNGBLOOD (E) W/ Copies	

**INTERNAL DISTRIBUTION**

<b>REG FILE</b> NRC PDR OGC, ROOM P-509A GOSSICK/STAFF CASE GIAMBUSO BOYD MOORE (L) DEYOUNG (L) SKOVHOLT (L) GOLLER (L) (Ltr) P. COLLINS DENISE REG OPR FILE & REGION (2) T.R. WILSON (2) STEELE	<b>TECH REVIEW</b> SCHROEDER MACCARY KNIGHT PAWLICKI SHAO STELLO HOUSTON NOVAK ROSS IPPOLITO TEDESCO LONG LAINAS BENAROYA VOLLMEYER	<b>DENTON</b> GRIMES GAMMILL KASTNER BALLARD SPANGLER  <b>ENVIRO</b> MULLER DICKER KNIGHTON YOUNGBLOOD REGAN PROJECT LDR  <b>HARLESS</b>	<b>LIC ASST</b> R. DIGGS (L) H. GEARIN (L) E. GOULDSBORNE (L) P. KREUTZER (E) J. LEE (L) M. MAIGRET (L) S. REED (E) M. SERVICE (L) S. SHEPPARD (L) M. SLATER (E) H. SMITH (L) S. TEETS (L) G. WILLIAMS (E) V. WILSON (L) R. INGRAM (L)	<b>A/T IND</b> BRAITHMAN SALTZMAN MELTZ  <b>PLANS</b> MCDONALD CHAPMAN DUBE (Ltr) E. COUPE PETERSON HARTFIELD (2) KLECKER EISENHUT WIGGINTON  F. WILLIAMS LEWIS
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**EXTERNAL DISTRIBUTION**

1 - LOCAL PDR <b>KEWAUNEE, WIS.</b>	1 - NATIONAL LABS	1 - PDR-SAN/LA/NY
1 - TIC (ABERNATHY) (1)(2)(10)	1 - W. PENNINGTON, Rm E-201 GT	1 - BROOKHAVEN NAT LAB
1 - NSIC (BUCHANAN)	1 - CONSULTANTS	1 - G. ULRIKSON, CIVIL
1 - ASLB	NEWMARK, BLUME/AGBASIAN	1 - AGUED RUTH CUBERMAN
1 - Newton Anderson		Rm B-127 GT
1 - ACRS SENT TO LIC ASST		1 - J. D. RUEKLES, Rm E-201
** SEND ONLY TEN DAY REPORTS		GT

WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

May 2, 1975

Mr. Benard Rusche, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Regulatory Docket File

Dear Mr. Rusche:

Subject: Docket 50-305  
Operating License DPR-43  
Abnormal Occurrence Report AO 75-8

In accordance with the requirements of Technical Specifications, paragraph 6.6.2, we submit the attached Licensee Event Report form.

This abnormal occurrence is the first trip of the current overload breakers for valve SI 2A. The redundant valve, which was verified operable immediately following this occurrence, has tripped its current overload breakers on previous occasions as reported by Abnormal Occurrence Reports AO 74-20 and AO 75-6. This overcurrent tripping is an intermittent occurrence which has not occurred during our investigation program which was established to resolve the first abnormal occurrences of this nature. Our investigative program has been intensified and will continue until resolution is determined.

To provide indication that a current overload trip breaker has actuated, a design change has been initiated. This design change will result in annunciation within the control room whenever valves SI 2A or SI 2B trip on overload. This modification should be completed within one month. The installation of annunciators for monitoring of the current overload breakers will not correct the cause for the trip, however, it will remove the possibility of overlooking the existence of the tripped condition for these valves.

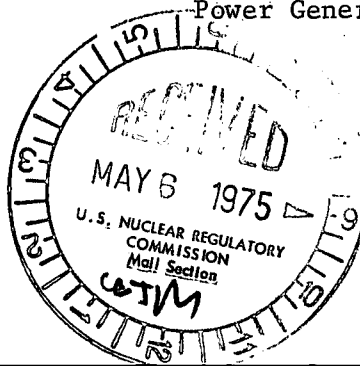
An additional report will be transmitted whenever resolution of the cause for these overcurrent trips is reached.

Very truly yours,

*Carl W. James*  
for

E. W. James  
Senior Vice President  
Power Generation & Engineering

EWJ:sna  
Attach.  
cc - Mr. James G. Keppler  
Mr. Dwane Boyd



4993

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME														LICENSE NUMBER														LICENSE TYPE						EVENT TYPE			
01	W	I	K	N	P	I	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	1													
7	8	9				14	15										25	26					30	31	32												

CATEGORY				REPORT TYPE		REPORT SOURCE		DOCKET NUMBER								EVENT DATE						REPORT DATE					
01	CON'T	M	I	L	L	0	5	0	-	0	3	0	5	0	4	2	4	7	5	0	5	0	2	7	5		
7	8		57	58	59	60	61						68	69						74	75					80	

**EVENT DESCRIPTION**

02	Valve SI-2A failed to open when the manual control switch was placed in the open																																																																																80
03	position. The valve was being operated to measure the operator drive current in																																																																																80
04	conjunction with the investigation of the failure of a parallel valve (SI-2B) is																																																																																80
05	pe orted in A0-75-6. The drive current and torque switch settings were checked																																																																																80
06	and found to be normal. Investigation is continuing.																																																																																80

SYSTEM CODE				CAUSE CODE		COMPONENT CODE						PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	S	F	E	V	A	L	V	O	P	N	L	2	0	0	N				
7	8	9	10	11	12				17	43	44			47	48				

**CAUSE DESCRIPTION**

08	The drive motor thermal overloads were found to be tripped. The overloads were																																																																																80
09	reset and the valve operated satisfactorily.																																																																																80
10																																																																																	80

FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION															
11	E	0	9	0	NA				C	Operator Observation																			
7	8	9	10	12	13				44	45			46																80

FORM OF ACTIVITY RELEASED				CONTENT OF RELEASE				AMOUNT OF ACTIVITY				LOCATION OF RELEASE																
12	Z	Z	NA				NA				NA																	
7	8	9	10	11						44	45																	80

**PERSONNEL EXPOSURES**

NUMBER				TYPE		DESCRIPTION																						
13	0	0	0	Z	NA																							
7	8	9	11	12	13																							80

**PERSONNEL INJURIES**

NUMBER				DESCRIPTION																							
14	0	0	0	NA																							
7	8	9	11	12																							80

**OFFSITE CONSEQUENCES**

15	No danger to Public Health and Safety																																																																																80
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**LOSS OR DAMAGE TO FACILITY**

TYPE				DESCRIPTION																							
16	Z	NA																									
7	8	9	10																								80

**PUBLICITY**

17	NA																																																																																80
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**ADDITIONAL FACTORS**

18																																																																																	80
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19																																																																																	80
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NAME: M. E. Stern

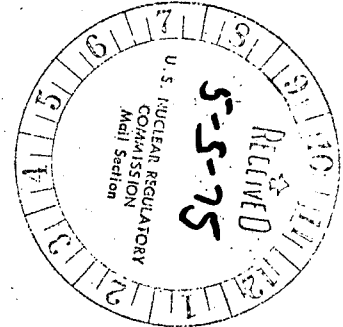
PHONE: 414/432-3311

TELEGRAM

April 25, 1975

Mr. Duane Boyd  
Resident Inspector - US Nuclear Regulatory Commission  
Directorate of Regulatory Operations - Region III  
Two Rivers Municipal Hospital  
Two Rivers, Wisconsin 54241

SUBJECT: Docket 50-305  
OPERATING LICENSE: DPR-43  
ABNORMAL OCCURRENCE REPORT



In accordance with the requirements of Technical Specifications, Paragraph 6.6.2.a and 1.0.3.1.(d), we submit the following:

Report Number: 50-305/75-8

Occurrence Date: April 24, 1975

Facility: Kewaunee Nuclear Plant  
Kewaunee, Wisconsin 54216

Identification of Occurrence:

Thermal overloads on BREAKER for SI-2A Valve were found tripped.

Condition Prior to Occurrence:

Reactor Critical - 90% Power  
NORMAL Operating Temperature - 556 f Tavg  
NORMAL Operating Pressure - 2235 psig

Description of Occurrence:

While performing the procedure to determine the cause of the failures of SI-2B (MU 32105), SI-2A was to be cycled so that comparable data could be obtained. The main line contacts did not pick up when an attempt to open valve was made. The thermal overloads were found tripped in the breaker.

The overloads were reset and the valve functioned properly.

*CPH*

4960