



FPL Energy
Seabrook Station

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November 3, 2008

Docket 50-443
SBK-L-08172

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

Seabrook Station

**10CFR50.59 Report, Revision 12 to the Seabrook Station Updated Final
Safety Analysis Report, Revision 9 to Appendix R, "Fire Protection Safe
Shutdown Capability," Revision 10 to Appendix A, "Evaluation and Comparison
to BTP APCSB 9.5-1," and Revision 112 to the Technical Requirements Manual**

FPL Energy Seabrook, LLC (FPL Energy Seabrook) encloses the 10CFR50.59 Report, Revision 12 to the Seabrook Station Updated Final Safety Analysis Report (UFSAR), Revision 9 to Appendix R, "Fire Protection Safe Shutdown Capability," Revision 10 to Appendix A, "Evaluation and Comparison to BTP APCSB 9.5-1," and Revision 112 to the Technical Requirements Manual. The 10CFR50.59 Report and the UFSAR are submitted pursuant to the requirements of 10CFR50.59(d)(2) and 10CFR50.71(e). The 10CFR50.59 report and the UFSAR report cover the period from March 9, 2007 through October 3, 2008. UFSAR Revision 12 incorporates approved and implemented design changes and UFSAR changes identified through October 3, 2008. The incorporated changes to the UFSAR have been reviewed in accordance with 10CFR50.59. The reviews determined that these changes did not require prior NRC approval.

The UFSAR is provided in its entirety on CD-ROM in Portable Document Format (PDF). Changes from Revision 11 are indicated by a change in revision number and a vertical line (revision bar) in the margin next to the change. The List of Effective Pages contained within the UFSAR provides a listing of each page and its revision number with a revision bar indicating which pages contain changes. The controlled drawings referenced in the UFSAR are provided on a separate CD-ROM. Appendix R, Appendix A and the Technical Requirements Manual are also provided on CD-ROM. Summaries of the 10CFR50.59 evaluations for the changes incorporated in Revision 12 of the UFSAR are attached as Enclosure 1. Enclosure 2 provides a summary of the 10CFR50.59 evaluations performed to support facility and procedure changes during the reporting period that were determined to be reportable pursuant to 10CFR50.59(d)(2), but did not require a change to the UFSAR. Enclosure 3 provides a summary for changes to the UFSAR incorporated using the guidance of NEI 98-03, "Guidelines for Updating Final Safety Analysis Reports."

Enclosure 4 is a listing of UFSAR Change Requests (UFCRs) incorporated in UFSAR Revision 12 during the reporting period. The affected Sections, Tables and Figures are provided for each UFCR. Enclosure 5 contains UFCRs incorporated in Appendix R, Enclosure 6 provides the UFCRs incorporated in Appendix A and Enclosure 7 contains UFCRs incorporated in the Technical Requirements Manual.

One copy of the UFSAR revision on CD-ROM is being submitted to the Document Control Desk, Washington, DC, along with a copy to the Region I Regional Office and a copy to the Resident Inspector at Seabrook Station.

Should you have any questions regarding this matter, please contact Mr. Michael O'Keefe, Licensing Manager, at (603) 773-7745.

Very truly yours,

FPL Energy Seabrook, LLC



Gene St. Pierre
Site Vice President

cc: S. J. Collins, NRC Region I Administrator
G. E. Miller, NRC Project Manager, Project Directorate I-2
W. J. Raymond, NRC Senior Resident Inspector

OATH AND AFFIRMATION

I, Gene St. Pierre, Site Vice President of FPL Energy Seabrook, LLC, hereby affirm that the information and statements contained within this submittal are based on facts and circumstances which are true and accurate to the best of my knowledge and belief.

Sworn and Subscribed
before me this

3rd day of November, 2008

Gene St. Pierre

Gene St. Pierre
Site Vice President

Victoria S. Brown

Notary Public



Enclosure 1 to SBK-L-08172

Summary Report of Facility Changes, Tests, and Experiments
Completed in Accordance with the Requirements of 10CFR50.59
for Revision 12 of the Updated Final Safety Analysis Report

Design Change Records/Minor Modifications

Design changes documented in the following Design Change Records (DCR) and on their Design Change Notices (DCN) or Minor Modifications (MMOD) were installed during the period covered by the 10CFR50.59 Report. A 10CFR50.59 evaluation was performed for each MMOD, DCR or DCN. For each of the evaluations performed, there were no activities requiring prior NRC approval identified.

10CFR50.59 Evaluation 07-005

DCR 07-006, Extreme Outside Air
Minimum and Maximum Hourly
Temperature Change (UFCR 07-042)

Summary Description and Purpose: DCR 07-006, Extreme Outside Air Minimum and Maximum Hourly Temperature Change, implemented the changes to the 100-year environs temperature extremes, based on a new calculation using a newer methodology based on American Society of Heating, Refrigerating and Air-Conditioning (ASHRAE) 2005. The UFCR revised UFSAR Section 2.3 to reference ASHRAE 2005 and change the maximum temperature from 102 to 100.9°F and the minimum temperature from -21 to -16.8°F.

Evaluation Summary: The evaluation addressed Criterion 8 to determine if the use of the ASHRAE 2005 temperature calculation resulted in a departure from a method of evaluation described in the UFSAR. Seabrook recalculated the 100-Year Environs Temperature Extremes (or 100-year return period temperature) using the ASHRAE 2005 methodology. The ASHRAE 2005 methodology is referenced in NUREG-0800 as an acceptable "method for establishing 100-year return period extreme temperature values as a function of annual extreme temperature values."

The 10 CFR 50.59 evaluation concluded that since the ASHRAE 2005 method is previously approved by the NRC and is appropriate for the specific application it is not a departure from a method of evaluation, therefore prior NRC approval is not required.

10CFR50.59 Evaluation 08-003

DCR 07-003, DCN 02, Appendix R
Time Critical Operator Actions and
Reactor Coolant Pump (RCP) Seal
Cooling

Summary Description and Purpose: DCN 02 of DCR 07-003 changes the administrative controls for the Main Steam Isolation Valve (MSIV) bypass valves MS-V204, MS-V205, MS-V206, and MS-V207 from normally closed with the associated breakers open (off) to normally locked closed with the associated breakers locked open (off). The new administrative control is an enhancement for control of the desired post accident position of closed. The administrative control complies with General Design Criterion 57 and meets locked valve criterion 1 of Engineering Design Standard (EDS) 37190.

Evaluation Summary: The 10 CFR 50.59 evaluation performed for this activity determined that changing the administrative controls for the MSIV bypass valves from normally closed with the associated breakers open (off) to normally locked closed with the associated breakers locked open (off) is an enhancement for control of the desired post accident position of closed. The use of locks on the valves would result in a delay in operator action to reposition the valves. However, there is not an accident sequence that requires repositioning of the valves to open, therefore there is no impact on consequences of an accident. The evaluation determined that prior NRC approval is not required.

Miscellaneous Evaluations

The following 10CFR50.59 evaluations were associated with plant activities conducted during the reporting period covered by this 10CFR50.59 Report. The evaluations concluded that the activities did not require prior NRC approval.

10CFR50.59 Evaluation 07-003 Departure from Nucleate Boiling (DNB) UFSAR Changes (UFCR 07-033)

Summary Description and Purpose: UFCR 07-033 revised the UFSAR Section 4.4 description of the power shape adjustment factor applied to Departure from Nucleate Boiling (DNB) predictions and the rod bow Departure from Nucleate Boiling Ratio (DNBR) penalty. Westinghouse reported that the bow-to-contact DNBR penalty which is an input to the rod bow DNBR penalty calculation, may be non-conservative for plant conditions associated with uprated power levels and increased peaking factors. As a result, Westinghouse recalculated the rod bow penalties for all fuel products using an input value for the bow-to contact DNBR penalty that accounts for the increased heat flux associated with current and planned uprated conditions. No change was made to the current NRC-approved methodology. Westinghouse also reported that DNB tests on fuel products similar to the RFA fuel assemblies used by Seabrook suggested that DNB margin predictions for RFA fuel assemblies using their licensed WRB-2M DNB were potentially non-conservative for certain axial power shapes. As a result, Westinghouse applied an adjustment factor to the WRB-2M DNB margin predictions to account for this potential non-conservatism. Westinghouse indicated that the minimum DNBR was reduced by up to 6% for several events. However, margin still remains to the WRB-2M Safety Analysis Limit DNBR value of 1.47.

Evaluation Summary: The 10CFR50.59 Evaluation for this activity concluded that the UFSAR changes can be implemented without prior NRC approval. The DNB-related UFSAR corrections do not involve a physical change to the reactor core or any other structure, system, or component (SSC), therefore, the ability of the Reactor Coolant System (RCS) and Emergency Core Coolant System (ECCS) to adequately remove heat and prevent DNB is not adversely affected. Margin to the 95/95 DNB limit remains for all Condition I and Condition II transients following these changes.

10CFR50.59 Evaluation 07-004 Non-LOCA Safety Analysis UFSAR Changes (UFCR 07-036)

Summary Description and Purpose: UFCR 07-036 provided changes to UFSAR Chapter 15 Non-LOCA Safety Analysis results for the Uncontrolled RCCA Bank Withdrawal at Power event, the Loss of Forced Reactor Coolant Flow events, Locked Rotor event and Feedwater Malfunction events. These events were reanalyzed by Westinghouse after determination that an incorrect RCCA trip reactivity curve was assumed in the Non-LOCA safety analysis for Seabrook Station. Specifically, the normalized RCCA drop distance (insertion) versus time curve utilized by the Non-LOCA RETRAN base deck model were transposed during development of the "Normalized RCCA position versus time curve." The reanalysis required revisions to UFSAR Sections 15.3.1 and 15.3-2, Tables 15.3-1, 15.3-2 and 15.4-1 and Figures 15.1-1, 15.3-1, 15.3-2, 15.3-3, 15.3-4, 15.3-5, 15.3-6, 15.4-4, 15.4-5, 15.4-6, 15.4-7 and 15.4-8.

Evaluation Summary: The 10CFR50.59 Evaluation for this activity concluded that the UFSAR changes can be implemented without prior NRC approval. Implementation of the changes does not result in a physical change to the reactor core or any other structure, system, or component (SSC). The ability of the reactor trip system to prevent any Safety Analysis Limit from being exceeded following any non-LOCA transient has been verified by reanalysis or evaluation.

Enclosure 2 to SBK-L-08172

Summary Report of Facility Changes, Tests, and Experiments
Completed in Accordance with the Requirements of 10CFR50.59
for Design Changes, Temporary Modifications, Temporary Setpoint Requests,
Procedure Changes, Commitment Changes, and Miscellaneous Changes

There were no facility changes, tests, and experiments completed in accordance with the requirements of 10CFR50.59 for design changes, temporary modifications, temporary setpoint requests, procedure changes and miscellaneous changes that required performance of a 10CFR50.59 evaluation during the reporting period.

The following pages contain a brief description of commitment changes approved during the reporting period.

Commitment Change Request 2007-2 Change to Commitment Detailed in
Letter NYN-90179, Response to Generic
Letters 83-28 and 90-03, dated October
2, 1990

Summary Description and Purpose: This commitment change request (CCR) revised a 1990 commitment made in response to Generic Letters 83-28 and 90-03 which addressed the establishment of a Vendor Interface Program for safety-related components. In its response, Seabrook Station committed to a two tier Vendor Interface Program. The first tier included the establishment of a technical program with Seabrook's NSSS supplier, Westinghouse, to cover safety-related components within the NSSS scope of supply. The interactions between FPL Energy Seabrook and Westinghouse are well established and not the subject of the commitment change request. The second tier of the program included the establishment of periodic contact with those vendors that supply key safety-related (KSR) components. A total of 70 key safety related vendors were identified and initial contact was established via letter. In NYN-90179, Seabrook Station committed to establish contact with the non-NSSS vendors on a yearly basis. In 1995, CCR 95-06 revised the annual documented commitment to a triennial vendor contact frequency, implemented yearly with 1/3 of the vendors contacted. During the most recent KSR vendor reviews, establishing an accurate vendor contact list was difficult. Many of the "original" vendors are no longer in business under their original company names or are no longer in the nuclear business. Upon completion of the reviews, the time required to establish a current contact list was extensive and the payback was minimal. CCR 2007-2 eliminates the requirement to have direct contact with those key safety-related vendors identified in GL 90-03 and described in NYN-90179. The existence of established ongoing programs meets the intent of maintaining vendor manuals current.

Evaluation Summary: A 10CFR50.59 evaluation was not required to be performed for this Commitment Change. The combination of existing programs provide a level of information that meets the intent of the requirements of GL 83-28 and GL 90-03 to ensure key safety related vendor manuals are current. These programs are 10 CFR Part 21 Tracking, Maintenance Rule Implementation Guidelines, Operating Experience Review Program, Approved Supplier/Vendor List (AVL), Purchasing, Receipt Processing and Approval of Vendor Documentation and Electronic Document Management System – Document Control.

The requirement to have direct contact with our key safety-related vendors was made in a time when computers were not a part of the everyday work tool. The relationship between the utility and supplier/vendor was only established when there was a need. The creation of owner groups provides a conduit to share information about components and systems. It is Seabrook's position that the activities currently implemented (listed above) satisfy the recommendation for the control of key safety-related vendor manuals as defined in Generic Letters 83-28 and 90-03 and direct vendor contact is not required and the intent of the original commitment is maintained.

Commitment Change Request 2007-3

Change to Commitment Detailed in Letter SBN-1027, Response to Generic Letter 86-04; Policy Statement on Engineering Expertise on Shift, dated May 2, 1986

Summary Description and Purpose: This commitment change request (CCR) revised a commitment made in 1986 in response to Generic Letter 86-04; Policy Statement on Engineering Expertise on Shift. In its response, Seabrook Station informed the NRC that it had elected Option 1, Alternative 3 of the Policy Statement. Option 1 of the policy statement is using a combined SRO/STA position. Seabrook combined the Shift Technical Advisor (STA) position with that of the Shift Manager. Option 1 requires that the Shift Manager possess the Technical Degree required for the STA. CCR 2007-3 allows the use of Generic Letter 86-04 Option 1 and Option 2. Option 2 is placing on each shift a dedicated Shift Technical Advisor (STA) who meets the STA criteria of NUREG-07037, Clarification of TMI Action Plant Requirements, I.A.1.1.

Evaluation Summary: A 10CFR50.59 evaluation was not required to be performed for this Commitment Change. Generic Letter 86-04 allows either Option 1 or Option 2 to be used on each shift. Option 1 may be used on some shifts and Option 2 on other shifts, or the same option may be used on every shift. If Option 1 is used for a shift, then a separate STA position may be eliminated for that shift. Technical Specification Table 6.2-1 allows the use of the dedicated STA. Table Notation (3) states "The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Manager or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC." The commitment change allows the assignment of a dedicated STA on those shifts where the Shift Manager does not have the necessary qualifications to assume the STA function. The change fully meets the intent of Generic Letter 86-04 and the original commitment to provide engineering expertise on shift.

Commitment Change Request 2008-1

Change to Commitment Detailed in
Letter L-2003-201, Response to NRC
Bulletin 2003-01, dated August 8,
2003

Summary Description and Purpose: This commitment change request (CCR) retired a commitment made as a compensatory measure in the response to NRC Bulletin 2003-01, Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized Water Reactors. The commitment was to include a protected step in procedure OX1406.012, 18 Month Containment and Containment Spray Recirculation Sump Surveillance, to inspect the ECCS recirculation sump screens using a go-no go gage fabricated from a drill bit to inspect questionable screen openings. To meet the requirements of Generic Letter 2004-02, Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized Water Reactors, FPL Energy Seabrook replaced the existing containment sump screens with passive strainers constructed of stainless steel plate perforated with circular holes. The perforated plate is not subject to distortion as woven wire screen is and the gap size of the new strainers cannot be increased unless the strainer is damaged. The perforated holes will not distort in shape and size and the inspection criteria of procedure OX1406.012 can identify any damage to the strainer assembly. The compensatory measure to inspect the ECCS recirculation sump screens with the go-no go gage is no longer required. The CCR removed the protected step added as a compensatory measure to procedure OX1406.012.

Evaluation Summary: A 10CFR50.59 evaluation was not required to be performed for this Commitment Change. The commitment to add a protected step to the procedure was a compensatory measure made in response to Bulletin 2003-01 to improve the response to potential debris blockage of the ECCS recirculation sump screens. As part of its response to Generic Letter 2004-02, FPL Energy Seabrook replaced the existing containment sump screens. GL 2004-02 states that upon resolution of the potential concerns identified in the generic letter and the completion of any corrective actions resulting from that resolution, addressees may consider continuing, revising, or retiring their compensatory measures as appropriate. Based on the new design of the sump screen, CCR 2008-1 retired the commitment to enhance the procedure for the sump screen inspection and removed the step protected to Bulletin 2003-01.

Enclosure 3 to SBK-L-07062

Summary of Changes to the Updated Final Safety Analysis Report
Incorporated Using the Guidance of NEI 98-03,
“Guidelines for Updating Final Safety Analysis Reports”

The following pages provide a summary of changes incorporated in Revision 12 of the Updated Final Safety Analysis Report using the guidance contained in NEI 98-03, "Guidelines for Updating Final Safety Analysis Reports." The summaries provide the UFSAR Change Request (UFCR) number, affected UFSAR, Sections, Tables, or Figures and a description of the change.

UFCR Number	Affected Sections, Tables, and Figures
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05-018	9.5.1, 9.5.1.3, Table 9.5-2, Appendix A, Section B (pg. 5-13)
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Description of Change: This UFCR classified the information contained in PSNH letter SBN-1102 as historical as the information has been incorporated in the Station Fire Protection Program. Additionally, the UFCR revised the UFSAR and Appendix A to be consistent with detector terminology in Technical Requirement 12, Fire Detection Instrumentation. Photoelectric and Ionization detectors were changed to the generic term "smoke" detectors.

07-005	16.3 (Technical Requirement 2, Technical Requirement 33)
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Description of Change: This UFCR corrected an error made during incorporation of a previously approved design change (03DCR012). In TR 2, table notation 5, 30 seconds was revised to 27 seconds. This UFCR also added clarification to the TR 33 basis to enhance execution of the secondary calorimetric power determination. The revision to TR 33 did not alter the intent of the Technical Requirement.

07-019	2.3, 7.7, 9.5, 16.3, TRM 1.0, TRs 7, 8, 9, 10, 11, 19, 20, 21, 22, 23, 24, 26, 27, 29, 32, 33
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Description of Change: This UFCR replaced the reference to NRC Generic Letter 91-18 with Regulatory Issue Summary in Chapter 1 of the Technical Requirements Manual (TRM). The change also replaced the Technical Specification (TS) defined term "Operable" with the term "functional" in the UFSAR and TRM in those instances that refer to an attribute of a system, structure, or component (SSC) that is not controlled by the technical specifications (TS). This UFCR is of an administrative nature and clarifies user understanding by distinguishing between operability of TS SSCs and functionality of TRM controlled SSCs. The change is unrelated to the operation and control of any SSCs and is therefore outside the scope of 10 CFR 5.59.

07-021	13.1
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Description of Change: This UFCR reflected approved changes to the organizational structure and deleted redundant information contained in the Quality Assurance Topical Report (QATR). The changes were to reporting relationships and did not change required functions performed by Station staff.

UFCR Number	Affected Sections, Tables, and Figures
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07-024	13.1
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Description of Change: This UFCR changed the UFSAR job description for the Assistant Operations Manager – Support to be consistent with the job description provided in the Operations Management Manual (OPMM), the administrative program manual controlling the conduct of operations

07-032	9.3.3.2.b.5
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Description of Change: This UFCR corrected an inconsistency in the UFSAR. The change added the recovery test tank (RTT) to UFSAR Section 9.3.3.2.b.5 to be consistent with UFSAR Section 11.2.3.3 that states chemical drain treatment tanks can be directed to the waste tank or recovery test tank.

07-035	Table 12.3-15
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Description of Change: This UFCR deleted an incorrect reference to loop diagram 1-NHY-506131 in Table 12.3-15. The referenced drawing has no association with instrument tag 1-RM-RE-6526-2.

07-040	Tables 6.2-75 and 9.3-7
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Description of Change: This UFCR corrected errors in the incorporation of previously approved data. Specifically, during the implementation of 03DCR012, UFSAR tables 6.2-75 and 9.3-7 were not revised to reflect the approved reduction in RWST boron concentration from 2700-2900 ppm to 2400-2600 ppm.

07-046	13.1, 13.2, 13.5
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Description of Change: This UFCR changed titles in UFSAR Chapter 13 to reflect titles used in the current organization, added additional detail to the description for operations training supervisors, and corrected the senior management representative responsible for certification license applications from the Station Director to the Senior Vice President. The UFCR did not change any required functions performed by station staff.

07-054	Figures 15.6-50, 15.6-51, 15.6-52, 15.6-53, 15.6-54, 15.6-55, 15.6-57, 15.6-58, 15.6-59, 15.6-60
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Description of Change: This UFCR corrected an error in incorporating previously approved data. During incorporation for the Stretch Power Uprate, the UFSAR markup transmittal provided by Westinghouse contained pre-uprate figures instead of the required post-uprate figures. The UFCR incorporated the correct figures as approved by the NRC in License Amendment 101.

UFCR Number	Affected Sections, Tables, and Figures
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07-055	13.1.2.2
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Description of Change: This change added descriptive detail to §13.1.2.2 to describe the Shift Technical Advisor (STA) function when a dedicated STA is assigned to a shift when the Shift Manager is not STA qualified. Adding this description is a clarification to improve reader understanding by increasing the level of detail regarding the STA function without changing the scope or modifying the meaning of the information provided.

08-009	13.1
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Description of Change: This UFCR corrected titles and reporting relationships as a result of the implementation of a standard fleet organization and did not change required functions performed by station staff.

08-010	9.5.2.2.b.1, 9.5.2.2.b.2
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Description of Change: This UFCR removed excessive detail from §9.5.2.2.b.1 to provide a more general description of the telephone system. The change also removed an obsolete item “off-premise extensions” from §9.5.2.2.b.2.

08-021	16.3, Technical Requirements Manual, Section 1.0
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Description of Change: This change replaced the reference to deleted procedure OE 4.9, Plant Nonconformance/Degraded Condition Evaluation, with OE 3.6, Condition Reports. The change replaced a reference to obsolete information to assist the manual user and is unrelated to the operation and control of any plant structure, system or component.

08-022	9.3.3
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Description of Change: This change removed excessive detail regarding the Primary Auxiliary Building (PAB) sump pump run time totalizer and clarified that there are various inputs to the PAB sump.

08-023	9.5.1
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Description of Change: This change replaced an incorrect reference to Westinghouse Technical Specifications for Fire Protection limiting conditions for operation, action statements, and surveillance requirements with the Technical Requirements Manual.

Enclosure 4 to SBK-L-08172

Listing of UFSAR Change Requests (UFCRs) Incorporated in
Updated Final Safety Analysis Report, Revision 12

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
00-041	9.5.2		
03-001	3.9(B)18, 3.9(B)19		9.2-2
04-003	8.2.1.3a.		8.2-7
04-027	9.3.1.2, 9.3.1.5		
05-018	9.5.1, 9.5.1.3	9.5-2	
05-021	16.3		
05-028	9.4.15	9.4-19	
06-002		9.5-2	
06-003	9.3.3.3e, 9.3.5.2b, 9.3.5.2d, 11.2.2.1a, 11.2.3.3, 11.5.2.3b		
06-019	16.3		
06-028	16.3		
06-031	11.2.2.1		6.2-74, 9.1-2, 11.2-4, 10.4-11
07-004	8.2.1.1, 8.2.4		
07-005	16.3		
07-008	7.2, 10.2		
07-010	16.3		
07-011	9.1.1.3, 9.1.4.2.c.3	3.2-1	
07-012	16.3		
07-013	7.7.1.4.a	7.7-1	7.2-5, 7.2-15
07-014	13.2.2.9		
07-015	10.4.5.3		
07-016	1.8, 3.1, 3.8, 6.2.2.2.g,	1.1-1, 3.2-1, 6.2-75,	6.2-78, 6.2-79

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
	6.2.2.2.j, 6.2.2.3.d, 6.2.2.4, 6.2.7	6.2-78, 6.2-79, 6.2-87, 6.3-1	
07-017		6.2-87	
07-018, Rev 0	6.2.4.2, 7.4.5.2, 7.4.5.3		
07-018, Rev 2	6.2.4	6.2-83	6.2-91
07-019	2.3, 7.7, 9.5, 16.3		
07-020	15.0.1.1, 15.1.5.3, 15.3.4.3, 15.4.8.3, 15.6.3.3		
07-021	13.1		13.1-3, 13.1-4
07-022	9.3.4.2		
07-023	9.1.2.1.o, 9.1.2.2, 9.1.4.2.c.1, 9.1.4.2.c.2, 9.1.4.3.a.2.(a), 9.1.4.3.a.2(f), 9.1.3.1.a.3		
07-024	13.1		
07-029	10.2.2.1		
07-030	16.3		
07-031	3.4.1.2		
07-032	9.3.3.2.b.5		
07-033	4.4		
07-035		12.3-15	
07-036	15.3.1, 15.3.2	15.3-1, 15.3-2, 15.4-1	15.1-1, 15.3-1, 15.3-2, 15.3-3, 15.3-4, 15.3-5, 15.3-6, 15.4-4, 15.4-5, 15.4-6, 15.4-7, 15.4-8
07-037	16.3		
07-038		5.5-2	

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
07-038, R1		5.1-1	
07-039		2.3-36	
07-040		6.2-75, 9.3-7	
07-041	16.3		
07-042	2.5		3.11-1
07-043	9.5.1.2.5	9.5-2	1.2-1, 9.5-1, 9.5-2, 9.5-3
07-045	8.2.1.3a.1		
07-046	13.1, 13.2, 13.5		
07-047	6.8.2		
07-051	8.2.1.4.e		
07-052	1.1, 1.2, 9.1, 12.5, 13.2, 13.4, 16.3	9.3-3	1.2-1, 2.1-4, 2.1-5, 2.4-1
07-054			15.6-50, 15.6-51, 15.6-52, 15.6-53, 15.6-54, 15.6-55, 15.6-57, 15.6-58, 15.6-59, 15.6-60
07-055	13.1.2.2		
07-058	9.5.4		
07-059			7.7-7
07-062	16.3		
08-003	7.6		
08-004	10.4.7.2		
08-007	10.2.2.3		
08-009	13.1		

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
08-010	9.5.2.2		
08-013	9.1.4.2.b.4, 9.1.5		
08-015		3.9(B)-27	8.3-23
08-016	5.2.3, 9.3.4, 12.2.1	9.3-6, 12.2-5	
08-017		8.3-1, 8.3-2	
08-021	16.3		
08-022	9.3.3		
08-023	9.5.1		
08-026	16.3		
08-028	Appendix 3A, 9.3.3.3		

Enclosure 5 to SBK-L-08172

Listing of UFSAR Change Requests (UFCRs) Incorporated in
Appendix R, Fire Protection Safe Shutdown Capability, Revision 9

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables/ Tabulations</u>	<u>Affected Figures</u>
07-018, Rev 0	3.1.2, 3.1.8, 3.2.1, 3.2.2, 3.2.3, 3.2.4, 3.2.5, 3.2.6, 3.2.7, 3.3.2, 3.3.3, 3.3.5, 3.3.6, 3.3.7, 3.3.8, 3.4.1, 3.4.2, 3.4.3, 3.5.3, 3.6.2	3.2.7.1, 3.2.7.2, 3.2.7.3, 3.2.7.10, 3.2.7.12, 3.2.7.17, 3.2.7.24, 3.2.7.30, 3.2.7.39, 3.2.7.40, 3.2.7.41, 3.2.7.48, 3.2.7.49, 3.2.7.50, 3.2.7.51, 3.2.7.52, 3.2.7.58, 3.2.7.59, 3.2.7.62, 3.2.7.63, 3.2.7.66, 3.2.7.68, 3.2.7.69, 3.2.7.70, 3.2.7.71, 3.2.7.72, 3.2.7.77, 3.2.7.80, 3.2.7.81, 3.2.7.88, 3.2.7.89, 3.3.9.1 MCR 3.1.3.1, MCR 3.1.3.2, MCR 3.1.3.3, MCR 3.1.3.4, MCR 3.1.3.6, MCR 3.1.3.7, MCR 3.1.3.17, MCR 3.1.3.8 RSS 3.1.3.1, RSS 3.1.3.3, RSS 3.1.3.7, RSS 3.1.3.17, RSS 3.1.3.18, RSS 3.1.3.19 (EFW) 3.2.3 (HI-LO) 3.3.2	
07-018, Rev 1		3.2.7.2, 3.2.7.62, 3.2.7.88, 3.2.7.89, MCR 3.1.3.2, MCR 3.1.3.4, MCR 3.1.3.6, MCR 3.1.3.7, MCR 3.1.3.17, RSS 3.1.3.6	
07-018, Rev 2	3.2.2.3, 3.3.3.3	MCR 3.1.3.1, RSS 3.1.3.1	
08-015	3.3.6	MCR 3.1.3.7-6	

Enclosure 6 to SBK-L-08172

Listing of UFSAR Change Requests (UFCRs) Incorporated in
Appendix A, Evaluation and Comparison to BTP APCS 9.5-1, Revision 10

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
00-041	9.5-1, Page F-89		
04-027	F.2, Tab 15		
05-018	Section B, Page 5		
06-002	F.2, Tab 15		
07-028	F.2, Tab 9		
07-043	Section B, Page 3, F.3, Page 102		
08-015	F.2, Tab 8		

Enclosure 7 to SBK-L-08172

Listing of UFSAR Change Requests (UFCRs) Incorporated in
Revisions 104 through 112 of the Technical Requirements Manual

<u>UFCR Number</u>	<u>Affected Sections</u>	<u>Affected Tables</u>	<u>Affected Figures</u>
05-021	TR 13		
06-019	TR 4		
06-028	TR 24		
07-005	TR 2, TR 33		
07-010	TR Program 5.1		
07-012	TR 12		
07-019	TRM Section 1.0, TRs 7, 8, 9, 10, 11, 19, 20, 21, 22, 23, 24, 26, 27, 29, 32, 33		
07-030	TR 12		
07-037	TR 29		
07-041	TR 7		
07-052	TR 27		
07-062	Chapter 6 COLR		
08-021	TRM, Section 1.0		
08-026	TR 2		