Entergy

Entergy Nuclear Operations, Inc.
Palisades Nuclear Plant
27780 Blue Star Memorial Highway
Covert, MI 49043
Tel 269 764 2000

March 31, 2009

10 CFR 72.48
10 CFR 50.59

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Big Rock Point Plant
Docket 50-155 and 72-043
License No. DPR-6

Report of Changes, Tests and Experiments and Summary of Commitment Changes

Dear Sir or Madam:

Entergy Nuclear Operations, Inc. (ENO) is providing the Report of Facility Changes, Tests and Experiments for the time period from April 1, 2007, through March 31, 2009, for the Big Rock Point Plant. The report is contained in Enclosure 1. This report is being submitted in accordance with the requirements of 10 CFR 72.48 and 10 CFR 50.59. There were no commitment changes made in accordance with NEI 99-04, "Guidelines for Managing Commitment Changes," during the period.

Laurie Lahti

Laurie A. Lahti
Licensing Manager
Palisades Nuclear Plant

Enclosure

CC Administrator, Region III, USNRC
Director, Spent Fuel Storage and Transportation Office, USNRC
NRC NMSS Project Manager

ENCLOSURE 1
SUMMARY OF 72.48 AND 50.59 EVALUATION
BIG ROCK POINT PLANT

Log No.	Description of Change and Summary of 72.48 and 50.59 Evaluation Performed
063-07	The Big Rock Point (BRP) plant quality assurance (QA) program was changed from the Consumers Energy QA program to the Entergy Operations, Inc. QA program. The Entergy Nuclear Operations QA program was previously approved by the NRC in a safety evaluation dated November 6, 1998. Under 10 CFR 72.140(d), a QA program accepted as satisfying 10 CFR 50, Appendix B, is acceptable to satisfy 10 CFR 72.140. The adoption of the Entergy Nuclear Operations QA program has no effect on the frequency of any accidents previously evaluated in the Safety Analysis Reports. The likelihood of occurrence of a malfunction of a system, structure, or component important to safety is not increased, the consequences of previously evaluated accidents are not affected, no consequences of malfunctions of important to safety components are increased, no accidents of a different type are created, the possibilities of different malfunctions are not created, no design basis limits for fission product barriers are exceeded or altered, and no methods of evaluation are affected.