

August 16, 2007

MEMORANDUM TO: Lynn Mrowca, Chief  
PRA Licensing, Operations Support & Maintenance Branch 1  
Division of Safety Systems and Risk Assessment  
Office of New Reactors

FROM: Martin Stutzke **R/A**  
PRA Licensing, Operations Support & Maintenance Branch 1  
Division of Safety Systems and Risk Assessment  
Office of New Reactors

SUBJECT: SUMMARY OF JULY 19, 2007 CATEGORY 2 MEETING TO  
DISCUSS PROBABILISTIC RISK ASSESSMENT INFORMATION  
TO SUPPORT DESIGN CERTIFICATION AND COMBINED  
LICENSE APPLICATIONS (TAC NO. MC4370)

On July 19, 2007, a public meeting was held between the U.S. Nuclear Regulatory Commission (NRC), and representatives from the ABWR, AP1000, EPR, ESBWR, and US-APWR Design-Centered Working Groups (DCWGs) and the Nuclear Energy Institute (NEI) at the Hilton Washington DC/Rockville Executive Meeting Center, 1750 Rockville Pike, Rockville, MD. The purpose of the meeting was to discuss the guidance in RG 1.206 concerning the format and content of probabilistic risk assessment (PRA) information to be included in an application for a combined license, and the staff's plan for the review of this PRA information as described in the Standard Review Plan (NUREG-0800), Section 19.0.

The staff made a presentation that provided information useful to getting oriented to the new guidance (e.g., section renumbering, how to locate the guidance on the NRC public website, etc.), summarized the applicable regulations, discussed a timeline that showed the application submittal and review process, provided an overview of RG 1.206, listed an acceptable format and content for Chapter 19 of the FSAR, and answered some "frequently asked questions."

Meeting attendees queried the staff about several issues:

1. Does the format and content of an application need to follow the guidance in Appendix A to Section C.I.19 of RG 1.206?
2. Does the Tier 2 change process apply to Chapter 19 of the FSAR?
3. Does Chapter 19 of the FSAR need to include the screening process used for PRA maintenance, or can it simply state that such a process exists?
4. What Capability Category from the AMSE PRA Standard is acceptable?

5. Why is large release frequency (LRF) used to license new plants, but large early release frequency (LERF) is used to evaluate risk-informed license amendment requests for current plants?
6. When does a combined operating license (COL) holder need to commence PRA maintenance and upgrade?
7. What is the relationship among the PRA scope, the generic DCD, and NRC-endorsed consensus standards?
8. Will the staff routinely use requests for additional information (RAI)s and onsite audits to obtain information about the PRA?
9. What is the meaning of the term “significant change” in RG 1.206, Section C.I.19?
10. Should the severe accident mitigation design alternatives (SAMDA) evaluation be presented in the FSAR or the environmental report (ER)?
11. What is meant by “regulatory oversight processes?”

The staff agreed to provide answers to these questions at a later date.

Members of the public were in attendance. Public Meeting Feedback forms were not received.

A list of meeting attendees is attached [ADAMS Accession Number ML072250190]. Also attached is a copy of the staff's slides presented at the meeting [ADAMS Accession Number ML072220293].

Please direct any inquiries to Martin Stutzke at 301-415-4105, or [mas7@nrc.gov](mailto:mas7@nrc.gov).

Attachments: 1. List of attendees  
2. Staff Handout

5. Why is large release frequency (LRF) used to license new plants, but large early release frequency (LERF) is used to evaluate risk-informed license amendment requests for current plants?
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ADAMS ACCESSION NUMBER: ML

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