

August 17, 2007

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BYRON STATION, UNIT NOS. 1 AND 2 - REQUEST FOR ADDITIONAL
INFORMATION RELATED TO SECOND TEN-YEAR INTERNAL RELIEF
REQUESTS (TAC NOS. MD4097, MD4098, MD4099, MD4100, MD4101,
MD4102, MD4103, MD4104, MD4105, AND MD4106)

Dear Mr. Crane:

By letter to the Nuclear Regulatory Commission (NRC) dated January 12, 2007, Exelon Generation Company, LLC submitted requests for relief from the American Society of Mechanical Engineers Boiler and Pressure Vessel Code for the second inservice inspection interval, for Byron Station, Unit Nos. 1 and 2.

The NRC staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure to this letter. During a discussion with your staff on August 15, 2007, it was agreed that you would provide a response within 30 days from the date of this letter.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. If circumstances result in the need to revise the requested response date, please contact me at (301) 415-3733.

Sincerely,

/RA/

Robert F. Kuntz, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-454 and 50-455

Enclosure:
Request for Additional Information

cc w/encl: See next page

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Byron Station, Unit Nos. 1 and 2

cc:

Illinois Emergency Management Agency
Division of Disaster Assistance &
Preparedness
1035 Outer Park Dr
Springfield, IL 62704

Document Control Desk - Licensing
via e-mail

Mr. Dwain W. Alexander, Project Manager
Westinghouse Electric Corporation
via e-mail

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
35 East Wacker Drive
Suite 1300
Chicago, IL 60601-2110

U.S. Nuclear Regulatory Commission
Byron Resident Inspector's Office
via e-mail

Ms. Lorraine Creek
RR 1, Box 182
Manteno, IL 60950

Chairman, Ogle County Board
P.O. Box 357
Oregon, IL 61061

Mrs. Phillip B. Johnson
1907 Stratford Lane
Rockford, IL 61107

Attorney General
500 S. Second Street
Springfield, IL 62701

Plant Manager - Byron Station
via e-mail

Site Vice President - Byron Station
via e-mail

Senior Vice President - Operations Support
via e-mail

Chairman
Will County Board of Supervisors
Will County Board Courthouse
Joliet, IL 60434

Director - Licensing and Regulatory Affairs
via e-mail

Manager Regulatory Assurance - Byron
via e-mail

Associate General Counsel
via e-mail

Vice President - Regulatory Affairs
via e-mail

Manager Licensing - Braidwood/Byron
via e-mail

Senior Vice President - Midwest Operations
via e-mail

Vice President - Regulatory Affairs
via e-mail

Manager Licensing - Braidwood, Byron
and LaSalle
via e-mail

Senior Vice President - Midwest Operations
via e-mail

Mr. Barry Quigley
3512 Louisiana
Rockford, IL 61108

REQUEST FOR ADDITIONAL INFORMATION

BYRON STATION, UNITS 1 AND 2

DOCKET NOS. STN 50-454 AND STN 50-455

In reviewing the Exelon Generation Company's (Exelon's) submittal dated January 12, 2007, related to requests for relief from the American Society of Mechanical Engineers Boiler and Pressure Vessel Code for the second inservice inspection interval (ISI), for Byron Station, Units 1 and 2 (Byron), the NRC staff has determined that the following information is needed in order to complete its review:

Relief Request (RR) I2R-21

I2R-21-1:

RR I2R-21 did not indicate whether the limited scope surface examination of the seismic lug welds provided any indication of the presence of unacceptable flaws or conditions in accordance with the acceptance criteria of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Article IWB-3000. Therefore, the Nuclear Regulatory Commission (NRC) staff requests that you discuss whether the limited scope surface examination of these welds provided any indication of the presence of flaws or other relevant conditions that were determined to be unacceptable according to the acceptance criteria of the ASME Code, Section XI, Article IWB-3000.

I2R-21-2:

The NRC staff requests that you discuss the extent to which the seismic lug welds were examined during the first ISI interval and the preservice exam, including the percentage of credible surface examination coverage that was achieved during these previous examinations. Discuss any relevant conditions that were found during these previous examinations.

I2R-21-3:

The NRC staff notes that a similar request was authorized for Braidwood Station, Units 1 and 2 on January 6, 2000. However, the authorization of this request was granted with the understanding that the licensee would perform a VT-1 visual examination of the accessible areas in the vicinity of the seismic weld lugs during the forthcoming ISI interval. Please discuss whether a VT-1 visual examination has been or will be performed for the accessible areas of the seismic lugs.

RR I2R-22

I2R-22-1:

Please indicate the percentage of credible surface examination coverage that was achieved during the examination of the accessible welded attachments located on the exposed outside surface of the containment penetration. If less than essentially 100 percent coverage was

Enclosure

achieved for any of these welds, please provide supplemental information justifying why compliance with the ASME Code, Section XI requirements for essentially 100 percent surface examination coverage of these welds was impractical.

I2R-22-2:

RR I2R-22 did not indicate whether the surface examination of the accessible attachment welds provided any indication of the presence of unacceptable flaws or conditions in accordance with the acceptance criteria of the ASME Code, Section XI, Article IWC-3000. Therefore, the NRC staff requests that you discuss whether the surface examination of these welds provided any indication of the presence of flaws or other relevant conditions that were determined to be unacceptable according to the acceptance criteria of the ASME Code, Section XI, Article IWC-3000.

RR I2R-23

I2R-23-1:

RR I2R-23 did not indicate whether the limited scope volumetric examination of the specified pressurizer nozzle-to-vessel welds provided any indication of the presence of unacceptable flaws or conditions in accordance with the acceptance criteria of the ASME Code, Section XI, Article IWB-3000. Therefore, the NRC staff requests that you discuss whether the limited scope volumetric examination of these welds provided any indication of the presence of flaws or other relevant conditions that were determined to be unacceptable according to the acceptance criteria of the ASME Code, Section XI, Article IWB-3000.

I2R-23-2:

The NRC staff requests that you discuss the extent to which the specified pressurizer nozzle-to-vessel welds were examined during the first ISI interval and the preservice exam, including the percentage of credible surface examination coverage that was achieved during these previous examinations. Discuss any relevant conditions that were found during these previous examinations.

RR I2R-25

I2R-25-1:

RR I2R-25 did not indicate whether the limited scope volumetric examination of the reactor vessel head-to-flange weld provided any indication of the presence of unacceptable flaws or conditions in accordance with the acceptance criteria of the ASME Code, Section XI, Article IWB-3000. Therefore, the NRC staff requests that you discuss whether the limited scope volumetric examination of this weld provided any indication of the presence of flaws or other relevant conditions that were determined to be unacceptable according to the acceptance criteria of the ASME Code, Section XI, Article IWB-3000.

RR I2R-53

I2R-53-1:

RR I2R-53 did not indicate whether the limited scope volumetric examination of the residual heat removal heat exchanger (RHRHX) shell-to-flange weld provided any indication of the presence of unacceptable flaws or conditions in accordance with the acceptance criteria of the ASME Code, Section XI, Article IWC-3000. Therefore, the NRC staff requests that you discuss whether the limited scope volumetric examination of this weld provided any indication of the presence of flaws or other relevant conditions that were determined to be unacceptable according to the acceptance criteria of the ASME Code, Section XI, Article IWC-3000.

I2R-53-2:

The NRC staff requests that you discuss the extent to which the RHRHX shell-to-flange weld was examined during the first ISI interval and the preservice exam, including the percentage of credible volumetric examination coverage that was achieved during these previous examinations. Discuss any relevant conditions that were found during these previous examinations.