



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

August 14, 2007

Tennessee Valley Authority
ATTN: Mr. William R. Campbell
Chief Nuclear Officer and
Senior Vice President
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: NOTIFICATION OF BROWNS FERRY NUCLEAR PLANT COMPONENT
DESIGN BASES INSPECTION - NRC INSPECTION REPORT NOS.
05000259/2007007, 05000260/2007007, AND 05000296/2007007

Dear Mr. Campbell:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Browns Ferry Nuclear Plant during the weeks of November 13 - 16, 2007, November 26 - 30, 2007, and December 10 - 14, 2007. The inspection team will be led by Mr. David A. Jones, a Senior Reactor Inspector from the NRC's Region II Office. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued June 22, 2006.

The inspection will evaluate the capability of risk significant / low margin components to function as designed and support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on August 7, 2007, Mr. Jones confirmed with Mr. T. Langley of your staff, arrangements for an information gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of October 29 - November 2, 2007
- Onsite weeks: November 13 - 16, November 26 - 30, and December 10 - 14, 2007

The purpose of the information gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. R. Bernhard, a Region II Senior Reactor Analyst, may accompany Mr. Jones during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection.

The enclosure lists documents that will be needed prior to the information gathering visit. Please contact Mr. Jones prior to preparing copies of the materials listed in the enclosure and provide this information to the Region II office by October 15, 2007.

The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for the inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space, supplemental documents requested to be made available to the team in the Region II office prior to the inspection preparation week of November 5 - November 9, 2007; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Jones at (404) 562-4631 or me at (404) 562-4876.

Sincerely,

/RA/

Lloyd M. Cain, Acting Branch Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.:50-259, 50-260, 50-296
License Nos.:DPR-33, DPR-52, DPR-68

Enclosure: Information Request for Browns Ferry Plant Component Design Basis Inspection

cc w/encl:
Ashok S. Bhatnagar
Senior Vice President
Nuclear Generation Development
and Construction
Tennessee Valley Authority
Electronic Mail Distribution

Preston D. Swafford
Senior Vice President
Nuclear Support
Tennessee Valley Authority
Electronic Mail Distribution

(cc w/encl cont'd - See page 3)

TVA

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(cc w/encl cont'd)

Walter M. Justice, II
Interim Vice President
Nuclear Engineering &
Technical Services
Tennessee Valley Authority
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Brian J. O'Grady
Site Vice President
Browns Ferry Nuclear Plant
Tennessee Valley Authority
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General Counsel
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Robert G. Jones, General Manager
Browns Ferry Site Operations
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Beth A. Wetzel, Manager
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D. T. Langley, Manager
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John C. Fornicola, General Manager
Nuclear Assurance
Tennessee Valley Authority
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(cc w/encl cont'd - See page 4)

TVA

4

(cc w/encl cont'd)

Robert H. Bryan, Jr., General Manager
Licensing & Industry Affairs
Tennessee Valley Authority
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State Health Officer
Alabama Dept. of Public Health
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P. O. Box 303017
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Chairman
Limestone County Commission
310 West Washington Street
Athens, AL 35611

Distribution w/encl:

J. Boska, NRR
E. Brown, NRR
L. Raghavan, NRR
RIDSNNRRDIRS
PUBLIC

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(cc w/encl cont'd - See page 3)

☒ PUBLICLY AVAILABLE

☐ NON-PUBLICLY AVAILABLE

☐ SENSITIVE

☒ NON-SENSITIVE

ADAMS: ☒ Yes ACCESSION NUMBER: _____

OFFICE	RII:DRS	RII:DRS	RII:DRP				
SIGNATURE			RA				
NAME	BERNHARD	JONES	TLIU				
DATE	8/ /2007	8/ /2007	8/14/2007	8/ /2007	8/ /2007	8/ /2007	8/ /2007
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

DOCUMENT NAME: C:\FileNet\ML072260413.wpd

Browns Ferry Information Request for CDBI

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Lists should contain enough information to be easily understood to someone who has a knowledge of boiling water reactor technology.

1. From your most-recent probabilistic safety analysis (PSA) **excluding** external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
2. From your most-recent probabilistic safety analysis (PSA) **including** external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide copies of your human reliability worksheets for these items.
4. Any pre-existing evaluation or list of components and calculations with low design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, MOV risk-margin rankings, etc.).
5. A list of station applicability evaluations/reviews performed and documented in the station corrective action program in the past two years for industry events, critical equipment failures, and safety related equipment vulnerabilities [as communicated by NRC generic communications, industry communications, 10 CFR part 21 notifications, etc.].
6. A list of **safety related SSC** design modifications implemented within the last two years, sorted by affected system.
7. This item deleted. Corrective action lists will be requested after the component sample is determined.
8. A list of common-cause failure of components that have occurred at Browns Ferry and have been identified within the last five years.
9. A list of operability evaluations completed within the last two years, sorted by associated component or system.
10. Contact information for a person to discuss PRA information prior to the information gathering trip: name, title, phone number, and e-mail address.

Enclosure

11. List of equipment currently on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address those reasons.
12. List of equipment currently in RIS 05-020 (formerly GL 91-18) status.
13. List of equipment currently in MR (a)(1) status.