

ENCLOSURE 1

REVISED PAGES OF RENEWED FACILITY OPERATING LICENSE NOS. DPR-63 and NPF-69

DOCKET NOS. 50-220 AND 50-410

NINE MILE POINT NUCLEAR STATION, UNIT NOS. 1 AND 2

Replace the following pages of the Renewed Facility Operating Licenses. The revised pages are identified by the date of the letter issuing these pages and contain marginal lines indicating the areas of change.

REMOVE

License DPR-63

Page 7

Page 8

License NPF-69

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INSERT

License DPR-63

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- (11) NMP LLC shall take all necessary steps to ensure that the decommissioning trusts are maintained in accordance with the Application for approval of the transfer of the NMP-1 license to NMP LLC (Application), the requirements of the Order approving the transfer, and the related safety evaluation.
- (12) At the time of the transfer of NMP-1 to NMP LLC, NMP LLC shall enter or shall have entered into an intercompany credit agreement with Constellation Energy Group (CEG), Inc. or New Controlled, whichever entity is the ultimate parent of NMP LLC at that time, in the form and on the terms represented in the Application for license transfer. Should New Controlled become the ultimate parent of NMP LLC following the direct transfer of the license to NMP LLC, NMP LLC shall enter or shall have entered into a substantially identical intercompany credit agreement with New Controlled at the time New Controlled becomes the ultimate parent; in such case, any existing intercompany credit agreement with CEG, Inc. may be canceled once the intercompany credit agreement with New Controlled is established. Except as otherwise provided above, NMP LLC shall take no action to void, cancel, or modify any intercompany credit agreement referenced above, without the prior written consent of the Director of the Office of Nuclear Reactor Regulation.
- (13) Mitigation Strategy License Condition

The licensee shall develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

 - a. Fire fighting response strategy with the following elements:
 - (1) Pre-defined coordinated fire response strategy and guidance
 - (2) Assessment of mutual aid fire fighting assets
 - (3) Designated staging areas for equipment and materials
 - (4) Command and control
 - (5) Training of response personnel
 - b. Operations to mitigate fuel damage considering the following:
 - (1) Protection and use of personnel assets
 - (2) Communications
 - (3) Minimizing fire spread
 - (4) Procedures for implementing integrated fire response strategy
 - (5) Identification of readily-available pre-staged equipment
 - (6) Training on integrated fire response strategy
 - (7) Spent fuel pool mitigation measures

- c. Actions to minimize release to include consideration of:
 - (1) Water spray scrubbing
 - (2) Dose to onsite responders
- (14) The licensee shall implement and maintain all Actions required by Attachment 2 to NRC Order EA-06-137, issued June 20, 2006, except the last action that requires incorporation of the strategies into the site security plan, contingency plan, emergency plan and/or guard training and qualification plan, as appropriate.
- E. This license is effective as of the date of issuance and shall expire on August 22, 2029.
- F. The UFSAR supplement, as revised, submitted pursuant to 10 CFR 54.21(d), shall be included in the next scheduled update to the UFSAR required by 10 CFR 50.71(e)(4) following the issuance of this renewed operating license. Until that update is complete, NMP LLC may make changes to the programs and activities described in the supplement without prior Commission approval, provided that NMP LLC evaluates such changes pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.
- G. The UFSAR supplement, as revised, describes certain future activities to be completed prior to the period of extended operation. NMP LLC shall complete these activities in accordance with the schedule in Appendix A of NUREG-1900, "Safety Evaluation Report Related to the License Renewal of Nine Mile Point Nuclear Station, Units 1 and 2", dated September 2006, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
- H. All capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of the most recent NRC-approved version of the Boiling Water Reactor Vessels and Internals Project (BWRVIP) Integrated Surveillance Program (ISP) appropriate for the configuration of the specimens in the capsule. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC, as required by 10 CFR Part 50, Appendix H.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by

J. E. Dyer, Director
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A – Technical Specifications

Date of Issuance: October 31, 2006

Renewed License No. DPR-63
Revised by letter dated August 23, 2007

- (15) At the time any subject direct transfer is effected, NMP LLC shall enter or shall have entered into an intercompany credit agreement with Constellation Energy Group (CEG), Inc., or New Controlled, whichever entity is the ultimate parent of NMP LLC at that time, in the form and on the terms represented in the Application for license transfer. Should New Controlled become the ultimate parent of NMP LLC following the direct transfer of the license to NMP LLC, NMP LLC shall enter or shall have entered into a substantially identical intercompany credit agreement with New Controlled at the time New Controlled becomes the ultimate parent; in such case, any existing intercompany credit agreement with CEG, Inc. may be canceled once the intercompany credit agreement with New Controlled is established. Except as otherwise provided above, NMP LLC shall take no action to void, cancel, or modify any intercompany credit agreement referenced above, without the prior written consent of the Director of the Office of Nuclear Reactor Regulation.

(16) Reactor Vessel Integrated Surveillance Program

NMP LLC is authorized to revise the Updated Safety Analysis Report (USAR) to allow implementation of the Boiling Water Reactor Vessel and Internals Project reactor pressure vessel Integrated Surveillance Program as the basis for demonstrating compliance with the requirements of Appendix H to Title 10 of the *Code of Federal Regulations*, Part 50, "Reactor Vessel Material Surveillance Program Requirements," as set forth in the licensee's application dated January 9, 2004, and as supplemented on June 17, 2004.

(17) Mitigation Strategy License Condition

The operating licensee shall develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- a. Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel

b. Operations to mitigate fuel damage considering the following:

1. Protection and use of personnel assets
2. Communications
3. Minimizing fire spread
4. Procedures for implementing integrated fire response strategy
5. Identification of readily-available pre-staged equipment
6. Training on integrated fire response strategy
7. Spent fuel pool mitigation measures

c. Actions to minimize release to include consideration of:

1. Water spray scrubbing
2. Dose to onsite responders

- (18) The operating licensee shall implement and maintain all Actions required by Attachment 2 to NRC Order EA-06-137, issued June 20, 2006, except the last action that requires incorporation of the strategies into the site security plan, contingency plan, emergency plan and/or guard training and qualification plan, as appropriate.

D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70.

- i) An exemption from the critically alarm requirements of 10 CFR Part 70.24 was granted in the Special Nuclear Materials License No. SNM-1895 dated November 27, 1985. This exemption is described in Section 9.1 of Supplement 4 to the SER. This previously granted exemption is continued in this operating license.
- ii) Exemptions to certain requirements of Appendix J to 10 CFR Part 50 are described in Supplements 3, 4, and 5 to the SER. These include (a) (this item left intentionally blank); (b) an exemption from the requirement of Option B of Appendix J, exempting main steam isolation valve measured leakage from the combined leakage rate limit of 0.6 La. (Section 6.2.6 of SSER 5)*; (c) an exemption from Option B of Appendix J, exempting the hydraulic control system for the reactor recirculation flow control valves from Type A and Type C leak testing (Section 6.2.6 of SSER 3); (d) an exemption from Option B of Appendix J, exempting Type C testing on traversing incore probe system shear valves. (Section 6.2.6 SSER 4)

* The parenthetical notation following the discussion of each exemption denotes the section of the Safety Evaluation Report (SER) and/or its supplements wherein the safety evaluation of the exemption is discussed.

- iii) An exemption to Appendix A to 10 CFR Part 50 exempting the Control Rod Drive (CRD) hydraulic lines to the reactor recirculation pump seal purge equipment from General Design Criterion (GDC) 55. The CRD hydraulic lines to the reactor recirculation pump seal purge equipment use two simple check valves for the isolation outside containment (one side). (Section 6.2.4, SSER 3)
- iv) A schedular exemption to GDC 2, Appendix A to 10 CFR Part 50, until the first refueling outage, to demonstrate the adequacy of the downcomer design under the plant faulted condition. This exemption permits additional analysis and/or modifications, as necessary, to be completed by the end of the first refueling outage. (Section 6.2.1.7.4, SSER 3)
- v) A schedular exemption to GDC 50, Appendix A to 10 CFR Part 50 to allow the operating licensee until start-up following the "mini-outage," which is to occur within 12 months of commencing power operation (entering Operational Condition 1), to install redundant fuses in circuits that use transformers for redundant penetration protection in accordance with their letter of August 29, 1986 (NMP2L 0860). (Section 8.4.2, SSER 5)
- vi) A schedular exemption to 10 CFR 50.55a(h) for the Neutron Monitoring System until completion of the first refueling outage to allow the operating licensee to provide qualified isolation devices for Class 1E/non-1E interfaces described in their letters of June 23, 1987 (NMP2L 1057) and June 25, 1987 (NMP2L 1058). (Section 7.2.2.10, SSER 6).

For the schedular exemptions in iv), v), and vi), above, the operating licensee, in accordance with its letter of October 31, 1986, shall certify that all systems, components, and modifications have been completed to meet the requirements of the regulations for which the exemptions have been granted and shall provide a summary description of actions taken to ensure that the regulations have been met. This certification and summary shall be provided 10 days prior to the expiration of each exemption period as described above.

The exemptions set forth in this Section 2.D are authorized by law, will not present an undue risk to public health and safety, and are consistent with the common defense and security. These exemptions are hereby granted. The special circumstances regarding each exemption are identified in the referenced section of the Safety Evaluation Report and the supplements thereto. The exemptions in ii) through vi) are granted pursuant to 10 CFR 50.12.

With these exemptions, the facility will operate to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. Nine Mile Point Nuclear Station, LLC shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21 is entitled "Nine Mile Point Nuclear Station, LLC Physical Security, Safeguards Contingency, and Security Training and Qualification Plan, Revision 1," and was submitted by letter dated April 26, 2006. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. Nine Mile Point Nuclear Station, LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility through Amendment No. 27 and as described in submittals dated March 25, May 7 and 9, June 10 and 25, July 11 and 16, August 19 and 22, September 5, 12, and 23, October 10, 21, and 22, and December 9, 1986, and April 10 and May 20, 1987, and as approved in the SER dated February 1985 (and Supplements 1 through 6) subject to the following provision:

Nine Mile Point Nuclear Station, LLC may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.
- G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- H. This license is effective as of the date of issuance and shall expire at midnight on October 31, 2046.
- I. The UFSAR supplement, as revised, submitted pursuant to 10 CFR 54.21(d), shall be included in the next scheduled update to the USAR required by 10 CFR 50.71(e)(4) following the issuance of this renewed operating license. Until that update is complete, NMP LLC may make changes to the programs and activities described in the supplement without prior Commission approval, provided that NMP LLC evaluates such changes pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- J. The UFSAR supplement, as revised, describes certain future activities to be completed prior to the period of extended operation. NMP LLC shall complete these activities in accordance with Appendix A of NUREG-1900, "Safety Evaluation Report Related to the License Renewal of Nine Mile Point Nuclear Station, Units 1 and 2", dated September 2006, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
- K. For the renewed license term, all capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of the most recent NRC-approved version of the Boiling Water Reactor Vessels and Internals Project (BWRVIP) Integrated Surveillance Program (ISP) appropriate for the configuration of the specimens in the capsule. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC, as required by 10 CFR Part 50, Appendix H.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by

J. E. Dyer, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Appendix A – Technical Specifications (NUREG-1253)
- 2. Appendix B – Environmental Protection Plan

Date of Issuance: October 31, 2006