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U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555-0001

Joseph M. Farley Nuclear Plant Units 1 and 2
10 CFR 72.48(d)(2) Biennial Report

Ladies and Gentlemen:

Enclosed is the biennial report of changes to the Joseph M. Farley Nuclear Plant Independent Spent Fuel Storage Installation (ISFSI) cask designs in accordance with the provisions of 10 CFR 72.48. The enclosed report contains a description of the change implemented during the period from July 1, 2005 through July 1, 2007, and a summary of the 10 CFR 72.48 evaluation.

This letter contains no NRC commitments. If you have any questions, please advise.

Sincerely,

B. J. George
Manager, Nuclear Licensing

BJG/CHM/phr

Enclosure: Farley Nuclear Plant 10 CFR 72.48(d)(2) Biennial Report

cc: Southern Nuclear Operating Company
Mr. J. T. Gasser, Executive Vice President
Mr. J. R. Johnson, Vice President – Farley
Mr. D. H. Jones, Vice President – Engineering
RTYPE: CFA04.054; LC # 14614

U. S. Nuclear Regulatory Commission
Dr. W. D. Travers, Regional Administrator
Ms. K. R. Cotton, NRR Project Manager – Farley
Mr. E. L. Crowe, Senior Resident Inspector – Farley

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10 CFR 72.48 Evaluation Summary

The Holtec International reports HI-2043231, Rev. 1, "Dose Versus Distance from a HI-STORM 100S Version B Containing The MPC-32," HI-2053364, Rev. 2, "HI-STORM CoC Radiation Protection Program Dose Rate Limits," and the related Southern Nuclear Operating Company (SNC) calculation, SM-02-9809-01, Version 2, "Site Boundary Dose Assessment for Dry Cask Storage," were revised to increase the cobalt impurity level of 2.0 g/kg for stainless steel fuel assembly hardware above and below the active fuel region, as well as for the grid spacers. This resulted in minimal increase in the normal operation annual offsite dose for Farley Nuclear Plant (FNP) of 0.1 mR. The annual offsite dose for FNP remains well below the 10 CFR 72.104 limits by more than a factor of 3. The FNP 10 CFR 72.212 Report was revised to reflect the increase in the annual offsite dose, the change in allowable dose rates at different locations on the HI-TRAC and HI-STORM, and the difference between the Holtec Updated Dry Storage FSAR and FNP use of the dry storage system. This change does not impact the frequency of occurrence of any accident; does not impact the likelihood of occurrence of a malfunction of a structure, system, or component (SSC); results in a minimal increase in the normal operation annual offsite dose for FNP but remains well below the 10 CFR 72.104 limits; does not result in more than a minimal increase in the consequences of a malfunction of an SSC; does not change any accident assumptions; does not create the possibility of a malfunction of an SSC; and does not impact the fuel or the Multi-Purpose Canister (MPC) integrity.

The proposed change does not impact the conclusions contained in the FNP 10 CFR 72.212 Report regarding acceptability of the HI-STAR 100 and HI-STORM 100S cask systems for use in conjunction with the Farley ISFSI. Accordingly, the proposed change does not require a CoC Amendment pursuant to 72.48(c)(2).