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L-2006-240
10CFR50.4
10CFR2.390

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

OCT 17 2006

Subject: Florida Power and Light Company
Turkey Point Unit 3
Docket Nos. 50-250
Presentation Material Pertaining to Regulatory Conference
On Turkey Point Preliminary White Finding Held October 10, 2006

Reference: Letter, Mr. C. A. Casto to Mr. J. A. Stall, Turkey Point Nuclear Plant – NRC
Integrated Inspection Report 05000250/2006015; EA-06-200, Preliminary White
Finding, dated August 24, 2006

On October 10, 2006, a meeting was held between Florida Power and Light Company (FPL) and the Nuclear Regulatory Commission (NRC) in Atlanta, Georgia regarding a Preliminary White Finding discussed in the above referenced letter.

Provided in the attached are copies of the presentation material presented at the October 10, 2006 meeting. Attachment 1 is non-proprietary. Attachment 2 is considered proprietary and contains potentially sensitive security information. FPL requests that Attachment 2 be withheld from disclosure in accordance with 10CFR2.390.

If there are any questions regarding this letter, please contact Jim Connolly at 305-246-6632.

Sincerely Yours,

Terry O. Jones
Vice President
Turkey Point Nuclear Plant

Attachments: 1) Turkey Point Non-Proprietary Presentation Material Regarding Preliminary
White Finding 05000250/2006015-01
2) Turkey Point Proprietary Presentation Material Regarding Preliminary White
Finding 05000250/2006015-01

cc: NRC Regional Administrator
Senior Resident Inspector, USNRC, Turkey Point

IFD

ATTACHMENT 1
Turkey Point Non-Proprietary Presentation Material
Regarding Preliminary White Finding
05000250/2006015-01



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Regulatory Conference NRC Region II

Turkey Point Nuclear Plant Unit 3

Loss of Decay Heat Removal Event



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Agenda

- **Introductions**
- **Overview**
- **Topics of Discussion**
 - Event description
 - Corrective actions
 - Thermal-hydraulic analysis of event
 - Mitigating actions
 - SDP Analysis
- **Closing Remarks**



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Overview

- **FPL agrees that it did not comply with requirements of 10 CFR 50.65(a)(4)**
- **FPL has learned from the loss of decay heat removal event and has taken actions to prevent recurrence**
- **FPL evaluation concludes that the change in core damage frequency is less than 1.0E-6/yr**

Event Description

- **Initial conditions**
 - Unit 3 in Mode 5
 - Draindown in progress to support reactor head removal
- **Sequence of events**
 - While restoring power to 3C 480V load center, spurious undervoltage signal sent to 3A load sequencer
 - 3A load sequencer de-energized 3A 4kV bus, causing loss of running 3A RHR pump
 - 3A EDG re-energized 3A 4kV bus
 - 3A load sequencer does not automatically re-start the 3A RHR pump after loss of offsite power
 - Operator started 3B RHR pump and terminated the event in approximately 9 minutes



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Causes

-
- **Insufficient defense in depth to prevent the event**
 - **The outage risk assessment procedure was insufficient**
 - **Experience in maneuvering plant was low with significant shutdown maintenance in progress**
 - **Vendor human error in the configuration of auxiliary switch contacts on a 480V load center breaker that went undetected**



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Immediate Corrective Actions Taken

- **Senior management team augmented by fleet after event for additional oversight**
- **Additional reviews of remaining outage schedule performed**
- **Additional controls of protected plant and switchyard equipment implemented**
- **Outage schedule changes subject to more rigorous review and approval process**

Long Term Corrective Actions

- **Outage risk assessment and control procedure upgraded**
 - Responsibility for procedure transferred to Operations
 - PNSC approval required for procedure changes
 - Clearly identifies required protected in-service equipment for higher risk evolutions
 - Provides logic ties for risk significant activities
- **Use of dedicated and more experienced licensed operators for outage planning and risk assessment (complete)**

Long Term Corrective Actions (cont'd)

- **As-left auxiliary switch contact configuration to be verified by Nuclear Receipt Inspection for 4kV & 480V breakers (complete)**
- **Plant procedures for safety-related breakers revised to check auxiliary switch contact configuration on 4kV & 480V breakers (completed for procedures needed for Fall outage breaker work)**
- **Applicable plant procedure revised to defeat the sequencer during replacement of 480V load center breakers (complete)**

Long Term Corrective Actions (cont'd)

- **Fleet peer reviews of outage schedule (complete)**
- **Management challenge of outage schedule (prior to Fall outage)**
- **Enhanced operator and staff training on shutdown risk assessment (in-progress, complete prior to Fall outage)**
- **Outage risk management improvements (perform prior to RCS draindown)**
 - Pressurizer code safety removed
 - At least two Core Exit Thermocouples available (until just prior to detensioning reactor vessel head)
 - Containment closure ability confirmed

FPL Analysis of Loss of RHR Event

- **Thermal-hydraulic simulation to determine effects of loss of RHR scenarios**
 - Case 1 - No operator actions
 - Case 2 - HHSI feed only
 - Case 3 - HHSI feed & PORV bleed
- **Use results to develop FPL SDP event tree**
- **Using event tree and failure probabilities, calculate change in core damage frequency**

Initial Plant Conditions

- 63 hours 50 minutes after shutdown
 - prior to shutdown reactor was at ~50% power for 24 hours
- RCS being drained to support reactor vessel head lift
- RCS level near reactor vessel flange
- RCS temperature ~115 °F
- RCS vented via:
 - Reactor vessel head vent line with 0.219" diameter orifice
 - Pressurizer vent line 0.742" diameter
- A-RHR in service
- B-RHR in standby

Initial Plant Conditions (cont'd)

- SG secondary side water levels average 84 % wide range
- SG atmospheric steam dumps full open
- Both RWSTs with inventory ~295,000 gal per unit available for HHSI pump use while maintaining NPSH
- Equipment required to mitigate loss of RHR in service
- 2nd qualified Unit Supervisor supervising draindown

Case 1 – No Operator Action

- **Conclusion:**
 - With no operator action, RHR cooling will be restored simply by starting an RHR pump within approximately 9 hours after event initiation
 - No core damage with RHR pump start anytime during first 9 hrs of event

Case 2 – HHSI Feed Only

- **Conclusion:**
 - Able to sustain steady state condition for at least 24 hours with single RWST
 - No core damage for at least 24 hours
 - Sufficient time available to implement RWST inventory management or SG secondary water makeup

Case 3 – HHSI Feed & PORVs Bleed

- **Conclusion:**
 - No core damage for at least 16 hrs using both RWSTs
 - Sufficient time available to restore RHR or implement RWST inventory management

Thermal-hydraulic Analysis Conclusions

- SG reflux cooling will prevent core damage without operator action for at least 9 hours
- The minimum time to start a RHR pump is at least 9 hours (time to boil is overly conservative as the criterion for RHR pump start)
- Feed & bleed prevents core damage regardless of pressurizer PORVs position
- Managing RWST inventory is proceduralized with options to:
 - Throttle HHSI flow
 - Establish RWST makeup
 - Use opposite unit RWST

Key Factors for Additional NRC Consideration

- Base RHR restoration time on NPSH requirements (9 hr) rather than core boiling (21 min)
- Failure of PORVs to open for feed & bleed does not result in core damage
- Late restoration of RHR based on additional time provided by SG reflux cooling and feed & bleed
- Additional RWST inventory management strategies to extend availability of HHSI suction source
 - Throttling HHSI pump flow
 - Using opposite unit RWST

Summary of SDP Results

- Based on a more detailed SDP analysis FPL estimated the total CDF increase for this event to be approximately $2.0\text{E-}7/\text{yr}$
- CDF increase below risk significance threshold of $1.0\text{E-}6/\text{yr}$
- FPL concluded this violation to be GREEN

ROP Cornerstone

- NRC ROP Cornerstone for this finding should be “Initiating Events”
 - ROP “Initiating Events” Cornerstone objective: limit frequency of events that upset plant stability and challenge critical safety functions
- Definitions: NRC Manual Chapter 0308 – ROP Basis Document
 - Initiating Events- “such events include reactor trips due to turbine trips, loss of feedwater, loss of off-site power . . .”
 - Mitigating Systems- “include those systems associated with safety injection, residual heat removal, and their support systems. . .”
- Event attributable to the loss of 3A 4kV bus normal electrical power to the running 3A RHR pump, not involving a failure attributable to the RHR System

Conclusions

- FPL agrees that it did not comply with requirements of 10 CFR 50.65(a)(4)
- Review of SDP analysis shows low safety significance with delta CDF < 1.0E-6/yr
- FPL has taken timely and aggressive corrective actions to prevent recurrence

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Open Discussion

Questions



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Final Remarks