



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

September 18, 2006

Virginia Electric and Power Company
ATTN.: Mr. David A. Christian
Sr. Vice President and
Chief Nuclear Officer
Innsbrook Technical Center - 2SW
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: NOTIFICATION OF NORTH ANNA NUCLEAR POWER PLANT - COMPONENT
DESIGN BASES INSPECTION - NRC INSPECTION REPORT NOS.
05000338/2007006, 05000339/2007006

Dear Mr Christian:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your North Anna Nuclear Power Plant during the weeks of January 8 - 12, 2007; January 22 - 26, 2007; and February 5 - 9, 2007. The inspection team will be led by Mr. Bob Hagar, a Senior Resident Inspector in Region II. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued June 22, 2006.

The inspection will evaluate the capability of risk significant / low margin components to function as designed and support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on September 13, 2006, Mr. Hagar confirmed with Mr. J. Leberstien of your staff, arrangements for an information-gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information-gathering visit: December 12-14, 2006
- Onsite inspection: January 8 - 12, January 22 - 26, and February 5 - 9, 2007

The purpose of the information-gathering visit is to meet with members of your staff, to identify risk-significant components and operator actions, and to identify information and documentation needed to support the inspection. Mr. Walt Rogers, a Region II Senior Reactor Analyst, may accompany Mr. Hagar during the information-gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection.

The enclosure lists documents that will be needed prior to the information-gathering visit. Please contact Mr. Hagar prior to preparing copies of the materials listed in the enclosure.

The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for the inspection preparation.

During the information-gathering visit, the team leader will also discuss the following inspection support administrative details: office space, supplemental documents requested to be made available to the team in the Region II office prior to the inspection preparation week of January 2, 2007; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Hagar at (843) 339-2822 or me at (404) 562-4605.

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-338, 50-339
License Nos.: NPF-4, NPF-7

Enclosure
cc w/enc/: (See page 3)

VEPCO

3

cc w/encl:

Chris L. Funderburk, Director
Nuclear Licensing and
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Richmond, VA 23219

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Docket Nos.: 50-338, 50-339
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cc w/enc/: (See page 3)

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OFFICE	RII:DRS	RII:DRSP					
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NAME	E. Guthrie	R. Hagar					
DATE	9/18/2006	9/18/2006		6/ /2006	6/ /2006	6/ /2006	6/ /2006
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INFORMATION REQUEST FOR NORTH ANNA NUCLEAR POWER PLANT COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Lists should contain enough information to be easily understood to someone who has a knowledge of pressurized water reactor technology.

1. From your most-recent probabilistic safety analysis (PSA) excluding external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
 - c. A risk ranking of all operator actions, sorted by RAW.
2. From your most-recent probabilistic safety analysis (PSA) including external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
 - c. A risk ranking of all operator actions, sorted by RAW.
3. Human reliability worksheets for the operator actions included in your PSA model.
4. Any pre-existing evaluation or list of components and calculations with low design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, MOV risk-margin rankings, etc.).
5. A list of operating experience evaluations completed within the last two years, sorted by associated component and/or system.
6. A list of design modifications implemented within the last two years, sorted by affected system.
7. A list of corrective-action evaluations that involved plant equipment and were completed within the last two years, sorted by involved component or system.
8. A list of common-cause failure of components that have occurred at North Anna and have been identified within the last five years.

Enclosure

9. A list of operability evaluations completed within the last two years, sorted by associated component or system.
10. Contact information for a person to discuss PRA information prior to bag man trip: name, title, phone number, and e-mail address
11. List of equipment currently on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address those reasons.
12. List of equipment currently in GL 91-18 status.
13. List of equipment currently in MR (a)(1) status.