

- (3) SCE, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
 - (4) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of San Onofre Nuclear Generating Station, Units 1 and 2 and by the decommissioning of San Onofre Nuclear Generating Station Unit 1.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern California Edison Company (SCE) is authorized to operate the facility at reactor core power levels not in excess of full power (3438 megawatts thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 203, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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1.1 Definitions

OPERABLE – OPERABILITY

A system, subsystem, train, component, or device shall be OPERABLE when it is capable of performing its specified safety function(s) and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication, and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s).

PHYSICS TESTS

PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are:

- a. Described in Chapter 14, Initial Test Program of the SONGS Units 2 and 3 UFSAR;
- b. Authorized under the provisions of 10 CFR 50.59; or
- c. Otherwise approved by the Nuclear Regulatory Commission.

PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.7.1.6.

RATED THERMAL POWER (RTP)

RTP shall be a total reactor core heat transfer rate to the reactor coolant of 3438 MWt.

(continued)

1.1 Definitions

REACTOR PROTECTIVE
SYSTEM (RPS) RESPONSE
TIME

The RPS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RPS trip setpoint at the channel sensor until electrical power to the CEAs drive mechanism is interrupted. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.

SHUTDOWN MARGIN (SDM)

SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming:

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.3 RCS Pressure and Temperature (P/T) Limits

LCO 3.4.3 The combination of RCS pressure, RCS temperature and RCS heatup and cooldown rates shall be maintained within the limits as specified in the RCS PRESSURE-TEMPERATURE LIMITS REPORT (PTLR).

APPLICABILITY: At all times.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. -----NOTE----- Required Action A.2 shall be completed whenever this Condition is entered. ----- Requirements of LCO not met in MODE 1, 2, 3, or 4.</p>	A.1 Restore parameter(s) to within limits.	30 minutes
	<p><u>AND</u></p> <p>A.2 Determine RCS is acceptable for continued operation.</p>	72 hours
<p>B. Required Action and associated Completion Time of Condition A not met.</p>	B.1 Be in MODE 3.	6 hours
	<p><u>AND</u></p> <p>B.2 Be in MODE 5 with RCS pressure < 500 psia.</p>	36 hours
<p>C. -----NOTE----- Required Action C.2 shall be completed whenever this Condition is entered. ----- Requirements of LCO not met any time in other than MODE 1, 2, 3, or 4.</p>	C.1 Initiate action to restore parameter(s) to within limits.	Immediately
	<p><u>AND</u></p> <p>C.2 Determine RCS is acceptable for continued operation.</p>	Prior to entering MODE 4

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.4.3.1 -----NOTE----- Only required to be performed during RCS heatup and cooldown operations and RCS inservice leak and hydrostatic testing. ----- Verify RCS pressure, RCS temperature, and RCS heatup and cooldown rates within limits specified in the PTLR.</p>	<p>30 minutes</p>
<p>SR 3.4.3.2 The reactor vessel material irradiation surveillance specimens shall be removed and examined, to determine changes in material properties, as required by 10 CFR 50 Appendix H. The results of these examinations shall be used to update the PTLR.</p>	<p>In accordance with requirements of 10CFR 50 Appendix H</p>

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FIGURE 3.4.3-2
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FIGURE 3.4.3-3
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FIGURE 3.4.3-4
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FIGURE 3.4.3-5
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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.6 RCS Loops – MODE 4

LCO 3.4.6 Two loops or trains consisting of any combination of RCS loops and shutdown cooling (SDC) trains shall be OPERABLE and at least one loop or train shall be in operation.

-----NOTES-----

1. All reactor coolant pumps (RCPs) and SDC pumps may be de-energized for ≤ 1 hour per 8 hour period, provided:
 - a. No operations are permitted that would cause introduction into the RCS, coolant with boron concentration less than required to meet the SDM of LCO 3.1.1; and
 - b. Core outlet temperature is maintained at least 10°F below saturation temperature.
2. No RCP shall be started with any RCS cold leg temperature less than or equal to the LTOP enable temperature specified in the PTLR unless:
 - a. Pressurizer water volume is $< 900 \text{ ft}^3$, or
 - b. Secondary side water temperature in each steam generator (SG) is $< 100^\circ\text{F}$ above each of the RCS cold leg temperatures.

APPLICABILITY: MODE 4.

- NOTES (continued)-----
4. No reactor coolant pump (RCP) shall be started with one or more of the RCS cold leg temperatures \leq the temperature in the PTLR unless:
 - a. The pressurizer water volume is $< 900 \text{ ft}^3$ or
 - b. The secondary side water temperature in each steam generator (SG) is $< 100^\circ\text{F}$ above each of the RCS cold leg temperatures.
 5. A containment spray pump may be used in place of a low pressure safety injection pump in either or both shutdown cooling trains to provide shutdown cooling flow provided the reactor has been subcritical for a period > 24 hours and the RCS is fully depressurized and vented in accordance with LCO 3.4.12.1.
 6. All SDC trains may be removed from operation during planned heatup to MODE 4 when at least one RCS loop is in operation.
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APPLICABILITY: MODE 5 with RCS loops filled.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Less than the required SDC trains/RCS loops OPERABLE. <u>AND</u> Any SG with secondary side water level not within limit.	A.1 Initiate action to restore the required SDC trains/RCS loops to OPERABLE status.	Immediately
	<u>OR</u> A.2 Initiate action to restore SG secondary side water levels to within limits.	Immediately

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12.1 Low Temperature Overpressure Protection (LTOP) System

RCS Temperature \leq PTLR Limit

LC0 3.4.12.1 No more than two high pressure safety injection pumps shall be OPERABLE, the safety injection tanks shall be isolated or depressurized to less than the limit specified in the PTLR and at least one of the following overpressure protection systems shall be OPERABLE:

a. The Shutdown Cooling System Relief Valve (PSV9349) with:

- 1) A lift setting of 406 ± 10 psig,
- 2) Relief Valve isolation valves 2HV9337, 2HV9339, 2HV9377, and 2HV9378 open,

or,

b. The Reactor Coolant System depressurized with an RCS vent of greater than or equal to 5.6 square inches.

APPLICABILITY: MODE 4 when the temperature of any one RCS cold leg is less than or equal to the enable temperatures specified in the PTLR,

MODE 5, and

MODE 6 when the head is on the reactor vessel and the RCS is not vented.

-----NOTE-----

SIT isolation or depressurization to less than the limits in the PTLR is only required when SIT pressure is greater than or equal to the maximum RCS pressure for the existing RCS cold leg temperature allowed by the PTLR.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. With more than two HPSI pumps capable of injecting into the RCS.	A.1 Initiate action to verify a maximum of two HPSI pumps capable of injecting into the RCS.	Immediately
B. SIT pressure is greater than or equal to the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	B.1 Isolate affected SIT.	1 hour
C. Required Action and associated Completion Time of Condition B not met.	C.1 Depressurize affected SIT to less than the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	12 hours
D. With one or both SDCS Relief Valve isolation valves in a single SDCS Relief Valve isolation valve pair (valve pair 2HV9337 and 2HV9339 or valve pair 2HV9377 and 2HV9378) closed.	D.1 Open the closed valve(s). <u>OR</u> D.2 Power-lock open the OPERABLE SDCS Relief Valve isolation valve pair.	24 hours 24 hours

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.4.12.1.1 -----NOTE----- A HPSI pump is secured by verifying that its motor circuit breaker is not racked-in, or its discharge valve is locked closed. The requirement to rack out the HPSI pump breaker is satisfied with the pump breaker racked out to its disconnected or test position. ----- Verify a maximum of two HPSI pumps are capable of injecting into the RCS.</p>	<p>12 hours</p>
<p>SR 3.4.12.1.2 -----NOTE----- Required to be performed when complying with the LCO 3.4.12.1 Note. ----- Verify each SIT is isolated or depressurized less than the limit specified in the PTLR.</p>	<p>12 hours</p>
<p>SR 3.4.12.1.3 Verify RCS vent \geq 5.6 square inches is open when in use for overpressure protection.</p>	<p>12 hours for unlocked open vent valve(s) <u>AND</u> 31 days for locked, sealed, or otherwise secured open vent valve(s), or open flanged RCS penetrations</p>

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12.2 Low Temperature Overpressure Protection (LTOP) System

RCS Temperature > PTLR Limit

- LC0 3.4.12.2 At least one of the following overpressure protection systems shall be OPERABLE:
- a. The Shutdown Cooling System Relief Valve (PSV9349) with:
 - 1) A lift setting of 406 ± 10 psig,
 - 2) Relief Valve isolation valves 2HV9337, 2HV9339, 2HV9377, and 2HV9378 open,
 - or,
 - b. A minimum of one pressurizer code safety valve with a lift setting of $2500 \text{ psia} \pm 1\%$.

APPLICABILITY: MODE 4 when the temperature of all RCS cold legs are greater than the enable temperatures specified in the PTLR.

- NOTES-----
1. The lift setting pressure of the pressurizer code safety valve shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.
 2. The SDCS Relief Valve lift setting assumes valve temperatures less than or equal to 130°F .
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5.7 Reporting Requirements (continued)

5.7.1.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

1. CENPD-132P, "Calculative Methods for the C-E Large Break LOCA Evaluation Model"
 2. CENPD-137P, "Calculative Methods for the C-E Small Break LOCA Evaluation Model"
 3. CEN-356(V)-P-A, "Modified Statistical Combination of Uncertainties"
 4. SCE-9801-P-A, "Reload Analysis Methodology for the San Onofre Nuclear Generating Station Units 2 and 3"
 5. CEN-635(S), "Identification of NRC Safety Evaluation Report Limitations and/or Constraints on Reload Analysis Methodology"
 6. Letter, dated May 16, 1986, G. W. Knighton (NRC) to K. P. Baskin (SCE), "Issuance of Amendment No. 47 to Facility Operating License NPF-10 and Amendment No. 36 to Facility Operating License NPF-15," San Onofre Nuclear Generating Station Units 2 and 3 (Cycle 3 SER)
 7. Letter, dated January 9, 1985, G. W. Knighton (NRC) to K. P. Baskin, "Issuance of Amendment No. 30 to Facility Operating License NPF-10 and Amendment No. 19 to Facility Operating License NPF-15," San Onofre Nuclear Generating Station Units 2 and 3 (Cycle 2 SER)
 8. "Implementation of ZIRLO™ Cladding Material in CE Nuclear Power Fuel Assembly Designs," CENPD-404-P-A
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal hydraulic limits, Emergency Core Cooling System (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any mid-cycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.7.1.6 REACTOR COOLANT SYSTEM (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:

(continued)

5.7 Reporting Requirements (continued)

5.7.1.6 REACTOR COOLANT SYSTEM (RCS) PRESSURE AND TEMPERATURE LIMITS
REPORT (PTLR) (continued)

Technical Specification 3.4.3 RCS Pressure and Temperature
(P/T) Limits,

Technical Specification 3.4.6 RCS Loops - MODE 4,

Technical Specification 3.4.7 RCS Loops - MODE 5, Loops
Filled,

Technical Specification 3.4.12.1 Low Temperature
Overpressure Protection (LTOP) System RCS Temperature
 \leq PTLR Limit, and

Technical Specification 3.4.12.2 Low Temperature
Overpressure Protection (LTOP) System RCS Temperature
 $>$ PTLR Limit.

- b. The analytical methods used to determine the RCS pressure
and temperature limits shall be those previously reviewed
and approved by the NRC, specifically those described in the
following document:

CE NPSD-683-A, The Development of a RCS Pressure and
Temperature Limits Report for the Removal of P-T Limits and
LTOP Setpoints from the Technical Specifications.

- c. The PTLR shall be provided to the NRC upon issuance for each
reactor vessel fluence period and for any revision or
supplement thereto.

5.7.1.7 Hazardous Cargo Traffic Report

Hazardous cargo traffic on Interstate 5 (I-5) and the AT&SF
railway shall be monitored and the results submitted to the NRC
Regional Administrator once every three years.

(continued)

- (3) SCE, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
 - (4) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear materials as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) SCE, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of San Onofre Nuclear Generating Station, Units 1 and 3 and by the decommissioning of San Onofre Nuclear Generating Station Unit 1.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

Southern California Edison Company (SCE) is authorized to operate the facility at reactor core power levels not in excess of full power (3438 megawatts thermal).
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 195, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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(continued)

1.1 Definitions

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PHYSICS TESTS

PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are:

- a. Described in Chapter 14, Initial Test Program of the SONGS Units 2 and 3 UFSAR;
- b. Authorized under the provisions of 10 CFR 50.59; or
- c. Otherwise approved by the Nuclear Regulatory Commission.

RCS PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.7.1.6.

RATED THERMAL POWER (RTP)

RTP shall be a total reactor core heat transfer rate to the reactor coolant of 3438 MWt.

(continued)

1.1 Definitions

REACTOR PROTECTIVE
SYSTEM (RPS) RESPONSE
TIME

The RPS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RPS trip setpoint at the channel sensor until electrical power to the CEAs drive mechanism is interrupted. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.

SHUTDOWN MARGIN (SDM)

SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming:

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.4.3.1 -----NOTE----- Only required to be performed during RCS heatup and cooldown operations and RCS inservice leak and hydrostatic testing. ----- Verify RCS pressure, RCS temperature, and RCS heatup and cooldown rates within limits specified in the PTLR.</p>	<p>30 minutes</p>
<p>SR 3.4.3.2 The reactor vessel material irradiation surveillance specimens shall be removed and examined, to determine changes in material properties, as required by 10 CFR 50 Appendix H. The results of these examinations shall be used to update the PTLR.</p>	<p>In accordance with requirements of 10CFR 50 Appendix H</p>

FIGURE 3.4.3-1
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FIGURE 3.4.3-2
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FIGURE 3.4.3-3
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FIGURE 3.4.3-4
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FIGURE 3.4.3-5
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TABLE 3.4.3-1
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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.6 RCS Loops – MODE 4

LCO 3.4.6 Two loops or trains consisting of any combination of RCS loops and shutdown cooling (SDC) trains shall be OPERABLE and at least one loop or train shall be in operation.

-----NOTES-----

1. All reactor coolant pumps (RCPs) and SDC pumps may be de-energized for ≤ 1 hour per 8 hour period, provided:
 - a. No operations are permitted that would cause introduction into the RCS, coolant with boron concentration less than required to meet the SDM of LCO 3.1.1; and
 - b. Core outlet temperature is maintained at least 10°F below saturation temperature.
2. No RCP shall be started with any RCS cold leg temperature less than or equal to the LTOP enable temperature specified in the PTLR unless:
 - a. Pressurizer water volume is $< 900 \text{ ft}^3$, or
 - b. Secondary side water temperature in each steam generator (SG) is $< 100^\circ\text{F}$ above each of the RCS cold leg temperatures.

APPLICABILITY: MODE 4.

-----NOTES (continued)-----

4. No reactor coolant pump (RCP) shall be started with one or more of the RCS cold leg temperatures \leq the temperature in the PTLR unless:
 - a. The pressurizer water volume is $< 900 \text{ ft}^3$ or
 - b. The secondary side water temperature in each steam generator (SG) is $< 100^\circ\text{F}$ above each of the RCS cold leg temperatures.
 5. A containment spray pump may be used in place of a low pressure safety injection pump in either or both shutdown cooling trains to provide shutdown cooling flow provided the reactor has been subcritical for a period > 24 hours and the RCS is fully depressurized and vented in accordance with LCO 3.4.12.1.
 6. All SDC trains may be removed from operation during planned heatup to MODE 4 when at least one RCS loop is in operation.
-

APPLICABILITY: MODE 5 with RCS loops filled.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Less than the required SDC trains/RCS loops OPERABLE. <u>AND</u> Any SG with secondary side water level not within limit.	A.1 Initiate action to restore the required SDC trains/RCS loops to OPERABLE status.	Immediately
	<u>OR</u> A.2 Initiate action to restore SG secondary side water levels to within limits.	Immediately

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12.1 Low Temperature Overpressure Protection (LTOP) System

RCS Temperature \leq PTLR Limit

LCO 3.4.12.1 No more than two high pressure safety injection pumps shall be OPERABLE, the safety injection tanks shall be isolated or depressurized to less than the limit specified in the PTLR and at least one of the following overpressure protection systems shall be OPERABLE:

- a. The Shutdown Cooling System Relief Valve (PSV9349) with:
 - 1) A lift setting of 406 ± 10 psig,
 - 2) Relief Valve isolation valves 3HV9337, 3HV9339, 3HV9377, and 3HV9378 open,

or,

- b. The Reactor Coolant System depressurized with an RCS vent of greater than or equal to 5.6 square inches.

APPLICABILITY: MODE 4 when the temperature of any one RCS cold leg is less than or equal to the enable temperatures specified in the PTLR,

MODE 5, and

MODE 6 when the head is on the reactor vessel and the RCS is not vented.

-----NOTE-----
SIT isolation or depressurization to less than the limits in the PTLR is only required when SIT pressure is greater than or equal to the maximum RCS pressure for the existing RCS cold leg temperature allowed by the PTLR.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. With more than two HPSI pumps capable of injecting into the RCS.	A.1 Initiate action to verify a maximum of two HPSI pumps capable of injecting into the RCS.	Immediately
B. SIT pressure is greater than or equal to the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	B.1 Isolate affected SIT.	1 hour
C. Required Action and associated Completion Time of Condition B not met.	C.1 Depressurize affected SIT to less than the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	12 hours
D. With one or both SDCS Relief Valve isolation valves in a single SDCS Relief Valve isolation valve pair (valve pair 3HV9337 and 3HV9339 or valve pair 3HV9377 and 3HV9378) closed.	D.1 Open the closed valve(s).	24 hours
	<u>OR</u> D.2 Power-lock open the OPERABLE SDCS Relief Valve isolation valve pair.	24 hours

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.4.12.1.1 -----NOTE----- A HPSI pump is secured by verifying that its motor circuit breaker is not racked-in, or its discharge valve is locked closed. The requirement to rack out the HPSI pump breaker is satisfied with the pump breaker racked out to its disconnected or test position. ----- Verify a maximum of two HPSI pumps are capable of injecting into the RCS.</p>	<p>12 hours</p>
<p>SR 3.4.12.1.2 -----NOTE----- Required to be performed when complying with the LCO 3.4.12.1 Note. ----- Verify each SIT is isolated or depressurized less than the limit specified in the PTLR.</p>	<p>12 hours</p>
<p>SR 3.4.12.1.3 Verify RCS vent \geq 5.6 square inches is open when in use for overpressure protection.</p>	<p>12 hours for unlocked open vent valve(s) <u>AND</u> 31 days for locked, sealed, or otherwise secured open vent valve(s), or open flanged RCS penetrations</p>

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12.2 Low Temperature Overpressure Protection (LTOP) System

RCS Temperature > PTLR Limit

- LC0 3.4.12.2 At least one of the following overpressure protection systems shall be OPERABLE:
- a. The Shutdown Cooling System Relief Valve (PSV9349) with:
 - 1) A lift setting of 406 ± 10 psig,
 - 2) Relief Valve isolation valves 3HV9337, 3HV9339, 3HV9377, and 3HV9378 open,
 - or,
 - b. A minimum of one pressurizer code safety valve with a lift setting of $2500 \text{ psia} \pm 1\%$.

APPLICABILITY: MODE 4 when the temperature of all RCS cold legs are greater than the enable temperatures specified in the PTLR.

- NOTES-----
- 1. The lift setting pressure of the pressurizer code safety valve shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.
 - 2. The SDCS Relief Valve lift setting assumes valve temperatures less than or equal to 130°F .
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5.7 Reporting Requirements (continued)

5.7.1.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

1. CENPD-132P, "Calculative Methods for the C-E Large Break LOCA Evaluation Model"
 2. CENPD-137P, "Calculative Methods for the C-E Small Break LOCA Evaluation Model"
 3. CEN-356(V)-P-A, "Modified Statistical Combination of Uncertainties"
 4. SCE-9801-P-A, "Reload Analysis Methodology for the San Onofre Nuclear Generating Station Units 2 and 3"
 5. CEN-635(S), "Identification of NRC Safety Evaluation Report Limitations and/or Constraints on Reload Analysis Methodology"
 6. Letter, dated May 16, 1986, G. W. Knighton (NRC) to K. P. Baskin (SCE), "Issuance of Amendment No. 47 to Facility Operating License NPF-10 and Amendment No. 36 to Facility Operating License NPF-15," San Onofre Nuclear Generating Station Units 2 and 3 (Cycle 3 SER)
 7. Letter, dated January 9, 1985, G. W. Knighton (NRC) to K. P. Baskin, "Issuance of Amendment No. 30 to Facility Operating License NPF-10 and Amendment No. 19 to Facility Operating License NPF-15," San Onofre Nuclear Generating Station Units 2 and 3 (Cycle 2 SER)
 8. "Implementation of ZIRLO™ Cladding Material in CE Nuclear Power Fuel Assembly Designs," CENPD-404-P-A
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal hydraulic limits, Emergency Core Cooling System (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any mid-cycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.7.1.6 REACTOR COOLANT SYSTEM (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:

(continued)

5.7 Reporting Requirements (continued)

5.7.1.6 REACTOR COOLANT SYSTEM (RCS) PRESSURE AND TEMPERATURE LIMITS
REPORT (PTLR) (continued)

Technical Specification 3.4.3 RCS Pressure and Temperature
(P/T) Limits,

Technical Specification 3.4.6 RCS Loops - MODE 4,

Technical Specification 3.4.7 RCS Loops - MODE 5, Loops
Filled,

Technical Specification 3.4.12.1 Low Temperature
Overpressure Protection (LTOP) System RCS Temperature
 \leq PTLR Limit, and

Technical Specification 3.4.12.2 Low Temperature
Overpressure Protection (LTOP) System RCS Temperature
 $>$ PTLR Limit.

- b. The analytical methods used to determine the RCS pressure
and temperature limits shall be those previously reviewed
and approved by the NRC, specifically those described in the
following document:

CE NPSD-683-A, The Development of a RCS Pressure and
Temperature Limits Report for the Removal of P-T Limits and
LTOP Setpoints from the Technical Specifications.

- c. The PTLR shall be provided to the NRC upon issuance for each
reactor vessel fluence period and for any revision or
supplement thereto.

5.7.1.7 Hazardous Cargo Traffic Report

Hazardous cargo traffic on Interstate 5 (I-5) and the AT&SF
railway shall be monitored and the results submitted to the NRC
Regional Administrator once every three years.

(continued)