September 21, 2006

Mr. James A. Spina, Vice President
Calvert Cliffs Nuclear Power Plant, Inc.
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 - AMENDMENT RE: DELETION OF TERM CORE ALTERATIONS (TAC NOS. MC7330 AND MC7331)

Dear Mr. Spina:

The Commission has issued the enclosed Amendment No. 279 to Renewed Facility Operating License No. DPR-53 and Amendment No. 256 to Renewed Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated June 7, 2005, as supplemented on May 12, 2006.

These amendments revise the TSs to eliminate the use of the defined term Core Alterations. The amendments incorporate the changes reflected in TS Task Force (TSTF) Travelers 471-T (TSTF-471-T), "Eliminate use of term CORE ALTERATIONS in ACTIONS and Notes," and TSTF-51-A, "Revise containment requirements during handling irradiated fuel and core alterations." The amendments also include a revision to TS-471-T to replace Core Alterations with "positive reactivity additions" in TS 3.9.2, "Nuclear Instrumentation."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Patrick D. Milano, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosures:
1. Amendment No. 279 to DPR-53
2. Amendment No. 256 to DPR-69
3. Safety Evaluation

cc w/encls: See next page

September 21, 2006

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1650 Calvert Cliffs Parkway
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cc w/encls: See next page
Accession Number: ML062350447

OFFICE	LPLI-1/PM	LPLI-1/LA	SPWB/BC	EEEB/BC	ITSB/BC	OGC	LPLI-1/BC
NAME	PMilano	SLittle	JNakoski	GWilson	TKobetz	DRoth	RLaufer
DATE	09/20/06	09/20/06	08/24/06	08/29/06	09/19/06	09/18/06	09/21/06

OFFICIAL RECORD COPY

Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2

cc:

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Resident Inspector
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DATED: September 21, 2006

AMENDMENT NO. 279 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-53
CALVERT CLIFFS UNIT 1

AMENDMENT NO. 256 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69
CALVERT CLIFFS UNIT 2

PUBLIC	
LPLI-1 R/F	
R. Laufer	RidsNrrDorlLpl1-1
S. Little	RidsNrrLASLittle
P. Milano	RidsNrrPMPMilano
J. Nakoski	RidsNrrDssSpwb
G. Wilson	RidsNrrDeEeeb
K. Desai	
T. Boyce	RidsNrrDirsItsb
G. Hill (2)	
OGC	RidsOgcRp
ACRS	RidsAcrsAcnwMailCenter
B. McDermott, RI	RidsRgn1MailCenter

cc: Plant Service list

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 279
Renewed License No. DPR-53

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) dated June 7, 2005, as supplemented on May 12, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the License and Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-53 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 279, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: September 21, 2006

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 256
Renewed License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) dated June 7, 2005, as supplemented on May 12, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the License and Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 256, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: September 21, 2006

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 279 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 256 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	Insert Pages
3 (DPR-53)	3
3 (DPR-69)	3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	Insert Pages
1.1-2	1.1-2
1.1-3	1.1-3
1.1-4	1.1-4
1.1-5	1.1-5
1.1-6	1.1-6
3.3.7-1	3.3.7-1
3.8.2-2	3.8.2-2
3.8.2-3	3.8.2-3
3.8.2-4	3.8.2-4
3.8.5-1	3.8.5-1
3.8.5-2	3.8.5-2
3.8.8-1	3.8.8-1
3.8.8-2	3.8.8-2
3.8.10-1	3.8.10-1
3.8.10-2	3.8.10-2
3.9.1-1	3.9.1-1
3.9.2-1	3.9.2-1
3.9.3-1	3.9.3-1
3.9.3-2	3.9.3-2
3.9.4-1	3.9.4-1
3.9.6-1	3.9.6-1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 279 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-53

AND AMENDMENT NO. 256 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By letter dated June 7, 2005, as supplemented on May 12, 2006 (Agencywide Documents Access and Management System Accession Nos. ML051660207 and ML061380129, respectively), the Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) submitted a request for changes to the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2 Technical Specifications (TSs). The requested changes would revise the TSs to eliminate the use of the defined term Core Alterations. The proposed amendment would incorporate the changes reflected in TS Task Force (TSTF) Travelers 471-T (TSTF-471-T), "Eliminate use of term CORE ALTERATIONS in ACTIONS and Notes," and TSTF-51-A, "Revise containment requirements during handling irradiated fuel and core alterations." In addition, the proposed amendment would revise TS 3.9.2, "Nuclear Instrumentation," by replacing "Core Alterations" with "positive reactivity additions" in the Required Action for an inoperable source range monitor during refueling operations. The limiting conditions for operation in TS 3.9.4, "Shutdown Cooling (SDC) and Coolant Recirculation - High Water Level," would also be revised by replacing "core alterations" with "movement of fuel assemblies within containment."

The May 12, 2006, letter provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on July 5, 2005 (70 FR 38716).

2.0 REGULATORY EVALUATION

2.1 Background

In TS 1.1, "Definitions," the term "Core Alteration" is defined as "Core Alteration shall be the movement of any fuel, sources, or reactivity control components within the reactor vessel with the vessel head removed and fuel in the vessel. Suspension of Core Alterations shall not preclude completion of movement of a component to a safe position."

2.2 Proposed TSs Changes

In accordance with TSTF-471-T, the licensee proposed to delete: (a) the definition of Core Alteration from TS 1.1 and (b) the Required Actions to suspend core alterations from the Limiting Conditions for Operation (LCOs) for TS 3.8.2, "AC Sources-Shutdown," TS 3.8.5, "DC Sources-Shutdown," TS 3.8.8, "Inverters-Shutdown," TS 3.8.10, "Distribution Systems-Shutdown," TS 3.9.1, "Boron Concentration," TS 3.9.4, "Shutdown Cooling and Coolant Circulation-High Water Level," and TS 3.9.6, "Refueling Pool Water Level." In addition, the Required Action A.1 for TS 3.9.2, "Nuclear Instrumentation," is proposed to be modified by replacing Core Alteration with "positive reactivity additions," and Note 2.b in LCO for TS 3.9.4, "Shutdown Cooling and Coolant Circulation-High Water Level," would replace Core Alterations with "Movement of fuel assemblies within Containment."

The licensee stated that the deletion of the required Action to suspend Core Alterations has no effect on the initial conditions or mitigation of any design-basis accident (DBA) or transient. These requirements impose an operational burden with no corresponding safety benefit. The licensee's application is the initial request for approval (lead plant application) for TSTF-471-T from the industry. When approved, a TSTF becomes available for adoption by other licensees in applications for changes to their plant TSs.

TSTF-51 is an industry initiated method, which was accepted by the Nuclear Regulatory Commission (NRC) staff on November 1, 1999, for allowing some engineered safety feature systems and components to be non-operable when moving irradiated fuel, subject to a defined decay period, and acceptable shutdown administrative controls. TSTF-51-A eliminated the use of the term Core Alterations from certain Applicability Statements and Required Actions. On April 7, 2003, the NRC issued Amendment Nos. 257 and 234 for Calvert Cliffs Unit Nos. 1 and 2, respectively, which revised TS 3.7.11, "Spent Fuel Pool Exhaust Ventilation System," to limit the types of fuel assemblies to which it applies. These amendments in part changed the applicability from "movement of irradiated fuel assemblies" to "movement of recently irradiated fuel assemblies." These TS changes were consistent with changes that were previously accepted in TSTF-51-A. In its June 7, 2005, application, the licensee is now proposing to adopt portions of TSTF-51-A that were not adopted in Amendment Nos. 257 and 234. In this regard, the licensee proposed to (a) remove "During Core Alterations" from the Applicability and LCO Required Action A.2.1 from TS 3.3.7, "Containment Radiation Signal (CRS)," and (b) remove the Applicability and LCO Required Action from TS 3.9.3, "Containment Penetrations."

2.3 Regulatory Requirements

The Commission's regulatory requirements related to the contents of TSs are set forth in Section 50.36 of Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.36), which assures the TS specified LCOs are consistent with assumed values of the initial conditions in the licensee's safety analyses. 10 CFR 50.36(c)(2)(ii)(B) requires an LCO in the TS for an operating restriction that is an initial condition of a DBA. In this regard, the LCOs specify the minimum requirements for ensuring safe operation. The TS LCOs also contain associated Actions that are prescribed to be taken should certain designated conditions exist such that the LCO requirements are not met.

General Design Criterion (GDC) 19, "Control room," of Appendix A to 10 CFR Part 50 provides requirements for maintaining a habitable control room and includes limitations on radiological dose that may be received by control room operators.

GDC 61, “Fuel storage and handling and radioactivity control,” requires that the fuel storage and handling ... systems ... shall be designed to assure adequate safety under normal and postulated accident conditions. The systems shall be designed (1) with a capability to permit appropriate periodic inspection and testing of components important to safety, (2) with suitable shielding for radiation protection, (3) with appropriate containment, confinement, and filtering systems.

GDC 62, “Prevention of criticality in fuel storage and handling,” requires that criticality be prevented by physical systems and processes.

GDC 64, “Monitoring radioactivity releases,” requires that the means shall be provided for monitoring the reactor containment atmosphere effluent discharge paths, and the plant environs for radioactivity that may be released from normal operations, including anticipated operational occurrences, and from postulated accidents.

3.0 TECHNICAL EVALUATION

3.1 Accidents and Transients

When the reactor vessel head is unbolted and removed, core alterations take place during operating Mode 6 (refueling operation). There are only two accidents considered during Mode 6. These are: (1) a fuel-handling accident (FHA), and (2) a boron dilution accident. An FHA is initiated by the dropping of an irradiated fuel assembly, either in the containment or in the auxiliary building. There are no mitigation actions, except for taking credit for ventilation systems to reduce the dose consequences. Thus, the suspension of core alterations, except for suspension of movement of irradiated fuel, will not prevent or impair the mitigation of an FHA.

The analysis for an FHA assumes that a fuel assembly is dropped during fuel handling in the containment or the spent fuel pool. Interlocks and procedural and administrative controls make such an event highly unlikely. However, if an assembly were damaged to the extent that one or more fuel rods were broken, the accumulated fission product gases and iodines in the fuel element gap would be released to the surrounding water. Release of the solid fission products in the fuel would be negligible because of the low fuel temperature during refueling, which greatly limits their diffusion.

A boron dilution accident is initiated by a dilution source that results in the boron concentration dropping below the value required to maintain the shutdown margin. TS 3.9.1, “Boron Concentration,” applies in Mode 6, and the refueling boron concentration limit is specified in the Core Operating Limits Report (COLR). This accident is mitigated by stopping the dilution.

The suspension of core alterations has no effect on the mitigation of a boron dilution accident. Also, the control rods or fuel do not affect the initial conditions of a boron dilution accident.

Thus, the FHA and boron dilution accident are not impacted by deleting “Suspend Core Alterations” from TS applicability statement.

3.2 Changes to TSs

TS 1.1, Definitions

The licensee proposed to remove the definition for Core Alterations from TS Section 1.1. Since a TS Definition has no actions or surveillance requirements, the removal of this definition to coincide with the deletion from other TS section is acceptable.

TS Applicability

Since the assumed values of the initial conditions in the licensee’s safety analyses for an FHA and boron dilution accident continue to be met, the applicable conditions wherein these accidents could occur and the required operability of the associated systems are not reduced by the deletion of “During Core Alterations” from the Applicability of TS 3.3.7 and TS 3.9.3. Therefore, the NRC staff finds these changes acceptable.

TS Required Actions

The current TS Sections 3.3.7, 3.8.2, 3.8.5, 3.8.8, and 3.8.10 require core alterations be suspended if one containment radiation monitor or one required electric power source channel or distribution system, as applicable, is inoperable. These systems and components would be used, in part, to mitigate the consequences of postulated events during shutdown, such as an FHA. The licensee has proposed to delete the Required Action to suspend core alterations from each of these TS sections.

Except for the actions to suspend the movement of irradiated fuel and/or to suspend operations involving positive reactivity additions, suspending core alterations does not affect the initiation or mitigation of the postulated FHA. Since the actions to suspend the movement of irradiated fuel and to suspend operations involving positive reactivity additions will remain in the TS Actions, the NRC staff finds the deletion of core alteration acceptable.

TS 3.9.1 - Boron Concentration

The current TS 3.9.1 requires Core Alterations be suspended if the required boron concentration is not maintained within the limit specified in the COLR. The boron concentration limit during refueling operations assures that the reactor remains subcritical during Mode 6.

The term Core Alteration is not included in the Applicability statement for this TS. However, if core alterations are being performed during Mode 6, these operations must be suspended if the required boron concentration is not maintained. Also, the TS Required Actions still require positive reactivity additions be suspended if boron concentration is not within limit. Since this action provides reasonable assurance that an accidental criticality will be avoided, the NRC staff finds proposed deletion of the action to suspend core alterations acceptable.

TS 3.9.2 - Nuclear Instrumentation

TS 3.9.2 requires that Core Alterations be suspended if the required source range nuclear instrumentation is determined to be inoperable. The source range monitors (SRMs) are used during refueling operations to monitor the core reactivity conditions. The SRMs provide a signal to the operators of unexpected changes in core reactivity such as by a boron dilution accident or an improperly loaded fuel assembly. These detectors are located external to the reactor vessel and detect neutrons leaking from the core. Since these instruments are the only direct means of monitoring core reactivity conditions, positive reactivity additions must be suspended immediately if the SRMs are inoperable, to preclude an accidental criticality.

The licensee has proposed to replace the Required Action to “Suspend Core Alterations” from this TS with the Action to “Suspend Positive Reactivity Additions,” when required SRM instrumentation is inoperable. Since the requirement provides reasonable assurance that an accidental criticality will be avoided, the NRC staff finds the change acceptable.

TS 3.9.4 - SDC and Coolant Circulation - High Water Level

The purposes of the SDC system in Mode 6 are to remove decay heat and other residual heat, to provide mixing of borated coolant, to provide sufficient circulation to minimize the effects of a boron dilution accident, and to prevent boron stratification. TS LCO 3.9.4 requires that one SDC loop be OPERABLE and in operation in Mode 6 with the water level greater than 23 feet above the top of the irradiated fuel.

The licensee proposed to revise Note 2b, which currently states that “Core Alterations are suspended,” by replacing it with “Movement of fuel assemblies within containment is suspended.”

Note 2 states that the SDC pumps may be removed from operation during the time required for local leak rate testing of the containment penetration or to permit maintenance on the valves located in the common SDC suction line. The wording in 2.b to suspend core alterations is used as one of the conditions for the removal of the pumps from operation. This is a plant-specific note, which is based on plant design, to allow work to be performed on a common system during Mode 6. The Note has no effect on initial conditions or mitigation of any DBA or transient. The revised Note condition 2.b, “Movement of fuel assemblies within containment is suspended,” provides reasonable assurance that an accidental criticality will be avoided and is acceptable to the NRC staff.

TS 3.9.6 - Refueling Pool Water Level

The current TS 3.9.6 requires core alteration be suspended if the refueling pool water level is not maintained within its limit. Sufficient water is necessary to retain iodine fission product activity in the water in the event of an FHA. The refueling pool water level is credited in the safety analysis for an FHA while moving irradiated fuel assemblies. It is not credited for other situations involving core alterations. This requirement imposes an administrative burden on the operators, who have to verify that the water level meets the LCO requirement. The administrative burden of tracking water levels and responding to a change in the water level during Core Alterations has no benefit in the safety analyses, and other controls are in place for safe operation.

Since the requirement to suspend movement of irradiated fuel assemblies within containment will remain, the removal of core alterations does not affect the initiation or mitigation of an FHA. Therefore, the NRC staff finds this acceptable.

Summary

The NRC staff has reviewed the proposed TS changes and finds that elimination of the term Core Alterations from TSs will facilitate the refueling operations during Mode 6. It will provide operational flexibility to operators during Core Alterations activities. Since the requirements to suspend the movement of irradiated fuel assemblies within the containment will remain, the TS Action item, “Suspend Core Alterations,” has no effect on the initial conditions or mitigation of any design accident or transient, and the licensee will eliminate the Action item from these TSs.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (70 FR 38716). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Desai

Date: September 21, 2006