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Omaha NE 68102-2247

August 18, 2006
LIC-06-0077

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

- References:
1. Docket No. 50-285
 2. Letter from OPPD (R. T. Ridenoure) to NRC (Document Control Desk) dated October 11, 2004, "Fort Calhoun Station Unit No. 1 Relief Request: Alternative Test Requirements for Containment Repairs" (LIC-04-0093)
 3. Letter from NRC (D. S. Collins) to OPPD (R. T. Ridenoure) dated October 3, 2005, "Fort Calhoun Station Unit No. 1 Relief Request: Alternative Test Requirements for Containment Repairs" (TAC No. MC4653) (NRC-05-0120)

SUBJECT: Corrections to Fort Calhoun Station Unit No. 1 Relief Request: Alternative Test Requirements for Containment Repairs (TAC No. MC4653)

In Reference 2, the Omaha Public Power District (OPPD) requested approval for alternative testing to demonstrate the leak-tight integrity of a required repair resulting from cutting an access opening through the post tensioned containment concrete and metallic liner. The access opening is necessary for the replacement of the steam generators, pressurizer and reactor vessel head during the Fall 2006 outage. In Reference 3, the Nuclear Regulatory Commission (NRC) approved the relief request.

The purpose of this letter is to correct two administrative errors in Reference 2. First, Reference 2 incorrectly stated that the applicable code edition and addenda is American Society of Mechanical Engineers (ASME) Section XI 1998 edition with 2000 Addenda. The correct reference is ASME Section XI 1992 edition with the 1992 Addenda, which is the applicable code of record. Second, due to a typographical error, Reference 2 cited IWL-5500, which does not exist. The correct reference is IWL-5000.

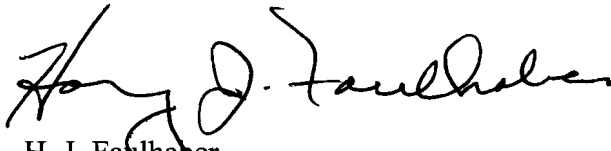
The above code edition is consistent with the Federal Register (FR) dated August 8, 1996 (61 FR 41303), where the NRC amended its regulations to incorporate by reference the 1992 edition with 1992 addenda of Subsections IWE and IWL of Section XI of the ASME Boiler and Pressure Vessel Code. Subsections IWE and IWL provide the

requirements for inservice inspection (ISI) of Class CC (Concrete Containment) and Class MC (metallic containment) of light-water cooled power plants.

In conclusion, this submittal documents the applicable code requirements, as discussed in the Safety Evaluation Report (Reference 3) approving the relief request.

No commitments are made to the NRC in this letter. If you have any questions or require additional information, please contact Mr. Thomas C. Matthews at (402) 533-6938.

Sincerely,

A handwritten signature in black ink, appearing to read "H. J. Faulhaber", written in a cursive style.

H. J. Faulhaber
Division Manager
Nuclear Engineering Division

HJF/rlj