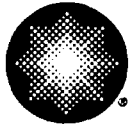


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Constellation Energy

Nine Mile Point Nuclear Station

August 10, 2006

U. S. Nuclear Regulatory Commission
Washington, DC 20555-0001

ATTENTION: Document Control Desk

SUBJECT: Nine Mile Point Nuclear Station
Unit No. 1; Docket No. 50-220

Licensee Event Report 06-001, "Technical Specification Required Shutdown due to Increased Drywell Leakage"

In accordance with 10 CFR 50.73(a)(2)(i)(A), we are submitting Licensee Event Report 06-001, "Technical Specification Required Shutdown due to Increased Drywell Leakage."

Should you have questions regarding the information in this submittal, please contact M. H. Miller, Licensing Director, at (315) 349-1510.

Very truly yours,

JAH/RF/sac

Attachment: (1) Licensee Event Report

cc: S. J. Collins, NRC Regional Administrator, Region I
L. M. Cline, NRC Senior Resident Inspector

IE22

ATTACHMENT (1)

LICENSEE EVENT REPORT (LER)

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by Internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Nine Mile Point Unit 1	2. DOCKET NUMBER 05000220	3. PAGE 1 OF 3
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4. TITLE Technical Specification Required Shutdown due to Increased Drywell Leakage
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5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
06	11	2006	2006	- 001 -	00	08	10	2006	FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE N	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)									
10. POWER LEVEL 45	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)						
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER						
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A							

12. LICENSEE CONTACT FOR THIS LER

NAME Mary H. Miller, Licensing Director	TELEPHONE NUMBER (Include Area Code) (315) 349-1510
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
D	AD	ISV	R344	Y					

14. SUPPLEMENTAL REPORT EXPECTED

☐ YES (If yes, complete 15. EXPECTED SUBMISSION DATE) ☒ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On June 10, 2006, Nine Mile Point Unit 1 (NMP1) commenced a planned downpower to perform a drywell entry to determine the cause of increased drywell leakage. Packing on a Reactor Coolant Recirculation System (RCS) pump drain valve was found leaking. The pump suction and discharge valves were then closed. The normally closed drain valve was backseated, which reduced the leak however, a leak in a weld downstream of the valve was discovered. The drain valve was subsequently closed to isolate the weld leak, and NMP1 began power ascension. On June 11, 2006, at 0126 hours NMP1 entered Technical Specification (TS) 3.2.5, "Reactor Coolant System Leakage," required shutdown due to an increase in leakage exceeding 2 gallons per minute in a 24 hour period. NMP1 was placed in cold shutdown mode at 1417 hours on June 11, 2006.

The source of the increased leakage was determined to be the drain valve packing. The cause of the packing leak was installation of incorrect packing in March 1997. The packing that was installed did not have the same diameter as the inside diameter of the stuffing box. Adequate performance standards for valve packing installation did not exist in the early 1990s. The NMP1 valve packing program was initially issued in November of 1998.

During the shutdown, NMP1 replaced the packing in the leaking RCS pump drain valve and repaired the weld leak. Extent of condition review identified the need for packing replacement of the redundant valve, which was entered in the corrective action process. The current NMP1 valve packing program provides adequate performance standards for packing installation.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET (2) NUMBER (2)	LER NUMBER (6)			PAGE (3)		
Nine Mile Point Unit 1	05000220	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3		
		2006	-- 001	-- 00			

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event

On June 10, 2006, Nine Mile Point Unit 1 (NMP1) commenced a downpower to 12% power to perform a drywell entry to determine the cause of increased drywell leakage that began in January 2006. Packing on a Reactor Coolant Recirculation System (RCS) pump drain valve was found leaking. The drain valve is one of two normally closed drain valves on a line that is capped. The pump suction and discharge valves were then closed. The leaking drain valve was placed in a back seat position in an effort to stop the leak. Placing the drain valve in a back seat position reduced the leak rate, but a pin hole leak was discovered in a weld at the RCS piping to pump drain valve weld, on the downstream side of the valve. The drain valve was then closed to isolate the weld leak and permit power ascension, which began at 1420 on June 10, 2006, with a plan to perform necessary repairs in a near term planned outage. On June 11, 2006, at 0126 hours (NMP1) entered Technical Specification (TS) 3.2.5, "Reactor Coolant System Leakage," required shutdown due to an increase in leakage exceeding 2 gallons per minute in a 24 hour period. NMP1 was placed in cold shutdown mode at 1417 hours on June 11, 2006.

II. Cause of Event

The source of the increased leakage was determined to be the drain valve packing. The cause of the packing leak was installation of incorrect packing in March 1997. The packing that was installed did not have the same diameter as the inside diameter of the stuffing box. Adequate performance standards for valve packing installation did not exist in the early 1990s. The NMP1 valve packing program was initially issued in November of 1998.

A contributing cause was a leak at the RCS piping to pump drain valve weld, on the downstream side of the valve. The cause of the leaking weld was determined to be outer diameter or inner diameter initiated mechanical fatigue that propagated from an original defect at the root of the weld.

III. Analysis of Event

This event is reportable in accordance with 10 CFR 50.73(a)(2)(i)(A), "The completion of any nuclear plant shutdown required by the plant's Technical Specifications..."

There were no direct safety consequences associated with this event. There were no structures, systems or components that were inoperable that contributed to this event. All available plant safety systems operated as designed and the operators effectively stabilized reactor parameters. Based on the NMP1 probabilistic risk assessment, the risk increase was not significant.

Based on the above, the event did not pose a threat to the health and safety of the public or plant personnel.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET (2) NUMBER (2)	LER NUMBER (6)			PAGE (3)
Nine Mile Point Unit 1	05000220	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
		2006	-- 001 --	00	

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

IV. Corrective Actions

The RCS pump drain valve packing was replaced and the leaking weld downstream of the RCS pump drain valve was repaired. Replacement of incorrect packing on the redundant downstream valve, which was identified through extent of condition review, was entered in the corrective action program. The current NMP1 valve packing program provides adequate performance standards for packing installation.

V. Additional Information

A. Failed Components:

Reactor Coolant Recirculation System pump drain valve.

B. Previous similar events:

Two similar events occurred in 1997 where lack of a valve packing program resulted in incorrect packing installation and caused shutdowns due to increased drywell leakage. The NMP1 valve packing program was established in November 1998. The failed valve in this event was in scope for replacement during the refueling outage in 2005, but was chosen not to be replaced because it was assessed as a low risk due to its function, infrequent use and absence of known seat leakage.

C. Identification of component referred to in this Licensee Event Report:

<u>Component</u>	<u>IEEE 805 System ID</u>	<u>IEEE 803.A Function</u>
Pump Drain Valve	AD	ISV
Pump Discharge Valve	AD	ISV
Pump Suction Valve	AD	ISV