

August 15, 2006

Mr. D. E. Grissette  
Vice President  
Southern Nuclear Operating  
Company, Inc.  
Post Office Box 1295  
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1 AND VOGTLE ELECTRIC  
GENERATING PLANT, UNITS 1 AND 2, REQUEST FOR ADDITIONAL  
INFORMATION FOR RELIEF REQUEST NO. ISI-GEN-ALT-06-02  
(TAC NOS. MD2482, MD2483 and MD2484)

Dear Mr. Grissette:

By letter dated June 29, 2006, Southern Nuclear Company (SNC) submitted Relief Request No. ISI-GEN-ALT-06-02 proposing an alternative to certain requirements of the American Society of Mechanical Engineers, Boiler and Pressure Vessel Code (ASME Code) at Joseph M. Farley Nuclear Plant, Unit 1 and Vogtle Electric Generating Plant, Units 1 and 2. Specifically, SNC proposed using ASME Code Case N-696, "Qualification Requirements for Appendix VIII Piping Examinations Conducted From the Inside Surface."

The Nuclear Regulatory Commission staff has reviewed that response and has identified a need for additional information as stated in the enclosure. We request your most expeditious response to these issues to support continuation of our review.

Sincerely,

**/RA/**

Robert E. Martin, Senior Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-348, 50-424, and 50-425

Enclosure: Request for Additional Information

cc w/encl: See next page

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DATE	08/11/06	08/11/06	08/10/06	08/10/06

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REQUEST FOR ADDITIONAL INFORMATION  
RELATED TO RELIEF REQUEST NO. ISI-GEN-ALT-06-02  
FOR RENEWED FACILITY OPERATING LICENSE NO. NPF-2  
DOCKET NO. 50-348  
AND FACILITY OPERATING LICENSE NO. NPF-68  
DOCKET NO. 50-424  
AND FACILITY OPERATING LICENSE NO. NPF-81  
DOCKET NO. 50-425

The licensee requested to use ASME Code Case N-696 (N-696) for the examination of selected dissimilar metal welds. N-696, Paragraph 3.3(d) states that Supplement 2 or Supplement 3 procedures, equipment, and personnel are qualified for depth-sizing when the flaw depths estimated by ultrasonics, as compared with the true depths, do not exceed 0.125 in. (3mm) RMS [root mean square], when they are combined with a successful Supplement 10 qualification. The proposed alternative is specifically for the use of N-696. In the section identified as the "Basis for Use," there is an implied alternative for handling performance demonstrated depth-sizing values that are greater than 0.125 in. RMS.

In the submittal section "Basis for Use," a discussion on depth sizing values greater than 0.125 in. RMS was presented. The purpose of the discussion is not clear because it is suggesting another alternative separate from the submittal section "Proposed Alternative." If a depth sizing value greater than 0.125 in. is part of the proposed alternative, provide the alternative depth-sizing value. The basis for the acceptability of an RMS value greater than the ASME Code required value is that any required flaw evaluation adds to the measured flaw size, the difference between the ASME Code 0.125" RMS performance demonstration acceptance criterion and the larger RMS value from the vendor's procedure performance demonstration.

The staff considers the licensee's approach to address the current inability to meet the ASME Code-required UT depth-sizing RMS requirement as a short-term solution. Vendors continue to make improvements to their UT sizing capabilities. These improvements may reduce the difference between the depth-sizing RMS Code-requirement and currently achievable RMS values. Discuss options Farley and Vogtle have considered for the examination of the subject welds. Explain why the ASME Code RMS value is not achievable for the selected vendor.

Joseph M. Farley Nuclear Plant, Unit 1

cc:

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Vogtle Electric Generating Plant, Units 1 & 2

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