

August 15, 2006

Mr. Randall K. Edington
Vice President-Nuclear and CNO
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION
RE: REQUEST FOR A ONE-TIME EXTENSION OF CONTAINMENT
INTEGRATED LEAKAGE RATE TEST INTERVAL (MC9732)

Dear Mr. Edington:

The Nuclear Regulatory Commission (NRC) staff has reviewed the information provided in the January 30, 2006, submittal and has determined that the additional information identified in this enclosure is required in order for the NRC staff to complete its review. To meet our target date, the NRC staff requests that the licensee provide its response no later than August 20, 2006.

Sincerely,

/RA/

Brian Benney, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: Request for Additional Information

cc w/encl: See next page

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NRR-106

OFFICE	LPL4/PM	LPL4/LA	LPL4/BC
NAME	BBenney	LFeizollahi	DTerao
DATE	0/10/06	08/10/06	08/15/06

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REQUEST FOR ADDITIONAL INFORMATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
REQUEST FOR A ONE-TIME EXTENSION OF
CONTAINMENT INTEGRATED LEAKAGE RATE TEST INTERVAL
FACILITY OPERATING LICENSE NO. DPR-46
NEBRASKA PUBLIC POWER DISTRICT
COOPER NUCLEAR STATION
DOCKET NO. 50-298

1. IWE-1240 requires licensees to identify the containment surface areas requiring augmented examinations. Provide the locations of the steel containment surfaces that have been identified as requiring augmented examination and a summary of the findings of the examinations performed.
2. For the examination of penetration seals and gaskets, and examination and testing of bolted connections associated with the primary containment pressure boundary, the licensee requested relief from the requirements of the code. As an alternative, the licensee proposed to examine the above items during the leak-rate testing of the primary containment. Option B of Appendix J for Type B and Type C testing (per Nuclear Energy Institute (NEI) 94-01 and Regulatory Guide (RG) 1.163), and the integrated leakage rate test extension requested in this amendment for Type A testing, provide flexibility in the scheduling of these inspections. Discuss your schedule for examination and testing of seals, gaskets, and bolted connections that provide assurance regarding the integrity of the containment pressure boundary.
3. Inspections of some steel containment structures have identified degradation of uninspectable areas (gap between the shield wall and drywell). These degradations cannot be found by visual (i.e., VT-1 or VT-3) examinations unless they are through the thickness of the shell or when 100 percent of the uninspectable surfaces are periodically examined by ultrasonic testing. Discuss what approach is used in the Cooper Nuclear Station (CNS) to identify degradation of uninspectable areas of the containment.
4. In response to Generic Letter 87-05, CNS committed to implement new surveillance procedures for inspection of sand cushion drains during refueling outages. Discuss the results obtained from the inspection.

Cooper Nuclear Station

cc:

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