

July 31, 2006

Mr. Michael Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601-1839

SUBJECT: REQUESTS FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
PILGRIM NUCLEAR POWER STATION LICENSE RENEWAL APPLICATION
(TAC MC9669)

Dear Mr. Kansler:

By letter dated January 25, 2006, Entergy Nuclear Operations, Inc. submitted an application pursuant to 10 CFR Part 54, to renew the operating license for Pilgrim Nuclear Power Station for review by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, areas where additional information is needed to complete the review. These requests for additional information address license renewal scoping and screening aspects for reactor coolant system (Section 2.3.1), engineered safety features (Section 2.3.2), and auxiliary systems (Section 2.3.3).

These questions were discussed with a member of your staff, Bryan Ford, and a mutually agreeable date for this response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-1478 or e-mail RXS2@nrc.gov.

Sincerely,

/RA/

Ram Subbaratnam, Project Manager
License Renewal Branch A
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure:
Requests for Additional Information

cc w/encl: See next page

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NAME	RSubbaratnam	IKing	LLund
DATE	7/ 26 /06	7/ 21 /06	7/ 28 /06

OFFICIAL RECORD COPY

Letter to Michael Kansler from Ram Subbaratnam dated July 31, 2006

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PILGRIM NUCLEAR POWER STATION LICENSE RENEWAL APPLICATION
(TAC MC9669)

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REQUESTS FOR ADDITIONAL INFORMATION (RAIs)
PILGRIM NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION SECTIONS 2.3.1, 2.3.2 and 2.3.3

2.3.1.1 Reactor Vessel

RAI 2.3.1.1-1

In Table 2.3.1-1 of the license renewal application (LRA), the reactor vessel leakage monitoring piping was not identified as a component within scope requiring an aging management review (AMR). The staff requests the applicant to identify the subject components within scope, because it is considered as part of the pressure boundary, and accordingly, it should be within the scope of license renewal and subject to AMR. If, however, the applicant believes that the components do not require an AMR, then they should provide plant-specific justification based on the description of the subject components, or any other relevant information, as to why the components need not be subjected to an AMR.

RAI 2.3.1.1-2

The staff believes that the scram discharge piping and volume should be in scope requiring aging management. However, it appears that the subject component was not identified in Table 2.3.1-1 of the LRA. Please justify.

RAI 2.3.1.1-3

The staff understands that the CRD Housing Supports (CRDHS) limit the travel of a control rod in the event that a control rod housing is ruptured. The supports prevent a nuclear excursion as a result of a housing failure, thus protecting the fuel barrier, and limiting radioactive releases. In addition, following a postulated failure of the drive housing at the attachment weld at the same time the control rod is withdrawn, and if the collet were to stay unlatched, the housing would separate from the vessel, and the drive and housing would be blown downward against the CRDHS. If credit is taken for the CRDHS; and since, the CRDHS are passive and long-lived, the staff believes that the subject components should be within the scope of license renewal requiring aging management. It appears, however, that the subject components and their intended function of limiting travel of the control rod following control rod housing rupture have not been identified in Table 2.3.1-1 of the LRA. Therefore, the staff requests the applicant to provide an explanation.

2.3.1.2 Reactor Vessel Internals (RVI)

RAI 2.3.1.2-1

In Section 2.3.1.2, "Reactor Vessel Internals," it was stated that RVI include local power range monitors. The staff understands that the neutron monitoring system (NMS) includes additional neutron monitors, such as, intermediate range monitors, rod block monitors, etc.; and these monitoring circuits, along with their electrical cables should be within scope of license renewal requiring aging management. The staff also noted that in Table 2.2-1b of the LRA, it was

Enclosure

indicated that a bounding approach was used for the NMS. The staff requests the applicant to clarify which neutron monitors and the related cables are considered within scope based on the bounding approach.

2.3.2.1 Residual Heat Removal System

RAI 2.3.2.1-1

The low pressure coolant injection coupling was identified in the Boiling Water Reactor Vessel and Internals Project (BWRVIP)-06 report as a safety-related component. It appears, however, that the component was not identified in Table 2.3.2-1 of the LRA requiring an AMR. If the component exists at Pilgrim Nuclear Power Station, then the staff requests the applicant to justify its exclusion from aging management; otherwise, submit an AMR for the subject component.

RAI 2.3.2.1-2

Please clarify whether the passive components, namely, vortex breakers used in pump suction lines, which could be located inside the emergency core cooling system tanks or in the sump, and whose intended functions are to protect the pumps from cavitation, are subject to an AMR. If so, identify which of these tanks are equipped with such passive components, and where in the LRA are the AMRs for these components, or provide justifications for exclusion of these components from AMRs.

2.3.2.4 & 2.3.2.5 High Pressure Coolant Injection(HPCI) & Reactor Core Isolation Cooling (RCIC)

RAI 2.3.2.4-1

The steam supply and return lines for HPCI and RCIC perform safety functions, and therefore, should be in scope of license renewal in accordance with 10 CFR 50.4(a)(1). The staff requests the applicant to clarify whether the subject components are in scope requiring an AMR.