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Thomas A. Marlow
Director,
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2CAN070601

July 28, 2006

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: License Amendment Request
Corrected Technical Specification Page for ANO-2 Steam Generator Tube
Inservice Inspection Program
Arkansas Nuclear One, Unit 2
Docket No. 50-368
License No. NPF-6

REFERENCE: 1 Entergy letter dated June 19, 2006, *Revised Technical Specification
Pages for ANO-2 Steam Generator Tube Inservice Inspection Program*
(2CAN060602)

Dear Sir or Madam:

In Reference 1, Entergy Operations, Inc. (Entergy) provided the revised Technical Specification (TS) pages for the Arkansas Nuclear One, Unit 2 (ANO-2) Operating License (OL) amendment associated with the Steam Generator Tube Inservice Inspection Program. However, it was discovered that one of the TS pages (page 6-22) did not reflect the changes contained in OL Amendment 262 which was received after the original Steam Generator Tube Inservice Inspection Program OL application. Entergy is providing the corrected TS page in the attachment to this letter.

There are no new commitments associated with this letter. If you have any questions or require additional information, please contact Steve Bennett at 479-858-4626.

I declare under penalty of perjury that the foregoing is true and correct. Executed on July 28, 2006.

Sincerely,

Thomas A. Marlow

TAM/sab

A047

Attachment: Corrected Technical Specification Page 6-22 for the ANO-2 Steam Generator
Tube Inservice Inspection Program

cc: Dr. Bruce S. Mallett
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
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U. S. Nuclear Regulatory Commission
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Mr. Bernard Bevill
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Attachment

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**Corrected Technical Specification Page 6-22 for the ANO-2 Steam Generator Tube
Inservice Inspection Program**

ADMINISTRATIVE CONTROLS

6.6.6 Containment Inspection Report

Any degradation exceeding the acceptance criteria of the containment structure detected during the tests required by the Containment Tendon Surveillance Program shall undergo an engineering evaluation within 60 days of the completion of the inspection surveillance. The results of the engineering evaluation shall be reported to the NRC within an additional 30 days of the time the evaluation is completed. The report shall include the cause of the condition that does not meet the acceptance criteria, the applicability of the conditions to the other unit, the acceptability of the concrete containment without repair of the item, whether or not repair or replacement is required and, if required, the extent, method, and completion date of necessary repairs, and the extent, nature, and frequency of additional examinations.

6.6.7 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into HOT SHUTDOWN following completion of an inspection performed in accordance with the Specification 6.5.9, *Steam Generator (SG) Program*. The report shall include:

- a. The scope of inspections performed on each SG,
- b. Active degradation mechanisms found,
- c. Nondestructive examination techniques utilized for each degradation mechanism,
- d. Location, orientation (if linear), and measured sizes (if available) of service induced indications,
- e. Number of tubes plugged during the inspection outage for each active degradation mechanism,
- f. Total number and percentage of tubes plugged to date,
- g. The results of condition monitoring, including the results of tube pulls and in-situ testing, and
- h. The effective plugging percentage for all plugging in each SG.

6.6.8. Specific Activity

The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.4.8. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded the results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Clean-up system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.