

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 256, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 234 to Facility Operating License DPR-33, the first performance is due at the end of the first surveillance interval that begins at implementation of the Amendment 234. For SRs that existed prior to Amendment 234, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment 234.

SURVEILLANCE REQUIREMENTS

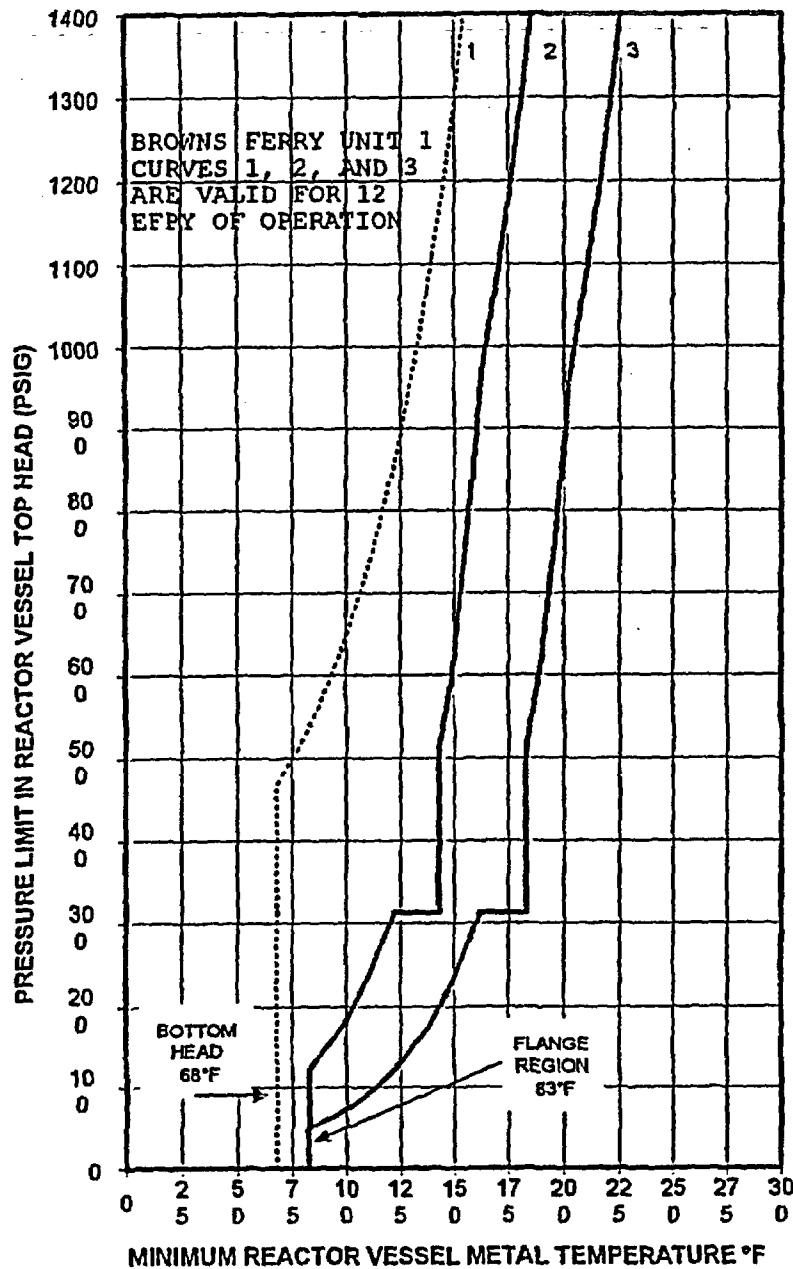
SURVEILLANCE	FREQUENCY
<p>SR 3.4.9.1</p> <p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. Only required to be performed during RCS heatup and cooldown operations or RCS inservice leak and hydrostatic testing when the vessel pressure is > 312 psig. 2. The limits of Figure 3.4.9-2 may be applied during nonnuclear heatup and ambient loss cooldown associated with inservice leak and hydrostatic testing provided that the heatup and cooldown rates are $\leq 15^{\circ}\text{F}/\text{hour}$. 3. The limits of Figures 3.4.9-1 and 3.4.9-2 do not apply when the tension from the reactor head flange bolting studs is removed. <p>-----</p> <p>Verify:</p> <ol style="list-style-type: none"> a. RCS pressure and RCS temperature are within the limits specified by Curves No. 1 and No. 2 of Figures 3.4.9-1 and 3.4.9-2; and b. RCS heatup and cooldown rates are $\leq 100^{\circ}\text{F}$ in any 1 hour period. 	<p>30 minutes</p>
<p>SR 3.4.9.2</p> <p>Verify RCS pressure and RCS temperature are within the criticality limits specified in Figure 3.4.9-1, Curve No. 3.</p>	<p>Once within 15 minutes prior to control rod withdrawal for the purpose of achieving criticality</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.9.5	<p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. Only required to be performed when tensioning the reactor vessel head bolting studs. 2. The reactor vessel head bolts may be partially tensioned (four sequences of the seating pass) provided the studs and flange materials are > 70°F. 	30 minutes
	<p>Verify reactor vessel flange and head flange temperatures are > 83°F.</p>	
SR 3.4.9.6	<p>-----NOTE-----</p> <p>Not required to be performed until 30 minutes after RCS temperature ≤ 85°F in MODE 4.</p>	30 minutes
	<p>Verify reactor vessel flange and head flange temperatures are > 83°F.</p>	
SR 3.4.9.7	<p>-----NOTE-----</p> <p>Not required to be performed until 12 hours after RCS temperature ≤ 100°F in MODE 4.</p>	12 hours
	<p>Verify reactor vessel flange and head flange temperatures are > 83°F.</p>	

Reactor Steam Dome Pressure
3.4.9



Curve No. 1
Minimum temperature for
bottom head during
mechanical heatup or
cooldown following
nuclear shutdown.

Curve No. 2
Minimum temperature for
upper RPV and beltline
during mechanical
heatup or cooldown
following nuclear
shutdown.

Curve No. 3
Minimum temperature for
core operation
(criticality).

Notes
These curves include
sufficient margin to
provide protection
against feedwater
nozzle degradation.
The curves allow for
shifts in RT_{NDT} of the
Reactor vessel beltline
materials, in
accordance with Reg.
Guide 1.99 Rev. 2 to
compensate for
radiation embrittlement
for 12 EFPY.

The acceptable area for
operation is to the
right of the applicable
curves.

Figure 3.4.9-1
Pressure/Temperature Limits for
Mechanical Heatup, Cooldown following Shutdown, and
Reactor Critical Operations

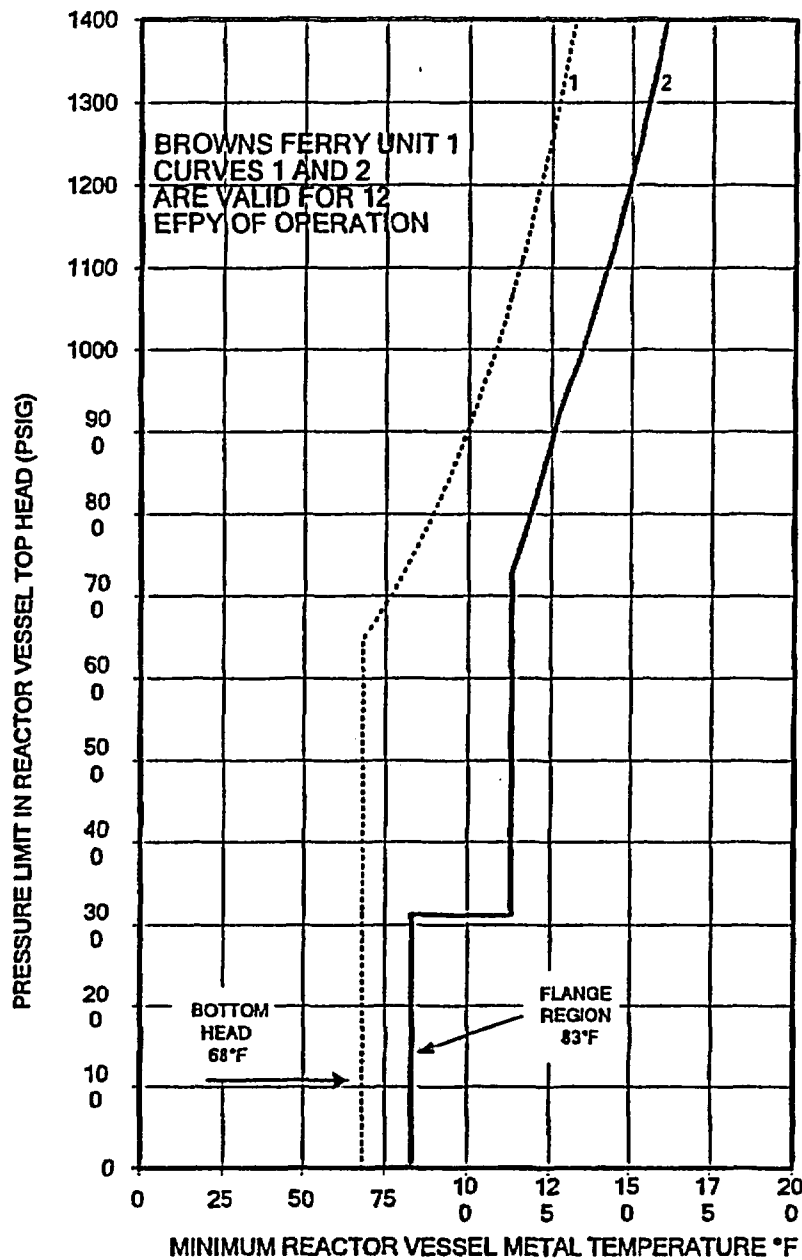
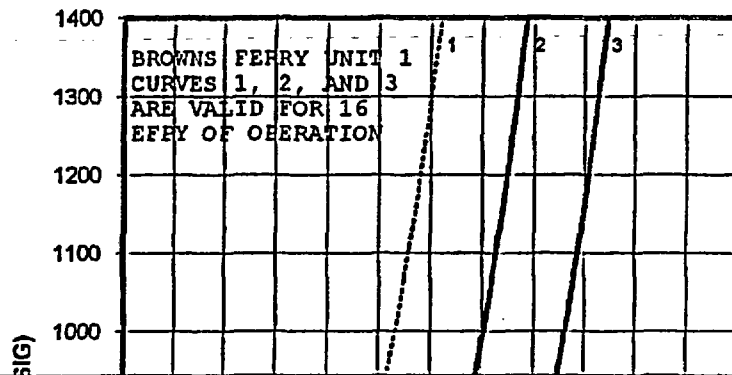


Figure 3.4.9-2
Pressure/Temperature Limits for
Reactor In-Service Leak and Hydrostatic Testing

Reactor Steam Dome Pressure
3.4.9



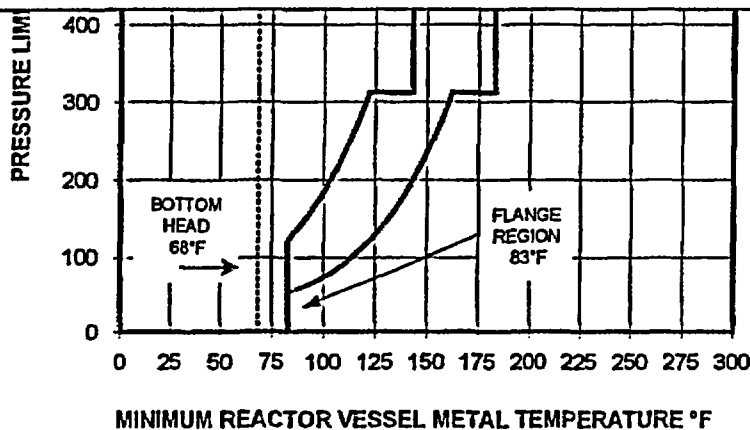
Curve No. 1
Minimum temperature for bottom head during mechanical heatup or cooldown following nuclear shutdown.

Curve No. 2
Minimum temperature for upper R/V and beltline during mechanical heatup or cooldown following nuclear

DO NOT USE THIS FIGURE

This curve applies to operations >12 EFPY.

For current operation, use previous curve, which is valid up to 12 EFPY.



Reactor vessel beltline materials, in accordance with Reg. Guide 1.99 Rev. 2 to compensate for radiation embrittlement for 16 EFPY.

The acceptable area for operation is to the right of the applicable curves.

Figure 3.4.9-1
Pressure/Temperature Limits for
Mechanical Heatup, Cooldown following Shutdown, and
Reactor Critical Operations

Reactor Steam Dome Pressure
3.4.9

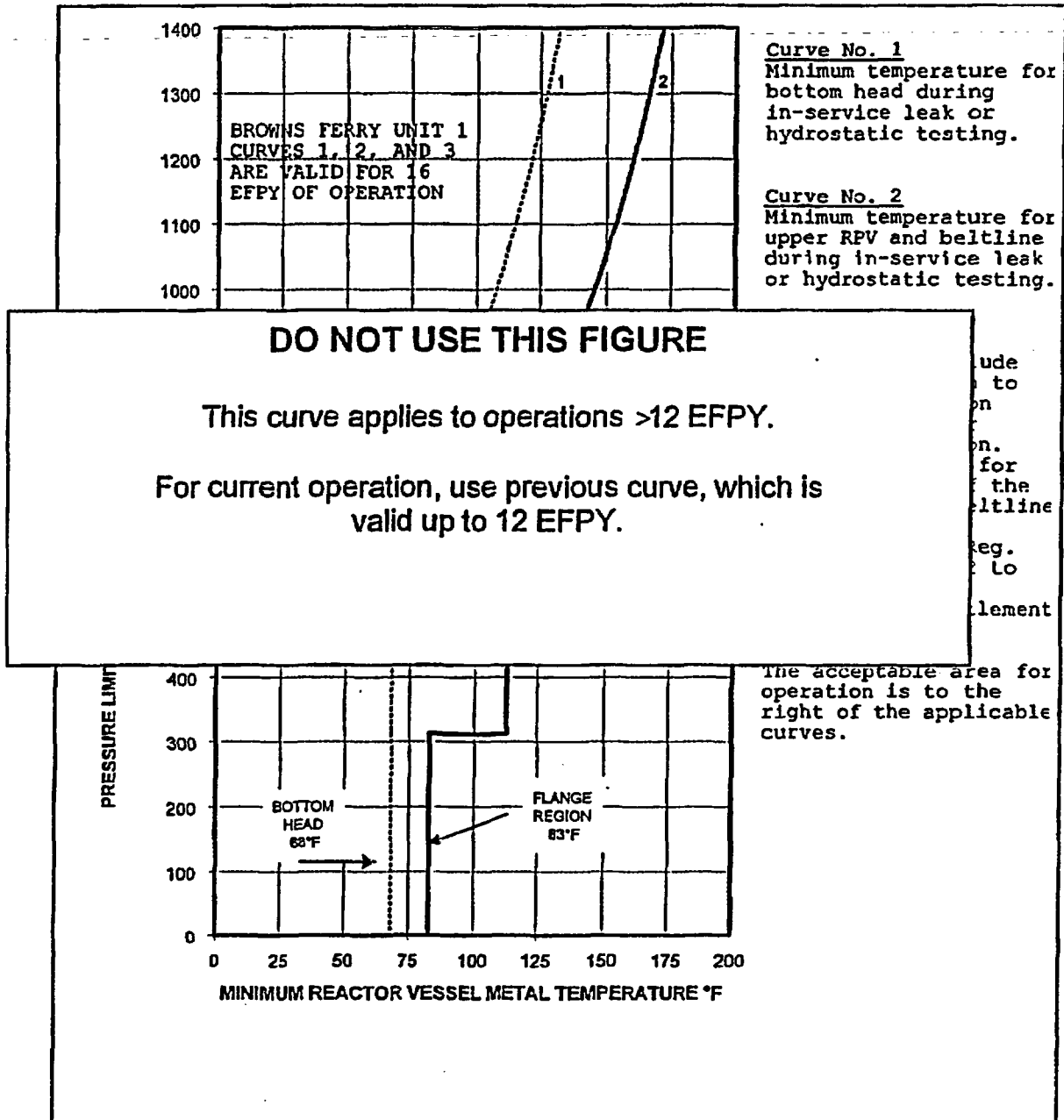


Figure 3.4.9-2
Pressure/Temperature Limits for
Reactor In-Service Leak and Hydrostatic Testing