

Facility: Arkansas Nuclear One Unit 2 RO Written Outline														Date of Exam: 07/14/2006			
Tier	Group	RO K/A Category Points											SRO – Only Points				
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A2	G*	TOTAL	
1. Emergency & Abnormal Plant Evolutions	1	3	3	3	N/A			3	3	N/A			3	18			
	2	2	1	2	N/A			1	1	N/A			2	9			
	Tier Totals	5	4	5	N/A			4	4	N/A			5	27			
2. Plant Systems	1	2	2	3	3	3	2	2	3	2	3	3	28				
	2	1	1	1	0	1	1	1	1	1	1	1	10				
	Tier Totals	3	3	4	4	4	3	3	4	2	4	4	38				
3. Generic Knowledge and Abilities Categories						1	2	3	4	10							
						3	2	2	3								
<p>Note:</p> <ol style="list-style-type: none"> Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO Outline and the SRO only outlines (i.e. except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two). The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ± 1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO –only exam must total 25 points. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution. Absent a plant specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories. * The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IR) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. Use duplicate pages for RO and SRO-only exams. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A number, descriptions, importance ratings, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43 																	

ES-401		PWR Examination Outline Emergency and Abnormal Plant Evolutions – Tier1 /Group1 (RO/SRO)						Form ES-401-2	
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
00007 (BW/E02 & E10; CE/E02) Reactor Trip – Stabilization – Recovery / 1			X				Knowledge of the reasons for the following responses as they apply to the (Reactor Trip Recovery): EK3.4 RO or SRO function within the control room team as appropriate to the assigned position, in such a way that procedures are adhered to and the limitations in the facilities license and amendments are not violated	3.2	1
00008 Pressurizer Vapor Space Accident / 3					X		Ability to determine and interpret the following as they apply to the Pressurizer Vapor Space Accident: AA2.09 PZR spray block valve controls and indicators	3.6	2
000009 Small Break LOCA / 3		X					Knowledge of the interrelations between the small break LOCA and the following: EK2.03 S/Gs	3.0	3
000011 Large Break LOCA / 3					X		Ability to determine and interpret the following as they apply to a Large Break LOCA: EA2.02 Consequences to RHR of not resetting safety injection	3.3*	4
000015/17 RCP Malfunctions / 4		X					Knowledge of the interrelations between the Reactor Coolant Pump Malfunctions and the following: AK2.08 CCWS	2.6	5
000022 Loss of Rx Coolant Makeup / 2	X						Knowledge of the operational implications of the following concepts as they apply to Loss of Reactor Coolant Pump Makeup: AK1.03 Relationship between charging flow and PZR level.	3.0	6
000025 Loss of RHR System / 4	X						Knowledge of the operational implications of the following concepts as they apply to Loss of Residual Heat Removal System: AK1.01 Loss of RHRS during all modes of operation	3.9	7
000026 Loss of Component Cooling Water / 8						X	Generic: 2.4.6 Knowledge symptom based EOP mitigation strategies.	3.1	8
000027 Pressurizer Pressure Control System Malfunction / 3						X	Generic: 2.4.11 Knowledge of abnormal condition procedures.	3.4	9
000029 ATWS / 1				X			Ability to operate and/or monitor the following as they apply to a ATWS: EA1.01 Charging pumps	3.4*	10
000038 Steam Gen. Tube Rupture / 3							Not Selected		
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4	X						040 Knowledge of the operational implications of the following concepts as they apply to Steam Line Rupture: AK1.04 Nil ductility temperature	3.2	11
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4				X			CE/E05 Ability to operate and/or monitor the following as they apply to the (Excess Steam Demand): EA1.1 Components, and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes, and automatic and manual features.	3.9	12

000054 (CE/E06) Loss of Main Feedwater / 4					X	054 Ability to determine and interpret the following as they apply to the Loss of Main Feedwater (MFW): AA2.01 Occurrence of reactor and/or turbine trip	4.3	13
000054 (CE/E06) Loss of Main Feedwater / 4		X				CE/E06 Knowledge of the interrelations between the (Loss of Feedwater) and the following: EK2.2 Facility's heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems, and relations between the proper operation of these systems to the operation of the facility.	3.5	14
000055 Station Blackout / 6						Not Selected		
000056 Loss of Off-site Power / 6			X			Knowledge of the reasons for the following responses as they apply to the Loss of Offsite Power: AK3.01 Order and time to initiation of power for the load sequencer	3.5	15
000057 Loss of Vital AC Inst. Bus / 6			X			Knowledge of the reasons for the following responses as they apply to the Loss of Vital AC Instrument Bus: AK3.01 Actions contained in EOP for loss of vital ac electrical instrument bus.	4.1	16
000058 Loss of DC Power / 6				X		Ability to operate and/or monitor the following as they apply to the Loss of DC Power: AA1.02 Static inverter dc input breaker, frequency meter, ac output breaker, and ground fault detector	3.1*	17
000062 Loss of Nuclear Svc Water / 4					X	Generic 2.1.28 Knowledge of the purpose and function of major system components and controls.	3.2	18
000065 Loss of Instrument Air / 8						Not selected		
W/E04 LOCA Outside Containment / 3						Not applicable to this Unit.		
W/E11 Loss of Emergency Coolant Recirc. / 4						Not applicable to this Unit.		
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4						Not applicable to this Unit.		
K/A Category Totals:	3	3	3	3	3	Group Point Total:		18

ES-401		PWR Examination Outline						Form ES-401-2	
		Emergency and Abnormal Plant Evolutions – Tier1 /Group2 (RO/SRO)							
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1		X					Knowledge of the interrelations between the Continuous Rod Withdrawal and the following: AK2.06 T-ave./ref. deviation meter	3.0*	19
000003 Dropped Control Rod / 1							Not Selected.		
000005 Inoperable/Stuck Control Rod / 1				X			Knowledge of the interrelations between the Inoperable/Stuck Control Rod and the following: AA1.05 RPI	3.4	20
000024 Emergency Boration / 1							Not Selected.		
000028 Pressurizer Level Malfunction / 2							Not Selected.		
000032 Loss of Source Range NI / 7							Not Selected.		
000033 Loss of Intermediate Range NI / 7							Not applicable to this Unit.		
000036 (BW/A08) Fuel Handling Accident / 8			X				Knowledge of the reasons for the following responses as they apply to the Fuel Handling Incidents: AK3.03 Guidance contained in EOP for fuel handling incident.	3.7	21
000037 Steam Generator Tube Leak / 3						X	Generic 2.2.22 Knowledge of limiting conditions for operations and safety limits.	3.4	22
000051 Loss of Condenser Vacuum / 4						X	Generic 2.1.32 Ability to explain and apply all system limits and precautions.	3.4	23
000059 Accidental Liquid RadWaste Rel. / 9							Not Selected.		
000060 Accidental Gaseous Radwaste Rel. / 9							Not Selected.		
000061 ARM System Alarms / 7	X						Knowledge of the operational implications of the following concepts as they apply to Area Radiation Monitoring (ARM) AK1.01 Detector limitations	2.5*	24
000067 Plant Fire On-site / 8							Not Selected.		
000068 (BW/A06) Control Room Evac. / 8							Not Selected.		
000069 (W/E14) Loss of CTMT Integrity / 5							Not Selected.		
000074 (W/E06&E07) Inad. Core Cooling / 4							Not Selected.		
000076 High Reactor Coolant Activity / 9					X		Ability to determine and interpret the following as they apply to the High Reactor Coolant Activity: AA2.02 Corrective actions required for high fission product activity in RCS	2.8	25
W/E01 & E02 Rediagnosis & SI Termination / 3							Not applicable to this Unit.		
W/E13 Steam Generator Over-pressure / 4							Not applicable to this Unit.		
W/E15 Containment Flooding / 5							Not applicable to this Unit.		

W/E16 High Containment Radiation / 9							Not applicable to this Unit.		
BW/A01 Plant Runback / 1							Not applicable to this Unit.		
BW/A02&A03 Loss of NNI-X/Y / 7							Not applicable to this Unit.		
BW/A04 Turbine Trip / 4							Not applicable to this Unit.		
BW/A05 Emergency Diesel Actuation / 6							Not applicable to this Unit.		
BW/A07 Flooding / 8							Not applicable to this Unit.		
BW/E03 Inadequate Subcooling Margin / 4							Not applicable to this Unit.		
BW/E08; W/E03 LOCA Cooldown - Depress. / 4							Not applicable to this Unit.		
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4							Not Selected.		
BW/E13&E14 EOP Rules and Enclosures							Not applicable to this Unit.		
CE/A11; W/E08 RCS Overcooling - PTS / 4	X						Knowledge of the operational implications of the following concepts as they apply to the (RCS Overcooling): EK1.1 Components, capacity, and function of emergency systems	3.1	26
CE/A16 Excess RCS Leakage / 2			X				Knowledge of the reasons for the following responses as they apply to the (Excess RCS Leakage): EK3.2 Normal, abnormal and emergency operating procedures associated with (Excess RCS Leakage)	2.8	27
CE/E09 Functional Recovery							Not Selected.		
K/A Category Point Totals:	2	1	2	1	1	2			9

ES-401		PWR Examination Outline											Form ES-401-2	
		Plant systems – Tier 2/Group 1 (RO / SRO)												
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump				X								Knowledge of RCPS design feature(s) and/or interlock(s) which provide for the following: K4.04 Adequate cooling of RCP motor and seals	2.8	28
004 Chemical and Volume Control							X					Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CVCS controls including: A1.06 VCT level	3.0	29
005 Residual Heat Removal		X										Knowledge of bus power supplies to the following: K2.03 RCS Pressure boundary motor operated valves.	2.7*	30
005 Residual Heat Removal										X		Ability to manually operate and/or monitor in the control room: A4.03 RHR temperature, PZR heaters and flow, and nitrogen	2.8*	31
006 Emergency Core Cooling									X			Ability to (a) predict the impacts of the following malfunctions or operations on the ECCS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A3.08 Automatic transfer of ECCS flowpaths	4.2	32
007 Pressurizer Relief/ Quench Tank					X							Knowledge of the operational implications of the following concepts as they apply to the PRTS: K5.02 Method of forming a steam bubble in the PZR.	3.1	33
008 Component Cooling Water								X				Ability to (a) predict the impacts of the following malfunctions or operations on the CCWS, and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.05 Effect of loss of instrument and control air on the position of the CCW valves that are airoperated	3.3*	34
010 Pressurizer Pressure Control		X										Knowledge of bus power supplies to the following: K2.01 PZR heaters.	3.0	35
012 Reactor Protection			X									Knowledge of the effect that a loss or malfunction of the RPS will have on the following: K3.03 SDS	3.1*	36
013 Engineered Safety Features Actuation	X											Knowledge of the physical connections and/or cause-effect relationships between the ESFAS and the following systems: K1.18 Premature reset of ESF actuation	3.7	37
013 Engineered Safety Features Actuation											X	Generic 2.1.25 Ability to obtain and interpret station reference materials such as graphs, monographs, and tables which contain performance data.	2.8	38

022 Containment Cooling										X		Ability to manually operate and/or monitor in the control room: A4.02 CCS pumps	3.2*	39
022 Containment Cooling										X		Ability to manually operate and/or monitor in the control room: A4.04 Valves in the CCS	3.1*	40
025 Ice Condenser												Not applicable to this unit.		
026 Containment Spray										X		Ability to monitor automatic operation of the CSS, including: A3.02 Verification that cooling water is supplied to the containment spray heat exchanger	3.9*	41
039 Main and Reheat Steam			X									Knowledge of the effect that a loss or malfunction of the MRSS will have on the following: K3.04 MFW pumps	2.5*	42
059 Main Feedwater			X									Knowledge of the effect that a loss or malfunction of the MFW System will have on the following: K3.02 AFW System	3.6	43
059 Main Feedwater										X		Generic 2.4.14 Knowledge of general guidelines for EOP flowchart use.	3.0	44
061 Auxiliary/Emergency Feedwater	X											Knowledge of the physical connections and/or cause-effect relationships between the AFW System and the following systems: K1.09 PRMS	2.6*	45
062 AC Electrical Distribution							X					Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the A.C. Distribution System controls including: A1.03 Effect on instrumentation and controls of switching power supplies	2.5	46
063 DC Electrical Distribution								X				Ability to (a) predict the impacts of the following malfunctions or operations on the D.C. Electrical System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.01 Grounds	2.5	47
064 Emergency Diesel Generator						X						Knowledge of the effect of a loss or malfunction of the following will have on the ED/G System: K6.08 Fuel oil storage tanks	3.2	48
064 Emergency Diesel Generator						X						Knowledge of the effect of a loss or malfunction of the following will have on the ED/G System: K6.07 Air receivers	2.7	49
073 Process Radiation Monitoring				X								Knowledge of the operational implications of the following concepts as they apply to the PRM System: K5.01 Radiation theory, including sources, types, units, and effects	2.5	50

073 Process Radiation Monitoring					X													Knowledge of the operational implications of the following concepts as they apply to the PRM System: K5.02 Radiation intensity changes with source distance	2.5	51
076 Service Water					X													Knowledge of SWS design feature(s) and/or interlock(s) which provide for the following: K4.02 Automatic start features associated with SWS pump controls	2.9	52
078 Instrument Air					X													Knowledge of IAS design feature(s) and/or interlock(s) which provide for the following: K4.02 Cross-over to other air systems	3.2	53
078 Instrument Air																X		Generic 2.2.27 Knowledge of the refueling process.	2.6	54
103 Containment													X					Ability to (a) predict the impacts of the following malfunctions or operations on the Containment System; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.05 Emergency Containment Entry	2.9	55
K/A Category Point Totals:	2	2	3	3	3	2	2	3	2	3	3							Group Point Total:		28

ES-401		PWR Examination Outline											Form ES-401-2	
		Plant systems – Tier 2/Group 2 (RO / SRO)												
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive												Not selected		
002 Reactor Coolant										X		Ability to manually operate and/or monitor in the control room: A4.01 RCS leakage calculation program using the computer	3.5*	56
011 Pressurizer Level Control			X									Knowledge of the effect that a loss or malfunction of the PZR LCS will have on the following: K3.03 PZR PCS	3.2	57
014 Rod Position Indication												Not selected		
015 Nuclear Instrumentation		X										Knowledge of bus power supplies to the following: K2.01 NIS channels, components, and interconnections	3.3	58
016 Non-nuclear Instrumentation												Not selected		
017 In-core Temperature Monitor									X			Ability to monitor automatic operation of the ITM System, including: A3.01 Indications of normal, natural, and interrupted circulation of RCS	3.6*	59
027 Containment Iodine Removal												Not applicable to this unit		
028 Hydrogen Recombiner and Purge Control												Not selected		
029 Containment Purge												Not selected		
033 Spent Fuel Pool Cooling	X											Knowledge of the physical connections and/or cause-effect relationships between the Spent Fuel Pool Cooling System and the following systems: K1.02 RHRS	2.5	60
034 Fuel Handling Equipment												Not selected		
035 Steam Generator						X						Knowledge of the effect of a loss or malfunction of the following will have on the S/GS: K6.01 MSIVs	3.2	61
041 Steam Dump/Turbine Bypass Control												Not selected		
045 Main Turbine Generator								X				Ability to (a) predict the impacts of the following malfunctions or operations on the MT/G System and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.17 Malfunction of electrohydraulic control	2.7*	62
055 Condenser Air Removal												Not selected		
056 Condensate										X		Generic 2.3.1 Knowledge of 10CFR20 and related facility radiation control requirements.	2.6	63
068 Liquid Radwaste												Not selected		
071 Waste Gas Disposal												Not selected		

072 Area Radiation Monitoring					X											Knowledge of the operational implications of the following concepts as they apply to the ARM system: K5.01 Radiation theory, including sources, types, units, and effect.	2.7	64
075 Circulating Water																Not selected		
079 Station Air																Not selected		
086 Fire Protection								X								Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the Fire Protection System controls including: A1.05 FPS lineups	2.9	65
K/A Category Totals:	1	1	1	0	1	1	1	1	1	1	1	1				Group Point Total:		10

Facility: Arkansas Nuclear One Unit 2 RO Written Outline		Date of Exam: 07/14/2006		
Category	K/A #	Topic	RO	
			IR	#
1. Conduct of Operations	2.1.7	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	3.7	66
	2.1.22	Ability to determine Mode of Operation.	2.8	67
	2.1.27	Knowledge of system purpose and or function	2.8	68
Subtotal				3
2. Equipment Control	2.2.1	Ability to perform pre-startup procedures for the facility, including operating those controls associated with plant equipment that could affect reactivity.	3.7	69
	2.2.26	Knowledge of refueling administrative requirements	2.5	70
Subtotal				2
3. Radiation Control	2.3.9	Knowledge of the process for performing a containment purge.	2.5	71
	2.3.11	Ability to control radiation releases.	2.7	72
Subtotal				2
4. Emergency Procedures/ Plan	2.4.35	Knowledge of local auxiliary operator tasks during emergency operations including system geography and system implications.	3.3	73
	2.4.47	Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.	3.4	74
	2.4.49	Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4.0	75
Subtotal				3
Tier 3 Point Total				10

Facility: Arkansas Nuclear One Unit 2 SRO Written Outline														Date of Exam: 07/14/2006				
Tier	Group	RO K/A Category Points											SRO – Only Points					
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A2	G*	TOTAL		
1. Emergency & Abnormal Plant Evolutions	1													4	2	6		
	2													3	1	4		
	Tier Totals													7	3	10		
2. Plant Systems	1													3	2	5		
	2													1	2	3		
	Tier Totals													4	4	8		
3. Generic Knowledge and Abilities Categories														1	2	3	4	7
														2	2	1	2	
<p>Note:</p> <ol style="list-style-type: none"> Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO Outline and the SRO only outlines (i.e. except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two). The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ± 1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO –only exam must total 25 points. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution. Absent a plant specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories. * The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IR) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. Use duplicate pages for RO and SRO-only exams. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A number, descriptions, importance ratings, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43 																		

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E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#	
00007 (BW/E02 & E10; CE/E02) Reactor Trip – Stabilization – Recovery / 1						X	2.1.19 Ability to use plant computer to obtain and evaluate parametric information on system or component status.	3.0	76	
00008 Pressurizer Vapor Space Accident / 3							Not selected			
000009 Small Break LOCA / 3							Not selected			
000011 Large Break LOCA / 3							Not selected			
000015/17 RCP Malfunctions / 4							Not selected			
000022 Loss of Rx Coolant Makeup / 2							Not selected			
000025 Loss of RHR System / 4							Not selected			
000026 Loss of Component Cooling Water / 8							Not selected			
000027 Pressurizer Pressure Control System Malfunction / 3							Not selected			
000029 ATWS / 1							Not selected			
000038 Steam Gen. Tube Rupture / 3					X		Ability to determine and interpret the following as they apply to a SGTR: EA2.02 Existence of an S/G tube rupture and its potential consequences	4.8	77	
000038 Steam Gen. Tube Rupture / 3					X		Ability to determine and interpret the following as they apply to a SGTR: EA2.12 Status of MSIV activating system	4.2	78	
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4							Not selected			
000054 (CE/E06) Loss of Main Feedwater / 4							Not selected			
000055 Station Blackout / 6						X	2.2.20 Knowledge of the process for managing troubleshooting activities.	3.3	79	
000056 Loss of Off-site Power / 6							Not selected			
000057 Loss of Vital AC Inst. Bus / 6							Not selected			
000058 Loss of DC Power / 6							Not selected			
000062 Loss of Nuclear Svc Water / 4							Not selected			
000065 Loss of Instrument Air / 8					X		Ability to determine and interpret the following as they apply to the Loss of Instrument Air: AA2.02 Relationship of flow readings to system operation	2.6*	80	
000065 Loss of Instrument Air / 8					X		Ability to determine and interpret the following as they apply to the Loss of Instrument Air: AA2.05 When to commence plant shutdown if instrument air pressure is decreasing	4.1	81	
W/E04 LOCA Outside Containment / 3							Not applicable to this Unit.			
W/E11 Loss of Emergency Coolant Recirc. / 4							Not applicable to this Unit.			
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4							Not applicable to this Unit.			
K/A Category Totals:					4	2	Group Point Total:		6	

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E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#	
000001 Continuous Rod Withdrawal / 1							Not selected			
000003 Dropped Control Rod / 1					X		Ability to determine and interpret the following as they apply to the Dropped Control Rod: AA2.03 Dropped rod, using in-core/ex-core instrumentation, in-core or loop temperature measurements	3.8	82	
000005 Inoperable/Stuck Control Rod / 1							Not selected			
000024 Emergency Boration / 1					X		Ability to determine and interpret the following as they apply to the Emergency Boration: AA2.02 When use of manual boration valve is needed	4.4	83	
000028 Pressurizer Level Malfunction / 2							Not selected			
000032 Loss of Source Range NI / 7							Not selected			
000033 Loss of Intermediate Range NI / 7							Not applicable to this Unit			
000036 (BW/A08) Fuel Handling Accident / 8							Not selected			
000037 Steam Generator Tube Leak / 3							Not selected			
000051 Loss of Condenser Vacuum / 4							Not selected			
000059 Accidental Liquid RadWaste Rel. / 9							Not selected			
000060 Accidental Gaseous Radwaste Rel. / 9							Not selected			
000061 ARM System Alarms / 7							Not selected			
000067 Plant Fire On-site / 8							Not selected			
000068 (BW/A06) Control Room Evac. / 8							Not selected			
000069 (W/E14) Loss of CTMT Integrity / 5					X		Ability to determine and interpret the following as they apply to the Loss of Containment Integrity: AA2.01 Loss of containment integrity	4.3	84	
000074 (W/E06&E07) Inad. Core Cooling / 4							Not selected			
000076 High Reactor Coolant Activity / 9							Not selected			
W/E01 & E02 Rediagnosis & SI Termination / 3							Not applicable to this Unit.			
W/E13 Steam Generator Over-pressure / 4							Not applicable to this Unit.			
W/E15 Containment Flooding / 5							Not applicable to this Unit.			
W/E16 High Containment Radiation / 9							Not applicable to this Unit.			
BW/A01 Plant Runback / 1							Not applicable to this Unit.			
BW/A02&A03 Loss of NNI-X/Y / 7							Not applicable to this Unit.			
BW/A04 Turbine Trip / 4							Not applicable to this Unit.			
BW/A05 Emergency Diesel Actuation / 6							Not applicable to this Unit.			
BW/A07 Flooding / 8							Not applicable to this Unit.			

BW/E03 Inadequate Subcooling Margin / 4							Not applicable to this Unit.		
BW/E08; W/E03 LOCA Cooldown - Depress. / 4							Not applicable to this Unit.		
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4							Not selected		
BW/E13&E14 EOP Rules and Enclosures							Not applicable to this Unit.		
CE/A11; W/E08 RCS Overcooling - PTS / 4							Not selected		
CE/A16 Excess RCS Leakage / 2							Not selected		
CE/E09 Functional Recovery						X	2.3.10 Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure.	3.3	85
K/A Category Point Totals:					3	1	Group Point Total:	4	

ES-401		PWR Examination Outline										Form ES-401-2		
Plant systems – Tier 2/Group 1 (RO/SRO)														
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump											X	2.2.18 Knowledge of the process for managing maintenance activities during shutdown operations.	3.5	86
004 Chemical and Volume Control												Not selected		
005 Residual Heat Removal												Not selected		
006 Emergency Core Cooling								X				Ability to (a) predict the impacts of the following malfunctions or operations on the ECCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.09 Radioactive release from venting RWST to atmosphere	3.2*	87
007 Pressurizer Relief/ Quench Tank												Not selected		
008 Component Cooling Water												Not selected		
010 Pressurizer Pressure Control												Not selected		
012 Reactor Protection												Not selected		
013 Engineered Safety Features Actuation												Not selected		
022 Containment Cooling												Not selected		
025 Ice Condenser												Not applicable to this Unit		
026 Containment Spray											X	2.4.46 Ability to verify that the alarms are consistent with the plant conditions.	3.6	88
039 Main and Reheat Steam												Not selected		
059 Main Feedwater												Not selected		
061 Auxiliary/Emergency Feedwater								X				Ability to (a) predict the impacts of the following malfunctions or operations on the AFW; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.05 Automatic control malfunction	3.4*	89
062 AC Electrical Distribution												Not selected		
063 DC Electrical Distribution												Not selected		
064 Emergency Diesel Generator												Not selected		
073 Process Radiation Monitoring												Not selected		
076 Service Water												Not selected		
078 Instrument Air												Not selected		
103 Containment								X				Ability to (a) predict the impacts of the following malfunctions or operations on the containment system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.04 Containment evacuation (including recognition of the alarm)	3.6*	90
K/A Category Point Totals:								3			2	Group Point Total:		5

ES-401		PWR Examination Outline										Form ES-401-2		
Plant systems – Tier 2/Group 2 (RO/SRO)														
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive												Not selected		
002 Reactor Coolant												Not selected		
011 Pressurizer Level Control												Not selected		
014 Rod Position Indication												Not selected		
015 Nuclear Instrumentation												Not selected		
016 Non-nuclear Instrumentation								X				Ability to (a) predict the impacts of the following malfunctions or operations on the NNIS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: A2.03 Interruption of transmitted signal	3.3*	91
017 In-core Temperature Monitor												Not selected		
027 Containment Iodine Removal												Not applicable to this Unit		
028 Hydrogen Recombiner and Purge Control												Not selected		
029 Containment Purge												Not selected		
033 Spent Fuel Pool Cooling												Not selected		
034 Fuel Handling Equipment												Not selected		
035 Steam Generator												Not selected		
041 Steam Dump/Turbine Bypass Control												Not selected		
045 Main Turbine Generator												Not selected		
055 Condenser Air Removal												Not selected		
056 Condensate												Not selected		
068 Liquid Radwaste											X	2.4.41 Knowledge of the emergency action level thresholds and classifications.	4.1	92
071 Waste Gas Disposal											X	2.3.8 Knowledge of process for performing a planned gaseous radioactive release.	3.2	93
072 Area Radiation Monitoring												Not selected		
075 Circulating Water												Not selected		
079 Station Air												Not selected		
086 Fire Protection												Not selected		
K/A Category Totals:								1			2	Group Point Total:		3

Facility: Arkansas Nuclear One Unit 2 SRO Written Outline			Date of Exam: 07/14/2006	
Category	K/A #	Topic	SRO-Only	
			IR	#
1. Conduct of Operations	2.1.11	Knowledge of less than one hour technical specification action statements for systems.	3.8	94
	2.1.30	Ability to locate and operate components, including local controls.	3.4	95
	Subtotal			
2. Equipment Control	2.2.10	Knowledge of the process for determining if the margin of safety, as defined in the basis of any technical specification is reduced by a proposed change, test, or experiment.	3.3	96
	2.2.11	Knowledge of the process for controlling temporary changes.	3.4*	97
	Subtotal			
3. Radiation Control	2.3.6	Knowledge of the requirements for reviewing and approving release permits.	3.1	98
	Subtotal			
4. Emergency Procedures/ Plan	2.4.36	Knowledge of chemistry / health physics tasks during emergency operations.	2.8	99
	2.4.48	Ability to interpret control room indications to verify the status and operation of system, and understand how operator actions and directives affect plant and system conditions.	3.8	100
	Subtotal			
Tier 3 Point Total				7

Facility: <u>ANO-2</u> Examination Level: RO <input checked="" type="checkbox"/> SRO <input type="checkbox"/>		Date of Examination: <u>07/17/2006</u> Operating Test Number: <u>1</u>
Administrative Topic (see note)	Type Code*	Describe activity to be performed
Conduct of Operations 2.1.23 RO (3.9)	N	Ability to perform specific system and integrated plant procedures during all modes of plant operation. Determine volume of Boric acid and DI water to makeup to RWT. JPM-ANO-2-JPM-NRC-ADMIN RWT
Conduct of Operations 2.1.23 RO (3.9)	M	Ability to perform specific system and integrated plant procedures during all modes of plant operation. Determine CEDM temperature using OP 2105.009. Modified-JPM-ANO-2-JPM-NRC-ADMIN XTCEA
Equipment Control 2.2.12 RO (3.0)	D/P	Knowledge of Surveillance procedures. Review 2P89B Surveillance as RO. Direct-JPM-ANO-2-NRC-ADMIN-Surveillance review 2P89B
Radiation Control 2.3.10 RO (2.9)	M	Ability to perform procedure to reduce excessive levels of radiation and guard against personnel exposure. Review survey map and RWP to determine radiological requirements JPM-ANO-2-NRC-ADMIN-2P60B isolation requirements
Emergency Plan	NA	NA
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.		
*Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & ROretakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 Exams (≤ 1; randomly selected)		

Facility: <u>ANO-2</u>		Date of Examination: <u>07/17/2006</u>
Examination Level: RO <input type="checkbox"/> SRO <input checked="" type="checkbox"/>		Operating Test Number: <u>1</u>

Administrative Topic (see note)	Type Code*	Describe activity to be performed
Conduct of Operations 2.1.12 SRO (4.0)	D	Determine RPS trip set point due to inoperable MSSV is correct using Technical Specifications. Ability to apply technical specification for a system Direct- ANO-2-JPM-NRC-ADMIN MSSVINOP
Conduct of Operations 2.1.23 SRO (4.0)	M	Ability to perform specific system and integrated plant procedures during all modes of plant operation. Approve CEDM temperature calculation using OP 2105.009. Modified-ANO-2-JPM-NRC-ADMIN XTCEA
Equipment Control 2.2.12 SRO (3.4)	D/P	Knowledge of Surveillance procedures. Review 2P89B Surveillance. Direct-ANO-2-JPM-NRC-ADMIN-Surveillance review 2P89B
Radiation Control 2.3.10 SRO (3.3)	D	Ability to perform procedure to reduce excessive levels of radiation and guard against personnel exposure. Review survey map and RWP to determine radiological requirements Modified-JPM-ANO-2-JPM-NRC-ADMIN-2P60B Radiological requirements
Emergency Plan 2.4.29 SRO (4.0)	N	Knowledge of the emergency plan. Classify EAL and complete applicable Shift Manager forms. New-ANO-2-JPM-NRC-ADMIN-Classify event and Complete SM E-plan forms

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.

*Type Codes & Criteria:

- (C)ontrol room, (S)imulator, or Class(R)oom
- (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & ROretakes)
- (N)ew or (M)odified from bank (≥ 1)
- (P)revious 2 Exams (≤ 1 ; randomly selected)

Facility: <u>ANO UNIT 2</u>		Date of Examination: <u>07/17/2006</u>
Exam Level (circle one): RO <input checked="" type="checkbox"/> SRO(I) <input type="checkbox"/> SRO(U) <input type="checkbox"/>		Operating Test No.: <u>1</u>
Control Room Systems [®] (8 for RO; 7 for SRO-I; 2 or 3 for SRO- U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
a. ANO-2-JPM-NRC-ELEC EOP 2 062 A4.01 RO-3.3 SRO-3.1 Energize 2A2, non-vital 4160VAC bus following a Loss Of Offsite Power	A/L/N/S	6 Electrical
b. ANO-2-JPM-NRC-SIT006 006 A4.08 RO-4.2 SRO-4.3 Isolate Safety Injection Tank's with Safety Injection Actuation System actuated	A/L/D/S	2 Inventory
c. ANO-2-JPM-NRC-LTOP 010 K4.03 RO-3.8 SRO-4.1 Respond to Annunciator 2K10 C-4 and place low temperature overpressure relief valves inservice	L/N/S	3 Reactor Pressure Control
d. ANO-2-JPM-NRC-RCP03 008 A4.01 RO-3.3 SRO-3.1 Restore Component Cooling Water to Reactor Coolant Pumps	A/L/D/P/S	8 Plant Service Systems
e. ANO-2-JPM-NRC-CEA1 001 A4.03 RO-4.0 SRO-3.7 Exercise a Control Element Assembly	D/S	1 Reactivity
f. ANO-2-JPM-NRC-FWCS1 035 A4.01 RO-3.7 SRO- 3.6 Place Feed Water Control System in Automatic	D/S	4 Heat Removal
g. ANO-2-JPM-NRC-H2001 028 A4.01 RO-4.0 SRO-4.0 Manually start Hydrogen analyzer	C/D	5 Containment Integrity
h. ANO-2-JPM-NRC-ICI01 015 A2.02 RO-3.1 SRO-3.5 Remove Incore instrument from scan for Core Operating Limits Supervisory System	D/P/S	7 Instrumentation
In- Plant Systems [®] (3 for RO; 3 for SRO-I; 3 or 2 for SRO- U)		
i. ANO-2-JPM-NRC-AACGLS 064 A3.06 RO-3.3 SRO-3.4 Local start of Station Blackout Diesel	A/D	6 Electrical
j. ANO-2-JPM-NRC-P36ASD 004 A4.08 RO-3.8 SRO-3.4 Operate Charging Pump 2P36B Locally During Alternate Shutdown	D/E/R	2 Inventory
k. ANO-2-JPM-NRC-PRHTR 006 A2.01 RO-3.3 SRO-3.6 Locally control pressurizer proportional heaters	D/E	3 Reactor Pressure Control
@ All control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.		
Type Codes	Criteria for RO /SRO-I / SRO-U	
(A)lternate path (C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (L)ow-Power (N)ew or (M)odified from bank including 1(A) (P)revious 2 Exams (R)CA (S)imulator	4-6 / 4-6 / 2-3 ≤ 9 / ≤ 8 / ≤ 4 ≥ 1 / ≥ 1 / ≥ 1 ≥ 1 / ≥ 1 / ≥ 1 ≥ 2 / ≥ 2 / ≥ 1 ≤ 3 / ≤ 3 / ≤ 2 (randomly selected) ≥ 1 / ≥ 1 / ≥ 1	

Facility: <u>ANO UNIT 2</u>	Date of Examination: <u>07/17/2006</u>
Exam Level (circle one): RO <input type="checkbox"/> / SRO(I) <input checked="" type="checkbox"/> / SRO(U) <input type="checkbox"/>	Operating Test No.: <u>1</u>

Control Room Systems® (8 for RO; 7 for SRO-I; 2 or 3 for SRO- U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
a. ANO-2-JPM-NRC-ELEC EOP 2 062 A4.01 RO-3.3 SRO-3.1 Energize 2A2, non-vital 4160VAC bus following a Loss Of Offsite Power	A/L/N/S	6 Electrical
b. ANO-2-JPM-NRC-SIT006 006 A4.08 RO-4.2 SRO-4.3 Isolate Safety Injection Tank's with Safety Injection Actuation System actuated	A/L/D/S	2 Inventory
c. ANO-2-JPM-NRC-LTOP 010 K4.03 RO-3.8 SRO-4.1 Respond to Annunciator 2K10 C-4 and place low temperature overpressure relief valves inservice	L/N/S	3 Reactor Pressure Control
d. ANO-2-JPM-NRC-RCP03 008 A4.01 RO-3.3 SRO-3.1 Restore Component Cooling Water to Reactor Coolant Pumps	A/L/D/P/S	8 Plant Service Systems
e. ANO-2-JPM-NRC-CEA1 001 A4.03 RO-4.0 SRO-3.7 Exercise a Control Element Assembly	D/S	1 Reactivity
f. ANO-2-JPM-NRC-FWCS1 035 A4.01 RO-3.7 SRO- 3.6 Place Feed Water Control System in Automatic	D/S	4 Heat Removal
g. ANO-2-JPM-NRC-H2001 028 A4.01 RO-4.0 SRO-4.0 Manually start Hydrogen analyzer	C/D	5 Containment Integrity
h.		

In- Plant Systems® (3 for RO; 3 for SRO-I; 3 or 2 for SRO- U)		
i. ANO-2-JPM-NRC-AACGLS 064 A3.06 RO-3.3 SRO-3.4 Local start of Station Blackout Diesel	A/D	6 Electrical
j. ANO-2-JPM-NRC-P36ASD 004 A4.08 RO-3.8 SRO-3.4 Operate Charging Pump 2P36B Locally During Alternate Shutdown	D/E/R	2 Inventory
k. ANO-2-JPM-NRC-PRHTR 006 A2.01 RO-3.3 SRO-3.6 Locally control pressurizer proportional heaters	D/E	3 Reactor Pressure Control

@ All control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

Type Codes	Criteria for RO /SRO-I / SRO-U
(A)lternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1
(L)ow-Power	≥ 1 / ≥ 1 / ≥ 1
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1
(P)revious 2 Exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)
(R)CA	≥ 1 / ≥ 1 / ≥ 1
(S)imulator	

Facility: <u>ANO UNIT 2</u>	Date of Examination: <u>7/17/2006</u>
Exam Level : RO <input type="checkbox"/> / SRO(I) <input type="checkbox"/> / SRO(U) <input checked="" type="checkbox"/>	Operating Test No.: <u>1</u>

Control Room Systems [®] (8 for RO; 7 for SRO-I; 2 or 3 for SRO- U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
a. ANO-2-JPM-NRC-ELEC EOP 2 062 A4.01 RO-3.3 SRO-3.1 Energize 2A2, non-vital 4160VAC bus following a Loss Of Offsite Power	A/L/N/S	6 Electrical
b. ANO-2-JPM-NRC-SIT006 006 A4.08 RO-4.2 SRO-4.3 Isolate Safety Injection Tank's with Safety Injection Actuation System actuated	A/L/D/S	2 Inventory
c. ANO-2-JPM-NRC-LTOP 010 K4.03 RO-3.8 SRO-4.1 Respond to Annunciator 2K10 C-4 and place low temperature overpressure relief valves inservice	L/N/S	3 Reactor Pressure Control
d.		
e.		
f.		
g.		
h.		

In- Plant Systems [®] (3 for RO; 3 for SRO-I; 3 or 2 for SRO- U)		
i. ANO-2-JPM-NRC-AACGLS 064 A3.06 RO-3.3 SRO-3.4 Local start of Station Blackout Diesel	A/D	6 Electrical
j. ANO-2-JPM-NRC-P36ASD 004 A4.08 RO-3.8 SRO-3.4 Operate Charging Pump 2P36B Locally During Alternate Shutdown	D/E/R	2 Inventory
k.		

@ All control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

Type Codes	Criteria for RO /SRO-I / SRO-U
(A)lternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1
(L)ow-Power	≥ 1 / ≥ 1 / ≥ 1
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1
(P)revious 2 Exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)
(R)CA	≥ 1 / ≥ 1 / ≥ 1
(S)imulator	

Facility: ANO-2	Scenario No.: 1	Op-Test No.: 2006-1	
Page 1			
Examiners:	Operators:		
Initial Conditions: 20% MOL, All Engineered Safety Features systems are in standby. Plant startup following a five day stator water cooling outage. 2P27, MFP standby lube oil pump tagged out for maintenance. Green Train Maintenance Week.			
Turnover: 20%. 250 EFPD. EOOS indicates 'Minimal Risk.' Plant Startup in progress; OP 2104.004 section 9, raising power above 20%, is the controlling procedure. 2P27, MFP standby lube oil pump tagged out for maintenance. Green Train Maintenance Week.			
Event No.	Malf. No.	Event Type*	Event Description
1	Raise Power above 20%	R (ATC)	Raise reactor and turbine power.
2	XRCCHAPCNT	I (ATC)	Pressurizer control channel pressure fails high.
3	CCWFAILBAUTO CCW2P33CPWR	C (CBOT)	Loop II Component Cooling Water pump 'C' Trips and 'B' Component Cooling Water pump fails to automatically start.
4	RCP2P32AASLK	M (ATC) N (CBOT)	Reactor Coolant System inter-system leak into Component Cooling Water system resulting in Loss of Coolant Accident and Safety Injection Actuation System (post reactor trip).
5	BUS2H2	M (ALL)	2H2 lockout. Loss of 2 Reactor Coolant Pump's and one condenser circulating water pump.
6	RPSRXAUTO	C (ATC)	Reactor Protection System fails to automatically trip the reactor on loss of Reactor Coolant Pumps.
7	XMSHDRPRS	I (CBOT)	Steam Dump and Bypass Control System fails to automatically open bypass and dump valves to control Steam Generator pressure.
8	HPI2P89AFAL	C (CBOT)	2P89A, 'A' High Pressure Safety Injection pump fails to auto-start.

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Facility: ANO-2	Scenario No.: 2	Op-Test No.: 2006-1	
Page 1			
Examiners:	Operators:		
Initial Conditions: 100% MOL, All ESF systems in standby. Green Train Maintenance Week.			
Turnover: 100%. 250 EFPD. EOOS indicates 'Minimal Risk. Green Train Maintenance Week. 'B' main chiller is tagged out for oil change out.			
Event No.	Malf. No.	Event Type*	Event Description
1	COND2P2AWIND	C(CBOT)	'A' Condensate Pump motor winding with rise resulting in manual start of 'D' Condensate Pump and securing 'A' Condensate Pump.
2	XCV2LT4861	I (ATC)	Volume Control Tank level instrument fails low resulting in Refueling Water Tank being aligned to Coolant Charging Pump suction.
3	CWS2P3BBOL	R (ATC) N (CBOT)	Trip 2P3B, 'B' Circulating Water Pump, which causes a partial loss of main condenser circulating water flow resulting in a rapid down power to ~ 90% power.
4	MSSGBLK	M (CBOT) M (ATC)	'B' Steam Generator Excess Steam Demand (ESD) inside containment results in manual reactor trip and control of Reactor Coolant System heat up and Pressurizer pressure post SG blowdown.
5	CEA51STUCK	C (ATC)	Control Element Assembly #51 stuck on reactor trip results in Emergency Boration.
6	CV1036-2 CV1075-1	C (CBOT)	'B' Emergency Feed Water (EFW) Pump to 'B' Steam Generator valves fail to close from control room resulting in over feeding Steam Generator with ESD cooldown of RCS unnecessarily. Secure 'B' EFW pump.
7	BS2P35BFAL BS2P35AFAULT	C (CBOT)	'B' Spray Pump fails to start. Can be manually started. 'A' Spray Pump cannot be restarted.
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			

Facility: ANO-2	Scenario No.: 3	Op-Test No.: 2006-1	
Page 1			
Examiners:	Operators:		
Initial Conditions:			
100% MOL, All ESF systems in standby. Green Train Maintenance Week.			
Turnover:			
100%. 250 EFPD. EOOS indicates 'Minimal Risk. Green Train Maintenance Week. Tornado watch for Pope County in effect until 10:00pm today. Natural Emergency AOP actions have been completed.			
Event No.	Malf. No.	Event Type*	Event Description
1	CV-4816	C(ATC)	Letdown flow control valve fails closed. Restore Letdown using 2CV 4817.
2	XFW2FIS0735	I (CBOT)	'A' Main Feedwater Pump (MFP) suction flow transmitter fails low. Recirculation valves ('A' loop condensate and 'A' MFP) to manual and closed.
3	MFWWPMPBTRP	R (ATC) N (CBOT)	'B' MFP trips on thrust bearing wear. Emergency borate and reduce power to <90%. Start 'D' Condensate Pump.
4	500LOSE500 500LOSE161	M (CBOT) M (ATC)	Loss of offsite power due to tornado.
5	EDGDG1OIL	C (CBOT)	#1 Emergency Diesel Generator trips on low lube oil pressure. Loss of Red vital AC power. Loss of 2Y1, 120VAC non-vital power. CBOT energize Vital Red AC busses with Station Blackout diesel generator.
6	Channel 1 PZR level and pressure channels lose power.	I (ATC)	Pressurizer control channel 1 for level and pressure will fail low due to loss of power. Control manually / select opposite channel.
7	CV0332	C (CBOT)	Over speed trip of 'A' Emergency Feedwater Pump. Requires starting 'B' Emergency Feedwater Pump.
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			