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Energy to Serve Your WorldSM

June 29, 2006

Docket No.: 50-321

NL-06-1358

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555-0001

**Edwin I. Hatch Nuclear Plant - Unit 1
Inservice Inspection Program
Owner's Activity Report for Outage 1R22**

Ladies and Gentlemen:

The 3rd inservice inspection interval for Hatch Nuclear Plant Unit 1 ended on December 31, 2005. Southern Nuclear Company (SNC) completed the remaining 3rd interval examinations during the 22nd refueling outage (designated as 1R22), which began during February 2006 (just after the scheduled end of the 3rd interval). The extension of the examinations beyond the end of the interval to coincide with the refueling outage was allowable per IWB-2412 of the 1989 Edition of Section XI and per IWA-2430 in more current Section XI Codes. SNC also started the 4th interval examinations during the same outage (designated as 1R22a); therefore, both 3rd and 4th interval examinations were performed. In no case was an examination used to satisfy the requirements of both intervals.

Enclosed are the ASME Section XI Code Case N-532-1 OAR-1 Owner's Activity Reports for the closeout of the 3rd Interval ISI activities (Interval 3, Period 3, Outage 2) and the beginning of the 4th Interval ISI activities (Interval 4, Period 1, Outage 1).

This letter contains no NRC commitments. If you have any questions, please advise.

Sincerely,

L. M. Stinson

LMS/PAH/daj

- Enclosures: 1). ASME Section XI Code Case N-532-1 OAR-1 Owner's Activity Reports for the closeout of the 3rd Interval ISI activities (Interval 3, Period 3, Outage 2)
2.) ASME Section XI Code Case N-532-1 OAR-1 Owner's Activity Report for the beginning of the 4th Interval ISI activities (Interval 4, Period 1, Outage 1)

AO47

cc: Southern Nuclear Operating Company
Mr. J. T. Gasser, Executive Vice President
Mr. D. R. Madison, General Manager – Plant Hatch
RTYPE: CHA02.004

U. S. Nuclear Regulatory Commission
Dr. W. D. Travers, Regional Administrator
Mr. C. Gratton, NRR Project Manager – Hatch
Mr. D. S. Simpkins, Senior Resident Inspector – Hatch

Enclosure 1

Edwin I. Hatch Nuclear Plant

ASME Section XI Code Case N-532-1 OAR-1 Owner's Activity Reports for the Closeout
of the 3rd Interval ISI Activities (Interval 3, Period 3, Outage 2)

PLANT HATCH UNIT 1, 1R22 OUTAGE

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number 1-3-3-2 (Unit 1, 3rd Interval, 3rd Period, 2nd Report)

Owner Southern Nuclear Operating Co. (as agent for Georgia Power Company), 40 Inverness Center Parkway, Birmingham, AL 35242
(Name and Address of Owner)

Plant Edwin I. Hatch Nuclear Plant, P. O. Box 2010, Baxley, Georgia 31515
(Name and Address of Plant)

Unit No. 1 Commercial service date 12/31/75 Refueling outage no. 1R22
(if applicable)

Current inspection interval 3rd
(1st, 2nd, 3rd, 4th, other)

Current inspection period 3rd
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plan 1989 Edition/No Addenda

Date and revision of inspection plan 12/15/05, Revision 8

Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan Same

CERTIFICATE OF CONFORMANCE

I certify that the statements made in the Owner's Activity Report are correct, and that the exams, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

Certificate of Authorization No. N/A Expiration Date N/A
(if applicable)

Signed *Mark J. Cifone* AGM PS Date 6/26/2006
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Georgia and employed by Hartford Steam Boiler Inspection & Insurance Company of Hartford, CT have inspected the items described in this Owner's Activity Report, for Refueling Outage 1R22, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the exams, tests, repairs, replacements, evaluations and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Mark J. Cifone Commissions GA 572
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/27/06

This form (E00127) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. (Modified for an outage frequency versus a period frequency)

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
B-A/B1.11	0	0	0%	0%	NRC approved Relief Request RR-38 allows deletion of exams per BWRVIP-05.
B-A/B1.12	11	0	0%	100%	Third Interval exams were completed in the 1 ST Period as allowed by 10 CFR 50.55a(g)(6)(ii)(A). 100% of all the accessible welds have been examined. Weld C-4-B was inaccessible due to a tie rod obstruction.
B-A/B1.21	2	1	50%	100%	Exams are deferrable until the end of the 3 RD Interval. There is a total of three B1.21 welds; however, weld C-8 is completely inaccessible, and relief has been submitted. Both accessible welds have been examined.
B-A/B1.22	16	8	50%	100%	There is a total of 22 B1.22 welds; however, six welds are inaccessible and relief has been submitted. Exams are deferrable until the end of the 3 RD Interval.
B-A/B1.30	1	0	0%	100%	Code Case N-598 percentages not applicable for one weld. See NRC approved Relief Request RR-28.
B-A/B1.40	1	0	0%	100%	Code Case N-598 percentages not applicable for one weld. See Relief Request RR-28.
B-A Total	NA	NA	NA	NA	Each item was either deferrable, required to be examined in a specific period, or there was less than 3 items. Therefore, N-598 percentages did not apply. See Relief Request RR-28.
B-D/B3.90	28	4	14%	100%	There are 29 nozzles; however, the N15 RPV drain nozzle is located inside the support skirt and is not accessible. NRC approved Relief Request RR-3 allows deletion of the N15 examination. Therefore, 28 exams were scheduled.
B-D/B3.100	28	4	14%	100%	There are 29 nozzles; however, the N15 RPV drain nozzle is located inside the support skirt and is not accessible. NRC approved Relief Request RR-3 allows deletion of the N15 examination. Therefore, 28 exams were scheduled.
B-D Total	56	8	14%	100%	Exams were per Table IWB-2500-1, Category B-D, Footnote 2 which states, "At least 25% but no more than 50% (credited) of the nozzles shall be examined by the end of the 1 ST Period, and the remainder by the end of the inspection interval."

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
B-E/B4.11 B-E/B4.12 B-E/B4.13	NA	NA	NA	NA	Performed as part of the Category B-P pressure tests.
B-F/B5.10 B-F/B5.20 B-F/B5.130	NA	NA	NA	NA	NRC approved Relief Request RR-39 allows exemption from Code exam percentage requirements of dissimilar metal welds (4" and larger in diameter). In lieu of the Code requirements, the number of welds examined and the frequency of the exam was as specified in BWRVIP-075.
B-G-1/B6.10	52	18	35%	100%	NRC approved Relief Request RR-1 allows performance of a VT-1 exam in lieu of the surface exam.
B-G-1/B6.20	52	52	100%	100%	NRC approved Relief Request RR-35 implements Code Case N-307-3 and allows the deferral of these stud exams until the end of the interval as long as there is no more than 10 years between inspections
B-G-1/B6.40	52	52	100%	100%	NRC approved Relief Request RR-35 allows the deferral of these flange ligament exams until the end of the interval as long as there are no more than 10 years between inspections.
B-G-1/B6.50	52	18	35%	100%	Washer exams
B-G-1/B6.50B	4	4	100%	100%	Bushing Exams. There are 52 bushing; however, only four are made accessible by removal of the studs.
B-G-1/B6.180 B-G-1/B6.190 B-G-1/B6.200	NA	NA	NA	NA	These exams are not deferrable; however, they are required only when the B-L-2 visual exam of the internal surfaces of a pump are performed. During the 2 ND Period, the B-L-2 visual exam was performed on RC Pump B and the Item B6.180 pump studs, Item B6.190 flange surface, and Item B6.200 nuts were examined. Only one pump is required to be examined per B-L-2 requirements, thus, B-G-1 requirements were satisfied.
B-G-1 Total	NA	NA	NA	NA	Each item was either deferrable or was required only in conjunction with another exam. Therefore, N-598 percentages do not apply.

FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL

TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
B-G-2/B7.50	8	2	25%	100%	Piping Flange Bolting examinations are required for flange bolting located on piping subject to examination.
B-G-2/B7.70	NA	NA	NA	NA	Examination of pressure-retaining valve bolting is required only when Category B-M-2 visual exams are performed. Category B-M-2 exams have been performed on 10 valves (one in the 3 RD Period)
B-G-2/B7.80	NA	NA	NA	NA	CRD bolting is required to be examined when disassembled. A VT-1 is performed of existing bolting (for CRD's swapped out) prior to any bolting being replaced following removal of the bolting. A final VT-1 is performed for replacement bolting (for CRD's swapped out) prior to the bolting being installed.
B-G-2 Total	NA	NA	NA	NA	
B-H	NA	NA	NA	NA	Code Case N-509 used for selection. Re-named as Category B-K.
B-J/B9.12 B-J/B9.22	NA	NA	NA	NA	Code Case N-524 provides alternate examination criteria for longitudinal welds.
B-J/B9.11 B-J/B9.21 B-J/B9.31 B-J/B9.32	88	36	41%	100%	NRC approved Relief Request RR-39 allows exemption from Code examination percentage requirements for stainless steel Category B-J piping (4" and larger in diameter). Therefore, only carbon steel welds are shown in this row.
B-K/B10.10	1	0	0%	100%	NRC approved Relief Request RR-24 implements Code Case N-323-1 for the RPV skirt weld. The NRC also approved Relief Request RR-41 to allow the use of N-700 to select RPV welded attachments for examination versus Code Case N-509 footnote 4 requirements. As a result only one of the five was required to be examined (RPV skirt)
B-K/B10.20	8	3	38%	100%	Selected per Code Case N-509.
B-K/B10.30	1	0	0%	100%	Code Case N-509 was used; therefore, one integral attachment on one recirculation pump was examined during the 1 ST Period.
B-K Total	10	3	30%	100%	

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
B-L-2	NA	NA	NA	NA	Category B-L-2 applies only to the 2 RC pumps. During the 2 ND Period, the B-L-2 visual exam was performed on RC Pump B. Only one pump in a group is required to be examined per B-L-2 requirements, thus, B-L-2 requirements were satisfied.
B-M-2	NA	NA	NA	NA	Category B-M-2 applies to 17 groups of valves. Examination of the valve internal surface is limited to one valve in a group and is required only when a valve is disassembled for maintenance or repair. To-date examinations have been performed in 10 of the valve groups.
B-N-1/B13.10	3	1	33%	100%	Examination of vessel interior performed once per period.
B-N-2/ B13.20 B-N-2/ B13.30	38	9	24%	100%	Exams are deferrable until the end of the 3 RD interval. Item numbers are combined because a VT-1 is performed on all attachment welds.
B-N-2/B13.40	NA	16	NA	NA	Exams are deferrable until the end of the 3 RD interval; therefore, N-598 percentages do not apply. The accessible surfaces of the Shroud, Shroud Support Ledge, Control Rod Housing & Stub Tubes, are examined during the 10-year interval.
B-P/B15.10 B-P/B15.50 B-P/B15.60 B-P/B15.70	NA	NA	NA	NA	A system leakage test is performed each refueling outage.
B-P/B15.11 B-P/B15.51 B-P/B15.61 B-P/B15.71	NA	NA	NA	NA	The system leakage test with extended boundary (to 2 ND isolation valve) is deferrable to the end of the interval and was performed during 1R22 for the 3 RD Interval per ISI-CODE-1. Therefore, reporting percentage completion for this pressure test is not appropriate.
C-A/C1.10 C-A/C1.20	3	1	33%	100%	The Code requires that the equivalent of one RHR Heat Exchanger be examined in the 10-year interval.

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
C-B/C2.21 C-B/C2.22	4	2	50%	100%	There are two nozzles on each of the two RHR Heat Exchangers (4 exams). The Code requires that the equivalent of one RHR Heat Exchanger be examined in the 10-year interval.
C-C/C3.10	1	0	0%	100%	Code Case N-509 was used; therefore, one integral attachment on one RHR Heat Exchanger was examined during the 2 ND Period.
C-C/C3.20	21	5	24%	100%	Code Case N-509 was used. 10% of the non-exempt piping integral attachments are required to be examined.
C-C/Total	22	5	23%	100%	Code Case N-509 was used. 10% of the non-exempt piping integral attachments are required to be examined.
C-F-2/C5.52 C-F-2/C5.62 C-F-2/C5.82	NA	NA	NA	NA	Code Case N-524 provides alternate examination criteria for longitudinal welds.
C-F-2/C5.51 C-F-2/C5.81	97	26	27%	100%	
C-H/C7.10 C-H/C7.30 C-H/C7.50 C-H/C7.70	NA	NA	NA	NA	These operating pressure tests are performed twice each inspection interval (two out of three inspection periods). Therefore, reporting percentage completion for these pressure tests is not appropriate.
C-H/C7.20 C-H/C7.40 C-H/C7.60 C-H/C7.80	NA	NA	NA	NA	Plant Hatch utilized Code Case N-498-1 to meet hydrostatic test requirements during 3 RD Interval. Remaining 3 RD Interval pressure tests to be completed per IWA-2430(d). Since these tests are performed once per interval and since deferral to the end of the interval is permitted, reporting percentage completion of these pressure tests is not appropriate.

FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL
TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 3	Total Exams Credited for Period 3	Total Exams Credited (%) for Period 3	Total Exams Credited (%) To Date for Interval 3 (See Note 2)	Remarks (See Note 1)
F-A Total	122	54	44%	100%	<p>These numbers do not include Class 1 and 2 snubber visual examinations.</p> <p>Justification:</p> <p>HNP utilized Code Case N-491-1 for examination of component supports and para. -1220 states: "The inservice inspection requirements for snubbers shall be in accordance with the Section XI Edition and Addenda specified in the Owner's ISI Program".</p> <p>Therefore, per the 1989 Edition of Section XI, IWF-1220, neither Table IWF-2500-1 (SC XI) nor Table -2500-1 (N-491-1) percentages are applicable. IWF-5000 requires snubber examinations in accordance with ASME/ANSI OM-1987, Part 4. OM-1987, Part 4 does not contain any requirements for reporting of snubber examinations. Additionally, HNP utilizes Code Case N-532-1 for reporting of ISI for Class 1 and 2 pressure retaining components and their supports. N-532-1 does not require reporting of snubber examinations or activities other than repairs or replacements. HNP evaluated the snubber examination requirements of OM-1987, Part 4 and determined that the requirements of the snubber program contained in the Technical Requirements Manual (TRM), Section 3.7.1 exceeds the requirements of OM – 1987, Part 4, except the VT-3 inspection method required by ASME XI. Therefore, HNP performed snubber inspections in accordance with TRM 3.7.1 and approved Relief Request RR-11 as an alternative to performing a VT-3 during the 3rd ISI Interval.</p>

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22 OUTAGE – 3RD INTERVAL
TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

NOTES

- Note 1:** Interval 3 requirements (NDE and visual exams performed during an outage) were completed during 1R22.
- Note 2:** NRC approved Code Case N-598 was used to determine the required percentage of exams, except in cases where there were less than three items or welds to be examined in an Exam Category. In this case the item or welds could be examined in any two periods, or in any one period if there was only one item or weld. (See NRC approved Relief Request RR-28).
- Note 3:** Those Category/Items not shown in Table 1 are not applicable to Plant Hatch.

TABLE 2
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category	Item Number	Item Description	Flaw Characterization (IWA-3000)	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes / No)
B-D	B3.90	Main Steam Nozzle "A" (N3A) nozzle to RPV shell weld documented two Section XI defects, which were evaluated as acceptable per IWB-3600 without scope expansion (N-586) or re-examination (N-526)	Subsurface	Yes

Code Class	Repair Replacement Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found during Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
2	Repair	4" Piping	Downstream of 1E21-F015A, leak existed at the branch location where the 4" RHR line downstream of 1E11-F011A comes into the 10" CS test. The leak was at the weld joining the 4" RHR piping to the 10" CS piping. The degraded portion of the weld was ground out and then rewelded. Most likely cause of leak was porosity in weld during fabrication. Reference CR 2006102619.		3/16/2006	1060698001
2	Replacement	Main Steam Line Supp	During ISI VT-3 exam of support, 3 base plate nuts were found loose and cracking was noted in the paint for the 2" x 2" structural steel to base plate weld. Corrective measures performed were not Section XI R&R applicable, but due to the conditions being noted during an ISI exam and corrective measures being performed, this activity has been designated for inclusion in the 1R22 OAR-1, Table 3. NIS-2A not required for this activity. Reference CR 2006102066.	Yes	3/22/2006	1060604201
3	Replacement	CRD Room Cooler	During internals inspection of CRD room cooler with end cover removed, several Nelson studs were noted as being degraded due to normal inservice conditions. To ensure continued cooler performance, 10 Nelson studs were replaced by welding in new ones (attached to tube sheet of room cooler). Reference CR 2005110437.		1/12/2006	1052613701

Code Class	Repair Replacement Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found during Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
3	Replacement	3" AOV / Globe Valve	Replaced valve body due to degradation from normal inservice conditions. Valve did not fail but body was replaced as preventative measure. Also, pressure retaining bolting for various mechanical connections and a portion of piping was replaced to facilitate replacement activities. Reference CR's 2006102675 & 2006103101. Note: valve stem/plug was not replaced (parts returned to warehouse). Additionally, R&R Plan for WO 1041660802 covered by plan in		3/24/2006	1041660801
3	Replacement	1.5" Globe Valve	Replaced valve body due to degradation from normal inservice conditions. Valve did not fail but body was replaced as preventative measure. Reference CR 2006101940. Additionally, R&R Plan for WO 1041660602 covered by plan in WO1041660601.		3/27/2006	1041660601
3	Replacement	1.5" Globe Valve	Replaced valve body due to degradation from normal inservice conditions. Valve did not fail but body replaced as preventative measure. Reference CR 2006101940. Additionally, R&R Plan for WO 1041660402 covered by plan in WO1041660401.		3/27/2006	1041660401

Code Class	Repair Replacement Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found during Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
3	Replacement	1.5" Globe Valve	Replaced valve body due to degradation from normal inservice conditions. Valve did not fail but body was replaced as preventative measure. Reference CR 2006101756. Additionally, R&R Plan for WO 1060577401 covered by plan in WO1040683901.		3/27/2006	1040683901
3	Replacement	1.5" Globe Valve	Replaced valve body due to degradation from normal inservice conditions. Valve did not fail but body was replaced as preventative measure. Reference CR 2006101756. Additionally, R&R Plan for WO 1060577301 covered by plan in WO1040683801.		3/27/2006	1040683801
3	Replacement	8" Gate Valve	Replaced valve 1P41-F050 due to normal inservice degradation. Wedge seating surface was pitted, and the seat pattern was high on the wedge, as well as the wedge guides in the body were worn. This valve is the outboard PCIV for return PSW piping for the Drywell Coolers. Reference CR 2004102391.		3/27/2006	1040404901

Code Class	Repair Replacement Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found during Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
3	Replacement	6" Swing Check Valve	Replaced swing check valve bonnet due to thinning as noted by spot 3/4" UT thickness measurement. Ref CR 2004102026.		3/22/2006	1040386601
2	Replacement	20" Swing Check Valve	Replaced disc of swing check valve due to worn conditions of spud threads to which nut fastens to join and hold disc to the swing check valve arm. Conditions noted during IST check valve internals examination per 42SV-SUV-040-1. Reference CRs 2005108188 and 2005108184.		8/18/2005	1040188601
3	Replacement	18" Butterfly Valve	Replaced 4 studs due to condition noted during disassembly and inspection for leakage past valve seat noted on CR 2002000652. Ref CR 2006102647.		3/6/2006	1020025301

Enclosure 2

Edwin I Hatch Nuclear Plant

**ASME Section XI Code Case N-532-1 OAR-1 Owner's Activity Reports for the
beginning of the 4th Interval ISI activities (Interval 4, Period 1, Outage 1)**

PLANT HATCH UNIT 1, 1R22a OUTAGE

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number 1-4-1-1 (Unit 1, 4th Interval, 1st Period, 1st Report)

Owner Southern Nuclear Operating Co. (as agent for Georgia Power Company), 40 Inverness Center Parkway, Birmingham, AL 35242
(Name and Address of Owner)

Plant Edwin I. Hatch Nuclear Plant, P. O. Box 2010, Baxley, Georgia 31515
(Name and Address of Plant)

Unit No. 1 Commercial service date 12/31/75 Refueling outage no. 1R22a
(if applicable)

Current inspection interval 4th
(1st, 2nd, 3rd, 4th, other)

Current inspection period 1st
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plan 2001 Edition through the 2003 Addenda

Date and revision of inspection plan 2/06/06, Revision 1

Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan Same

CERTIFICATE OF CONFORMANCE

I certify that the statements made in the Owner's Activity Report are correct, and that the exams, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

Certificate of Authorization No. N/A Expiration Date N/A
(if applicable)

Signed [Signature] Date 6/26/2006
Owner or Owner's Designee, Title AGMPS

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Georgia and employed by Hartford Steam Boiler Inspection & Insurance Company of Hartford, CT have inspected the items described in this Owner's Activity Report, for Refueling Outage 1R22a, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the exams, tests, repairs, replacements, evaluations and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions GA 572
Inspector's Signature National Board, State, Province, and Endorsements

Date 6/27/06

This form (E00127) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300. (Modified for an outage frequency versus a period frequency)

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22a OUTAGE – 4th INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 4	Total Exams Credited for Period 1	Total Exams Credited (%) for Period 1	Total Exams Credited (%) To Date for Interval 4 (See Note 2)	Remarks (See Note 1)
B-A/B1.11	0	0	0%	0%	Relief to be submitted to the NRC to delete exams per BWRVIP-05.
B-A/B1.12	12	0	0%	0%	Fourth Interval exams to be completed in the 1 st Period as allowed by 10 CFR 50.55a(g)(6)(ii) (A).
B-A/B1.21	2	0	0%	0%	Exams are deferrable until the end of the 4th interval. Weld C-8 is completely inaccessible and relief will be requested.
B-A/B1.22	16	0	0%	0%	There is a total of 22 B1.22 welds; however, six welds are inaccessible and relief will be requested.
B-A/B1.30	1	1	100%	100%	
B-A/B1.40	1	1	100%	100%	
B-A Total	NA	NA	NA	NA	Each item is either deferrable, required to be examined in a specific period, or there are less than 3 items. Therefore, N-598 percentages do not apply.
B-D Total	56	22	39%	39%	There are 29 nozzles; however, the N15 RPV drain nozzle is located inside the support skirt and is not accessible. Relief will be required. Therefore, 28 nozzles will be examined. Exams were scheduled per Table IWB-2500-1, Category B-D, Footnote 2 which states, "At least 25% but no more than 50% (credited) of the nozzles shall be examined by the end of the first inspection period, and the remainder by the end of the inspection interval."
B-F/B5.20	2	1	50%	50%	NRC approved ISI Alternative ISI-ALT-3 allows exemption from Code exam percentage requirements of dissimilar metal welds (4" and larger in diameter). In lieu of the Code requirements, the number of welds examined and the frequency of the exam are as specified in BWRVIP-075. There are four small diameter welds in this B-F scope. However, the N16B Safe-End to nozzle weld (1C41-1INST-2B-1) and the N15 Bottom Head Drain Nozzle to Safe-End weld are inaccessible and relief is required. Therefore, the scope shown for this table is two.

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22a OUTAGE – 4th INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 4	Total Exams Credited for Period 1	Total Exams Credited (%) for Period 1	Total Exams Credited (%) To Date for Interval 4 (See Note 2)	Remarks (See Note 1)
B-G-1/B6.10	52	0	0%	0%	Closure Head Nuts - Deferrable
B-G-1/B6.20	52	0	0%	0%	Studs - Use Code Case N-307-3- Deferrable
B-G-1/B6.40	52	0	0%	0%	Threads in Flange- Deferrable
B-G-1/B6.50	52	0	0%	0%	Washers- Deferrable
B-G-1/B6.50B	4	0	0%	0%	Bushings – Only required to be examined when studs are removed.
B-G-1/B6.180 B-G-1/B6.190 B-G-1/B6.200	NA	0	0%	0%	These exams are required only when the B-L-2 visual exam of the internal surfaces of a pump are performed.
B-G-1 Total	NA	NA	NA	NA	
B-G-2/B7.50	NA	1	NA	NA	Piping Flange Bolting examinations are required only when a flange is disassembled. Exams may be limited to one flange in a group of bolted connections similar in design, size, function, and service. An examination was performed on 1B21-F013B flange bolting during 1R22a, thus completing 4 th Interval Group 2 flange bolting requirements.
B-G-2/B7.70	NA	2	NA	NA	Examination of pressure-retaining valve bolting is required only when Category B-M-2 visual exams are performed (and the valve is disassembled). Valve bolting was examined on 1B21-F028D during 1R22a, thus completing 4 th Interval Group 4 valve bolting requirements. 1B21-F013B valve bolting was examined, thus completing 4 th Interval Group 3 valve bolting requirements.
B-G-2/B7.80	NA	NA	NA	NA	CRD bolting is required to be examined if bolting is reused.
B-G-2 Total	NA	3	NA	NA	

FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22a OUTAGE – 4th INTERVAL
TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 4	Total Exams Credited for Period 1	Total Exams Credited (%) for Period 1	Total Exams Credited (%) To Date for Interval 4 (See Note 2)	Remarks (See Note 1)
B-J/B9.11 B-J/B9.21 B-J/B9.31 B-J/B9.32	88	6	7%	7%	NRC approved ISI Alternative ISI-ALT-3 grants relief from Code examination percentage requirements for stainless steel Category B-J piping (4" and larger in diameter). Therefore, only carbon steel welds are shown in this row.
B-J/B9.21 B-J/B9.32 B-J/B9.40	29	1	3%	3%	Small diameter welds previously exempted during the 3 rd Interval per IWB-1220(a).
B-J Total	117	7	6%	6%	
B-K Total	14	0	0%	0%	
B-L-2	NA	NA	NA	NA	Category B-L-2 applies only to the 2 RC pumps. Only one pump in a group is required to be examined per B-L-2 requirements.
B-M-2	NA	2	NA	NA	Category B-M-2 applies to 17 groups of valves. Examination of the valve internal surface is limited to one valve in a group and is required only when a valve is disassembled for maintenance or repair. During 1R22a, 1B21-F028D was examined thus completing 4 th Interval Group 4 B-M-2 requirements. 1B21-F013B valve bolting was also examined, thus completing 4 th Interval Group 3 B-M-2 requirements.
B-N-1/B13.10	3	0	0%	0%	Examination of vessel interior performed once per period.
B-N-2/ B13.20 B-N-2/ B13.30	38	0	0%	0%	Exams are deferrable until the end of the 4th interval. Item numbers are combined because a VT-1 is performed on all attachment welds.
B-N-2/B13.40	NA	0	NA	NA	Exams are deferrable until the end of the 4 th Interval; therefore, N-598 percentages do not apply. The accessible surfaces of the Shroud, Shroud Support Ledge, Control Rod Housing & Stub Tubes, are examined during the 10-year interval.

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22a OUTAGE – 4th INTERVAL**

**TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

Exam Category/Item Number (See Note 3)	Total Exams Required for Interval 4	Total Exams Credited for Period 1	Total Exams Credited (%) for Period 1	Total Exams Credited (%) To Date for Interval 4 (See Note 2)	Remarks (See Note 1)
B-O/B14.10	8	0	0%	0%	
B-P/B15.10	NA	NA	NA	NA	A system leakage test is performed each refueling outage.
C-A/C1.10 C-A/C1.20	3	0	0%	0%	
C-B/C2.21 C-B/C2.22	4	0	0%	0%	There are two nozzles on each of the two RHR Heat Exchangers and there are two exams per nozzle. The Code requires that the equivalent of one RHR Heat Exchanger be examined.
C-C Total	22	3	14%	14%	
C-F-2/C5.51 C-F-2/C5.81	91	14	15%	15%	
C-H/C7.10	NA	NA	NA	NA	These operating pressure tests are performed each inspection period; therefore they are performed three times per inspection interval. Therefore, reporting percentage completion for these pressure tests is not appropriate.
F-A/Total	151	15	10%	10%	

**FORM OAR-1 OWNER'S ACTIVITY REPORT
PLANT HATCH UNIT 1, 1R22a OUTAGE – 4th INTERVAL
TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS**

NOTES

- Note 1:** During 1R22, Interval 3 requirements were completed, and Interval 4 requirements were initiated. For clarity of reporting, Interval 3 requirements have been designated and reported as 1R22, and Interval 4 requirements have been designated and reported as 1R22a.
- Note 2:** NRC approved Code Case N-598 is used to determine the required percentage of exams.
- Note 3:** Those Category/Items not shown in Table 1 are not applicable to Plant Hatch.

TABLE 2
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE

NO APPLICABLE ITEMS FOR 1R22a (1-4-1-1)

TABLE 3
Abstract of Repairs, Replacements, or Corrective Measures
Required for Continued Service

FOR 1R22/1R22a, ALL APPLICABLE ITEMS HAVE BEEN INCLUDED IN OAR-1,
TABLE 3 FOR 1R22 (1-3-3-2)