

July 19, 2006

Mr. R. T. Ridenoure
Vice President - Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT 1 - REQUEST FOR ADDITIONAL
INFORMATION (RAI) RELATED TO REVISING THE FORT CALHOUN
STATION UPDATED SAFETY ANALYSIS REPORT (TAC NO. MC8857)

Dear Mr. Ridenoure:

By letter dated October 31, 2005, Omaha Public Power District (OPPD) submitted an amendment request to revise the Fort Calhoun Station, Unit 1, Updated Safety Analysis Report for the radiological consequences of events affected by the planned 2006 replacement of the steam generators and pressurizer.

The U.S. Nuclear Regulatory Commission staff has reviewed OPPD's submittal and has determined that additional information is needed to complete our review. A request for additional information is enclosed. This request was discussed with Thomas Matthews of your staff on July 5, 2006, and it was agreed that a response would be provided within 30 days of receipt of this letter.

If you have any questions, please contact me at (301) 415-1445.

Sincerely,

/RA/

Alan B. Wang, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: RAI

cc w/encl: See next page

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ADAMS ACCESSION NO.: **ML061930417**

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DATE	07/17/06	07/17/06	07/19/06

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING UPDATED SAFETY ANALYSIS REPORT
RADIOLOGICAL CONSEQUENCE ANALYSIS
OMAHA PUBLIC POWER DISTRICT
FORT CALHOUN STATION, UNIT 1
DOCKET NUMBER 50-285

By letter dated October 31, 2005, Omaha Public Power District (OPPD) submitted an amendment request to revise the Fort Calhoun Station, Unit 1 (FCS), Updated Safety Analysis Report (USAR) for the radiological consequences of events affected by the planned 2006 replacement of the steam generator and pressurizer. The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed OPPD's submittal and has determined that the following additional information is needed to complete our review:

1. The NRC staff notes that the atmospheric dispersion factors (χ/Q values) provided in the October 31, 2005, license amendment request (LAR) are the same as those approved in the safety evaluation associated with FCS Amendment No. 201. Amendment No. 201 implements use of the alternative source term in the FCS design-basis accident dose assessments. Therefore, for each design-basis accident or event addressed in the October 31, 2005, LAR, please discuss any differences between the two sets of release scenarios. If there are differences, please justify how the differences do not impact the applicability of the χ/Q values associated with Amendment No. 201 to the dose assessment associated with the October 31, 2005, LAR.
2. Which water mass inventories (both primary and secondary) were used in calculating coolant activities? Please indicate if the used values are the minimum, nominal or maximum.
3. The submitted S-RELAP5 analysis indicates 0 percent/0 percent fraction of failed/melted fuel. Is this a change from the analysis of record?
4. Attachment 4 (AREVA document 86-5064251-00) describes the thermal-hydraulic input in support of the consequence analysis. Please confirm that the main steam safety valves and atmospheric dump valves actuation setpoints used are those specified in the technical specifications.
5. Presumably, the cumulative steam releases used in the Attachment 6 (Stone & Webster report) came from the Attachment 4 (AREVA document). However, it is not immediately obvious which tables in Attachment 4 were used to generate Tables 7.6 - 7.9 in Attachment 6. Please clarify.

Ft. Calhoun Station, Unit 1

cc:

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April 2006