

Dominion Nuclear Connecticut, Inc.  
Millstone Power Station  
Rope Ferry Road  
Waterford, CT 06385



**Dominion™**

JUN 30 2006

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Serial No.	06-513
MPS Lic/GJC	R0
Docket No.	50-336
License No.	DPR-65

**DOMINION NUCLEAR CONNECTICUT, INC.**  
**MILLSTONE POWER STATION UNIT 2**  
**LICENSEE EVENT REPORT 2006-005-00**  
**INADVERTENT ACTUATION OF 'A' MOTOR DRIVEN**  
**AUXILIARY FEEDWATER PUMP**

This letter forwards Licensee Event Report (LER) 2006-005-00, documenting an event that occurred at Millstone Power Station Unit 2, on May 2, 2006. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(iv)(A), as an invalid actuation of a system listed in 10 CFR 50.73(a)(2)(iv)(B).

If you have any questions or require additional information, please contact Mr. David W. Dodson at (860) 447-1791, extension 2346.

Very truly yours,

J. Alan Price  
Site Vice President - Millstone

IE22

Attachments: 1

Commitments made in this letter: None.

cc: U.S. Nuclear Regulatory Commission  
Region I  
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Mr. V. Nerses  
Senior Project Manager  
U.S. Nuclear Regulatory Commission  
One White Flint North  
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Mr. S. M. Schneider  
NRC Senior Resident Inspector  
Millstone Power Station

**Attachment 1**

**Licensee Event Report 2006-005-00**  
**Inadvertent Actuation of 'A' Motor Driven Auxiliary Feedwater Pump**

**Millstone Power Station Unit 2**  
**Dominion Nuclear Connecticut, Inc. (DNC)**

NRC FORM 366 (6-2004)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104		EXPIRES 06/30/2007		
<b>LICENSEE EVENT REPORT (LER)</b> (See reverse for required number of digits/characters for each block)								
1. FACILITY NAME Millstone Power Station - Unit 2				2. DOCKET NUMBER 05000336		3. PAGE 1 OF 3		
4. TITLE Inadvertent Actuation of 'A' Motor Driven Auxiliary Feedwater Pump								
5. EVENT DATE			6. LER NUMBER		7. REPORT DATE		8. OTHER FACILITIES INVOLVED	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MO	DAY	YEAR
05	02	2006	2006 - 005 - 00			06	30	2006
							FACILITY NAME	
							DOCKET NUMBER 05000	
							FACILITY NAME	
							DOCKET NUMBER 05000	
9. OPERATING MODE		11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)						
1		20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)
10. POWER LEVEL		20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)
100		20.2203(a)(1)		50.36(c)(1)(i)(A)		X 50.73(a)(2)(iv)(A)		73.71(a)(4)
		20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)
		20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		OTHER
		20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or In NRC Form 366A
		20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)		
		20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)		
		20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)		
		20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)		
12. LICENSEE CONTACT FOR THIS LER								
NAME David W. Dodson, Supervisor Nuclear Station Licensing						TELEPHONE NUMBER (Include Area Code) 860-447-1791		
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT								
CAUSE	SYSTEM	COMPONENT	MANU- FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU- FACTURER
14. SUPPLEMENTAL REPORT EXPECTED						15. EXPECTED		
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE).						<input checked="" type="checkbox"/> NO		
						SUBMISSION DATE		
						MONTH		
						DAY		
						YEAR		
16. ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)								
<p>On May 2, 2006 at approximately 1430, with Millstone Power Station Unit 2 at 100% power, the 'A' Motor Driven Auxiliary Feedwater (MDAFW) pump automatically started during restoration following surveillance testing. The operators immediately realized that this was an unexpected response and secured the pump. Since the input parameters for the initiation of automatic auxiliary feed (i.e., steam generator level and pressurizer pressure), were within the normal ranges, the start signal for the 'A' MDAFW pump was not valid.</p> <p>The actuation of the 'A' MDAFW pump is reportable under the provisions of 10CFR50.73(a)(2)(iv)(A) as an invalid actuation of a system listed in 10CFR50.73(a)(2)(iv)(B).</p> <p>The cause of this event was personnel error, I&amp;C technicians did not ensure proper procedure place keeping protocol was followed and inadvertently skipped two steps in the procedure.</p> <p>There were no safety consequences associated with this event.</p>								

**LICENSEE EVENT REPORT (LER)**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
Millstone Power Station - Unit 2	05000336	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		2006	-- 005	-- 00	

**NARRATIVE** (If more space is required, use additional copies of NRC Form 366A) (17)**1. Event Description**

Millstone Power Station (MPS) Unit 2 Auxiliary Feedwater (AFW) System [BA] consists of two Motor Driven Auxiliary Feedwater (MDAFW) Pumps [P] and a Turbine Driven Auxiliary Feedwater (TDAFW) Pump [P]. All of the pumps provide feedwater to both of the Steam Generators (SG) [AB]. The normal use of the AFW System is to supply feedwater during plant startups until main feedwater is available. The system also provides for decay heat removal until the Reactor Coolant System (RCS) is placed on Shutdown Cooling. During power operations, the AFW System is aligned for automatic start of the MDAFW Pumps on either low SG level or high Pressurizer (PZR) [AB] pressure.

On May 2, 2006 at approximately 1430, with MPS Unit 2 at 100% power, the 'A' MDAFW pump started after the pump control switch was placed in the 'Normal' position following surveillance testing by Instrument & Controls (I&C) personnel.

The I&C surveillance was being performed to satisfy the monthly Engineering Safety Features (ESF) Channel Functional Test requirements identified in Technical Specification Surveillance Requirement 4.3.2.1.1, Table 4.3-2 for AFW (for SG Low Level and Automatic Actuation Logic) and SG Blowdown (for SG Low Level). In addition, this testing satisfies the monthly Reactor Protection (RPS) Channel Functional Test requirements identified in Technical Specification Surveillance Requirement 4.3.1.1.1, Table 4.3-1 for SG Water Level - Low.

The event occurred during the performance of the 'Time Delay Relay Test' section of the surveillance. This section verifies that after receiving a start signal from the system logic, the time delay relays initiate a signal to start the 'A'/'B' MDAFW pumps after approximately 3 minutes and 25 seconds. During this relay testing I&C personnel failed to remove a jumper as required by the surveillance procedure. This resulted in the 'A' MDAFW pump starting as soon as the operator restored the pump control switch to the 'Normal' position following completion of testing. The operators immediately realized that this was an unexpected response and secured the pump. Since the input parameters for the initiation of automatic auxiliary feedwater (i.e., SG level and PZR pressure), were within the normal ranges, the start signal for the 'A' MDAFW pump was not valid.

The actuation of the 'A' MDAFW pump is reportable under the provisions of 10CFR50.73(a)(2)(iv)(A) as an invalid actuation of a system listed in 10CFR50.73(a)(2)(iv)(B).

**2. Cause**

The cause of this event was personnel error in that the I&C technician did not adequately adhere to the requirements of a Continuous Use procedure as defined by the Dominion administrative procedure on procedure adherence and usage. This allowed the technician to miss two restoration steps in the surveillance procedure resulting in the inadvertent start of the 'A' MDAFW pump.

**3. Assessment of Safety Consequences**

The AFW System is normally used to supply feedwater during plant startups until main feedwater is available. The system also provides for decay heat removal until the RCS is placed on Shutdown Cooling. During power operations, the AFW System is aligned for automatic initiation on either low SG level or high PZR pressure. The AFW Pumps pump cold water into the SG from the condensate storage tank (CST). This could have lowered reactor coolant temperatures, causing a corresponding drop in PZR level and pressure. Additionally, the cooler RCS water could have added positive reactivity to the core, potentially causing a power excursion, however the operators immediately realized what had occurred and secured the pump within 30 seconds of the pump start. Therefore, there were no safety consequences associated with this event.

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**NARRATIVE** (If more space is required, use additional copies of NRC Form 366A) (17)

4. Corrective Action

Corrective actions are being taken in accordance with the station's corrective action program.

5. Previous Occurrences

None

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].