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Attn: Document Control Room
U.S. Nuclear Regulatory Commission
Washington, DC 20555

UFTR Emergency Plan
Revision 14 – Addendum

University of Florida Training Reactor (UFTR)
Facility License R-56, Docket No. 50-83

By letter dated June 19, 2006, the Revision 14 package to the approved UFTR Emergency Plan was submitted for review. As noted in the submittal letter, the changes are considered relatively minor in nature; they are all associated with the conversion from using high enriched uranium (HEU) to low enriched uranium (LEU) fuel in the UFTR. That submittal consists of a set of updates and revisions to the title page, page v, pages 1-1, 1-6, 1-12, 1-13, 1-14, 5-1 and 6-1.

The enclosed package is an addendum to the June 19, 2006 submittal consisting of one additional change to page 10-4 in the approved UFTR Emergency Plan. As usual, the new page is marked with a vertical line in the right margin for easy location of the specific change in Table 10.2, UFTR Safety Systems Operability Tests. This change is required for consistency and is again associated with the conversion from using high enriched uranium to low enriched uranium fuel in the UFTR. Specifically, the conversion analysis includes one additional test added to Tech Spec Table 3.2, Safety System Operability Tests, expressly requiring the test of the high average primary coolant inlet temperature with the daily checkout. Since Table 10.2 in the Emergency Plan is essentially the same as Table 3.2 in the Technical Specifications, this change is required as part of the conversion to low enriched uranium fuel.

As indicated, this change has been reviewed by UFTR management and by the Reactor Safety Review Subcommittee as part of the Tech Spec changes and does not decrease the effectiveness of the UFTR Emergency Plan. Its omission in the earlier submittal was an oversight. In general, this change and the earlier submittal changes update the Plan to reflect the conversion from HEU to LEU fuel and make the Plan better suited to assure a proper response to emergencies at the University of Florida Training Reactor.

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If there are any questions, please let us know. Thank you for your consideration.

Sincerely,

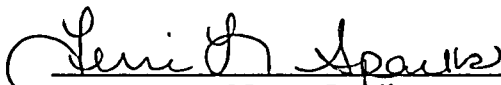


William G. Vernetson
Director of Nuclear Facilities

WGV/dms
Enclosures

cc (letter only): A. Adams, Sr. Project Manager, NRC
Reactor Safety Review Subcommittee

Sworn and subscribed this 29th day of June 2006


Notary Public

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Table 10.2

UFTR Safety System Operability Tests

Component or Scram Function	Frequency
Log-N period channel Power level safety channels	Before each reactor startup following a shutdown in excess of 6 hr, and after repair or deenergization caused by a power outage
10% reduction of safety channels high voltage	4/year (4-month maximum interval)
Loss of electrical power to console	4/year (4-month maximum interval)
Loss of primary coolant pump power	4/year (4-month maximum interval)
Loss of primary coolant level	4/year (4-month maximum interval)
Loss of primary coolant flow	4/year (4-month maximum interval)
High average primary coolant inlet temperature	With daily checkout
High average primary coolant outlet temperature	With daily checkout
Loss of secondary coolant flow (at power levels above 1 kW)	With daily checkout
Loss of secondary coolant well pump power	4/year (4-month maximum interval)
Loss of shield tank water level	4/year (4-month maximum interval)
Loss of power to vent system and dilution fan	4/year (4-month maximum interval)
Manual scram bar	With daily checkout