

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

February 23, 2006

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
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Rockville, MD 20852-2738

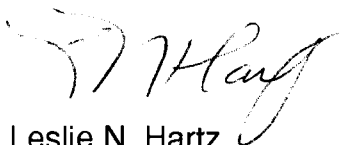
Serial No. 06-014A
NL&OS/PRW R0
Docket Nos. 50-280/281
License Nos. DPR-32/37

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
PROPOSED TECHNICAL SPECIFICATION CHANGE AND SUPPORTING SAFETY
ANALYSES REVISIONS TO ADDRESS GENERIC SAFETY ISSUE 191
REVISED MARKED-UP TECHNICAL SPECIFICATIONS PAGES

In a letter dated January 31, 2006 (Serial No. 06-014), Virginia Electric and Power Company (Dominion) submitted a Technical Specifications (TS) change request for Surry Power Station Units 1 and 2. The proposed change was submitted as part of Dominion's resolution to NRC Generic Safety Issue 191. In Attachment 2 to that letter, Dominion provided marked-up TS pages that included the requested changes. Subsequent to the submittal of the proposed TS change, it was discovered that two of the marked-up TS pages contained administrative errors. Specifically, TS Table 3.7-6, Item 9a, should have been marked-up to note the change of the Functional Unit description from "RWST Level-Low" to "RWST Level-Low-Low". In addition, the phrase "from 70°F to 95°F service water temperatures" in the last paragraph on TS page TS 3.8-4 was incorrectly inserted between the words "sloped line". The phrase should have been inserted after these words instead. Therefore, please replace the marked-up TS pages TS 3.7-26 and TS 3.8-4 included in Attachment 2 of our previous submittal with the corrected marked-up TS pages provided in the attachment to this letter. The discussion of the TS changes and the typed proposed TS pages included in Attachments 1 and 3 of the previous submittal, respectively, are correct and therefore do not require revision.

If you have any questions regarding this submittal, please contact Mr. Paul R. Willoughby at (804) 273-3572.

Very truly yours,



Leslie N. Hartz
Vice President – Nuclear Engineering

Attachment

cc: U. S. Nuclear Regulatory Commission
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COMMONWEALTH OF VIRGINIA)
)
COUNTY OF HENRICO)

The foregoing document was acknowledged before me, in and for the County and Commonwealth aforesaid, today by Leslie N. Hartz, who is Vice President – Nuclear Engineering, of Virginia Electric and Power Company. She has affirmed before me that she is duly authorized to execute and file the foregoing document in behalf of that Company, and that the statements in the document are true to the best of her knowledge and belief.

Acknowledged before me the 23rd day of February, 2006.

My Commission Expires: May 31, 2006

Vicki L. Hulse
Notary Public

(SEAL)

ATTACHMENT

CORRECTED MARKED-UP TECHNICAL SPECIFICATIONS PAGES

TS 3.7-26 AND TS 3.8-4

**Virginia Electric and Power Company
(Dominion)
Surry Power Station Units 1 and 2**

TABLE 3.7-4
ENGINEERED SAFETY FEATURE SYSTEM INITIATION LIMITS INSTRUMENT SETTING

<u>No.</u>	<u>Functional Unit</u>	<u>Channel Action</u>	<u>Setting Limit</u>
6	AUXILIARY FEEDWATER		
	a. Steam Generator Water Level Low-Low	Aux. Feedwater Initiation S/G Blowdown Isolation	$\geq 14.5\%$ narrow range
	b. RCP Undervoltage	Aux. Feedwater Initiation	$\geq 70\%$ nominal
	c. Safety Injection	Aux. Feedwater Initiation	All S.I. setpoints
	d. Station Blackout	Aux. Feedwater Initiation	$\geq 46.7\%$ nominal
	e. Main Feedwater Pump Trip	Aux. Feedwater Initiation	N.A.
7	LOSS OF POWER		
	a. 4.16 KV Emergency Bus Undervoltage (Loss of Voltage)	Emergency Bus Separation and Diesel start	≥ 2975 volts and ≤ 3265 volts with a 2 (+5, -0.1) second time delay
	b. 4.16 KV Emergency Bus Undervoltage (Degraded Voltage)	Emergency Bus Separation and Diesel start	≥ 3830 volts and ≤ 3881 volts with a 60 (± 3.0) second time delay (Non CLS, Non SI) 7 (± 0.35) second time delay (CLS or SI Conditions)
8	NON-ESSENTIAL SERVICE WATER ISOLATION		
	a. Low Intake Canal Level	Isolation of Service Water flow to non-essential loads	23 feet-6 inches
9	RECIRCULATION MODE TRANSFER		
	a. RWST Level-Low <u>- Low</u>	Initiation of Recirculation Mode Transfer System	$\geq 11.25\%$ $\leq 15.75\%$
10	TURBINE TRIP AND FEEDWATER ISOLATION		
	a. Steam Generator Water Level High-High	Turbine Trip Feedwater Isolation	$\leq 80\%$ narrow range

INSERT 2

Amendment Nos. 224 and 224-

TS 3.7-26
03-12-01
2

- (3) assuring that environmental conditions will not preclude access to close the valves and
4) that this administrative or manual action will prevent the release of radioactivity outside the containment.

The Reactor Coolant System temperature and pressure being below 350°F and 450 psig, respectively, ensures that no significant amount of flashing steam will be formed and hence that there would be no significant pressure buildup in the containment if there is a loss-of-coolant accident. Therefore, the containment internal pressure is not required to be subatmospheric prior to exceeding 350°F and 450 psig.

The allowable value for the containment air partial pressure is presented in TS Figure 3.8-1 for service water temperatures from 25 to 95°F. The RWST water shall have a maximum temperature of 45°F.

The horizontal limit line in TS Figure 3.8-1 is based on ^{upper} ~~LOCA~~ peak calculated pressure criteria, and the sloped line is based on ^{MSLB} ~~LOCA~~ depressurization, subatmospheric peak pressure criteria.

from 70°F to 95°F service water temperatures