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Subject: **NEDO-33201, Revision 1, "ESBWR Probabilistic Risk Assessment,"
Sections 2 through 6**

Enclosure 1 contains the subject partial ESBWR Probabilistic Risk Assessment (PRA) document (Revision 1). The complete PRA document (including the sections transmitted herein) will be issued in accordance with the Reference 1 letter.

If you have any questions about the information provided here, please let me know.

Sincerely,

David H. Hinds
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Reference:

1. MFN 06-015, Letter from David H. Hinds to U.S. Nuclear Regulatory Commission, *Submittal Schedule for Documents Related to ESBWR Probabilistic Risk Assessment*, January 16, 2006

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Enclosure:

1. MFN 06-049 – NEDO-33201, Revision 1, “ESBWR Probabilistic Risk Assessment.”

- Section 2 – Initiating Events
- Section 3 – Accident Sequence Analysis
- Section 4 – System Analysis
- Section 5 – Data Analysis
- Section 6 – Human Reliability Analysis

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Enclosure 1

ENCLOSURE 1

MFN 06-049

**NEDO-33201, Revision 1, "ESBWR Probabilistic Risk
Assessment:"**

- **Section 2 – Initiating Events**
- **Section 3 – Accident Sequence Analysis**
- **Section 4 – System Analysis**
- **Section 5 – Data Analysis**
- **Section 6 – Human Reliability Analysis**

2 INITIATING EVENTS

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2 INITIATING EVENTS

2.1 INTRODUCTION

Event trees are used in a PRA to model discrete accident sequences. An accident sequence is comprised of an initiating event followed by challenges to system successes and failures that lead to either safe, stable plant conditions or to a plant condition that is considered unacceptable. An initiating event is any occurrence that disrupts normal plant operations sufficiently to require a reactor trip by either automatic or manual action. An initiating event may occur due to a random component failure or a human action, thus requiring plant systems to respond to maintain the unit in a safe, stable condition. The initiating event marks the starting point of the accident sequence analysis.

One of the first and basic steps in a PRA is the identification and quantification of the initiating events to be used in the sequence analysis. Initiating events have historically been broadly classified as either "internal" or "external" events. The initiating events discussed in this section are limited to at-power internal initiating events, that is, those initiating events occurring during power operation either as a direct result of equipment failure, or as the result of errors while performing maintenance, testing, or any other operator action. External initiating events (e.g., seismic events, internal floods) and initiating events during shutdown are discussed in other sections.

Initiating event analysis involves the following two major steps:

- (1) Identification and grouping
- (2) Frequency quantification

Each of these key steps is discussed below.

2.2 INITIATING EVENT IDENTIFICATION AND CATEGORIZATION

In order to identify the list of initiating events to be considered in the analysis, the following steps are taken:

- (1) Identification of the set of initiating events applicable to the ESBWR plant by reviewing NUREG/CR-5750 (Reference 2-1)
- (2) Analysis of the failure of individual ESBWR systems that could result in additional initiating events

Human error induced initiating events are considered for all of these analyses.

Individual initiating events requiring the same response from front line and auxiliary systems and operators are combined into initiating event categories. Combining initiating events into categories reduces the number of event trees that need to be developed. The four major groups of internal initiating events are:

- (1) Transients
- (2) Loss of preferred power (LOPP)
- (3) Loss of coolant accidents (LOCA)
- (4) Special initiators

These major groups are consistent with the ASME PRA Standard. The categorization of initiating events within each of these major groups is discussed below.

The postulated initiating events to be addressed in the PRA are derived from a review of BWR operating experience, as summarized in the NUREG/CR-5750 (Reference 2-1). NUREG/CR-5750 builds upon previous industry studies with similar objectives, such as EPRI Report NP-2230 (Reference 2-2) published in 1982 and NUREG/CR-3862 (Reference 2-3), published in 1985. The NUREG/CR-5750 categories are applicable, in general, to all BWR (and PWR) plants that are currently in operation. The NUREG/CR-5750 initiating event categories are listed in Table 2.2-1.

Many systems in the ESBWR design differ from those for the operating BWR plants. In addition, the ESBWR design features several innovative systems; as such, certain NUREG/CR-5750 categories are not directly applicable to the ESBWR. These are indicated in Table 2.2-1.

2.2.1 Transients

The transient initiating event categories are developed based on the following considerations:

- Effect of the initiator on feedwater and power conversion system (PCS) availability.
- Effect of the initiator on systems required to prevent core damage or containment failure.
- Successful configuration (e.g., mode of operation, number of pumps in operation) of mitigating systems.
- Effect of the initiator on operator response.

The resulting transient initiating event categories for the ESBWR PRA are as follows:

- General Transient
- Transient with Power Conversion System (PCS) Unavailable
- Loss of Feedwater
- Inadvertent Opening of SRV (IORV)

The results of this analysis and the NUREG/CR-5750 initiating events grouped into each ESBWR PRA initiating event category are summarized in Table 2.2-2.

2.2.2 Loss of Preferred Power

The Loss of Preferred Power (LOPP) initiating event category of the ESBWR PRA corresponds to the NUREG/CR-5750 category (B1) Loss of Offsite Power.

2.2.3 LOCAs

Loss of coolant accident (LOCA) initiator categories are defined based on approaches in previous BWR PRAs, plant specific configuration, and success criteria.

LOCA initiators are classified according to the following main factors:

- (1) Level position: steam breaks (for pipes above Level 3) or liquid breaks (for pipes below Level 3)
- (2) Break size: large, medium, or small nominal classifications are used
- (3) Pipe function: Emergency Core Cooling System (ECCS) line break or non-ECCS line break (the former affects ECCS operability)
- (4) Location: inside/outside containment breaks (breaks outside containment potentially can be isolated)
- (5) Pipe class: LOCAs can be postulated for situations in which valves between the reactor coolant pressure boundary (RCPB) and non-RCPB segments fail, propagating high pressure to low pressure design pipes
- (6) Coolability: coolable LOCAs (all the above categories) or non-coolable LOCAs (vessel rupture)

2.2.3.1 LOCAs Inside Containment

LOCAs inside containment initiators are failures of the RCPB that occur inside the containment during power operation. LOCAs inside containment are categorized into the following LOCA sizes:

- Large: RPV depressurizes sufficiently (without the need for additional depressurization methods) to allow GDCS injection prior to uncovering the top of active fuel (TAF).

- Medium (Steam): RPV depressurizes sufficiently (without the need for additional depressurization methods) to allow Fire Protection System (FPS) injection prior to uncovering TAF.
- Medium (Liquid): CRD insufficient makeup source. RPV depressurization required for low pressure injection.
- Small: CRD sufficient makeup source. RPV depressurization required for low pressure injection.

The lower limit of LOCAs is defined as a primary system crack or leakage small enough to be managed by normal operation water makeup and containment cooling systems. The Turbine Trip initiating event category models this level of RCPB leakage.

An analysis was performed to establish the appropriate LOCA classification for each line connected to the RCPB. Table 2.2-3 summarizes the number, the diameter, and the nozzle elevation of lines connected to the RCPB. For each line, Table 2.2-3 classifies the breaks into Large, Medium, and Small LOCA sizes according to the pipe cross-section area, and whether it is a liquid or steam break. Criteria for the classifications are:

- Pipes connected to the RPV at elevations above Level 3 are classified as steam breaks (even though a liquid phase is initially discharged through the break).
- Pipes connected to the RPV at elevations below Level 3 are classified as liquid breaks.

The Emergency Procedure Guidelines drive the operator to maintain reactor vessel level between Level 3 and Level 8 after reactor trip, so any LOCA below Level 3 results in the continual release of water through the break, provided that an injection system into the vessel is available.

As such, the LOCA categories in terms of location and diameter are as follows:

- Large (steam): >305 mm (12") dia. piping above Level 3
- Large (liquid): >305 mm (12") dia. Piping below Level 3
- Medium (steam): 264mm (10.4") – 305 mm (12") dia. piping above Level 3
- Medium (liquid): 25 mm (1") – 305 mm (12") dia. piping below Level 3
- Small (steam): <264 mm (10.4") dia. piping above Level 3
- Small (liquid): <25 mm (1") dia. piping below Level 3

The diameter of an orifice or a nozzle in a line, when it exists, is considered in the break size classification.

The initiating event categorization for LOCAs inside containment is summarized below.

Large Break LOCA

The large break LOCA is sufficiently large to depressurize the reactor to permit low pressure system injection, without the need for additional depressurization methods. The ESBWR design does not contain any piping with diameters greater than 12" below Level 3, thus, all breaks

analyzed are steam breaks, (refer to Table 2.3-3). Inadvertent opening of all DPV/SRV are also included in this group.

This initiating event category corresponds to the NUREG/CR-5750 category (G7) Large Pipe Break LOCA.

Medium Break LOCA

- Steam break case - The medium break steam LOCA is large enough to depressurize the reactor sufficiently to permit low pressure system injection (except GDSCS) without the need for additional RPV depressurization methods. No ESBWR lines fall into this category (refer to Table 2.2-3).
- Liquid break case - The flow rate at reactor pressure for a medium break liquid LOCA is greater than the CRD makeup capacity. Depressurization is needed for GDSCS injection to be effective. A number of ESBWR lines fall into this category (refer to Table 2.2-3)

This initiating event category corresponds to the NUREG/CR-5750 category (G6) Medium Pipe Break LOCA.

Small Break LOCA

- Steam break case - The small break steam LOCA is sufficiently small such that RPV depressurization is needed for effective low pressure injection (including LPCI and condensate). ESBWR instrument lines above Level 3 fall into this category (refer to Table 2.2-3). Inadvertent opening of a DPV is included in this group.

The consequences of an inadvertent opening of one SRV (IORV) are similar to the consequences of a small steam LOCA; however, IORV is analyzed separately as a transient because of the steam discharge to the suppression pool versus the drywell.

- Liquid break case - CRD is a viable high pressure injection source, as the flow rate for a small break liquid LOCA is less than the CRD makeup capacity. RPV depressurization is needed for effective low pressure system injection. ESBWR instrument lines below Level 3 fall into this category (refer to Table 2.3-3).

This initiating event category corresponds to the NUREG/CR-5750 category (G3) Small Pipe Break LOCA.

Vessel Rupture

The vessel rupture initiating event category is defined as a random failure of the reactor vessel below Level 3.

2.2.3.2 LOCAs Outside Containment

LOCAs outside containment are defined as breaks in high pressure systems outside the containment. The break can be isolated by closure of the isolation valves (break isolation is addressed in the accident sequence analysis).

This initiating event category corresponds to NUREG/CR-5750 categories (K1) Steam Line Break Outside Containment and (K2) Feedwater Line Break.

2.2.3.3 Interfacing LOCAs

An interfacing LOCA is a break in a low pressure system connected to the reactor vessel as a result of a failure of the low/high pressure barrier leading to an un-isolable loss of coolant outside the containment. This initiating event category corresponds to NUREG/CR-5750 category (N1) Interfacing System LOCA.

2.2.4 Special Initiators

Special initiating events refer to plant-specific systemic malfunctions that can lead to a plant trip. A new initiating category is identified when the event is not associated with any of the categories defined before; otherwise, it is grouped into one of the defined categories and the frequency of the existing category is modified to account for the contribution of the systemic malfunction.

In general, all systems that can influence the parameters involved with the scram or isolation signals (either directly or as a consequence) are analyzed. The list of systems analyzed is as follows:

Front line systems

- Control Rod Drive System (CRD)
- Feedwater / Condensate
- Isolation Condenser System (IC)
- Depressurization System (DPV/SRV)
- Gravity Driven Cooling System (GDCS)
- Fuel and Auxiliary Pool Cooling (FAPC) System
- Reactor Water Cleanup / Shutdown Cooling System (RWCU/SDCS)
- Standby Liquid Control System (SLCS)

Support Systems

- Reactor Component Cooling Water System (RCCW)
- Turbine Component Cooling Water System (TCCW)
- Plant Service Water System (PSWS)
- Air Systems (HPNSS, Service Air, Instrument Air)
- 13.8 kVAC and 6.9 kVAC bus system
- 250VDC bus system
- Reactor Water Level Instrumentation (RWLI) System
- Drywell Cooling System

For each of these systems, the possibility for the generation of scram or isolation signals due to a spurious actuation or a variation of the system configuration resulting from a human error or hardware failure is investigated. The effect of a pipe break in these systems is also considered.

A qualitative screening analysis has been performed to eliminate errors or failures which are not credible or which do not contribute to a scram initiation or isolation function. This analysis is described in the subsections below.

2.2.4.1 Control Rod Drive System

The CRD system has one pump in operation at all times during normal plant operation. The other pump is in standby and starts automatically upon a reactor water Level 2 signal.

In normal operation the system performs the following functions:

- Charging of hydraulic control units
- Supply of purging water to the control rod drives (CRD), RWCU/SDC pumps and reactor water level reference leg instrument lines

Refer to Section 4.3 for a description and simplified diagram of the system.

The following malfunctions are considered:

- a. Trip of the operating pump: No major effects are expected because at the time of the pump trip the hydraulic control units are charged and the check valve on the charging line prevents their discharge. Also, loss of purge does not impair the Fine Motion Control Rod Drive (FMCRD) function in the short term.
- b. Spurious actuation of the standby pump and opening of the injection valve: This malfunction results in additional inventory of cold water entering the reactor pressure vessel (RPV) from the feedwater line. Because the feedwater system is able to regulate the injection flow (by level control) and the temperature reduction of the total flow entering the vessel is negligible when feedwater is operating, no consequences are expected from this event.
- c. Injection line break downstream of the check valves (isolating CRD from RWCU line): This line is common to the injection of CRD and RWCU train A; because it belongs to RWCU, the consequences of this event are analyzed within RWCU below.

2.2.4.2 Feedwater / Condensate

The Feedwater and Condensate systems are in operation at all times during normal plant operation. Refer to Section 4.9 for a description and simplified diagrams of these systems.

The following malfunctions are considered:

- a. FW controller malfunction (reactor water level increase): The FW controller is single failure proof. Therefore, the FW controller failure is much less likely to occur in the ESBWR than in current generation reactors.

In addition, there is the added protection of a FW runback, closure signals to the high pressure makeup (CRD) flow control valves on reactor water Level 8, and trip of the FW pumps on reactor water Level 9. A scram occurs on reactor water Level 8; this event is included in the General Transient initiating event category.

In case a high reactor water level trip signal does not operate and the operator does not perform backup action, the following Main Steam Line (MSL) flooding scenarios can be postulated:

- MSL break: this event is similar to a steam LOCA event.
- MSL does not break and IC does not operate: in this case, the SRV opens and discharges water. A break of the SRV discharge might occur and cause a LOCA inside the drywell. The consequence of this event is be bounded by the Small Steam LOCA event.
- No pipe break occurs, the IC is not operating, and the SRV remains stuck open: The consequence of this event is bounded by the IORV event.

The frequency of all the above failures is estimated to be below any LOCA frequency.

- b. FW controller malfunctions (reactor water level decrease): A controller malfunction that results in reduced feedwater flow is included in the General Transient initiating event category. A controller malfunction that results in loss of all feedwater flow is included in the Loss of Feedwater initiating event category.
- c. Partial loss of feedwater: Malfunctions involving partial loss of the condensate or feedwater systems are included in the General Transient initiating event category.
- d. Complete loss of feedwater: Malfunctions that cause complete loss of condensate or complete loss of feedwater are included in the Loss of Feedwater initiating event category.

2.2.4.3 Isolation Condensers

This system is in standby during normal plant operation. It is automatically actuated in case of reactor isolation or high reactor pressure. Refer to Section 4.2 for a description and simplified diagram of this system.

The following malfunctions are considered:

- a. Spurious actuation of all four isolation condensers: the effect on the plant is an insertion of positive reactivity. The likely consequence of the event is a reactor trip. The water level changes are corrected by FW flow control. This event is included as part of the General Transient initiating event category.
- b. Spurious opening of one of the twelve vent lines venting the IC to the suppression pool: each isolation condenser is equipped with a top header vent line and a bottom header 3/4 inch vent line, each with two, automatic, solenoid-operated valves in series. All valves fail closed or fail as-is.

The automatic valves are signaled to open by high RPV pressure (above IC actuation value) and the two automatic condensate return valves open. The spurious opening of just one line produces a small LOCA with the steam flowing directly to the suppression pool and eventually leading to a high suppression pool temperature scram. This event is bounded by the IORV.

- c. System pipe breaks upstream of the isolation valves: These breaks are not isolable and they produce LOCA events. This event is considered in the frequency evaluation of LOCAs inside containment (refer to Section 2.3).
- d. System pipe breaks downstream from the isolation valves:
 - Significant break inside the containment: Automatic isolation valves are provided on the suction lines; if a break is sensed (by measuring the flow in the line) these valves close. If this occurs, it is expected that only a limited loss of RPV water occurs, such that no transient initiation is possible. Pipe breaks, combined with failure to isolate the line, have a very low frequency. The consequences are similar to a LOCA inside the containment, but the frequency of the event is much lower. Such events are assumed to be subsumed by the small steam LOCA initiators.
 - IC tube or pipe break outside containment: Automatic or manual break isolation is possible through the multiple leak detection systems, depending on the break size. The IC pipe break outside containment is analyzed as a separate initiating event category.

2.2.4.4 Depressurization and Safety/Relief Valves

Refer to Section 4.1 for a description and simplified diagram of this system.

For this system, only spurious actuation resulting either from operator error or from hardware failure is considered. Breaks of piping between valves and RPV nozzles are taken into account in the LOCA frequency evaluation (see Table 2.2-3).

The spurious actuation of an SRV constitutes an IORV event. An IORV is similar to a small break LOCA, but because of some important differences, a specific event tree is used. Its frequency is evaluated in Section 2.3.

The spurious actuation of only one DPV is included in the small steam LOCA initiating event category. Only the valve mechanical failure contributes to this event because the logic failure causes more than one valve to open, and thus, a different event.

The spurious actuation of all SRVs and DPVs is accounted for within the large LOCA category because of the large total flow area through the valves. The main causes for this event are discussed and quantified in Section 2.3.

2.2.4.5 Gravity Driven Cooling System

The GDCS is in standby and is automatically started by low reactor water Level 1. Refer to Section 4.6 for a description and simplified diagram of this system.

The following malfunctions are considered:

- a. Spurious actuation of a squib valve: the spurious actuation of a squib valve and the failure of the in-line check valve to close produce an event similar to a GDCS line break. The event has a low probability. Also, the event is easily mitigated, because 1)

the loss of coolant is through the GDCS pool and 2) as the reactor depressurizes, one GDCS line is already operating. This event is bounded by the Medium Liquid LOCA initiating event category.

- b. Spurious actuation of all squib valves: this event can occur for the same reasons as discussed earlier for the spurious opening of all DPVs; however, the frequency of this event is lower because, in this case, the check valves on the GDCS lines would need to fail to close. In this case, the event behaves like a medium steam LOCA, but with even less severe consequences than those discussed above, because all GDCS lines are already operating and the reactor is almost depressurized. Therefore, this event is not explicitly considered in the analysis.
- c. Injection line break between the nozzle and the check valve: This event is a LOCA and its frequency is evaluated in Section 2.3.

2.2.4.6 Fuel and Auxiliary Pools Cooling System

This system is always in operation with one pump providing spent fuel pool cooling during plant operation. Its reconfiguration into the suppression pool cooling mode is automatic upon a high Suppression Pool temperature signal. The LPCI mode is initiated manually by the operator. Refer to Section 4.7 for a description and simplified diagram of this system.

The following malfunctions are considered:

- a. Trip of the operating pump: tripping of the operating pump results in an increase of the spent fuel pool water temperature over the long term; however, the standby pump can be aligned. No effect on plant safety is expected.
- b. Spurious startup of the standby pump: the startup of the standby pump has no effect on plant safety. If the related suction valves are not open, then the pump could be damaged due to overtemperature. This is a plant availability problem and not a safety problem.
- c. Injection line break downstream of the check valves (isolating FAPCS from RWCU line): This line is common to the injection of FAPCS and RWCU train B; because it belongs to RWCU, the consequences of this event are analyzed within RWCU below.

2.2.4.7 Reactor Water Clean-Up System

This system is always in operation during normal plant operation with one pump working at a percentage of the rated speed to carry out the reactor water clean-up function. No automatic signals are provided for standby pump actuation or for increase of the operating pump speed. Refer to Section 4.8 for a description and simplified diagram of this system.

The following malfunctions are considered:

- a. Trip of the operating pump: tripping of the operating pump does not produce an immediate effect on plant safety because only the RWCU function is impaired. The loss of the RWCU system for a long time might result in high water conductivity and require a manual plant shutdown.
- b. Spurious startup of the standby pump: no effect on plant safety is expected.

- c. Injection line break downstream from the check valves (isolating RWCU from FW line): this event causes loss of FW because of the diverted flow, unavailability of the RWCU trains due to high temperature in the Main Steam Tunnel, loss of the FAPCS in half of the cases (FW line A break), and loss of the CRD injection in half of the cases (FW line B break). The event behaves like a loss of FW with the unavailability of some mitigating systems.
- d. RWCU line break outside of containment within Reactor Building (other than c above): this event causes loss of RWCU/SDC because of the LD&IS isolating signals, an RPV Level 2 containment IS isolating signal, and unavailability of both RWCU/SDC trains.

Breaks in the RWCU return line from the RPV lead to LOCA events. The frequency of such an event is evaluated in Section 2.3.

2.2.4.8 Standby Liquid Control System

The system is in standby during normal plant operation and is automatically actuated in response to ATWS signals. Refer to Section 4.4 for a description and simplified diagram of this system.

The following malfunctions are considered:

- a. Spurious system actuation: if one of the redundant squib valves spuriously actuates, boron injection into the RPV occurs. Plant availability is affected, but not plant safety. This event is considered as part of the General Transient initiating event category.
- b. Injection line break: this event causes a LOCA. Its contribution to LOCA frequency is evaluated in Section 2.3.

2.2.4.9 Complete Loss of RCCW

The Reactor Component Cooling Water System supplies cooling water to the following systems:

- CRD
- FAPCS
- RWCU/SDC
- Chilled Water System
- Instrument Air System

Refer to Section 4.10 for a description and simplified diagram of this system.

The consequences of total failure of RCCW are less severe than total failure of PSWS because BOP systems cooled by TCCW/PSWS remain available for the loss of RCCW event. However, because the consequences of this event are bounded by the Complete Loss of PSWS initiating event category, loss of RCCW is conservatively grouped together with the Complete Loss of PSWS initiator.

2.2.4.10 Complete Loss of TCCW System

Refer to Section 4.9 for a description and simplified diagram of this system.

The loss of this system results in loss of balance-of-plant (BOP) systems (i.e., a turbine trip with feedwater and condensate system failure and loss of service air system). Given the functional consequences of this event, loss of TCCW is grouped together with the Loss of Feedwater initiating event category.

2.2.4.11 Complete Loss of Plant Service Water System

Refer to Section 4.11 for a description and simplified diagram of this system.

The loss of this system produces a loss of RCCW and TCCW. The event results in the impairment of several mitigating systems (all non-safety systems, including complete loss of AC power to battery chargers). The consequences are similar to a station blackout event. This event is modeled with a separate initiating event category.

2.2.4.12 Complete Loss of Air Systems

This event consists of the simultaneous loss of HPNSS and both the instrument and service air (ISA) systems. Refer to Sections 4.12 and 4.13 for descriptions and simplified diagrams of these systems.

The following events are expected to occur in the plant as a result of complete loss of air:

- Isolation condenser condensate return valves F006A, B, C, D open. Other air-operated valves in the IC system fail as-is.
- Fuel and auxiliary pool cooling system is unavailable due to the failure of the LPCI configuration interface valves in the closed position
- MSIVs long term closure because of the discharge of the accumulators due to normal leakage

After the MSIV accumulators discharge, the event proceeds like an MSIV closure event. However, this event is not maintained as an initiator for accident sequence quantification because of the combination of the following:

- (1) Indications in the control room of the changed status of the above systems
- (2) Time available to the operator to shut down the plant
- (3) Beneficial effect of the IC operation
- (4) Low frequency of the initiating event

2.2.4.13 Loss of 13.8 kVAC or 6.9 kVAC Buses

Refer to Section 4.14 for a description and simplified diagram of the ac system.

The loss of the 13.8 or 6.9 kVAC buses of a single train produces a partial loss of plant service water system (PSWS), among other plant effects. The plant service water configuration is 2 x 100 percent capacity pumps; as such, the loss of one bus causes a partial failure of PSWS. Given the event behaves like a partial loss of PSWS event, it is not modeled as a separate initiator because the initiating event occurs only in case of failure to realign the redundant PSWS pumps (the frequency contributions of bus failure and operator error are judged not significant).

2.2.4.14 Loss of a Single 250 VDC Bus

Refer to Section 4.17 for a description and simplified diagram of the 250 DC system.

The loss of a single 250 VDC essential bus results in de-energization of one of two solenoids of the fail-open IC drain valve. The valves on the venting line to the suppression pool would not open. No effect on plant safety is expected.

The loss of a single 250 VDC non-essential bus does not produce any initiator because it does not result in the actuation of any component.

2.2.4.15 Reactor Water Level Instrumentation Failures

Failures of the RWLI system can initiate a transient event and at the same time jeopardize the operation of other mitigating systems. The failures considered are those involving an erroneous high level signal. Failures resulting in low level are already covered in other transient events in which initiation of the mitigating systems is not jeopardized.

The high level signal (Level 8) produces a trip of all feedwater pumps and closure of the turbine stop valves, resulting in a turbine trip. If the high level signal persists, it is not possible to restart the feedwater pumps and actuation of the CRD system would be inhibited.

The experience record of operating BWR plants shows that an RWLI failure could be caused by leaks in the instrumentation leg or by extreme environmental conditions, such as a high drywell temperature. The following discussion examines the effect of such phenomena, assuming the occurrence of only a single leg failure.

- a. Leaks: leaks occurring in the instrumentation legs could swell the level, inducing an error in all of the connecting instruments. The ESBWR plant design incorporates one separate leg for each level instrumentation division; this means that if only one leg fails, the RWLI system can still assure correct performance (i.e., one scram signal is not generated and mitigating systems are not prevented from starting).
- b. Extreme environmental conditions: a high DW temperature could be caused by accidents such as 1) LOCAs, 2) inadvertent opening of one DPV, or 3) loss of the drywell cooling system. However, the instrumentation is assumed to be designed for the maximum temperature attainable in the DW, so no instrumentation effects should occur. A possible local effect could be generated by a LOCA if the break is close to the instrumentation piping. For this case, only one division is expected to be affected, so no significant contribution to core damage frequency is expected. Safety systems (IC, GDCS, SRV) are actuated by various level sensors.

Given the above, RWLI failures are not included as an initiator for accident sequence quantification.

2.2.4.16 Loss of Drywell Cooling System

A scram is initiated when the drywell temperature increases above the Technical Specification limit. No mitigating systems are impaired. The event is similar to a spurious trip with an insignificant contribution to the spurious trip frequency. It is included in the General Transient initiating event category.

Table 2.2-1
NUREG/CR-5750 Initiating Event Categories

- B Loss of Offsite Power
 - (B1) Loss of Offsite Power (LOSP)

- C Loss of Safety-Related Bus
 - (C1) Loss of Vital Medium Voltage AC Bus²
 - (C2) Loss of Vital Low Voltage AC Bus²
 - (C3) Loss of Vital DC Bus

- D Loss of Instrument or Control Air
 - (D1) Loss of Instrument or Control Air

- E Loss of Safety-Related Cooling Water
 - (E1) Total Loss of Service Water
 - (E2) Partial Loss of Service Water³

- F Steam Generator Tube Rupture
 - (F1) Steam Generator Tube Rupture¹

- G Loss of Coolant Accident (LOCA)/Leak
 - (G1) Very Small LOCA/Leak
 - (G2) Stuck Open: 1 Safety/Relief Valve
 - (G3) Small Pipe Break LOCA
 - (G4) Stuck Open: Pressurizer PORV¹
 - (G5) Stuck Open: 2 or More Safety/Relief Valves
 - (G6) Medium Pipe Break LOCA
 - (G7) Large Pipe Break LOCA
 - (G8) Reactor Coolant Pump Seal LOCA: PWR¹

- H Fire
 - (H1) Fire

- J Flood
 - (J1) Flood

Table 2.2-1
NUREG/CR-5750 Initiating Event Categories

- K High Energy Line Break
 - (K1) Steam Line Break Outside Containment
 - (K2) Feedwater Line Break
 - (K3) Steam Line Break Inside Containment: PWR¹

- L Total Loss of Condenser Heat Sink
 - (L1) Inadvertent Closure of All MSIVs
 - (L2) Loss of Condenser Vacuum
 - (L3) Turbine Bypass Unavailable

- N Interfacing System LOCA
 - (N1) Interfacing System LOCA

- P Total Loss of Feedwater Flow
 - (P1) Total Loss of Feedwater Flow

- Q General Transients
 - (QC4) Loss of AC Instrumentation and Control Bus
 - (QC5) Loss of Non-safety-related Bus
 - (QG9) Primary System Leak
 - (QG10) Inadvertent Open/Close: 1 Safety/Relief Valve
 - (QK4) Steam or Feed Leakage
 - (QL4) Loss of Nonsafety-Related Cooling Water
 - (QL5) Partial Closure of MSIVs⁴
 - (QL6) Condenser Leakage
 - (QP2) Partial Loss of Feedwater Flow⁵
 - (QP3) Total Loss of Condensate Flow
 - (QP4) Partial Loss of Condensate Flow⁶
 - (QP5) Excessive Feedwater Flow
 - (QR0) RCS High Pressure (RPS Trip)
 - (QR1) RCS Low Pressure (RPS Trip): PWR¹
 - (QR2) Loss of Primary Flow (RPS Trip): PWR¹
 - (QR3) Reactivity Control Imbalance
 - (QR4) Core Power Excursion (RPS Trip)
 - (QR5) Turbine Trip
 - (QR6) Manual Reactor Trip
 - (QR7) Other Reactor Trip (Valid RPS Trip)

Table 2.2-1

NUREG/CR-5750 Initiating Event Categories

- (QR8) Spurious Reactor Trip
- (QR9) Spurious Engineered Safety Feature Actuation⁷

Notes to Table 2.2-1:

- 1 PWR event. Not applicable to ESBWR.
- 2 These event categories are not applicable to the ESBWR, as the Low and Medium Voltage AC Buses in the ESBWR design are nonsafety-related (with the exception of a portion of the uninterruptible power supply system that is backed up by batteries). Low and Medium voltage AC initiators for the ESBWR are addressed by the NUREG/CR-5750 category QC5 (Loss of Non-safety-Related bus).
- 3 Loss of a single PSWS train does not cause a reactor trip for the ESBWR design.
- 4 Closure of one MSIV does not cause a reactor trip for the ESBWR design.
- 5 Loss of a single feedwater train does not cause a reactor trip for the ESBWR design.
- 6 Loss of a single condensate train does not cause a reactor trip for the ESBWR design.
- 7 The ESBWR design does not have high pressure standby safety injection systems, and the ESBWR low pressure safety systems cannot inject into the RPV when it is at power (due to check valves). Spurious IC actuation is considered under Special Initiators, and spurious ADS is considered in the LOCA initiator categories.

Table 2.2-2
ESBWR Transient Initiating Event Categorization

| ESBWR Transient Initiating Category | Description | NUREG/CR-5750 Categories | |
|--|---|--|---|
| General Transient | Scram occurs. FW remains available or is promptly recoverable. Main condenser is available, if the mode switch is positioned in shutdown. | a. b. c. d. e. f. g. h. i. j. k. | (QG9) Primary System Leak (QK4) Steam or Feed Leakage (QL6) Condenser Leakage (QP5) Excessive Feedwater Flow (QR0) RCS High Pressure (RPS Trip) (QR3) Reactivity Control Imbalance (QR4) Core Power Excursion (RPS Trip) (QR5) Turbine Trip (QR6) Manual Reactor Trip (QR7) Other Reactor Trip (Valid RPS Trip) (QR8) Spurious Reactor Trip |
| Transient with PCS Unavailable | Scram occurs. FW remains available for RPV inventory makeup. Main condenser is unavailable. | a. b. c. | (L1) Inadvertent Closure of All MSIVs (L2) Loss of Condenser Vacuum (L3) Turbine Bypass Unavailable |
| Loss of Feedwater | Scram occurs. Feedwater is not available. Main condenser is unavailable due to MSIVs closure on Level 2. | a. b. c. | (P1) Total Loss of Feedwater Flow (QL4) Total Loss of TCCW (subset of QL4) (QP3) Total Loss of Condensate Flow |
| IORV | Scram occurs. FW remains available. One or more safety/relief valves initially stuck open | a. b. | (G2) Stuck Open: 1 Safety/Relief Valve (QG10) Inadvertent Open/Close: 1 Safety/Relief Valve |

Table 2.2-3
Lines Connected to Reactor Coolant Pressure Boundary (RCPB)

| Line | Number of lines | mm (inches) diameter | Nozzle Elevation (mm) | Break ^{7,8} Type | Notes |
|---|-----------------|------------------------------------|-----------------------|---------------------------|--------------|
| Main Steam (MSL) | 4 | 700 (28) ¹ | 22840 | L | Steam break |
| DPV/IC | 4 | 450 (18), 350 (14) ² | 21910 | L | Steam break |
| DPV/MSL | 4 | N/A ³ | 22840 | S or L ⁶ | Steam break |
| SRV/MSL | 18 | N/A ³ | 22840 | S or L ⁶ | Steam break |
| FW-12" | 6 | 300 (12) | 18915 | L | Steam break |
| 22" | 2 | 550 (22) | — | L | Steam break |
| RWCU/SDC | 2 | 300 (12) | 17215 | L | Steam break |
| IC return lines | 4 | 200 (8) | 13025 | M | Liquid break |
| GDCS | 8 | 150 (6) ⁴ | 10453 | M | Liquid break |
| Equalizing lines | 4 | 150 (6) ⁵ | 8453 | M | Liquid break |
| RWCU/RPV Drainlines | 4 | 75 (3) | 0 | M | Liquid break |
| SLCS | 2 | 50 (2) | | M | Liquid break |
| Instrument lines above L3 | 4 | 50 (2) | >L3 | S | Steam break |
| Instrument lines below TAF | 4+2 | 25 (1) | <TAF | S | Liquid break |
| Instrument lines above TAF and below L3 | 4 | 25 (1) | TAF-L3 | S | Liquid break |

Notes:¹ 9.75 10⁻² m² (1.05 sq. ft) throat diameter² 450 mm common pipe, 350 mm IC branch pipe³ DPV directly mounted on MSL pipe⁴ 75 mm (3 inches) throat diameter⁵ 50 mm (2 inches) throat diameter⁶ Break size depends upon whether single or multiple valves spuriously open (single DPV or SRV is Small; multiple DPVs or SRVs is Large)⁷ L = Large; M = Medium; S = Small⁸ Because the small and medium size steam LOCAs are consolidated, the most restrictive treatment of the two categories is used in the PRA model analysis.

2.3 INITIATING EVENTS FREQUENCY QUANTIFICATION

The initiating events are identified and grouped into initiating event categories as discussed in Section 2.2.

This section documents the frequencies calculated for each of the initiating event categories. Loss of coolant accidents (LOCA) are divided into further subcategories for more effective event tree modeling and quantification.

2.3.1 Transients

All transient frequencies are determined based on NUREG/CR-5750 results (Reference 2-1), which are based on U.S. nuclear power plant operational experience over the period 1987-1995. The resulting frequencies are summarized in Table 2.3-4 and the details are summarized below.

2.3.1.1 General Transient

The frequency of the General Transient initiating event category is quantified based on the NUREG/CR-5750 "General Transient (Q) - BWR" initiating event frequency, 1.5/yr. Those Q contributors that are included in other ESBWR PRA transient initiating event categories are subtracted out. As can be seen from Sections 2.3.1.3 and 2.3.1.4, these are:

- Total Loss of Condensate Flow (QP3) - 1.5E-2/yr
- Total Loss of TCCW (subset of QL4) - 7.5E-3/yr
- Inadvertent Open/Close: 1 Safety/Relief Valve (QG10) - non-significant contributor

The NUREG/CR-5750 "General Transient (Q) - BWR" category contributors identified in Table 2.2-1 as not applicable to the ESBWR design (QL5, QP2, and QP4) are also subtracted out. Based on Tables 4-7 and D-4 of NUREG/CR-5750, the percentage contribution to the "General Transient (Q) - BWR" frequency from these three contributors is approximately 12% (1.8E-1/yr).

Therefore, the frequency of the ESBWR PRA General Transient initiating event category is calculated as follows:

$$1.5/\text{yr} - 1.5\text{E-}2/\text{yr} - 7.5\text{E-}3/\text{yr} - 1.8\text{E-}1/\text{yr} = 1.30/\text{yr}$$

2.3.1.2 Transient With PCS Unavailable

As shown in Table 2.2-2, the ESBWR PRA Transient with PCS Unavailable initiating event category is comprised of the following contributors:

- Inadvertent Closure of All MSIVs (L1)
- Loss of Condenser Vacuum (L2)
- Turbine Bypass Unavailable (L3)

Based on the results of NUREG/CR-5750, the frequencies of the above contributors are 1.7E-1/yr, 2.0E-1/yr, and 4.1E-3/yr, respectively. Therefore, the frequency of the ESBWR PRA Transient with PCS Unavailable initiating event category is calculated as follows:

$$1.7\text{E-}1/\text{yr} + 2.0\text{E-}1/\text{yr} + 4.1\text{E-}3/\text{yr} = 3.74\text{E-}1/\text{yr}$$

2.3.1.3 Loss of Feedwater

As shown in Table 2.2-2, the following NUREG/CR-5750 categories are grouped into the ESBWR PRA Loss of Feedwater initiating event category:

- Total Loss of Feedwater Flow (P1)
- Total Loss of Condensate Flow (QP3)
- Total Loss of TCCW (subset of QL4)

The frequency of the NUREG/CR-5750 “Total Loss of Feedwater Flow (P1)” initiating event category is 8.5E-2/yr. Appendix A of NUREG/CR-5750 notes that QP3 events are already included in the calculation of the “Total Loss of Feedwater Flow (P1)” frequency (as well as separately in the “General Transient (Q)” frequency, representing about 1% of the Q frequency).

NUREG/CR-5750 does not provide a frequency estimate for the QL4 contributor (it is part of the larger NUREG/CR-5750 category “General Transient (Q)”; however, the frequency of QL4 can be estimated from the contribution breakdowns provided in NUREG/CR-5750. Based on Tables 4-7 and D-4 of NUREG/CR-5750, the percentage contribution of QL4 to the NUREG/CR-5750 category “General Transient (Q) - BWR” is approximately 3%. The NUREG/CR-5750 QL4 encompasses both partial system failures and total system failures for various non-safety cooling water systems (turbine building cooling water, circulating water system, cooling tower systems, etc.). Based on this information, the contribution to the NUREG/CR-5750 “General Transient (Q)” frequency from the subset of QL4 involving total failure of turbine building cooling water is estimated at less than 1% (a nominal value of 0.5% is used here). This is judged reasonable. Based on these NUREG/CR-5750 results, the frequency of total loss of TCCW is estimated at $1.5/\text{yr} \times 5\text{E-}3 = 7.5\text{E-}3/\text{yr}$. This frequency is similar to the loss of turbine building cooling water initiating event frequencies in other U.S. BWR PRAs.

Therefore, the frequency of the ESBWR PRA Loss of Feedwater initiating event category is calculated as follows:

$$8.5\text{E-}2/\text{yr} + 7.5\text{E-}3/\text{yr} = 9.25\text{E-}2/\text{yr}$$

2.3.1.4 IORV

As shown in Table 2.2-2, the ESBWR PRA IORV initiating event category is comprised of the following contributors:

- Stuck Open: 1 Safety/Relief Valve (G2)
- Inadvertent Open/Close: 1 Safety/Relief Valve (QG10)

The frequency of the NUREG/CR-5750 "Stuck Open: 1 Safety/Relief Valve (G2) - BWR" initiating event category is $4.6\text{E-}2/\text{yr}$.

NUREG/CR-5750 does not provide a frequency estimate for the QG10 contributor (it is part of the larger NUREG/CR-5750 category "General Transient (Q)"); however, the frequency of QG10 can be estimated from the contribution breakdowns provided in NUREG/CR-5750. Based on Tables 4-7 and D-4 of NUREG/CR-5750, the percentage contribution of QG10 to the NUREG/CR-5750 category "General Transient (Q) - BWR" is $\ll 1\%$. The QG10 frequency contribution is assessed here as a non-significant contributor.

Therefore, the frequency of the ESBWR PRA IORV initiating event category is estimated at $4.6\text{E-}2/\text{yr}$.

2.3.2 Loss of Preferred Power (LOPP)

The frequency of the Loss of Preferred Power (LOPP) initiating event category is quantified based on the NUREG/CR-5750 "Loss of Offsite Power (B1)" initiating event frequency, $4.6\text{E-}2/\text{yr}$.

Therefore, the frequency of the ESBWR PRA Loss of Preferred Power (LOPP) initiating event category is determined to be $4.6\text{E-}2/\text{yr}$.

2.3.3 LOCAs

The LOCA initiators considered in the ESBWR Probabilistic Risk Assessment are:

- Breaks of the reactor coolant pressure boundary (RCPB) during normal operation
- Breaks of pipes with design pressure lower than the reactor pressure due to the malfunction of the interfacing valves (interfacing breaks)
- Vessel rupture

2.3.3.1 LOCAs Inside Containment

The frequencies of LOCAs inside containment are quantified using of the NUREG/CR-5750 mean frequencies (based on BWRs operational experience) for large, medium and small pipe breaks. For each group of ESBWR lines, the associated NUREG/CR-5750 LOCA frequency is apportioned proportionally using three different methods:

- As a function of line length
- As a function of the number of lines
- As a function of the number of line segments

The results from the three different methods are averaged to determine the final frequency used in this analysis.

Table 2.3-1 summarizes the break frequency calculations for each group of lines. Table 2.3-1 summarizes for each pipe group: the number of lines, the number of sections (assessed on the

basis of layout drawings), the frequency apportionments, and the final averaged frequencies. These data are binned into the LOCA initiator classes, as summarized in Table 2.3-2. Note that contributions from events other than pipe breaks are also included in some of the LOCA categories.

Additional LOCA Contributions

In addition to pipe breaks, the following additional events contribute to the LOCA frequencies:

| | Event | Contribution To | Frequency |
|----|-------------------------------------|-------------------|-----------|
| a. | Spurious actuation of one DPV | Medium Steam LOCA | 4.9E-4 |
| b. | Spurious actuation of all SRVs/DPVs | Large LOCA | 3.2E-4 |

Spurious actuation of a single SRV is addressed as a separate initiating event (IORV), refer to Section 2.3.1.4.

The above two additional LOCA contributors can result from the following causes:

- Valve(s) mechanical failure
- Operator error
- Spurious generation of a level 1 water level signal

Operator errors are judged to be a non-significant contribution. The other causes contribute as follows:

- a. Opening of only one DPV: only valve mechanical failure contributes to this event because logic failure produces more than one valve opening and is already taken into account in item b below.
- b. Opening of all SRVs/DPVs: failure of the logic common to the other valves contributes to this event; mechanical failure of all valves can be postulated, but is statistically a non-significant contributor.

Opening of Only One DPV

Two causes found to contribute to this event:

- Spurious opening
- Shear disk rupture

A common cause failure of load drivers could result in a single spurious opening of a DPV. Assuming that the failure rate for spurious actuation is 50 percent of the total failure rate (1E-6/hr), the frequency of a CCF for load driver is:

$$0.5 \times 1\text{E-}6 \times 8760 \times 0.1 = 4.4\text{E-}4 \text{ events/year}$$

where 0.1 is the generic common cause failure factor assumed for spurious operation. Thus, the DPV spurious opening rate used is 4.4E-4 events/year.

The check valve internal rupture (shear disk rupture) failure rate is assumed; considering a verification of the integrity of the nozzle is performed at every refueling, the annual failure rate is about $4.4\text{E-}5$, based on data in Appendix A of Chapter 1 of the EPRI ALWR URD (Reference 2-6).

Summing these two values results in the frequency of $4.9\text{E-}4$ events/year for only one of eight DPVs spuriously opening. This is an additional contributor to the medium steam break LOCA (no FW or GDCS line break) initiator category.

Opening of All SRVs/DPVs

The operational experience of BWRs indicates that no events of more than one SRV stuck open have been observed. NUREG/CR-5750 estimates an upper bound frequency of $3.2\text{E-}4$ events/year for this initiating event (NUREG/CR-5750 category G5). This is an additional contributor to the large steam break LOCA (no FW line break) initiator category.

2.3.3.2 Breaks Outside Containment

In addition to loss of coolant accidents inside containment, lines outside containment that are exposed to the high pressure of the reactor coolant pressure boundary could lead to a LOCA if they experience a break. The high pressure lines where this event can occur are:

- Main steam
- Feedwater
- RWCU
- Isolation condenser (outside containment)

The largest pipes are the main steam lines, they are also the lines with the greatest number of sections. These lines isolate automatically if a break occurs downstream from the MSIVs. The frequency for this initiating event (refer to Table 2.3-3) is obtained from NUREG/CR-4832 (Reference 2-5). This frequency applies to experiencing the steam line break, isolation of the break is addressed in the accident sequence analysis.

The feedwater pipes isolate automatically if a break occurs upstream from the containment isolation valves, but result in the unavailability of those systems/trains that inject through these lines. The frequency for this initiating event (refer to Table 2.3-3) is obtained from NUREG/CR-5750 initiating event category K2. This frequency applies to experiencing the FW line break; isolation of the break is addressed in the accident sequence analysis.

The RWCU pipes isolate automatically if a break occurs downstream from the containment isolation valves, but result in the unavailability of the train affected by the break. The frequency for this initiating event is assumed to be similar to the frequency of the feedwater lines obtained from NUREG/CR-5750.

The IC pipes outside the containment isolate automatically if a break occurs downstream from the IC isolation valves, but result in the unavailability of the IC affected by the break. A conservative estimate of the frequency for this initiating event is the LOCA frequency

apportioned for IC inside the containment in Table 2.3-1. This frequency applies to experiencing the line break; isolation of the break is addressed in the accident sequence analysis.

2.3.3.3 Vessel Rupture

The value used for the frequency of this event in industry PRAs is typically $3E-7$ events/year (derived from the WASH 1400 report). It is judged that this value is applicable to older vessel designs. The adoption of improved materials, the absence of nozzles and welds at the core level, the reduction of the number of welds, the lower fluence, and the improved in-service inspection program all justify a lower value for the ESBWR. As such, vessel rupture is judged a negligible event and is not maintained as an initiator category for accident sequence quantification.

2.3.3.4 Interfacing LOCA

The interfacing LOCAs are of negligible frequency in the ESBWR design due to the numerous means of separation (i.e., check valves and air operated valves in the closed position, their opening interlocked with reactor pressure) between high and low pressure piping.

The FAPCS system (designed for low pressure) is not directly connected to the RCPB. For the low pressure core injection (LPCI) function, the system is connected to the FW line upstream of the FW isolation check valves. This FAPCS connecting line is protected by two FW isolation check valves, one testable FAPC check valve and one normally closed air operated valve (with an interlock preventing actuation at high reactor pressure). This protection is in addition to the residual structural resistance of the LPCI piping for pressure greater than the design pressure and of the greater pressure supplied by the FW pumps against the vessel pressure.

The frequency of a LOCA event due to the failure of all the above multiple (and, in many cases, independent) barriers is considered negligible. As such, interfacing LOCA is not maintained as an initiator category for accident sequence quantification.

Note that the GDSCS lines connected to the RCPB are all located inside containment and do not represent potential ISLOCA scenarios. They are addressed in LOCAs inside containment (refer to Section 2.3.3.1).

2.3.4 Special Initiators

As discussed in Section 2.2.4, the only systemic malfunction maintained as a separate special initiator is Complete Loss of PSWS. All other systemic malfunctions are either included in other existing initiator categories or do not cause a plant trip.

Complete Loss of PSWS

The frequency of the Complete Loss of PSWS initiating event category is obtained from NUREG/CR-5750. The ESBWR Plant Service Water Systems (PSWS) is designed with a high level of redundancy and diversity provided by the Cooling Tower Makeup pumps. Reactor Water Component Cooling (RCCW), which is grouped within the Complete Loss of PSWS initiator category (refer to Section 2.2.4.9), also has a design with a high level of redundancy

considering the normal operational requirements (two out of six pumps and heat exchangers). Given the design of these ESBWR systems, the frequency estimated in NUREG/CR-5750 for Total Loss of Service Water is taken as a conservative estimate for this initiating event frequency.

2.3.5 Summary of Initiating Event Frequencies

Table 2.3-3 summarizes the frequencies of the internal events initiators used in the event tree quantification.

The initiating event frequencies developed in this analysis are based on BWR historical experience. The total initiating event frequency estimated in this analysis is judged conservative because it is greater than the design goal of less than 1 event/year, to be demonstrated in a later stage for the ESBWR by the Reliability Availability Maintenance (RAM) program.

Table 2.3-4 provides a comparison of the ESBWR PRA internal events initiator frequencies to the following industry studies:

- EPRI ALWR Utility Requirements Document (Reference 2-6)
- NUREG/CR-3862 (Reference 2-3)
- NUREG/CR-4832 (Reference 2-5)
- NUREG/CR-4550, Vol. 4 (Reference 2-7)

Table 2.3-1
Summary of Lines Connected to RPV And Quantification Of Frequency Apportionment

| ID | Line | Length (m) | No. Lines | No. Sections | Apportionment Method | | | Final Frequency ⁴ (events/yr) | Notes |
|----|---|---------------|--------------|-----------------|---------------------------|------------------------------|---------------------------------|--|------------------------|
| | | | | | By Length ¹ | By No. Lines ² | By No. Sections ³ | | |
| a | Main Steam (MSL) | 92 | 4 | 44 | 6.22E-6 | 4.44E-6 | 4.13E-6 | 4.93E-6 | Large Steam break |
| b | DPV line/IC | 70 | 4 | 64 | 4.73E-6 | 4.44E-6 | 6.01E-6 | 5.06E-6 | Large Steam break |
| d | FW | 95 | 8 | 74 | 6.42E-6 | 8.89E-6 | 6.95E-6 | 7.42E-6 | Large Steam break |
| e | RWCU/SDC | 39 | 2 | 31 | 2.64E-6 | 2.22E-6 | 2.91E-6 | 2.59E-6 | Large Steam break |
| f | IC return lines | 12 | 4 | 40 | 2.83E-6 | 5.45E-6 | 8.96E-6 | 5.75E-6 | Medium Liquid break |
| g | GDCS | 44 | 8 | 32 | 1.04E-5 | 1.09E-5 | 7.16E-6 | 9.49E-6 | Medium Liquid break |
| g1 | Equalizing lines | 11 | 4 | 12 | 2.60E-6 | 5.45E-6 | 2.69E-6 | 3.58E-6 | Medium Liquid break |
| h | RWCU/RPV Drainlines | 20 | 4 | 20 | 4.72E-6 | 5.45E-6 | 4.48E-6 | 4.89E-6 | Medium Liquid break |
| i | SLCS | 40 | 2 | 30 | 9.45E-6 | 2.73E-6 | 6.72E-6 | 6.30E-6 | Medium Liquid break |
| j | Instrument lines above L3 | 50 | 4 | 100 | 1.14E-4 | 1.14E-4 | 1.14E-4 | 1.14E-4 | Small Steam break |
| j1 | Instrument lines below TAF | 75 | 6 | 150 | 1.71E-4 | 1.71E-4 | 1.71E-4 | 1.71E-4 | Small Liquid break |
| j2 | Instrument lines above TAF and under L3 | 50 | 4 | 100 | 1.14E-4 | 1.14E-4 | 1.14E-4 | 1.14E-4 | Small Liquid break |

Notes to Table 2.3-1:

¹ Calculated as: [(length of line) / (total length of all lines corresponding to LOCA size category)] x
NUREG/CR-5750 LOCA size category frequency

² Calculated as: [(number of individual lines) / (total number of lines corresponding to LOCA size category)] x
NUREG/CR-5750 LOCA size category frequency

³ Calculated as: [(number of sections) / (total number of sections corresponding to LOCA size category)] x
NUREG/CR-5750 LOCA size category frequency

⁴ Final frequency is the average of the three apportionment methods.

⁵ NUREG/CR-5750 frequencies for Large, Medium and Small BWR LOCAs: 2E-5, 3E-5, and 4E-4, respectively.

Table 2.3-2
Attribution of Line Frequencies to LOCA Categories

| LOCA Category | Contributors^{1, 2} | Frequency (events/yr) |
|---|------------------------------------|----------------------------------|
| Large steam LOCA (no FW line break) | $a + b + e + (A)^2$ | 3.33E-4 |
| Large steam LOCA in FW line | d | 7.42E-6 |
| Medium liquid LOCA (no RWCU break) | $f + g + g1 + i$ | 2.51E-5 |
| Medium liquid LOCA in RWCU | h | 4.89E-6 |
| Small steam LOCA | $j + (B)^2$ | 6.04E-4 |
| Small liquid LOCA (no RWCU break) | j2 | 1.14E-4 |
| Small liquid LOCA in RWCU | j1 | 1.71E-4 |
| Main Steam Line Break (Outside Containment) | -- | 3.00E-3 |
| Feedwater Line Break (Outside Containment) | -- | 3.40E-3 |
| RWCU Line Break (Outside Containment) | -- | 3.40E-3 |
| IC Line Break (Outside Containment) | -- | 5.06E-6 |

¹ Refer to Table 2.3-1 for line IDs.

² The additional LOCA contributors are as follows:

- (A) Spurious actuation of all SRVs and DPVs (3.2E-4/yr)
- (B) Spurious actuation of a single DPV (4.9E-4/yr)

Table 2.3-3
Internal Events Initiator Frequencies

| Initiating Event | Frequency (events/year) | PRA Basic Event | |
|---|----------------------------|-----------------|----------------------------|
| | | ID | Frequency (events/year) |
| <u>Transients</u> | | | |
| General Transient | 1.30 | %T-GEN | 1.30E+0 |
| Transient with PCS Unavailable | 3.74E-1 | %T-PCS | 3.74E-1 |
| Loss of Feedwater | 9.25E-2 | %T-FDW | 9.25E-2 |
| IORV | 4.60E-2 | %T-IORV | 4.60E-2 |
| | | | |
| Loss of Preferred Power (LOPP) | 4.60E-2 | %T-LOPP | 4.60E-2 |
| | | | |
| <u>LOCAs Inside Containment</u> | | | |
| Large Steam LOCA (no FW line break) | 3.33E-4 | %LL-S | 3.33E-4 |
| Large Steam LOCA in FW line | 7.42E-6 | %LL-S-FDWA | 3.71E-6 |
| | | %LL-S-FDWB | 3.71E-6 |
| Medium Liquid LOCA (no RWCU break) | 2.51E-5 | %ML-L | 2.51E-5 |
| Medium Liquid LOCA in RWCU | 4.89E-6 | %ML-L-RWCU | 4.89E-6 |
| Small Steam LOCA | 6.04E-4 | %SL-S | 6.04E-4 |
| Small Liquid LOCA (no RWCU break) | 1.14E-4 | %SL-L | 1.14E-4 |
| Small Liquid LOCA in RWCU | 1.71E-4 | %SL-L-RWCU | 1.71E-4 |
| | | | |
| <u>LOCAs Outside Containment</u> | | | |
| Main Steam Line Break (Outside Containment) | 3.00E-3 | %BOC-MS | 3.00E-3 |
| Feedwater Line Break (Outside Containment) | 3.40E-3 | %BOC-FDWA | 1.70E-3 |
| | | %BOC-FDWB | 1.70E-3 |
| RWCU Line Break (Outside Containment) | 3.40E-3 | %BOC-RWCU | 3.40E-3 |
| IC Line Break (Outside Containment) | 5.06E-6 | %BOC-IC | 5.06E-6 |
| | | | |
| <u>Special Initiators</u> | | | |
| Complete Loss of PSWS | 9.70E-4 | %T-SW | 9.70E-4 |

Table 2.3-4

Comparison of ESBWR PRA Internal Events Initiating Event Frequencies to Other Studies

| Initiating Event | Frequency (per year) | | | | |
|---|----------------------|---------------|---------------|---------------|---------------|
| | ESBWR PRA | EPRI ALWR URD | NUREG/CR-3862 | NUREG/CR-4832 | NUREG/CR-4550 |
| <u>Transients</u> | | | | | |
| General Transient | 1.30 | 2.3E+1 | 6.4E+1 | 4.5E+1 | 2.5E+1 |
| Transient with PCS Unavailable | 3.74E-1 | 4.9E-1 | 6.8E-1 | 1.54E+1 | 5.0E-2 |
| Loss of Feedwater | 9.25E-2 | 3.7E-1 | 7.0E-2 | 6.0E-1 | 6.0E-2 |
| IORV | 4.60E-2 | n/a | 1.4E-1 | 1.4E-1 | 1.9E-1 |
| | | | | | |
| Loss of Preferred Power (LOPP) | 4.60E-2 | 3.5E-2 | 8.0E-2 | 9.6E-2 | 7.9E-2 |
| | | | | | |
| <u>LOCAs Inside Containment</u> | | | | | |
| Large Steam LOCA (no FW line break) | 3.33E-4 | 5.8E-4 | n/a | 1.0E-4 | 1.0E-4 |
| Large Steam LOCA in FW line | 7.42E-6 | | | | |
| Medium Liquid LOCA (no RWCU break) | 2.51E-5 | n/a | n/a | 3.0E-4 | 3.0E-4 |
| Medium Liquid LOCA in RWCU | 4.89E-6 | | | | |
| Small Steam LOCA | 6.04E-4 | 5.1E-3 | n/a | 3.0E-2 | 3.0E-3 |
| Small Liquid LOCA (no RWCU break) | 1.14E-4 | | | | |
| Small Liquid LOCA in RWCU | 1.71E-4 | | | | |
| | | | | | |
| <u>LOCAs Outside Containment</u> | | | | | |
| Main Steam Line Break (Outside Containment) | 3.30E-3 | n/a | n/a | n/a | n/a |
| Feedwater Line Break (Outside Containment) | 3.40E-3 | | | | |
| RWCU Line Break (Outside Containment) | 3.40E-3 | | | | |

Table 2.3-4

Comparison of ESBWR PRA Internal Events Initiating Event Frequencies to Other Studies

| Initiating Event | Frequency (per year) | | | | |
|-------------------------------------|----------------------|------------------|-------------------|-------------------|-------------------|
| | ESBWR PRA | EPRI ALWR URD | NUREG/CR- 3862 | NUREG/CR- 4832 | NUREG/CR- 4550 |
| IC Line Break (Outside Containment) | 5.06E-6 | | | | |
| <u>Special Initiators</u> | | | | | |
| Complete Loss of PSWS | 9.70E-4 | n/a | n/a | n/a | n/a |

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3 ACCIDENT SEQUENCE ANALYSIS

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3 ACCIDENT SEQUENCE ANALYSIS

3.1 INTRODUCTION AND SCOPE

The purpose of this section is to document the event trees and accident sequence analysis performed as part of the Economic Simplified Boiling Water Reactor (ESBWR) Probabilistic Risk Assessment (PRA). This section includes the basis for the event trees modeled in the internal event analysis and a description of each event tree heading. These event trees also provide the baseline models for the external events analyses presented in later sections of this report.

The event trees developed for ESBWR are based on the initiating events included in Section 2. The event tree models include the set of systems needed to mitigate each initiating event. Each event tree provides a time independent, system-based response to each initiating event. The objective of these event tree analyses is to show which system combinations result in a safe, stable state, and which ones result in core damage.

The systems modeled include both safety-related and non-safety systems are modeled using the fault tree logic that is presented in Section 4. Support systems and operator actions are modeled explicitly within each system fault tree.

The accident sequence analysis numerical results are presented in Section 7.

The event tree end states are grouped into plant damage states in order to simplify the containment analyses that is presented in Sections 8 and 21.

The event tree logic diagrams are shown at the end of this section in Appendix A.3.

3.2 EVENT TREES AND ACCIDENT SEQUENCE METHODOLOGY

3.2.1 Acceptance Criteria

The acceptance criteria of the safety functions that are required to be accomplished for safe plant operation are described in this section. These safety functions are the following:

- Criterion for Reactivity Control

The acceptance criterion is to achieve subcriticality and maintain the reactor in a subcritical state.

- Criterion for RPV Overpressure Protection

A pressure of 150 percent of the reactor coolant pressure boundary (Service Level C) is defined as the acceptance criterion for the RPV overpressure protection. This limit is used to determine the number of Safety/Relief Valves (SRVs) that are required to operate following any event, including ATWS events.

- Criterion for Core Cooling

A peak cladding temperature (PCT) of 2200°F is chosen as the criterion for establishing the adequacy of coolant inventory. This is consistent with 10 CFR 50.46 and with criteria adopted in previous PRAs.

- Criterion for Containment Heat Removal

For event sequences in which core cooling is successful utilizing Passive Systems, the containment cooling function acceptance criteria is to maintain the containment pressure below the ultimate containment pressure. If the containment fails under these conditions, the inventory of water available for the passive systems is depleted through the containment breach, and consequently core coverage would be lost without recovery of some type. In all cases, containment does not fail until at least 24 hours following the initiating event, and any subsequent core uncover does not occur until 72 hours following the initiating event.

For sequences with successful core cooling using active water sources, containment failure does not affect continued indefinite core cooling.

3.2.2 Success Criteria

Success criteria are defined as the minimum grouping of systems, trains, or components that are required to operate in order to meet the acceptance criteria related to the safety functions identified in the previous section. The systems available for each of these functions and their success criteria are summarized in that Table 3.2-1.

3.2.3 Event Tree Development

The event tree methodology is used to represent the possible sequences of events following any one of the initiating event groups defined in Section 2. Each event tree sequence depicts a possible combination of system and operator action successes or failures leading to either the successful cooling of the core or to core damage according to the acceptance and success criteria defined in Sections 3.2.1 and 3.2.2.

The end point of each of these sequences could be either a stable and safe state of the plant (i.e., hot standby conditions), or a plant damage state (i.e., core damage) identified as an accident class. The list of accident classes used for the ESBWR plant is given in Section 3.2.4.

The event trees developed in the ESBWR PRA are presented in Appendix A.3 as follows:

- (1) General Transient
- (2) Transient with PCS unavailable
- (2a) Transient with PCS unavailable after BOC in IC or MS
- (3) Loss of Feedwater Transient
- (4) Loss of Service Water System
- (5) Loss of Preferred Power Transient
- (6) Inadvertent Opening of a Relief Valve
- (7) ATWS from General Transient or LOPP
- (7a) ATWS from Loss of Preferred Power
- (8) ATWS from Transient Loss of PCS
- (8a) ATWS from Loss of PCS after BOC in IC or MS
- (9) ATWS from Transient with Loss of Feedwater System
- (10) ATWS from Transient with Loss of Service Water System
- (11) ATWS from Inadvertent Opening of a RV
- (11a) ATWS from Small LOCA above Reactor Core
- (12) Large steam break (above L3) other than Feedwater lines
- (13) Large steam break (above L3) on FDW (A) lines
- (14) Large steam break (above L3) on FDW (B) line
- (15) Small steam break (above L3)
- (16) Medium liquid break (below L3) other than RWCU/SDC lines
- (17) Medium liquid break (below L3) in RWCU/SDC lines
- (18) Small liquid LOCA (below L3) other than RWCU/SDC lines
- (19) Small liquid LOCA (below L3) in RWCU/SDC lines
- (20) Reactor Vessel Rupture
- (20a) Reactor Vessel Rupture after Loss of Preferred Power
- (21) Steam break outside containment in Main Steam lines
- (22) Steam break outside containment in FDW A lines
- (23) Steam break outside containment in FDW B lines
- (24) Large steam break outside containment in IC lines

(25) Large liquid break outside containment in RWCU/SDC lines

The specific characteristics that are taken into consideration for the development of each event tree and its functions are described in Section 3.3.

3.2.4 End States of the Accident Sequences

The event trees presented in this section identify the potential sequences that can lead to core damage. Many of the sequences have common characteristics with respect to the challenge on the containment fission product barrier. These sequences are grouped into damage classes that are analyzed in the Level 2 portion of the PRA. The end states of the accident sequences developed for the ESBWR PRA are defined to facilitate the Level 2 Containment Performance Analysis and provide the link between the Level 1 and Level 2 analyses.

The core damage sequences are grouped together based upon the overall challenge to the containment barrier. They are defined as follows:

- OK: The core is successfully cooled and the containment is intact. There is no core damage in these events.
- CD I: The containment is intact when core damage occurs and the RPV is at low pressure.
- CD II: The containment fails while the core is successfully cooled. Core damage results if the containment failure affects the core cooling. In the ESBWR, it has been demonstrated that this end state is only applicable for scenarios where core cooling is being provided solely by the passive systems; and in these cases, the core does not become uncovered until more than 72 hours from the initiating event. In that time frame, there are many potential recovery actions, including repair of failed equipment. Because of this, CD II events are not included in the baseline CDF for ESBWR. Section 11 provides a sensitivity evaluation that assesses this modeling decision. Note that the event trees presented in Appendix A.3 include this end state so that a separate set of event trees specifically for the sensitivity analysis do not need to be presented.
- CD III: The containment is intact when core damage occurs and there is high RPV pressure at the time of core damage.
- CD IV: Core damage results from an accident sequence with a failure of effective reactivity control (e.g., ATWS without SLCS). This has the potential to affect the containment in a more severe manner than the CD I and CD III because more energy is deposited into the containment prior to RPV failure. The analysis of this end state (Section 8) demonstrates that in fact, all CD IV end states could be treated as CD I or CD III (depending on the RPV pressure) without affecting the results of the containment analysis. This end state has been retained in the Level 1 analysis to more easily allow for sensitivity analyses related to reactivity control.

- CD V: The containment is bypassed at the time of core damage.

The Level 2 analysis requires further discrimination between the end states to determine specific containment challenges. For example, in CD I sequences the water level in the lower drywell is required as input to the Level 2. These sub-classes cannot be determined solely from the sequence path definitions; rather the minimal cutsets must be reviewed for specific failures to determine the applicable plant state. This information is presented in Section 7 along with the results of the Level 1 quantification.

3.2.5 Mission Time

The mission time for the ESBWR PRA is 24 hours. The design of the ESBWR is such that the onsite inventory of cooling water available and plant battery capacity can keep the core covered using passive systems for more than 72 hours. However, the simplifying assumptions made in the PRA analysis are not always applicable for mission times longer than 24 hours. For example, the PRA assumes that once the initiator has occurred, no credit will be given for repair of failed equipment. This is a conservative assumption that is reasonable for a mission time of 24 hours; it provides unreasonable results and misleading insights for a 72 hour mission.

Reference is made to the following terms in the evaluation of containment cooling adequacy:

- Short Term Cooling: Time period less than 24 hours
- Longer Term Cooling: Time period of 24 to 72 hours

In cases where the core remains cooled during the short term but conditions are not stable, a core damage end state will be assigned if there are no effective ways to stabilize the plant in the long term. An example of this is shown in the BOC-RWCU event tree (Figure A.3-25). In cases where there is successful core cooling but the broken line has not isolated, the reactor building will continue to be flooded with water from inside the containment. This flooding can disable the means to align active makeup systems. Eventually (around 72 hours), the passive water sources inside the containment will be depleted and the core will become uncovered. The CD V end state is assigned to this sequence.

The PRA presents a sensitivity analysis in Section 11 to characterize the affect of extending the mission time to 72 hours.

Table 3.2-1
Success Criteria Summary

| EVENT TREE HEADING | DESCRIPTION | SUCCESS CRITERIA | NOTES¹ |
|-----------------------------------|--|---|---|
| ADS | Automatic Depressurization System | 4 of 8 DPVs Open | <ul style="list-style-type: none"> • ADS sequence starts on Level 1 • SRVs do not assist in depressurization • Only Group 2 and 3 DPVs credited • Success defined as adequate depressurization to allow gravity drain systems to function |
| C | Reactor Protection System | (RPS or ARI signal) and Control Rod Movement | |
| DL | Vapor Suppression System Close | All DW/WW Vacuum Breakers close after MDS/ADS | |
| DS | Vapor Suppression System Open in Large and Medium LOCA | 2 of 3 vacuum breakers close for pressure suppression; at least 1 of 3 vacuum breakers open to prevent excessive negative DW pressure | |
| I | Isolation Condensers | 3 of 4 IC and all PCCS pool connections | Pool connections only needed for 72 hour mission |
| IA | Automatic Depressurization System Inhibition | Automatic Inhibition Actuation in ATWS | |
| IIC | Isolation: C Steam Line Broken | Isolation by 1 of 2 Steam Valves | |
| IFDW A/B | Isolation: FDW Line Broken A/B | Isolation by 1 Valve or 1 check valve | |
| IML | Isolation: Main Steam Line Broken | Isolation by 1 of 2 MSIVs in each main steam line | |
| IRWCU/SDC | Isolation: RWCU Line Broken | Isolation by 1 of 2 Valves | |
| M | Overpressure Protection System | 1 of 18 SRVs Open No ATWS | <ul style="list-style-type: none"> • Only 1 SRV lifts on highest spring setpoint of 8.86 MPa (1270 psig) • Success is steam dome pressure less than 13.3 MPa (1914 psig) |
| MA | Overpressure Protection System | 9 of 18 SRVs Open ATWS | <ul style="list-style-type: none"> • FW Runback is successful • 9 ADS SRVs lift on highest spring setpoint of 8.86 MPa (1270 psig) |

Table 3.2-1
Success Criteria Summary

| EVENT TREE HEADING | DESCRIPTION | SUCCESS CRITERIA | NOTES¹ |
|-----------------------------------|---|---|--|
| | | | <ul style="list-style-type: none"> • Success is steam dome pressure less than 13.3 MPa (1914 psig) |
| MDS | Manual Depressurization System | 4 of 8 DPVs Open | <ul style="list-style-type: none"> • Same as calculation as ADS |
| PA | SRV closure | All SRV reclose in ATWS | |
| RF | Run-back FDW | Automatic Actuation | |
| SL | Standby Liquid Control System | 2 of 2 SLCS | |
| TPCS | Total Power Conversion System | 1 of 12 Bypass Valves Open, 1 of 4 Steam Lines Remain Open, 1 of 4 Circulating Water Pumps, 1 of 4 Feedwater Pumps, and 1 of 4 Condensate Pumps | |
| UCF | High Pressure Injection Systems | 2 of 2 CRD pumps or 1 of 4 Feedwater Pumps | <ul style="list-style-type: none"> • CRD starts on Level 2 • After initial transient, core remains covered |
| UC | Control Rod Drive Mechanism Injection System | 2 of 2 CRD pumps | <ul style="list-style-type: none"> • CRD starts on Level 2 • Core remains covered |
| U1C | CRD for RPV injection | 1 of 2 CRD pumps | <ul style="list-style-type: none"> • CRD starts on Level 2 • Core remains covered |
| U1CF | High Pressure Injection Systems | 1 of 2 CRD pumps or 1 of 4 Feedwater pumps | <ul style="list-style-type: none"> • CRD starts on Level 2 • Core remains covered |
| U2CISHORT | CRD and IC condensate return for short time | 2 of 2 CRD pumps for 15 minutes and 3 of 4 IC condensate return lines open | |
| UF | Feedwater Injection System | 1 of 2 Feedwater pumps | |
| VF | Fire Protection System | 1 of 1 Fire Protection System pumps inject | Design Requirement for FPS |
| VFL | Fire Protection System after Depressurization | 1 of 1 Fire Protection System pumps inject | Design Requirement for FPS |

Table 3.2-1
Success Criteria Summary

| EVENT TREE HEADING | DESCRIPTION | SUCCESS CRITERIA | NOTES¹ |
|-----------------------------------|---|--|---|
| VG | Gravity Driven Cooling System and Equalizing Lines | 2 of 8 lines and (2 of 3 GDCS pools or (1 of 3 GDCS pools and 1 of 4 Equalizing lines ²)) | <ul style="list-style-type: none"> • Core remains covered • If PCCS success, no equalizing lines needed • Without PCCS, core remains covered for > 72 hours with equalizing line open |
| VG2 | Gravity Driven Cooling System and Equalizing Lines | 2 of 8 lines and 2 of 3 GDCS pools and 1 of 4 Equalizing lines ² | <ul style="list-style-type: none"> • No core heatup • If PCCS success, no equalizing lines needed • Without PCCS, core remains covered for > 72 hours with equalizing line open |
| VG3 | Gravity Driven Cooling System and Equalizing Lines | 3 of 3 GDCS pools and 1 of 4 Equalizing lines ² | <ul style="list-style-type: none"> • Equalizing line opens on Level 0.5 • Core remains covered for 72 hours |
| VLFL | Low Pressure Injection Systems | 1 of 2 FAPCS pumps or 1 of 1 Fire Protection System pumps | <ul style="list-style-type: none"> • Core remains covered (FPS pump assumed to have the same head and flow capacity as one FAPCS pump) |
| VLFL | Low Pressure Injection Systems after Depressurization | 1 of 2 FAPCS pumps or 1 of 1 Fire Protection System pumps | <ul style="list-style-type: none"> • Core remains covered (FPS pump assumed to have the same head and flow capacity as one FAPCS pump) |
| WC | Containment Inerting System | System vent path | 2 inch vent prevents overpressure conditions in containment |
| WH | Reactor Water Clean Up and Shutdown Cooling System | 1 of 2 RWCU/SDC | |
| WHA | Reactor Water Clean Up and Shutdown Cooling System | 1 of 2 RWCU/SDC in ATWS | |

Table 3.2-1
Success Criteria Summary

| EVENT TREE HEADING | DESCRIPTION | SUCCESS CRITERIA | NOTES¹ |
|--|---|---|--|
| WL | Suppression Pool Cooling | 1 of 2 FAPCS Suppression Pool cooling | |
| WLL | Suppression Pool Cooling after Depressurization | 1 of 2 FAPCS Suppression Pool cooling | |
| WP | Passive Containment Cooling System | 4 of 6 PCCS | <ul style="list-style-type: none"> • PCC Pool makeup not credited • Containment remains below design pressure for 39 hours |
| WT | Continued PCCS Pool Inventory | All PCCS pools connected or external makeup available from FPS. | <ul style="list-style-type: none"> • 4 of 6 PCCS heat exchangers function • PCC Pool makeup successful Containment remains below design pressure for more than 72 hours |
| XS5 | Depressurization to FAPCS or FPS | 5 of 10 SRVs open | <ul style="list-style-type: none"> • FPS pump provides injection • Core remains covered • (FPS pump assumed to have the same head and flow capacity as one FAPCS pump) |
| <p>Notes for Table 3.2-1:</p> <ol style="list-style-type: none"> 1) Blank field indicates that the condition is covered by the design basis of the system. 2) Some sequences indicate that 2 of 4 equalizing valves are necessary. A review of the cutsets has shown that changing this to 2 of 4 will not alter the sequence results. | | | |

3.3 EVENT TREE AND ACCIDENT SEQUENCE ANALYSIS

The event tree provides a structured analysis of the integrated response of the plant systems to any initiating event. Following an initiating event, the objective of each safety function (Section 3.2.1) is queried to determine the overall state of the plant. They are analyzed from left to right, with the functions laid out in approximately the chronological order that they will be satisfied. In the ESBWR PRA, the safety functions are further broken down to the system level to determine whether or not the function objectives are met. Finally the event tree concludes with an end state (Section 3.2.4) for each analyzed sequence.

Some of the event tree figures show a "transfer" to another event tree. For example, the failure to SCRAM following a General Transient (Figure A.3-1) is transferred to the ATWS model (Figure A.3-7). The modeling of the sequence continues on the specified tree.

At each system query, or branch, the event tree shows either success of that system or failure. The convention used in the ESBWR model is that the branch upward represents success of the system while the branch downward represents failure. A path that does not branch under a particular system heading indicates that either the state of that system can be completely determined by the path leading to that point or that the success state of that system is not relevant to the remainder of the sequence. Using this method, the resulting end states represent all of the possible outcomes as a result of the initiating event.

The frequencies of each of the end states are determined by estimating the conditional probabilities of each branch on the event tree. The probabilities are assigned based on fault tree models of each of the systems. These are described in Section 4.

In the final model representation, the sequence of headings may not retain their chronological order. The re-ordering is done to prevent the quantification code from generating non-minimal cutsets. The re-ordering does not affect the outcome of Level 1 PRA results.

The numerical results of the event trees are presented in Section 7.

Event sequences for a general transient (turbine or reactor trip initiators) are described in the following section.

For the other initiators, the sequences and headings that differ from those of the general transient are given.

The event trees for all of the internal initiating events are shown in Appendix A.3.

3.3.1 Transients

3.3.1.1 General Transient (BASE Case)

(1) Development of the General Transient Event Tree (T GEN)

Uncomplicated turbine or reactor trips are typically the most frequent initiating events at nuclear power plants. In these events, the Feedwater System and the Power Conversion System, designated the Total Power Conversion System (TPCS) in the ESBWR PRA, are initially available.

Immediately following the trip, a scram signal is generated to initiate control rod insertion. The failure to insert control rods is transferred to the specific ATWS event tree for this initiating event.

If the TPCS remains available, all the safety functions required are assured, because both the short-term and long-term core cooling functions are accomplished.

In case of TPCS failure, the Isolation Condenser (I) function is initiated by high reactor pressure or due to low water level in reactor vessel. For cases that lead to low reactor water level, Main Steam Line Isolation Valve (MSIV) closure occurs. The I function is sufficient to perform the short-term and, with all the water that is in upper pools, the long-term core cooling functions.

If the I function fails, the pressure in the reactor rises until one or more Safety Relief Valves (SRV) open on their spring setpoint. Steam is relieved to the suppression pool and makeup water is required to keep the core covered. At this point in the sequence, the reactor pressure remains high so Feedwater or the Control Rod Drive (CRD) system in high pressure injection mode could provide adequate flow to the core. HPCRD initiates automatically on low water level. If either of these systems is successful, adequate core cooling is assured without any limitations.

In the case of failure of all SRVs to open, the situation is considered as a Reactor Vessel Rupture (RVR), and it is transferred to the specific RVR event tree for this kind of initiating event. The opening of one SRV is sufficient to prevent vessel overpressure failure.

If the high pressure injection systems fail to restore water level in the RPV, low pressure injection systems will need to operate. In order to do this, depressurization of the RPV is needed. This can occur either manually using SRVs as directed by plant EOPs, or automatically via ADS.

The opening of SRVs is sufficient for reactor depressurization to allow the low pressure system injection. Either the Fuel and Auxiliary Pool Cooling System (FAPCS) in the injection mode or the Fire Protection System (FPS) in the injection mode, have the capability to successfully perform the coolant makeup function for both the short and long term. Condensate pumps could also be used to inject water to the vessel. However, because of the strong dependence with both the TPCS and FDW headings, which are already resolved as failed in these sequence paths, the condensate pumps are conservatively not credited as a separate low pressure injection heading.

If the active low pressure injection systems fail, following successful depressurization using the SRVs, ADS will automatically actuate and the passive Gravity Driven Cooling System (GDCS) will provide core inventory. If manual depressurization with SRVs fails, the ADS will actuate to depressurize using the Depressurization Valves (DPV). Then GDCS, FAPCS, or FPS can support the core cooling functions as indicated above.

For the sequences in which FAPCS or FPS provide water injection, no further systems or actions are necessary to ensure long term cooling. For the sequences where the water injection function is performed by GDCS, the core will remain covered for more than 24 hours, however to preserve the state indefinitely without recovery of any of the active systems, the containment heat removal function may be necessary. As stated in 3.2.4, the baseline PRA considers these sequences as success paths. Section 11 presents a sensitivity analysis that shows the affect of extending these sequences. The sequence paths designated as CD II are used to quantify the sensitivity.

There are multiple methods to satisfy the containment heat removal function. These include the following:

- RWCU in shutdown cooling mode
- PCCS (For successful PCCS actuation, the vacuum breakers between the drywell and the wetwell must work as designed.)
- FAPCS in suppression pool cooling mode
- Containment vent

The event tree drawing for the General Transient is shown in Figure A.3-1.

(2) Heading Descriptions

SCRAM (C)

The self-sustaining nuclear reaction is terminated by rapidly inserting control rods so that the core remains shutdown for all moderator conditions. The ESBWR scram system is capable to drive control rods into RPV by multiple means. Preferentially, the Reactor Protection System (RPS) performs the scram. The RPS can fail to insert the control rods either electrically or hydraulically. The electrical part of the RPS has a backup system, the Alternate Rod Insertion System (ARI). A second backup is provided by a FMCRD run-in of the control rods that is independent of both the electrical and hydraulic function.

PCS and FDW (TPCS)

This functional heading includes the use of both the PCS and FDW to cool down the reactor after a trip. This is the preferred way to accomplish the short-term and long-term core cooling functions.

The FDW remains available following the scram because the RPV Level 9, the setpoint for FDW pump trip, is not reached. The success criteria of this function is the operation of the FDW level control system, at least one FDW pump, one Condensate pump, one Bypass valve, and one Circulation pump with at least one steam line able to connect the RPV to the Condenser throughout the sequence mission time.

If the FDW part of this heading fails, RPV level 2 setpoint is reached and subsequently MSIV closure and initiation of IC and CRD injection occurs.

If the PCS part of the function fails, the pressure in the RPV rises and the high RPV pressure IC initiation setpoint is reached. Even if upon reaching RPV level 2 setpoint, MSIV closure is automatically initiated, the operator could reopen them promptly in order to recover the function of Bypass if it is available. Nevertheless, as a conservative assumption, this scenario is not included in the model.

Isolation Condensers (I)

Auto initiation of I function occurs in case of TPCS failure, either on low RPV water level (Level 2), closure of MSIV, or high RPV pressure.

The I function is able to prevent RPV water Level 1.5 from being reached if there is little or no leakage from the RPV. The existence of leaks present prior to the initiating event, and equal to

the maximum RCPB leak rate permitted by Technical Specifications, are judged to not result in ADS actuation during the transient.

The success criterion of this function is the operation of at least three of four ICs during the sequence mission time.

At least 1 SRV Open (M)

If the TPCS and the I function both fail, the pressure in the RPV will rise to the actuation setpoint of the SRV. The criteria for this function are the operation of at least 1 SRV automatically. Conservatively, the failure of this function is assumed that will produce a RVR.

After the opening of at least one SRV, it is not necessary to take into consideration that all of the opened SRV close properly when the pressure in the RPV decreases below the SRV. That is because from this point in the event tree onwards, no further credit is given for the I function.

High Pressure Injection Systems (U1CF)

Injection into the RPV can be provided at high pressure by the FDW or by the CRD system. FDW continues running after the initiating event and CRD system automatic initiation occurs at RPV Level 2.

FDW is successful if at least 1 FDW pump and 1 Condensate pump taking suction from the Condenser, which is replenished from the Condensate Storage Tank (CST), for the mission time. If FDW fails to keep the RPV water level above Level 2, CRD injection automatically initiates. Success of CRD for this function is the effective injection into the reactor vessel of 1 CRD pump taking suction from the CST.

RPV depressurization is directed by procedure in this case due to the rise in the water level in the suppression pool. The continuous transfer of water, from the CST to the suppression pool via the SRVs, causes the increase. Neither failure to depressurize nor the depressurization itself cause failure of the high pressure systems credited in this heading.

At least 5 SRV open (XS5)

If no high pressure injection system is available, it is necessary to depressurize the RPV by opening SRVs to permit effective FACPS or FPS injection to the RPV.

Success of this function is the manually opening of at least five of eight SRVs.

Low Pressure Injection System (VLF)

After successful XS5 depressurization, both the FAPCS and the FPS can accomplish the core cooling function when configured in the RPV injection mode. Both systems are manually actuated and it is necessary to inhibit containment isolation signals if any are present.

Success of this function is the effective operation of at least 1 train of FAPCS or FPS, operating in RPV injection mode for the duration of the sequence.

At Least 4 DPV Open Automatically (ADS)

If water level drops to Level 1, the ADS logic will automatically start a depressurization sequence. This sequence starts with the opening of SRVs and continues through the opening of the DPVs.

The success criterion for this function is to open at least 4 of 8 DPVs.

Gravity Driven Cooling Systems (VG)

If all of the active injection systems are unavailable after a successful full RPV depressurization, the passive GDCS injects water into the RPV by gravity.

The success criteria for this function is discharge of at least 2 GDCS pools or the discharge of 1 GDCS pool and the opening of at least 1 equalizing line from the suppression pool. In addition, at least 2 GDCS discharge lines must open to provide the flow path from the GDCS pools.

Low Pressure Injection System after ADS (VLFL)

If the low pressure injection system could not provide RPV makeup due to the failure of XS5, then after a successful ADS the FAPCS or the FPS can accomplish the core cooling function in case of failure of the GDCS. Both systems are manually actuated and it is also necessary to inhibit the isolation signals, RPV Level 2 and high drywell pressure, that would be present in this case.

Success of this function is the success of at least 1 train of FAPCS or FPS operating in the RPV injection mode.

RWCU/SDC (WH)

The decay heat removal function can be accomplished by the RWCU/SDC system. This function is effective only if the reactor water level is recovered at normal level above Level 3. This system with the help of the steam sent by the SRV to the suppression pool, is able to support short-term decay heat removal, until the time that the RWCU/SDC alone is sufficient to cool the water in the vessel in the long-term.

The failure of this function is the failure of the system to operate with at least 1 pump at 100 % capability during the sequence mission time. A manual action is necessary to configure the system in this mode.

Vacuum Breaker Closed (DL)

The Steam Suppression System (SSS) has vacuum breakers that open in case the pressure in the wetwell is greater than the pressure in the drywell. PCCS effectiveness in containment heat removal requires that a small pressure differential exist between the drywell and wetwell. To this end, the vacuum breakers between the DW and WW must be leak tight to maintain this small DW to WW pressure differential.

The heading success criteria represented here is that all the vacuum breakers are closed or re-close following a previous actuation.

Passive Containment Cooling System (WP)

The passive containment heat removal function can be performed by the PCCS. The PCCS is effective when the drywell pressure is greater than the wetwell pressure, i.e., all of the vacuum breakers are closed and leak tight.

The system is always open to the containment atmosphere and it has no valves that require opening. Failure of this function is the loss of effectiveness of the heat exchangers in removing the decay heat from containment atmosphere (e.g., tube plugging).

The success criterion for this function is the operation of at least 4 of 6 heat exchangers.

Long Term Containment Cooling System (WT)

PCCS or the Isolation Condensers can perform the long term containment heat removal function as long as water remains in the pools of the upper part of the containment. There is at least enough water present during operation to remove decay heat for at least 24 hours. A connection to the refueling well in the upper reactor building will automatically open to extend this inventory to at least 72 hours. This is backed up by the ability to make up water to the pools using various water systems. In the PRA, the source provided by FPS is credited because it is completely independent, including support systems, of GDCS and the PCCS automatic water makeup.

The success criteria for this function are the connection of the pools to the refueling well or adequate external water sources to the pools.

Suppression Pool Cooling after Depressurization (WLL)

If the core is cooled with GDCS, the decay heat removal function can be accomplished by the FAPCS in the suppression pool cooling mode.

Success of this function is the effective manual initiation and operation of at least 1 FAPCS train.

Vent Containment (WC)

When no containment heat removal system is available, the pressure in the containment will rise. Containment venting is directed by procedure. The actuation of this function is required to avoid the failure of the containment and to provide a means for re-sealing the containment boundary.

The success criterion for the system is manual opening of the vent line.

3.3.1.2 Transient with PCS Unavailable

This group of initiating events (T PCS) includes initiators in which the steam pathway to the condenser is unavailable.

All the functions required to mitigate the events included in this group, except the TPCS, are the same as those following the General Transient. Therefore, the event tree developed for the General Transient is also valid for this event. The only difference is that the TPCS heading is initially failed, and therefore it is removed from the event tree.

The event tree for transient with PCS unavailable initiator is shown in Figure A.3-2.

In a break outside containment in either the ICS or main steam lines occurs, and the isolation of the broken line is successful, the event sequence is identical to the T PCS scenarios. For completeness, the event tree for Transient with PCS unavailable after BOC in IC or MS is shown in Figure A.3-2a.

In the case of RPS failure following the loss of PCS event, the transfer is to the ATWS T PCS event tree.

3.3.1.3 Loss of Feedwater Transient

(1) Development of the Loss of Feedwater Event Tree (T FDW)

This group of initiators consists of the events starting with the total loss of the FDW flow.

In a loss of feedwater event, the initial water level drop is much more severe than in a general transient. It goes down below Level 1.5 almost immediately. This starts a timer sequence that allows water level to be recovered above level 1.5 within a few minutes; otherwise the ADS logic is actuated. The main consequence is that if ADS actuates, ICS cannot be used to remove decay heat.

The design basis for the loss of feedwater event is such that if 3 of 4 isolation condensers actuate and both CRD pumps start in high pressure injection mode upon receipt of Level 2 signal, water level will be restored above Level 1.5 before the timer sequence expires.

If the water level is restored, the event tree proceeds in a similar manner to the general transient, except that TPCS and FDW are not available. TPCS is not available because the MSIVs close on water Level 2.

If ADS successfully activates, CRD is able to keep the core covered; however ICS is not available for core cooling. If CRD fails, then the sequence precedes the same as the general transient branch in which ADS is successful.

If the water level drops below level 1.5 but the ADS does not successfully actuate, ICS remains available for core cooling. Additionally, if the ICS fails too, CRD and FAPCS in LPCI mode (given depressurization using the SRVs only) could still provide core cooling. The event tree allows for this course of events.

The event tree for this event is shown in Figure A.3-3.

In the case of RPS failure, the transfer is to the ATWS T FDW event tree.

(2) Heading Description

An additional two top events are added to T FDW to treat special conditions introduced by the loss of feedwater.

U2CISHORT

The requirement for success at this node is to supply sufficient RPV inventory during the initial transient to avoid the automatic RPV depressurization that could be induced due to low RPV water level. The automatic depressurization is asked at the next node. The inventory requirement only requires operation of the CRD and IC condensate return lines for approximately 15 min to achieve success. Success at this node represents the most likely course of events given the total loss of FDW.

ADS

A second ADS top event is added to the event tree after U2CISHORT. This event is to incorporate the automatic plant response of depressurization on low RPV level that would defeat the IC effectiveness.

3.3.1.4 Loss of Service Water System

(1) Development of the Loss of Service Water and System Event Tree (T SW)

This initiating event produces the failure of the Reactor Closed Cooling Water System (RCCWS) and Turbine Building Closed Cooling Water (TCCW) systems. The event is similar to a loss of the FDW system, except there are three important differences:

- The timing of the loss of RCCWS and TCCWS is not immediate because of the energy absorption available in these two systems.
- Likewise, Feedwater is not lost immediately
- Additional systems that cannot be credited following TSW are RWCU/SCS FAPCS, and CRD.

All the functions required to mitigate the events included in this group (except for those lost as a direct consequence of the initiator) are the same as those following the General Transient. Therefore, the event tree developed for the General Transient is also valid for this event. However, considering that TPCS, UICF, WH, and WLL headings are initially failed, they are removed from the event tree. In addition, the VLF and VLFL headings are changed to the VF and VFL headings, respectively, which takes into consideration that the use of the FPS for low pressure injection is the only available system. The event tree for this event is shown in Figure A.3-4.

In the case of RPS failure, the transfer is to the ATWS T SW event tree.

(2) Heading Description

Low Pressure Fire Protection (VF)

After successful XS5 depressurization, the FPS can accomplish the core cooling function when configured in the RPV injection mode. This system is manually actuated only.

Success of this function is the success of at least 1 train of FPS, operating in the RPV injection mode for the sequence mission time.

3.3.1.5 Loss of Preferred Power Transient

The loss of the preferred power sources (T LOPP) causes a loss of feedwater. The discussion provided above relative to the loss of feedwater response remains applicable to the T LOPP event tree. In addition, the non-safety diesel generators start. However, in case of diesel failure, there is the possibility of restoration of the offsite power before the reactor water level decreases below RPV Level 1 to allow the use of the active systems given that the ICS are ineffective. The modeling of the diesel generator and the power recovery is included in the AC fault tree. There are no new scenarios associated with LOPP or SBO situations.

Therefore, the event tree developed for the Loss of Feedwater Transient is also valid for this event situation. The event tree for this event is shown in Figure A.3-5.

In the case of RPS failure, the transfer is to the ATWS T GEN LOPP event tree.

3.3.1.6 Inadvertent Opening of an RV

The Inadvertent Opening of an RV (T IORV) at power produces the need for a reactor trip when the temperature in the suppression pool increases above the allowed limit. For this initiating event, the TPCS and I functions are unavailable and the M function is guaranteed successful.

All the functions required to mitigate the events included in this group are the same as those following the General Transient. Therefore, the event tree developed for the General Transient is also valid for this event. However, considering that the TPCS and I headings are initially failed and the M heading is successful, they are removed from the event tree.

Additionally, the XS5 heading only requires 4 additional SRVs in this case; nevertheless, for simplification, the same success criteria as described in T GEN are used. The event tree for Inadvertent Opening of an RV is shown in Figure A.3-6.

In the case of RPS failure, the transfer is to the ATWS T IORV event tree.

3.3.2 Failure to Scram Following Transients

3.3.2.1 ATWS Following General Transient or LOPP (Base Case)

- (1) Development of the ATWS event tree sequence transferred from Transient with TPCS available or LOPP

The sequence of events following the failure of the RPS after the initiating event of this type in the ESBWR is described below.

ATWS sequences are initiated by any event in which there is a failure of either the hydraulic or electrical control rod insertion modes. To mitigate ATWS, sodium pentaborate at high pressure (in sufficient quantity to achieve subcriticality) is injected into the reactor core region. This is done by the Standby Liquid Control System (SLCS) that stores sodium pentaborate at high pressure inside an accumulator. The SLCS is activated, automatically, on high RPV pressure or an RPV Level 2 signal in combination with Average Power Range Monitors (APRM) not down-scale signal lasting for 3 minutes.

In order to control reactivity, ESBWR also uses FDW runback and ADS inhibit functions. FDW runback reduces the power by affecting natural circulation. ADS inhibit allows adequate reactor pressure to be maintained to take credit for IC operability. The ADS inhibit signal is sealed in. Under these conditions, power generation is reduced within the cooling capability of IC and/or CRD until SLCS injection is completed.

ADS inhibit also prevents uncontrolled reactor depressurization and the subsequent GDACS injection which could lead to boron washing out of the RPV. It is assumed that without ADS inhibit the resulting boron dilution causes recriticality.

Following an ATWS with the PCS remaining available, the nuclear boiler system overpressure protection is provided by the actuation of three possible systems: SRV, Bypass, or IC.

With successful SLC injection, core cooling can be performed: (1) by the IC if all the valves that are open also close correctly, (2) by the FDW or CRD with the heat removal by the FAPCS in the cooling mode; or, (3) by RWCU/SDC if isolation signals are inhibited and the filters are bypassed. In case of failure of high pressure systems, core damage is assumed because the

reactor is depressurized, and reactivity control is not credited in preventing core damage because of the potential for boron dilution.

The long term heat removal function is similar to that for transients except that no credit is taken for PCCS, because ATWS mitigation is not carried out at low pressure, as discussed earlier. The preferred source for the residual heat removal is FAPCS, because in ATWS situations the RWCU/SDC is automatically isolated. Nevertheless, if the FAPCS system fails the isolation signal can be overridden and the filters bypassed to allow the use of the system. The General Transient event tree is extensively modified to model ATWS response by eliminating TPCS, M, U1CF, XS5, VLF, VLFL, WH, MDS, ADS, VG, DL, WT, VLL and WP headings, and including the new headings at the beginning of the tree that are described below.

This leaves two headings that remain similar to the general transient: I and WC.

The event tree for ATWS is shown in Figure A.3-7. The event tree for ATWS from Loss of Preferred Power is shown in Figure A.3-7a.

(2) Heading Description

ATWS Initiator (ATWS-GEN-LOPP)

The initiating event for the ATWS event tree is a fault tree with the general transient ANDed with the failure of the RPS and/or ARI or LOPP ANDed with failure of RPS and/or ARI.

At least 9 SRV Open (MA)

In this case, RPV pressure increases to the SRV actuation setpoint value. The success of this function is to open automatically at least 9 SRVs. Conservatively, the failure of this function is assumed to lead to core damage due to RPV rupture and re-criticality at low RPV pressure.

ADS Inhibit (IA)

The ESBWR ATWS mitigation strategy is to maintain the reactor at high pressure until a controlled depressurization can be performed. To maintain the reactor at pressure, an ADS inhibit signal is provided on: (1) high RPV pressure; or, (2) RPV water Level 2 and APRM-not-down-scale signal. This avoids the potential consequences of boron dilution associated with reactor depressurization. Conservatively, the failure of this function is assumed to lead to core damage due to failure to successfully control power.

Standby Liquid Control System (SL)

A reactor shutdown can be achieved by the SLCS injection of concentrated boron at high pressure. Actuation of the SLCS is automatic on high pressure or RPV Level 2 and APRM-not-down-scale for 3 minutes. Conservatively, no back up or long term recovery of this function is considered. The failure of this function is assumed to lead to core damage due to failure to successfully control power.

All SRVs Close (PA)

After the opening of at least 9 SRVs, it is necessary to take into consideration that all of the opened SRVs close properly when the pressure in the RPV decreases below the setpoint of the SRVs. All SRVs must be closed to give credit to the isolation condenser function I, in bringing the plant to a safe state as in the General Transient.

High Pressure Injection Systems (UCF)

The high pressure injection function can be accomplished as was indicated for the General Transient, using FDW or CRD. However, in this case, the success of the CRD requires 2 CRD pumps taking suction from the CST.

Suppression Pool Cooling (WL)

The decay heat removal function preferred for ATWS is accomplished by FAPCS in suppression pool cooling mode, if FAPCS is not previously actuated.

Success of this function is the effective operation of at least 1 FAPCS train. This operational mode of FAPCS initiates automatically upon high temperature in the suppression pool, and if the FAPCS in the injection mode is not previously actuated.

RWCU/SDC (WHA)

The decay heat removal function can be accomplished by the RWCU/SDC system, as indicated for the General Transient, but in this case it is necessary to inhibit isolation signals developed by ATWS logic and bypass the filters.

3.3.2.2 ATWS Following Transient Loss of PCS

- (1) Development of the ATWS event tree sequences transferred from Transient with TPCS unavailable

The sequence of events following the failure of the RPS after the initiating event are the same as those described in Section 3.3.2.1, the ATWS following a Transient with TPCS available or LOPP. Therefore, the event tree developed for ATWS following a Transient with TPCS available or LOPP is also valid for this event, considering that a new heading, RF, is included after the MA heading. The RF heading is described below.

The event tree for ATWS following transient with loss of PCS is shown in Figure A.3-8. The event tree for ATWS from Loss of PCS after BOC in IC or MS is shown in Figure A.3-8a.

- (2) Heading Description

Feedwater Pumps Run-Back (RF)

An FDW runback is an automatic function that occurs on high RPV pressure and APRM-not-down-scale signal. The FDW is run back to a no-flow condition by tripping FDW pumps. This action is necessary to limit power production in the short term following the accident, in order to keep the pressure spike in the RPV below the reference limit. Conservatively, the failure of this function is assumed to lead to core damage due to a failure to successfully control power.

3.3.2.3 ATWS Following Transient with Loss of Feedwater System

The event tree developed for ATWS sequences transferred from the General Transient event tree is also valid for this scenario. However, the UCF heading is changed to the UC heading, which requires success of two CRD pumps for high pressure injection.

Note that ADS inhibit prevents the premature RPV depressurization due to low RPV water level as a result of the loss of feedwater. This is different than the non-ATWS loss of feedwater transient accident sequences.

The event tree for ATWS following a transient with loss of feedwater system is shown in Figure A.3-9.

3.3.2.4 ATWS Following Transient with Loss of Service Water System

The event tree developed for the ATWS following a General Transient is also valid for this scenario considering that the headings UCF, WHA, WC, and WL are initially failed or are not required and they are removed from the event tree.

The event tree for ATWS following Transient with Loss of Service Water System is shown in Figure A.3-10.

3.3.2.5 ATWS Following Inadvertent Opening of an RV

The event tree developed for ATWS following a General Transient is also valid for this scenario considering that the headings PA, I, and WT are initially failed or are not required and they are removed from the event tree.

The event tree for ATWS following inadvertent opening of an RV is shown in Figure A.3-11.

3.3.2.6 ATWS from Small LOCA above Reactor Core

While this event technically begins as a loss of coolant accident, the sequence of mitigating events is the same as the ATWS IORV event.

The event tree from ATWS from small LOCA above reactor core is shown in Figure A.3-11a.

3.3.3 LOCAs

3.3.3.1 Large Steam Break (above L3) Other than Feedwater Lines (Base Case)

(1) Development of the Large Steam Break Event Tree (LL S)

A large LOCA is an event leading to a rapid loss of coolant, resulting in a rapid consequential depressurization, such that no emergency depressurization is required in order to permit the low pressure injection systems, including GDSCS, to inject.

Immediately following the break, there is a drop in the RPV water level. The FDW system attempts to maintain the water level in the vessel. If FDW is not available, CRD automatically injects at RPV Level 2. FAPCS or FPS in injection mode can be manually actuated after defeating automatic isolation. As an alternative to these systems, GDSCS and the equalizing lines are sufficient to provide core cooling. Because the RPV is at low pressure, it is considered that the injection system components operability will not be impaired due to high pressure, given the pump head and containment ultimate capability. When the core is cooled with GDSCS, the RWCU/SDC is the preferred residual heat removal method after the water level in the vessel is restored. If RWCU/SDC is not available, core cooling requires that the LOCA blowdown energy be dissipated by the condensation of the steam that passes through the vents to the suppression pool. If the vapor suppression function is successful, the decay heat removal is accomplished by PCCS, by natural convection and steam condensation. One FAPCS system train can also accomplish the long-term heat removal. Finally, if all other heat removal systems fail, the containment vent can be initiated to reduce the pressure in the containment.

All of the functions required to mitigate the events included in this group, except for TPCS, I, XS5, ADS, VLFL, and MDS headings, are the same as those following the General Transient. Therefore, the event tree developed for the General Transient is also valid for this event, except that the TPCS and I headings are initially failed, and XS5, ADS, and VLFL headings are not required and are removed from the event tree. Also a new heading, DS heading, is included after the RPS heading, as described below.

The event tree for large steam breaks (above L3) other than feedwater lines is shown in Appendix A.3 (Figure A.3-12).

(2) Heading Description

Steam Suppression System (DS)

The Steam Suppression System has vacuum breakers that must be initially closed during the LOCA blow-down to allow steam condensation in the pool. These vacuum breakers must also subsequently open if drywell pressure decreases relative to the wetwell pressure. This would avoid negative pressure failures.

This heading takes into consideration the need of at least two of three vacuum breakers to be closed for the steam suppression function to be successful and at least one vacuum breaker to open after steam suppression to avoid containment failure due to negative pressures in the drywell.

3.3.3.2 Large Steam Breaks (above L3) in FDW (A) Line

The sequence of events subsequent to the initiating event is the same as those described in large steam breaks other than FDW lines. Therefore, the event tree developed for large steam breaks other than FDW lines is also valid for this initiating event, except that the U1CF heading is changed to U1C, and the VLF and VLFL headings are not applicable, due to the failure of FDW line A.

The FDW is assumed failed for a large break in either FDW line due to loss of FDW suction inventory.

The event tree for large steam breaks (above L3) in FDW (A) line is shown in Figure A.3-13

3.3.3.3 Large steam breaks (above L3) in FDW (B) line.

The sequence of events after the initiating event is the same as those described in large steam breaks other than FDW lines. Therefore, the event tree developed for large steam breaks other than FDW lines is also valid for this initiating event, except that the U1CF heading is not applicable due to the failure of the line B of the FDW.

This is because failure of the "B" FDW line results in the following:

- FDW is assumed failed for a large break in either FDW line due to loss of FDW suction inventory.
- CRD is failed due to the "B" FDW line failure because CRD injection is through the "B" FDW line.

The event tree for large steam breaks (above L3) in FDW (B) line is shown in Figure A.3-14.

3.3.3.4 Small Steam Breaks (above L3)

A small LOCA is an event leading to a slow loss of coolant, such that depressurization is required in order to permit the low pressure injection systems to inject. This group includes both the medium and small LOCA group included in Section 2, "PRA – Initiating Event Analysis". In this group of initiating events, it is considered that the size of the break does not affect the availability of any system that connects to the line containing the small LOCA.

All of the functions required to mitigate the events included in this group, except for TPCS and I headings, are the same as those following the General Transient. TPCS and I headings are initially failed and thus are removed from the event tree.

The event tree for small steam breaks (above L3) is shown in Figure A.3-15.

3.3.3.5 Medium Liquid Breaks (below L3) Other than RWCU/SDC Lines

(1) Development of the medium liquid breaks (below L3) other than RWCU/SDC lines.

A medium liquid LOCA is an event leading to a rapid loss of coolant, but with a smooth consequential depressurization, that allows low pressure injection system actuation, but requires that a full depressurization occurs in order to permit GDCS to inject.

In this initiating event, it is considered that the CRD pumps are not able to compensate for the inventory lost through the break. It is also considered that the FAPCS in injection mode, which takes suction from the suppression pool, is lost on suppression pool low water level, before the level of water outside the vessel can maintain the core covered. Therefore, FAPCS in the suppression pool cooling mode is not available in those cases. Finally, a greater amount of inventory must be discharged from the GDCS pool combined with equalizing lines to guarantee that the core is covered in all cases.

Accordingly, all the functions required to mitigate the events included in this group except for the TPCS and I headings, are the same as those following the General Transient. Therefore, the event tree developed for the General Transient is also valid for this event, except that the UCF heading has is changed to UF, to take into consideration that 2/2 CRD pumps operating is not enough to compensate for the leak from the break. The VLF and VLFL headings are changed to the VF and VFL, to take into consideration that FAPCS is not credited in these situations. Finally VG is changed to VG2, and is described below. TPCS and I headings are initially failed and are removed from the event tree, and a new heading is included after the RPS heading, DS heading, which is described under the large steam breaks event tree.

The event tree for this event is shown in Figure A.3-16.

(1) Heading Description

Two Gravity Driven Cooling Systems (VG2)

This function is similar to the one described in the General Transient section. However, in this case, the passive GDCS has to open two GDCS lines, discharge at least 2 GDCS pools and also one equalizing line must be opened for long-term cooling.

The success criteria for this function is the discharge of at least 2 GDCS pools and the opening of at least 1 equalizing line and functionality of 2 GDCS 150 mm dia. lines.

3.3.3.6 Medium Liquid Breaks (below L3) in RWCU/SDC Lines

The sequence of events after the initiating event is the same as described above for Medium liquid breaks. Therefore, the event tree developed for Medium liquid breaks in other lines (Section 3.3.3.5) is also valid for this initiating event, considering that the WH heading is not applicable in this case due to the break location.

The event tree for medium liquid breaks (below L3) in RWCU/SDC lines is shown in Figure A.3-17.

3.3.3.7 Small Liquid LOCA (below L3) other than RWCU/SDC Lines

All of the functions required to mitigate the events included in this group, except for the TPCS, are the same as those following the General Transient. Therefore, the structure of the event tree developed for the General Transient is modified to require that both CRD (or FDW) and ICS be successful to reach a safe condition similar to the General Transient. It is also considered that the FAPCS, which takes suction from the suppression pool, is lost on low level in the suppression pool, before the level of water outside the vessel can maintain the core cover, and that the FAPCS in the suppression cooling mode is not available in that case. Because TPCS is initially failed, it is removed from the event tree.

The event tree for small liquid LOCA (below L3) other than RWCU/SDC lines is shown in Figure A.3-18.

3.3.3.8 Small Liquid LOCA (below L3) in RWCU/SDC Lines

The sequence of events after the initiating event is the same as what is described in Section 3.3.3.7 for small liquid breaks other than RWCU/SDC lines. Therefore, the event tree developed for small liquid breaks other than in RWCU/SDC lines is also valid for this initiating event, considering that the WH heading is not applicable in this case.

The event tree for small liquid LOCA (below L3) in RWCU/SDC lines is shown in Figure A.3-19.

3.3.3.9 Reactor Vessel Rupture

A Reactor Vessel Rupture (RVR) is considered, in a conservative way, as an event leading to a rapid depressurization and a loss of coolant through a large break in the bottom of the RPV.

For this initiating event, it is considered that no active high pressure or low pressure system is able to compensate for the inventory lost through the break and maintain the level in the reactor above RPV Level 1 before core damage. Only the injection by the GDSCS can provide an amount of water quickly and with enough volume to allow the level of water outside the vessel to maintain the core covered.

The event tree structure developed for medium liquid break in "other than RWCU/SDC" lines (Section 3.3.3.5) is also valid for this initiating event, considering that the UF, VF, ADS, VFL and WH headings are not applicable due to the break size and location.

The event tree for reactor vessel rupture is shown in Figure A.3-20. The event tree for Reactor Vessel Rupture after Loss of Preferred Power is shown in Figure A.3-20a.

3.3.4 Break Outside Containment

In addition to a LOCA occurring within the drywell as described in Section 3.3.3, there is also the possibility of breaks outside the containment, which could lead to loss of primary coolant if not isolated in time.

3.3.4.1 Steam Break Outside Containment in Main Steam Lines (Base Case)

(1) Development of the Steam break outside containment in Main Steam Lines Event Tree

Immediately following the break there is a drop in the RPV pressure and an isolation signal is sent to the MSIVs to close. In the case of successful isolation, the scenario develops into a T PCS. If the isolation fails, the containment is bypassed and all the functions required to mitigate the event included in this group, except for TPCS, I, WH, M, DL, WP, WT, WLL and WC headings are the same as those following the General Transients. Therefore, the event tree developed for the General Transient is also valid for this event, considering that all of the above headings are failed or are not required and are removed from the event tree. Also, a new heading is included before the RPS heading, I ML heading, with is described below.

The event tree for this event is shown in Figure A.3-21.

(2) Heading Description

Isolation of all the Steam Lines (I ML)

This heading takes into consideration the need for at least one isolation valve to close in each steam line to stop the loss of inventory outside the containment before RPV Level 1 is reached.

3.3.4.2 Steam Break Outside Containment in FDW A Line

(1) Development of the Steam break outside containment in FDW A lines Event Tree

The sequence of events after the initiating event is the same as described in steam breaks in Main Steam lines. Conservatively, credit is not given to the WH heading because the RWCU isolation condition is not easily overridden. Therefore, the event tree developed for Steam breaks in Main Steam lines is also valid for this initiating event, considering that the UICF heading is changed to the UIC, the VLF and WH headings are not applicable, due to the failure of line A of the FDW or the isolation signal and a new heading is changed from I ML to I FDW A heading, which is described below.

The event tree for this event is shown in Figure A.3-22.

(2) Heading Description

Isolation of FDW A Line (I FDWA)

This heading takes into consideration the need for at least one isolation valve in FDW line A to prevent the loss of inventory outside the containment before, RPV Level 1 is reached.

3.3.4.3 Steam break outside containment on FDW B Lines

(1) Development of the Steam break outside containment in FDW B lines Event Tree

The sequence of events after the initiating event is the same as described in steam breaks in Main Steam lines. Conservatively, credit is not given to the WH heading because the RWCU isolation

condition is not easily overridden. Therefore, the event tree developed for steam breaks in Main Steam lines is also valid for this initiating event, considering that the UICF and the WH headings are not applicable, due to the failure of FDW line B, or failure of the isolation signal. Also, a new heading is included before the RPS heading, I FDW B heading, which is similar to the one described for Line A.

The event tree for this event is shown in Figure A.3-23.

(2) Heading Description

Isolation of the FDW B Line (I FDWB)

This heading takes into consideration the need for at least one isolation valve to close to stop the loss of inventory outside the containment before RPV Level 1 is reached.

3.3.4.4 Large Steam Break Outside Containment in IC Lines

(1) Development of the steam break outside containment in IC lines Event Tree

RPV pressure drops immediately following the break and an isolation signal is sent to the IC valves to close. In the case of successful isolation of the affected line, the scenario develops to a T PCS but with only 3 ICs available. If the isolation function fails, the containment is open and the sequence of events following this initiating event are the same as that described in steam breaks in Main Steam lines sequences. Therefore, the event tree developed for Steam breaks in Main Steam lines is also valid for this initiating event, considering that the I ML heading is changed to I IC.

The event tree for this event is shown in Figure A.3-24.

(2) Heading Description

Isolation of the IC Line (I IC)

This heading takes into consideration the need for at least one IC isolation valve of the affected IC to close to terminate the loss of inventory outside the containment before RPV Level 1 is reached.

3.3.4.5 Large Break Outside Containment in RWCU/SDC Lines

(1) Development Event Tree for the large break outside containment in RWCU/SDC lines

An isolation signal is sent to the RWCU/SDC valves to close, immediately following the line break. This scenario develops to a General Transient but with the WH function failed in the case of successful automatic isolation. In the case of the isolation heading failure, (with failure due to a break being located below the level of the TAF of the core), and after a reactor trip and ADS function, all of the water inventory in the containment is available, (heading VG3), or in case of failure of this heading by the use of the VFL heading, allows time necessary to proceed to a manual isolation of the broken RWCU/SDC line before the core could be uncovered in a long time frame. Isolation headings, I RWCU/SDC and MI RWCU/SDC and VG3 function are described below.

The event tree is shown in Figure A.3-25.

(1) Heading Description

Isolation of the RWCU/SDC Line (I RWCU)

This heading considers the need for at least one isolation valve to close to stop the loss of inventory outside the containment before, in the worst case, RPV Level 1 is reached.

Manual Isolation of the RWCU/SDC Line (MI RWCU)

This heading considers the need that the manual isolation valve, outside the Reactor Building, in the broken line closes to stop the loss of inventory outside the containment before core damage.

Three Gravity Driven Cooling Systems (VG3)

This function is similar to the one described in the General Transient section, but in the case of failure to isolate the break outside containment, the passive GDCS is required to discharge all of the available water to the RPV by gravity from the three pools. At least one equalizing line is required to open for long-term core cooling.

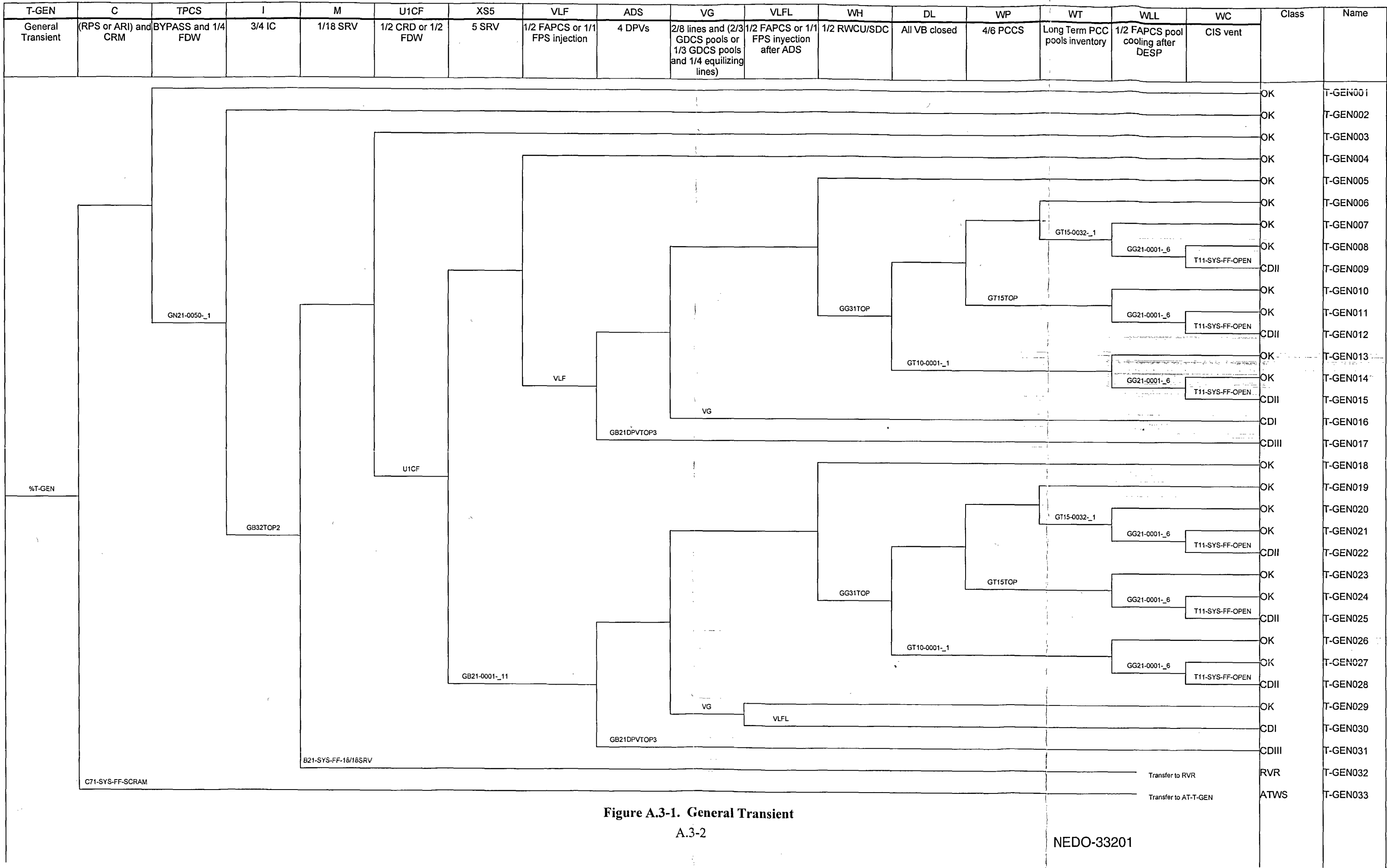
The success criteria for this function are the discharge of all three GDCS pools and the opening of at least 1 equalizing line.

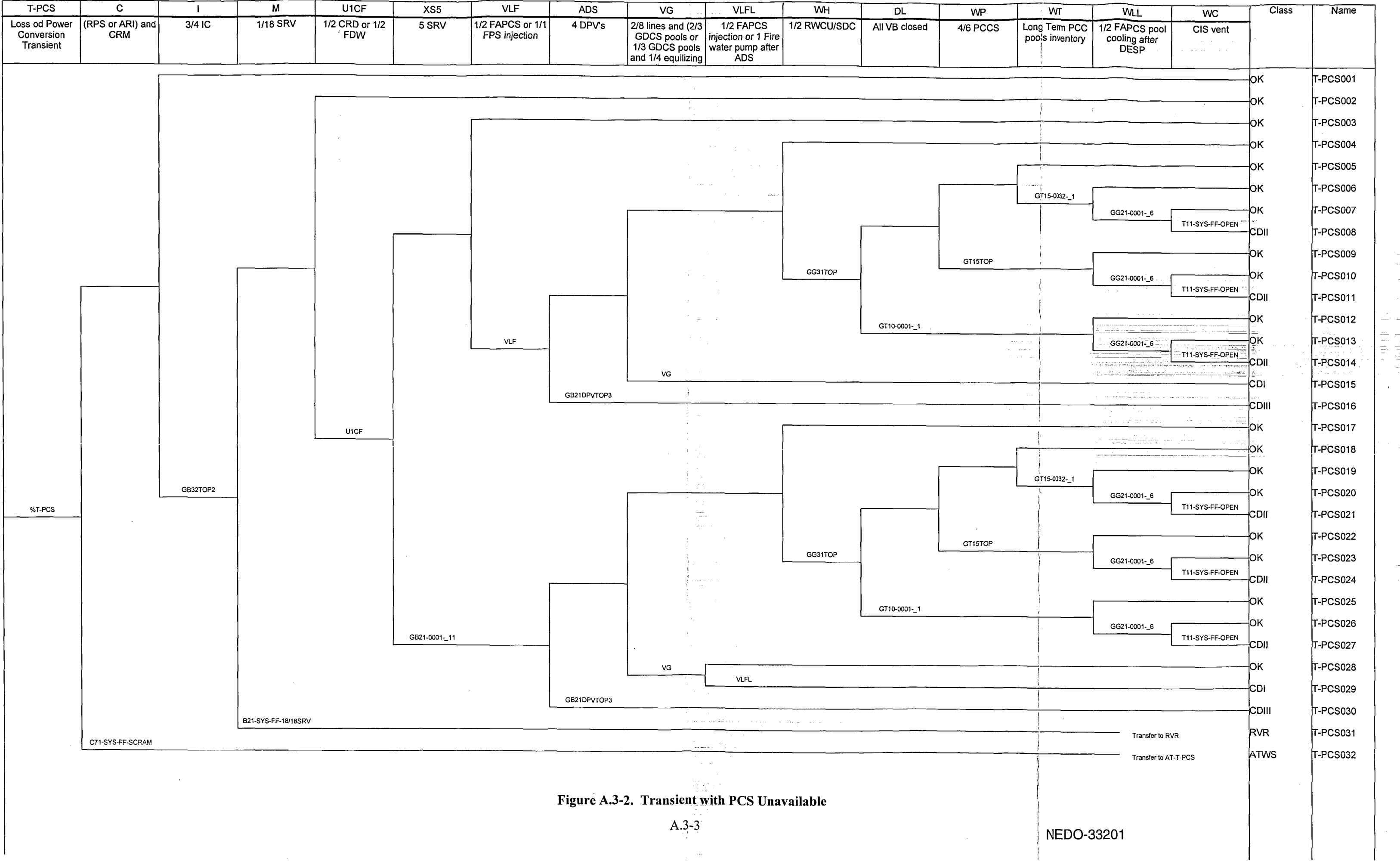
3.4 REFERENCES

- 3-1. ESBWR DCD Tier 2, Chapter 6, Engineering Safety Features, 26A6642AT, Rev. 1
- 3-2. ESBWR DCD Tier 2, Chapter 15, Safety Analysis, 26A6642BP Rev 1
- 3.3 MAAP 4.0.6 ESBWR Application and Comparison to TRACG Benchmarks; *MAAP Support of ESBWR Design Certification Document*, EPRI, Palo Alto, CA and General Electric Co.; 2005 1011712

APPENDIX A.3

- A.3-1. General Transient (BASE Case)
- A.3-2. Transient with PCS Unavailable
- A.3-2a Transient with PCS unavailable after BOC in IC or MS
- A.3-3. Loss of Feedwater Transient (T FDW)
- A.3-4. Loss of Service Water System (T SW)
- A.3-5. Loss of Preferred Power Transient (T-LOPP)
- A.3-6. Inadvertent Opening of a Relief Valve (IORV)
- A.3-7. ATWS from General Transient or LOPP (Base Case)
- A.3-7a ATWS from Loss of Preferred Power
- A.3-8. ATWS from Transient Loss of PCS
- A.3-8a ATWS from Loss of PCS after BOC in IC or MS
- A.3-9. ATWS from Transient with Loss of Feedwater System
- A.3-10. ATWS from Transient with Loss of Service Water System
- A.3-11. ATWS from Inadvertent Opening of a Relief Valve
- A.3-11a ATWS from Small LOCA above Reactor Core
- A.3-12. Large Steam Breaks (above L3) other than Feedwater lines
- A.3-13. Large Steam Breaks (above L3) on FDW (A) lines
- A.3-14. Large Steam Breaks (above L3) on FDW (B) line
- A.3-15. Small Steam Breaks (above L3)
- A.3-16. Medium Liquid Breaks (below L3) other than RWCU/SDC Lines
- A.3-17. Medium Liquid Breaks (below L3) in RWCU/SDC Lines
- A.3-18. Small Liquid LOCA (below L3) other than RWCU/SDC Lines
- A.3-19. Small Liquid LOCA (below L3) in RWCU/SDC Lines
- A.3-20. Reactor Vessel Break
- A.3-20a Reactor Vessel Rupture after Loss of Preferred Power
- A.3-21. Steam Break Outside Containment on Main Steam lines
- A.3-22. Steam Break Outside Containment on FDW A Lines
- A.3-23. Steam Break Outside Containment on FDW B Lines
- A.3-24. Large Steam Break Outside Containment on IC Lines
- A.3-25. Large Liquid Break Outside Containment on RWCU/SDC Lines





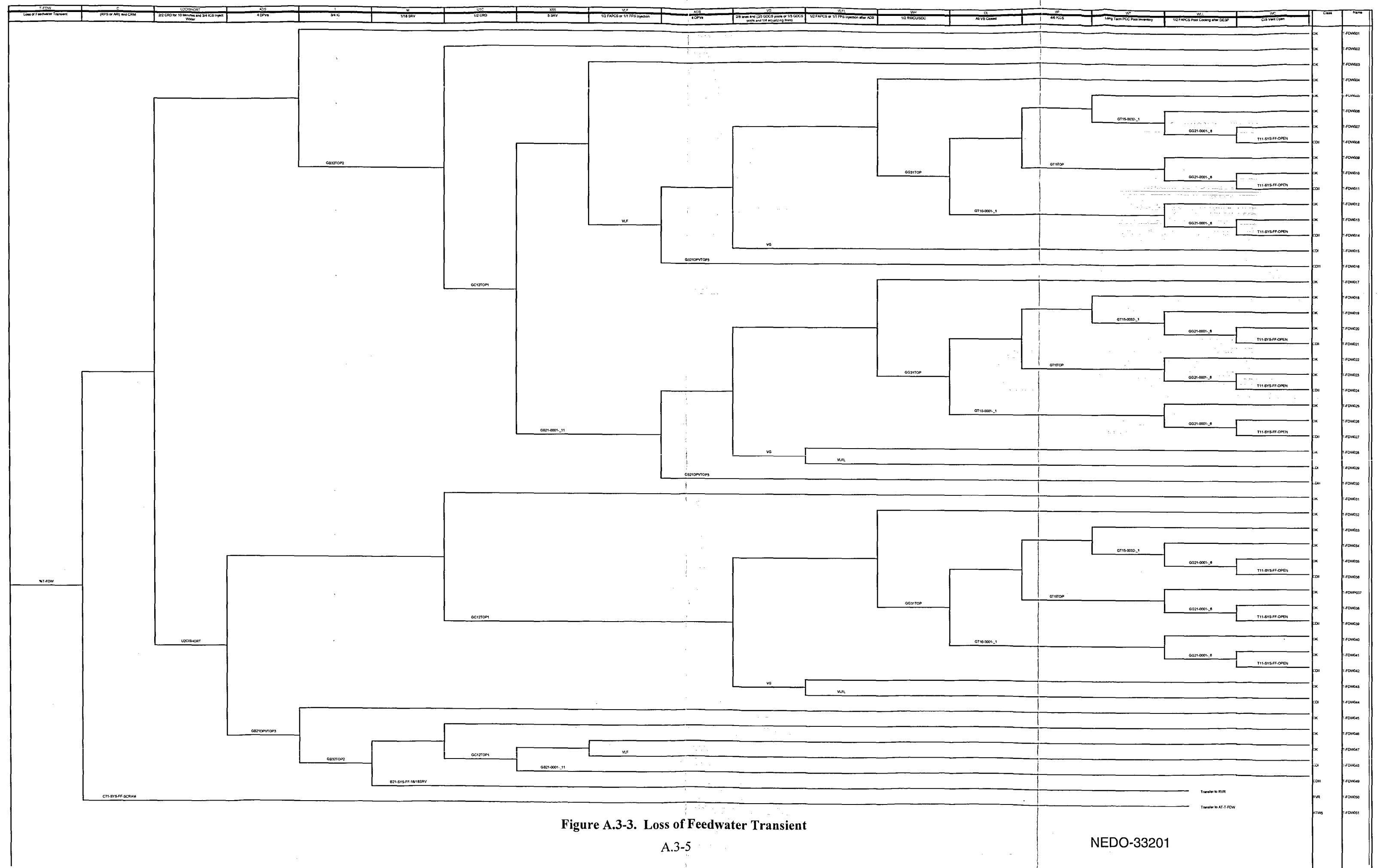
| T-PCSB | C | I | M | U1CF | XS5 | VLF | ADS | VG | VLFL | WH | DL | WP | WT | WLL | WC | Class | Name |
|---|----------------------|-----------|----------|--------------------|-------|--------------------------------|---------|--|--|--------------|---------------|----------|-------------------------------|-----------------------------------|----------|-------|-----------|
| Loss of Power Conversion System after BOC | (RPS or ARI) and CRM | 3/4 IC | 1/18 SRV | 1/2 CRD or 1/2 FDW | 5 SRV | 1/2 FAPCS or 1/1 FPS injection | 4 DPV's | 2/8 lines and (2/3 GDCS pools or 1/3 GDCS pools and 1/4 equilizing | 1/2 FAPCS injection or 1 Fire water pump after ADS | 1/2 RWCU/SDC | All VB closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| T-PCSB | | | | | | | | | | | | | | | | OK | T-PCSB001 |
| | | | | | | | | | | | | | | | | OK | T-PCSB002 |
| | | | | | | | | | | | | | | | | OK | T-PCSB003 |
| | | | | | | | | | | | | | | | | OK | T-PCSB004 |
| | | | | | | | | | | | | | | | | OK | T-PCSB005 |
| | | | | | | | | | | | | | | | | OK | T-PCSB006 |
| | | | | | | | | | | | | | | | | OK | T-PCSB007 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB008 |
| | | | | | | | | | | | | | | | | OK | T-PCSB009 |
| | | | | | | | | | | | | | | | | OK | T-PCSB010 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB011 |
| | | | | | | | | | | | | | | | | OK | T-PCSB012 |
| | | | | | | | | | | | | | | | | OK | T-PCSB013 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB014 |
| | | | | | | | | | | | | | | | | CDI | T-PCSB015 |
| | | | | | | | | | | | | | | | | CDIII | T-PCSB016 |
| | | | | | | | | | | | | | | | | OK | T-PCSB017 |
| | | | | | | | | | | | | | | | | OK | T-PCSB018 |
| | | | | | | | | | | | | | | | | OK | T-PCSB019 |
| | | | | | | | | | | | | | | | | OK | T-PCSB020 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB021 |
| | | | | | | | | | | | | | | | | OK | T-PCSB022 |
| | | | | | | | | | | | | | | | | OK | T-PCSB023 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB024 |
| | | | | | | | | | | | | | | | | OK | T-PCSB025 |
| | | | | | | | | | | | | | | | | OK | T-PCSB026 |
| | | | | | | | | | | | | | | | | CDII | T-PCSB027 |
| | | | | | | | | | | | | | | | | OK | T-PCSB028 |
| | | | | | | | | | | | | | | | | CDI | T-PCSB029 |
| | | | | | | | | | | | | | | | | CDIII | T-PCSB030 |
| | RVR | T-PCSB031 | | | | | | | | | | | | | | | |
| | ATWS | T-PCSB032 | | | | | | | | | | | | | | | |

Figure A.3-2a. Transient with PCS Unavailable after BOC in IC or MS

A.3-4

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Figure A.3-2a. Transient with PCS Unavailable after BOC in IC or MS



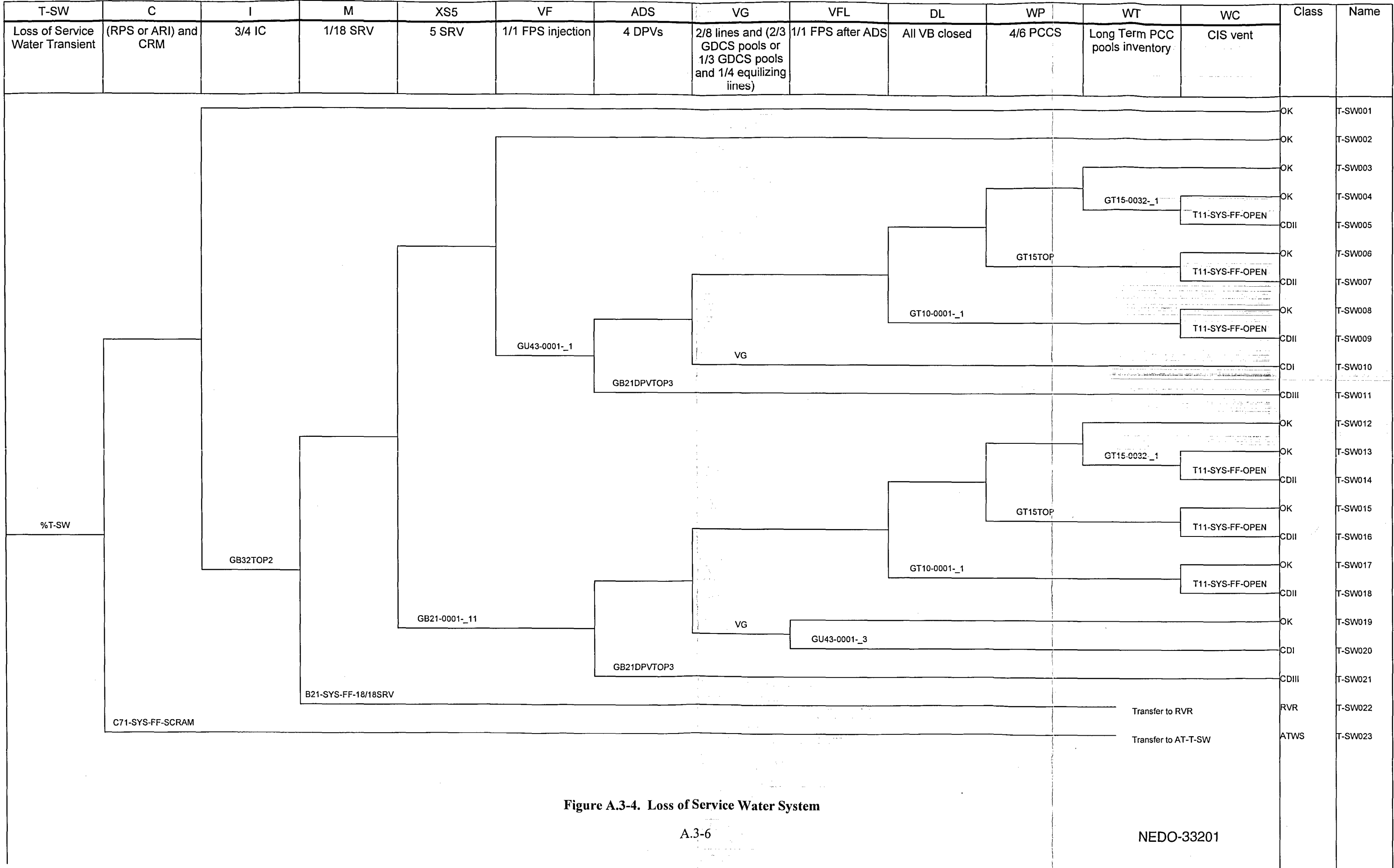


Figure A.3-4. Loss of Service Water System

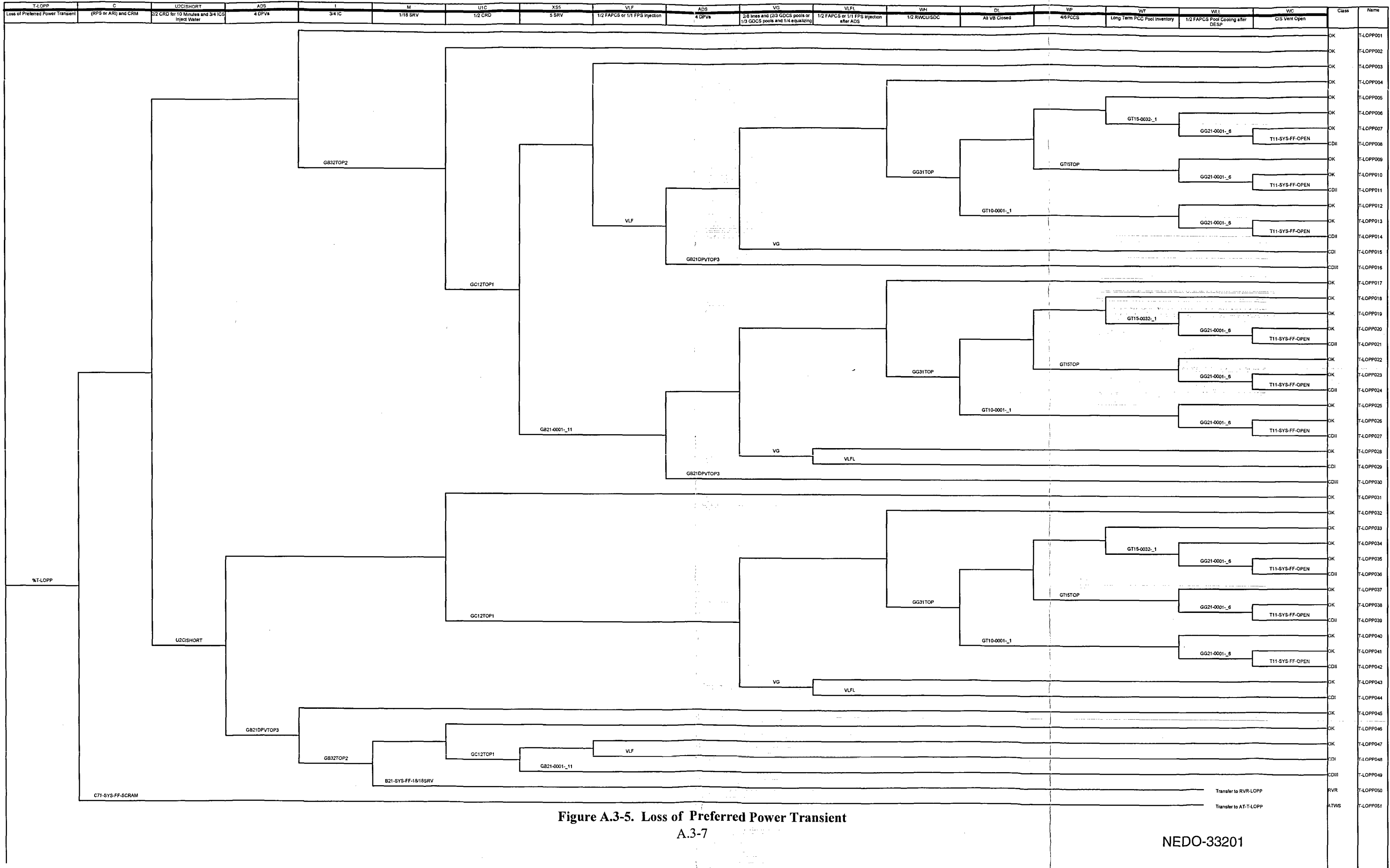
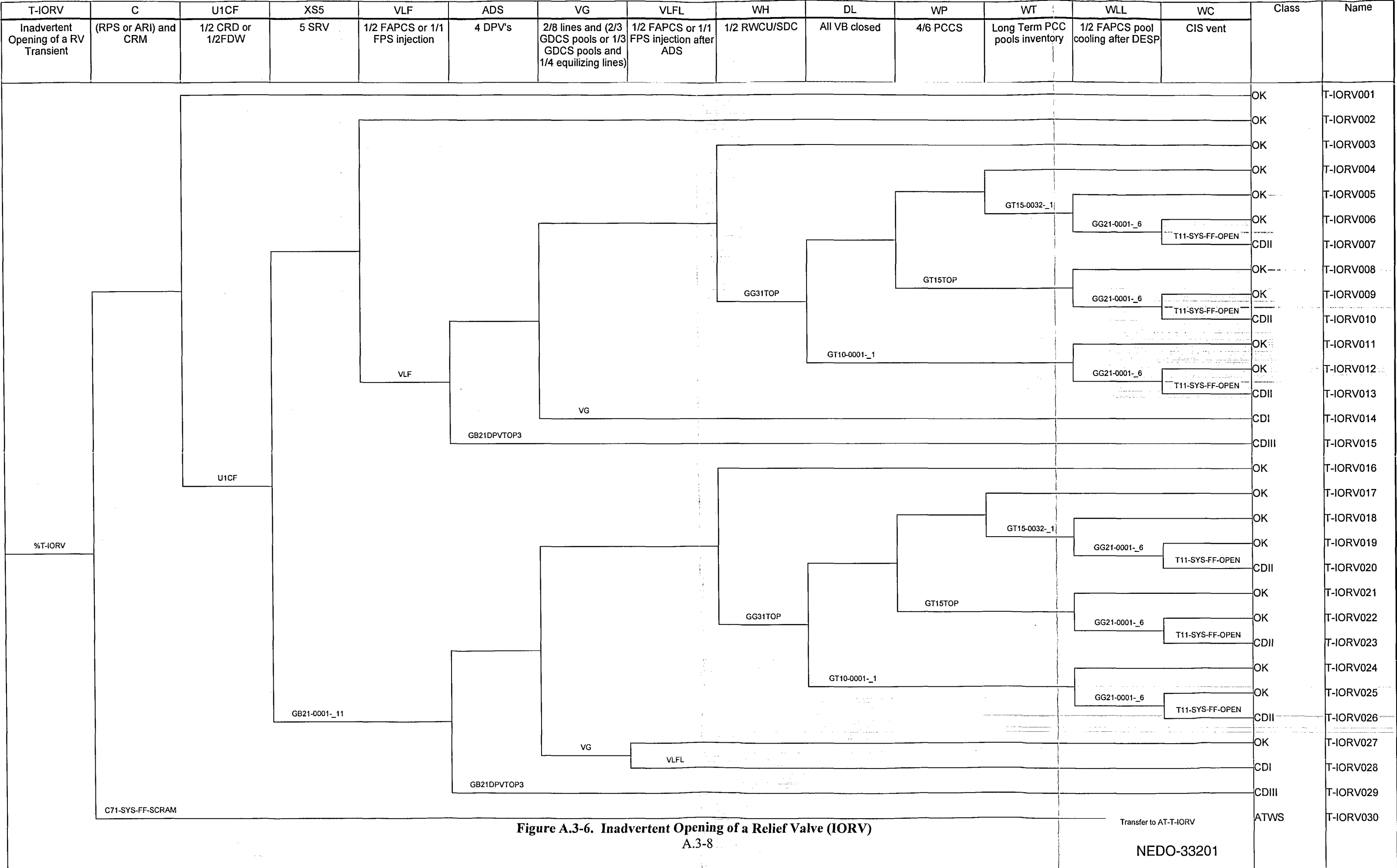


Figure A.3-5. Loss of Preferred Power Transient
A.3-7



| AT-T-GENERAL | MA | IA | SL | PA | I | UCF | WL | WHA | WC | Class | Name |
|------------------------------|------------|----------------|----------|---------------|--------|--------------------|------------------------|--------------|----------|-------|-------------|
| ATWS after General Transient | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | All SRV Close | 3/4 IC | 2/2 CRD or 1/2 FDW | 1/2 FAPCS pool cooling | 1/2 RWCU/SDC | CIS vent | | |
| AT-T-GEN | | | | | | | | | | OK | AT-T-GEN001 |
| | | | | | | | | | | OK | AT-T-GEN002 |
| | | | | | | | | | | OK | AT-T-GEN003 |
| | | | | | | | | | | OK | AT-T-GEN004 |
| | | | | | | | | | | CDII | AT-T-GEN005 |
| | | | | | | | | | | CDIII | AT-T-GEN006 |
| | | | | | | | | | | OK | AT-T-GEN007 |
| | | | | | | | | | | OK | AT-T-GEN008 |
| | | | | | | | | | | OK | AT-T-GEN009 |
| | | | | | | | | | | CDII | AT-T-GEN010 |
| | | | | | | | | | | CDI | AT-T-GEN011 |
| | | | | | | | | | | CDIV | AT-T-GEN012 |
| | | | | | | | | | | CDIV | AT-T-GEN013 |
| | | | | | | | | | | CDIV | AT-T-GEN014 |

Figure A.3-7. ATWS from General Transient

| AT-T-LOPP | MA | IA | SL | PA | I | UCF | WL | WHA | WC | Class | Name |
|------------------------------------|------------|----------------|----------|---------------|--------|--------------------|------------------------|--------------|----------|-------|--------------|
| ATWS after Loss of Preferred Power | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | All SRV Close | 3/4 IC | 2/2 CRD or 1/2 FDW | 1/2 FAPCS pool cooling | 1/2 RWCU/SDC | CIS vent | | |
| | | | | | | | | | | OK | AT-T-LOPP001 |
| | | | | | | | | | | OK | AT-T-LOPP002 |
| | | | | | | | | | | OK | AT-T-LOPP003 |
| | | | | | | | | | | OK | AT-T-LOPP004 |
| | | | | | | | | | | CDII | AT-T-LOPP005 |
| | | | | | | | | | | CDIII | AT-T-LOPP006 |
| | | | | | | | | | | OK | AT-T-LOPP007 |
| | | | | | | | | | | OK | AT-T-LOPP008 |
| | | | | | | | | | | OK | AT-T-LOPP009 |
| | | | | | | | | | | CDII | AT-T-LOPP010 |
| | | | | | | | | | | CDI | AT-T-LOPP011 |
| | | | | | | | | | | CDIV | AT-T-LOPP012 |
| | | | | | | | | | | CDIV | AT-T-LOPP013 |
| | | | | | | | | | | CDIV | AT-T-LOPP014 |

Figure A.3-7a. ATWS from Loss of Preferred Power

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| AT-T-PCS | RF | MA | IA | SL | PA | I | UCF | WL | WHA | WC | Class | Name |
|---|-------------|------------|----------------|----------|---------------|--------|--------------------|------------------------|--------------|----------|-------|-------------|
| ATWS after Loss of Power Conversion Transient | FDW Runback | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | All SRV Close | 3/4 IC | 2/2 CRD or 1/2 FDW | 1/2 FAPCS pool cooling | 1/2 RWCU/SDC | CIS vent | | |
| | | | | | | | | | | | OK | AT-T-PCS001 |
| | | | | | | | | | | | OK | AT-T-PCS002 |
| | | | | | | | | | | | OK | AT-T-PCS003 |
| | | | | | | | | | | | OK | AT-T-PCS004 |
| | | | | | | | | | | | CDII | AT-T-PCS005 |
| | | | | | | | | | | | CDIII | AT-T-PCS006 |
| | | | | | | | | | | | OK | AT-T-PCS007 |
| | | | | | | | | | | | OK | AT-T-PCS008 |
| | | | | | | | | | | | OK | AT-T-PCS009 |
| | | | | | | | | | | | CDII | AT-T-PCS010 |
| | | | | | | | | | | | CDI | AT-T-PCS011 |
| | | | | | | | | | | | CDIV | AT-T-PCS012 |
| | | | | | | | | | | | CDIV | AT-T-PCS013 |
| | | | | | | | | | | | CDIV | AT-T-PCS014 |
| | | | | | | | | | | | CDIV | AT-T-PCS015 |

Figure A.3-8. ATWS from Transient Loss of PCS

| AT-T-PCSB | RF | MA | IA | SL | PA | I | UCF | WL | WHA | WC | Class | Name |
|---------------------------------|-------------|------------|----------------|----------|---------------|--------|--------------------|------------------------|--------------|----------|-------|--------------|
| ATWS from Loss of PCS after BOC | FDW Runback | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | All SRV Close | 3/4 IC | 2/2 CRD or 1/2 FDW | 1/2 FAPCS pool cooling | 1/2 RWCU/SDC | CIS vent | | |
| | | | | | | | | | | | OK | AT-T-PCSB001 |
| | | | | | | | | | | | OK | AT-T-PCSB002 |
| | | | | | | | | | | | OK | AT-T-PCSB003 |
| | | | | | | | | | | | OK | AT-T-PCSB004 |
| | | | | | | | | | | | CDII | AT-T-PCSB005 |
| | | | | | | | | | | | CDIII | AT-T-PCSB006 |
| | | | | | | | | | | | OK | AT-T-PCSB007 |
| | | | | | | | | | | | OK | AT-T-PCSB008 |
| | | | | | | | | | | | OK | AT-T-PCSB009 |
| | | | | | | | | | | | CDII | AT-T-PCSB010 |
| | | | | | | | | | | | CDI | AT-T-PCSB011 |
| | | | | | | | | | | | CDIV | AT-T-PCSB012 |
| | | | | | | | | | | | CDIV | AT-T-PCSB013 |
| | | | | | | | | | | | CDIV | AT-T-PCSB014 |
| | | | | | | | | | | | CDIV | AT-T-PCSB015 |
| | | | | | | | | | | | | NEDO-33201 |

Figure A.3-8a. ATWS from Loss of PCS after BOC in IC or MS

| AT-T-SW | MA | IA | SL | PA | I | Class | Name |
|--|---------------------|----------------|--------------|--------------------|----------|-------|------------|
| ATWS after Loss of Service Water Transient | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | All SRV Close | 3/4 IC | | |
| AT-T-SW | | GC74-0001-_11 | GC41-0001-_1 | B21-SYS-FF-1/9OPEN | GB32TOP2 | OK | AT-T-SW001 |
| | | | | | | CDIII | AT-T-SW002 |
| | | | | | | CDI | AT-T-SW003 |
| | | | | | | CDIV | AT-T-SW004 |
| | | | | | | CDIV | AT-T-SW005 |
| | | | | | | CDIV | AT-T-SW006 |
| | B21-SYS-FF-10/18SRV | | | | | | |

Figure A.3-10. ATWS from Loss of Service Water Transient

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| AT-LOCA | MA | IA | SL | UCF | WL | WHA | WC | Class | Name |
|-------------------------------------|------------|----------------|----------|-----------------------|---------------------------|--------------|----------|-------|------------|
| ATWS after Small LOCA above Core | 9/18 SRV's | ADS Inhibition | 2/2 SLCS | 2/2 CRD or 1/2 FDW | 1/2 FAPCS pool cooling | 1/2 RWCU/SDC | CIS vent | | |
| | | | | | | | | OK | AT-LOCA001 |
| | | | | | | | | OK | AT-LOCA002 |
| | | | | | | | | OK | AT-LOCA003 |
| | | | | | | | | CDII | AT-LOCA004 |
| | | | | | | | | CDI | AT-LOCA005 |
| | | | | | | | | CDIV | AT-LOCA006 |
| | | | | | | | | CDIV | AT-LOCA007 |
| | | | | | | | | CDIV | AT-LOCA008 |

Figure A.3-11a. ATWS from Small LOCA above Reactor Core

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| LL-S | C | DS | U1CF | VLF | VG | WH | DL | WP | WT | WLL | WC | Class | Name |
|------------|----------------------|--------------------------|--------------------|--------------------------------|---|--------------|---------------|----------|-------------------------------|-----------------------------------|----------|-------|----------|
| Large LOCA | (RPS or ARI) and CRM | Steam suppression system | 1/2 CRD or 1/2 FDW | 1/2 FAPCS or 1/1 FPS injection | 2/8 lines and (2/3 GDCS pools or 1/3 GDCS pools and 1/4 equilizing lines) | 1/2 RWCU/SDC | All VB closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| %LL-S | | | | | | | | | | | | OK | LL-S-001 |
| | | | | | | | | | | | | OK | LL-S-002 |
| | | | | | | | | | | | | OK | LL-S-003 |
| | | | | | | | | | | | | OK | LL-S-004 |
| | | | | | | | | | | | | OK | LL-S-005 |
| | | | | | | | | | | | | OK | LL-S-006 |
| | | | | | | | | | | | | CDII | LL-S-007 |
| | | | | | | | | | | | | OK | LL-S-008 |
| | | | | | | | | | | | | OK | LL-S-009 |
| | | | | | | | | | | | | CDII | LL-S-010 |
| | | | | | | | | | | | | OK | LL-S-011 |
| | | | | | | | | | | | | OK | LL-S-012 |
| | | | | | | | | | | | | CDII | LL-S-013 |
| | | | | | | | | | | | | CDI | LL-S-014 |
| | | | | | | | | | | | | CDII | LL-S-015 |
| | | | | | | | | | | | | CDIV | LL-S-016 |

Figure A.3-12. Large Steam Breaks other than Feedwater Lines

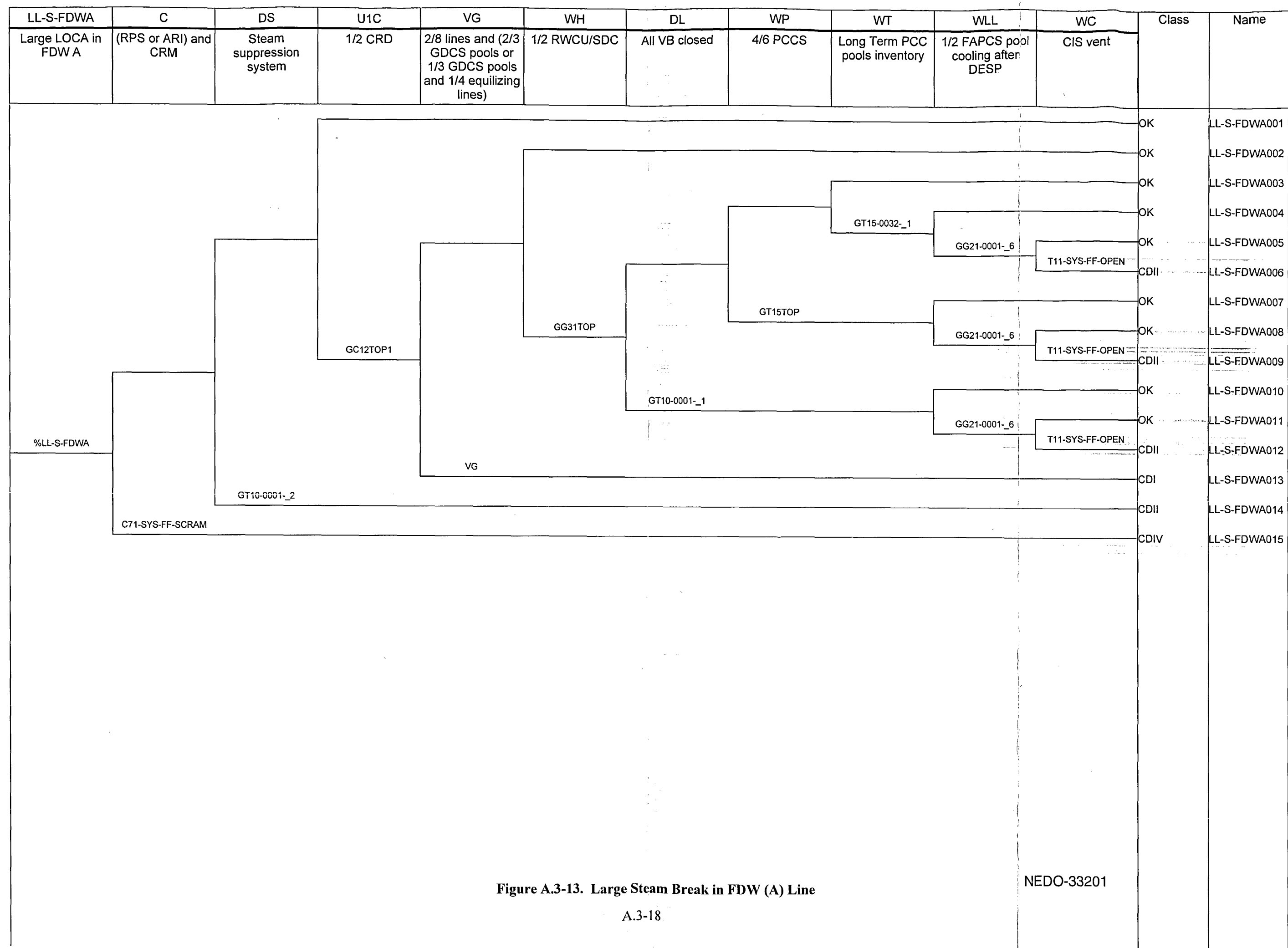


Figure A.3-13. Large Steam Break in FDW (A) Line

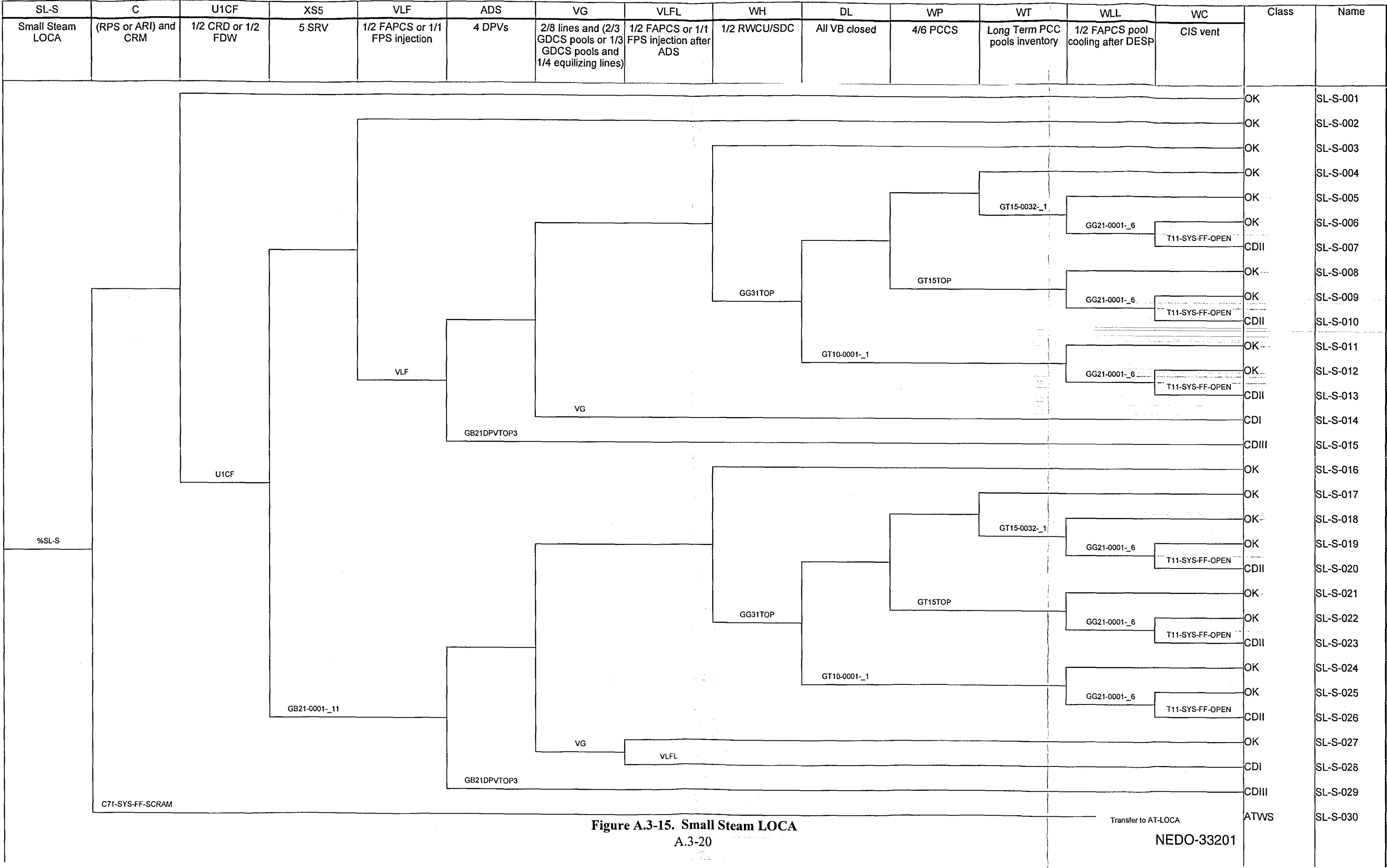
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| LL-S-FDWB | C | DS | VLF | VG | WH | DL | WP | WT | WLL | WC | Class | Name |
|---------------------|----------------------|--------------------------|--|---|--------------|---------------|----------|-------------------------------|-----------------------------------|----------|-------|--------------|
| Large LOCA in FDW B | (RPS or ARI) and CRM | Steam suppression system | 1/2 FAPCS injection or 1 Fire Water pump | 2/8 lines and (2/3 GDCS pools or 1/3 GDCS pools and 1/4 equilizing lines) | 1/2 RWCU/SDC | All VB closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| | | | | | | | | | | | OK | LL-S-FDWB001 |
| | | | | | | | | | | | OK | LL-S-FDWB002 |
| | | | | | | | | | | | OK | LL-S-FDWB003 |
| | | | | | | | | | | | OK | LL-S-FDWB004 |
| | | | | | | | | | | | OK | LL-S-FDWB005 |
| | | | | | | | | | | | OK | LL-S-FDWB006 |
| | | | | | | | | | | | OK | LL-S-FDWB007 |
| | | | | | | | | | | | OK | LL-S-FDWB008 |
| | | | | | | | | | | | OK | LL-S-FDWB009 |
| | | | | | | | | | | | OK | LL-S-FDWB010 |
| | | | | | | | | | | | OK | LL-S-FDWB011 |
| | | | | | | | | | | | OK | LL-S-FDWB012 |
| | | | | | | | | | | | OK | LL-S-FDWB013 |
| | | | | | | | | | | | OK | LL-S-FDWB014 |
| | | | | | | | | | | | OK | LL-S-FDWB015 |

Figure A.3-14. Large Steam Break in FDW (B) Line

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| ML-L | C | DS | UF | VF | ADS | VG2 | WH | DL | WP | WT | WLL | WC | Class | Name |
|---|----------------------|--------------------------|---------|-------------------|--------|--|--------------|---------------|----------|-------------------------------|-----------------------------------|-----------------|-------|----------|
| Medium LOCA not in RWCU | (RPS or ARI) and CRM | Steam suppression system | 1/2 FDW | 1/1 FPS injection | 4 DPVs | 2/8 lines, 2/3 GDCS pools and 1/4 equilizing lines | 1/2 RWCU/SDC | All VB Closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| <div> <div>%ML-L</div> <div> <div>GN21-0051_1</div> <div>GU43-0001_1</div> <div>GT10-0001_2</div> <div>C71-SYS-FF-SCRAM</div> </div> </div> | | | | | | | | | | | | | OK | ML-L-001 |
| | | | | | | | | | | | | | OK | ML-L-002 |
| | | | | | | | | | | | | | OK | ML-L-003 |
| | | | | | | | | | | | | | OK | ML-L-004 |
| | | | | | | | | | | GT15-0032_1 | | | OK | ML-L-005 |
| | | | | | | | | | | | GG21-0001_6 | | OK | ML-L-006 |
| | | | | | | | | | | | | T11-SYS-FF-OPEN | CDII | ML-L-007 |
| | | | | | | | | | | GT15TOP | | | OK | ML-L-008 |
| | | | | | | | GG31TOP | | | | GG21-0001_6 | | OK | ML-L-009 |
| | | | | | | | | | | | | T11-SYS-FF-OPEN | CDII | ML-L-010 |
| | | | | | | | | | | | | | OK | ML-L-011 |
| | | | | | | | | | | | | | OK | ML-L-012 |
| | | | | | | | | | | | | T11-SYS-FF-OPEN | CDII | ML-L-013 |
| | | | | | | | | | | | | | CDI | ML-L-014 |
| | | | | | | | | | | | | | CDIII | ML-L-015 |
| | | | | | | | | | | | | | CDII | ML-L-016 |
| | | | | | | | | | | | | | CDIV | ML-L-017 |

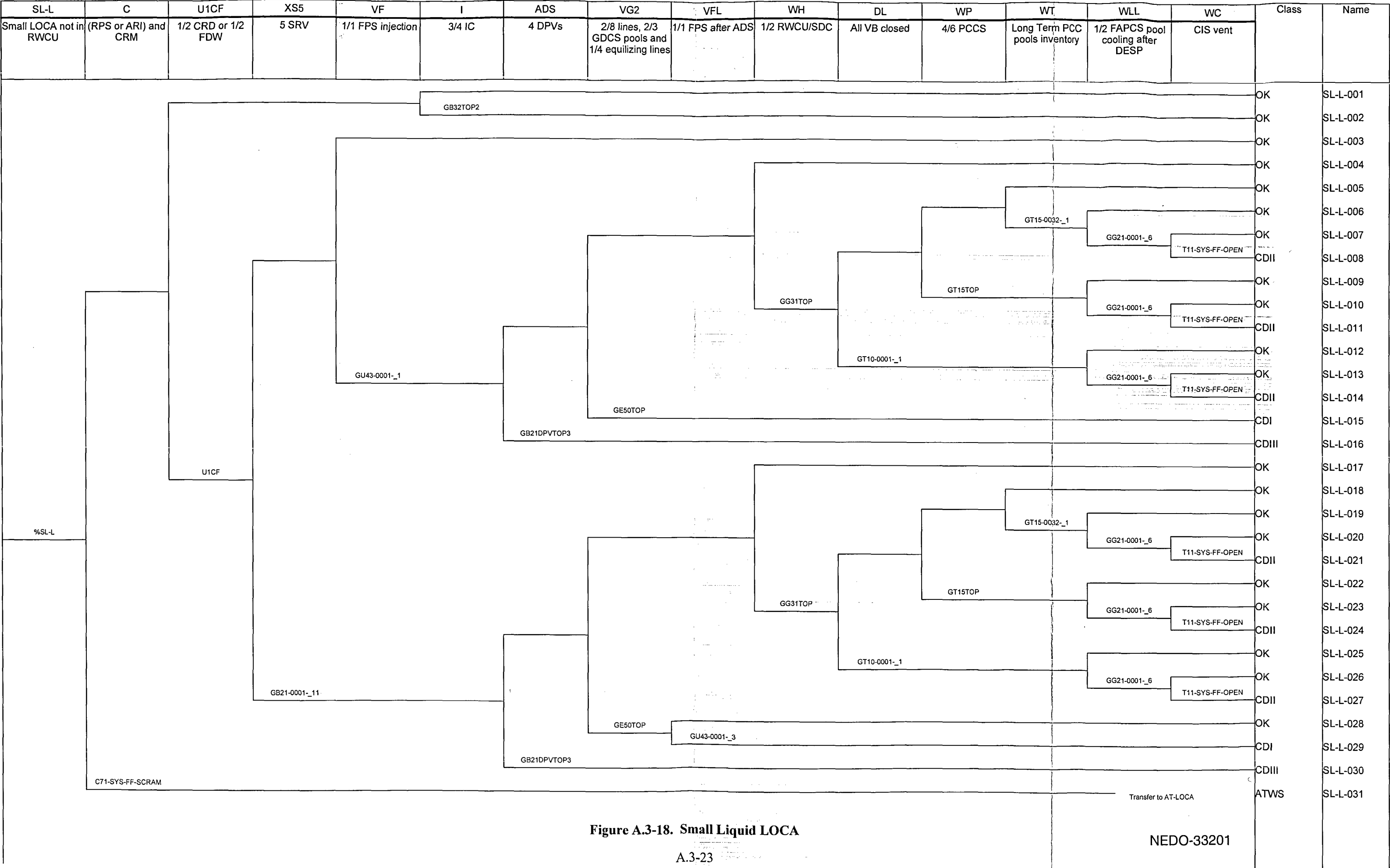
Figure A.3-16. Medium Liquid Breaks other than RWCU/SDC Lines

| ML-L-RWCU | C | DS | UF | VF | ADS | VG2 | DL | WP | WT | WLL | WC | Class | Name |
|---------------------|----------------------|--------------------------|---------|-------------------|--------|--|---------------|----------|-------------------------------|-----------------------------------|----------|-------|--------------|
| Medium LOCA in RWCU | (RPS or ARI) and CRM | Steam suppression system | 1/2 FDW | 1/1 FPS injection | 4 DPVs | 2/8 lines, 2/3 GDCS pools and 1/4 equilizing lines | All VB Closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| | | | | | | | | | | | | OK | ML-L-RWCU001 |
| | | | | | | | | | | | | OK | ML-L-RWCU002 |
| | | | | | | | | | | | | OK | ML-L-RWCU003 |
| | | | | | | | | | | | | OK | ML-L-RWCU004 |
| | | | | | | | | | | | | OK | ML-L-RWCU005 |
| | | | | | | | | | | | | CDII | ML-L-RWCU006 |
| | | | | | | | | | | | | OK | ML-L-RWCU007 |
| | | | | | | | | | | | | OK | ML-L-RWCU008 |
| | | | | | | | | | | | | CDII | ML-L-RWCU009 |
| | | | | | | | | | | | | OK | ML-L-RWCU010 |
| | | | | | | | | | | | | OK | ML-L-RWCU011 |
| | | | | | | | | | | | | CDII | ML-L-RWCU012 |
| | | | | | | | | | | | | CDI | ML-L-RWCU013 |
| | | | | | | | | | | | | CDIII | ML-L-RWCU014 |
| | | | | | | | | | | | | CDII | ML-L-RWCU015 |
| | | | | | | | | | | | | CDIV | ML-L-RWCU016 |

Figure A.3-17. Medium Liquid Breaks in RWCU/SDC Lines

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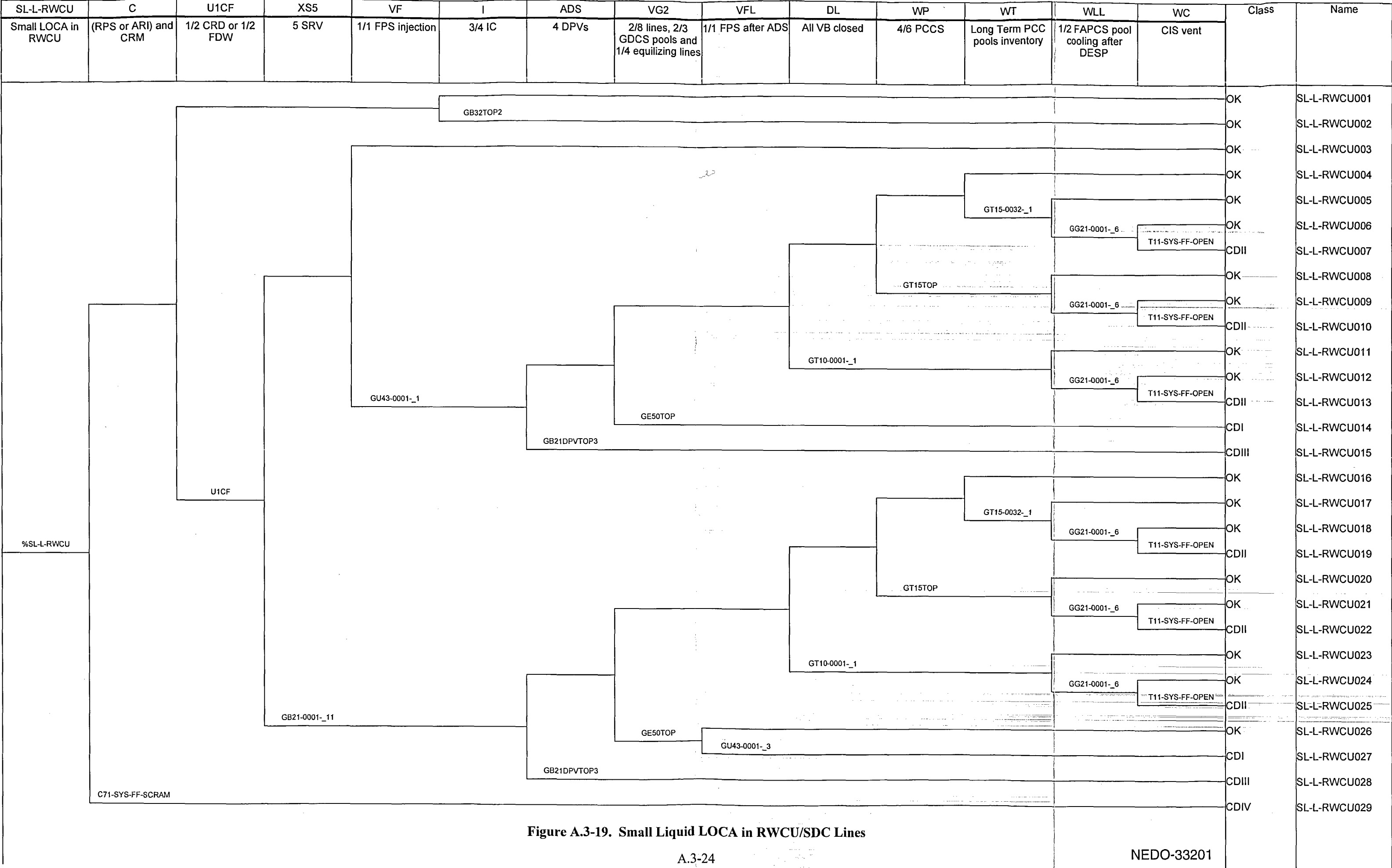


Figure A.3-19. Small Liquid LOCA in RWCU/SDC Lines

| RVR | DS | VG2 | DL | WP | WT | WLL | WC | Class | Name |
|------------------------|--------------------------|---|---------------|----------|-------------------------------|-----------------------------------|----------|-------|---------|
| Reactor Vessel Rupture | Steam suppression system | 2/8 lines and 2/3 GDCS pools and 1/4 equilizing lines | All VB Closed | 4/6 PCCS | Long Term PCC pools inventory | 1/2 FAPCS pool cooling after DESP | CIS vent | | |
| RVR | | | | | | | | OK | RVR-001 |
| | | | | | | | | OK | RVR-002 |
| | | | | | | | | OK | RVR-003 |
| | | | | | | | | CDII | RVR-004 |
| | | | | | | | | OK | RVR-005 |
| | | | | | | | | OK | RVR-006 |
| | | | | | | | | CDII | RVR-007 |
| | | | | | | | | OK | RVR-008 |
| | | | | | | | | OK | RVR-009 |
| | | | | | | | | CDII | RVR-010 |
| | | | | | | | | CDI | RVR-011 |
| | | | | | | | | CDII | RVR-012 |

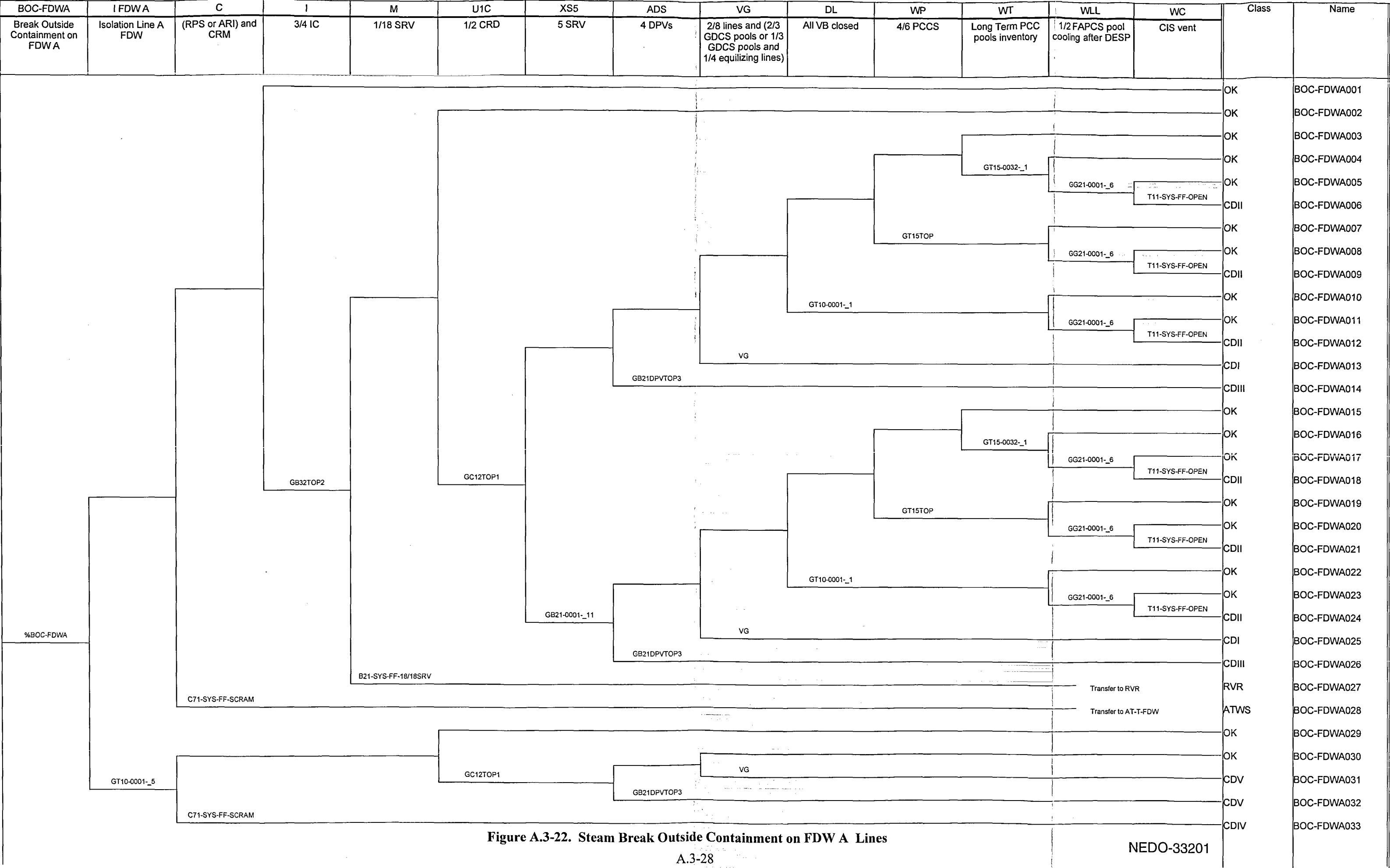
Figure A.3-20. Reactor Vessel Rupture

| BOC MS | I ML | C | U1CF | XS5 | VLF | ADS | VG | VLFL | Class | Name |
|--------------------------------------|---------------------------|----------------------|--------------------|---------------|--------------------------------|-------------|---|--|-------|------------|
| Break Outside Containment in MS Line | Isolation all MSteam Line | (RPS or ARI) and CRM | 1/2 CRD or 1/2 FDW | 5 SRV | 1/2 FAPCS or 1/1 FPS injection | 4 DPVs | 2/8 lines and (2/3 GDCS pools or 1/3 GDCS pools and 1/4 equilizing lines) | 1/2 FAPCS or 1/1 FPS injection after ADS | | |
| %BOC-MS | GT10-0001-_9 | C71-SYS-FF-SCRAM | U1CF | GB21-0001-_11 | VLF | GB21DPVTOP3 | VG | VLFL | TPCS | BOC-MS001 |
| | | | | | | | | | OK | BOC-MS002 |
| | | | | | | | | | OK | BOC-MS003 |
| | | | | | | | | | OK | BOC-MS004 |
| | | | | | | | | | CDV | BOC-MS005 |
| | | | | | | | | | CDV | BOC-MS006 |
| | | | | | | | | | OK | BOC-MS007 |
| | | | | | | | | | OK | BOC-MS008 |
| | | | | | | | | | CDV | BOC-MS009 |
| | | | | | | | | | CDV | BOC-MS010 |
| | | | | | | | | | CDIV | BOC-MS011 |
| Transfer to T-PCS | | | | | | | | | | NEDO-33201 |

Figure A.3-21. Steam Break Outside Containment on Main Steam Lines

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Figure A.3-21. Steam Break Outside Containment on Main Steam Lines



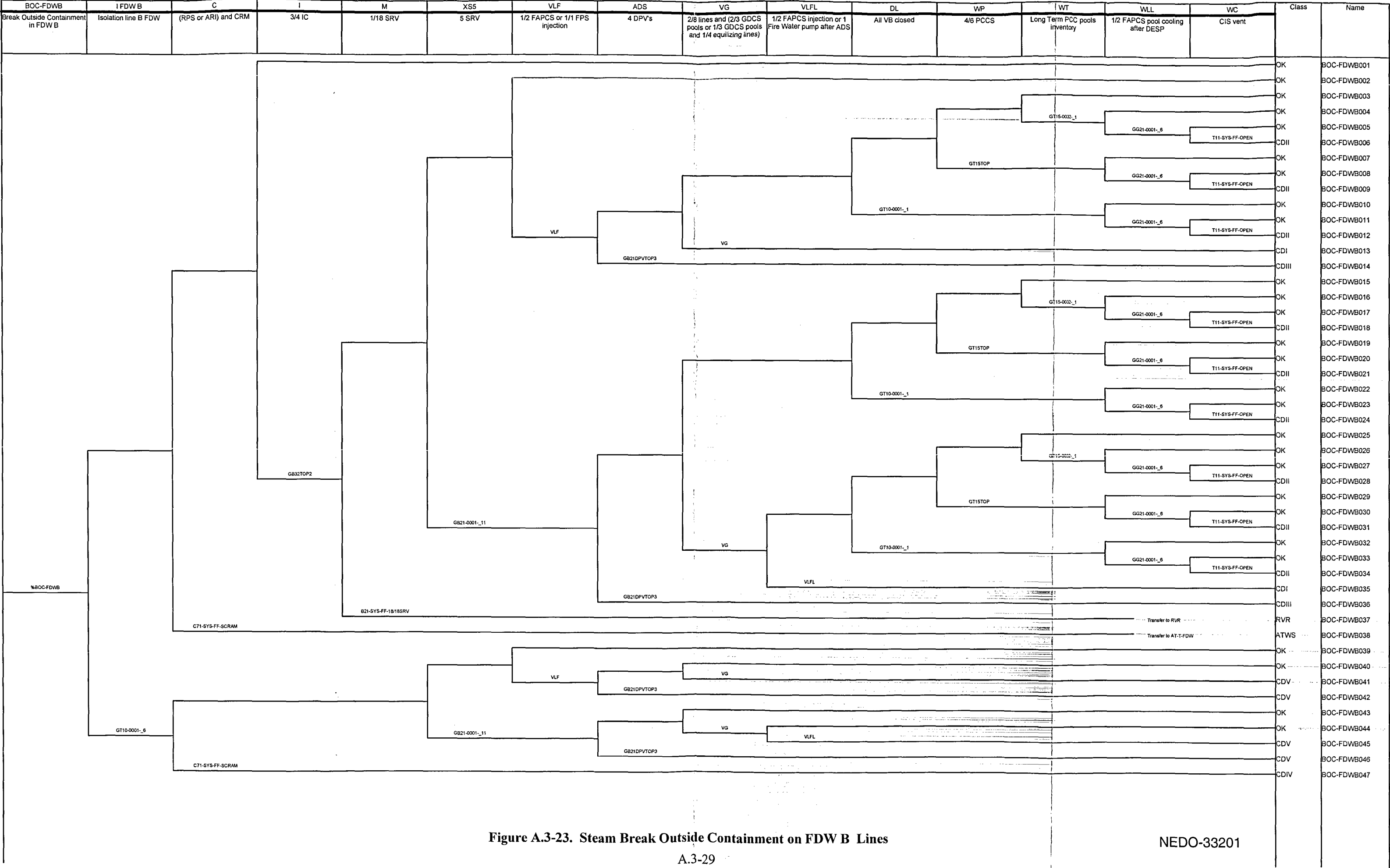


Figure A.3-23. Steam Break Outside Containment on FDW B Lines

| BOC-IC | IIC | C | U1CF | XS5 | VLF | ADS | VG | VLFL | Class | Name |
|---|-------------------------|----------------------|--------------------|-------|--------------------------------|---------|---|--|-------|-----------|
| Break Outside Containment in IC | Isolation IC Steam Line | (RPS or ARI) and CRM | 1/2 CRD or 1/2 FDW | 5 SRV | 1/2 FAPCS or 1/1 FPS injection | 4 DPV's | 2/8 lines and (2/3 GDCS pools or 1/3 GDCS pools and 1/4 equilizing lines) | 1/2 FAPCS or 1/1 FPS injection after ADS | | |
| <div> <div>%BOC-IC</div> <div> <div>GT10-0001_4</div> <div>C71-SYS-FF-SCRAM</div> </div> </div> | Transfer to T-PCS | | | | | | | | TPCS | BOC-IC001 |
| | | | | | | | | | OK | BOC-IC002 |
| | | | | | | | | | OK | BOC-IC003 |
| | | | | | | | | | OK | BOC-IC004 |
| | | | | | | | | | CDV | BOC-IC005 |
| | | | | | | | | | CDV | BOC-IC006 |
| | | | | | | | | | OK | BOC-IC007 |
| | | | | | | | | | OK | BOC-IC008 |
| | | | | | | | | | CDV | BOC-IC009 |
| | | | | | | | | | CDV | BOC-IC010 |
| | | | | | | | | | CDIV | BOC-IC011 |

Figure A.3-24. Large Liquid Break Outside Containment on IC Lines

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Figure A.3-24. Large Liquid Break Outside Containment on IC Lines

4 SYSTEM ANALYSIS

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4 SYSTEMS ANALYSIS

4.0 INTRODUCTION

4.0.1 System Analysis Purpose

The purpose of ESBWR system analysis is to the systems modeled in the ESBWR PRA. The systems correspond to the functional headings described in Chapter 3 plus any support systems needed to accomplish those functions. Each of Sections 4.1 through 4.19 contains the following typical items:

- System Purpose
- Design Assumptions
- System Description
- Automatic and Manual Control
- System Interfaces
- System Testing
- System Maintenance
- Potential for Common Cause Failure (CCF)
- Fault Tree Analysis
- Human Interactions

The documentation of the system analysis is constructed to facilitate certification of the ESBWR design. It describes the processes used for modeling and quantification. It also includes the generic modeling assumptions. It is constructed with consideration of future updates to the ESBWR PRA and for risk informed applications involving the ESBWR.

Table 4.0-1 contains a list of the systems and the associated functions that are modeled in the ESBWR PRA. Those that are identified as “front line” appear as event tree headings.

4.0.2 Fault Trees

There is at least one fault tree file for each system function included in the PRA model. If multiple trains can perform the function, then each train is modeled. All of the logic for a particular system can be contained in a single fault tree file. In fact, this is the preferred method for the ESBWR PRA.

The fault trees are constructed using the gate and basic event naming conventions described in Appendix A.4. The fault tree database is stored in the CAFTA Database file format.

Table 4.0-1
Section Number and Fault Trees of Key ESBWR Systems

| Section # | System ID | Description | Functions Modeled |
|------------------|-----------------------|---|---|
| Section 4.1 | B21 | Reactor Depressurization | Depressurization (SRV or DPV) Overpressure Protection |
| Section 4.2 | B32 | Isolation Condenser | Reactor Heat Removal SRV Leakage DPV Leakage |
| Section 4.3 | C12 | Control Rod Drive | RPV Injection at High Pressure |
| Section 4.4 | C41 | Standby Liquid Control | Boron Injection |
| Section 4.5 | C62, C63, C71, C74 | Instrument and Control | Essential Multiplexing Non-Essential Multiplexing Instrumentation (Sensors) Reactor Protection System Signals <ul style="list-style-type: none"> - SCRAM - Alternate Rod Insertion - Feedwater Runback - Manual Rod Insertion - ADS Inhibit |
| Section 4.6 | E50 | Gravity Driven Cooling System | RPV Injection at Low Pressure |
| Section 4.7 | G21 | Fuel & Auxiliary Pool Cooling System | Suppression Pool Cooling Low Pressure Coolant Injection Drywell Spray ICS Pool Makeup GDCS Pool Makeup |
| | U43 | Fire Protection System | RPV Injection at Low Pressure PCCS Pool Makeup IC Pool Makeup |

Table 4.0-1
Section Number and Fault Trees of Key ESBWR Systems (Continued)

| Section # | System ID | Description | Functions Modeled |
|------------------|------------------|--|---|
| Section 4.8 | G31 | Reactor Water Cleanup and Shutdown Cooling | Shutdown Cooling |
| Section 4.9 | N21 P22 | Feedwater and Condensate | RPV Injection at High Pressure RPV Injection at Low Pressure |
| Section 4.10 | P21 | Reactor Component Cooling Water | Removes heat from systems in the drywell and Reactor Building |
| Section 4.11 | P41 | Plant Service Water | Component Cooling |
| Section 4.12 | P52, P51 | Instrument Air System and Service Air System | Valve Motive Power |
| Section 4.13 | P54 | High Pressure Nitrogen | Valve Motive Power |
| Section 4.14 | R10, R11, R12 | A.C. Electric Power | AC Power |
| Section 4.15 | R13 | Uninterruptible AC Power | AC Power |
| Section 4.16 | | Deleted / Not Used | Not Used |
| Section 4.17 | R16 | DC Power Supply | DC Power |
| Section 4.18 | T10 | Containment System | Containment Isolation Vapor Suppression |
| Section 4.19 | T15 | Passive Containment Cooling | Ultimate Heat Sink |

4.1 AUTOMATIC DEPRESSURIZATION SYSTEM - (B21)

4.1.1 Purpose

In response to emergency conditions, the Automatic Depressurization System (ADS) provides automatic, effectively permanent depressurization of the reactor under a timely blowdown schedule. Once the depressurization is complete, this allows passive resupply of coolant to the reactor by the Gravity Driven Cooling System (GDCS). The depressurization is completed in time to allow GDCS injection flow to replenish core coolant and prevent uncovering the core assuming a failure of any single active component. The ADS uses depressurization valves (DPVs) together with safety-relief valves (SRVs) to achieve reactor depressurization. This function continues until the vessel pressure and temperature are nominally stabilized. For the long term, the DPVs provide a flow path for the steam to the drywell and from there to the Passive Containment Cooling System (PCCS). The SRVs involved in the ADS actuation are each piped separately to their respective quenchers located in the suppression pool to permit condensation of the discharged steam. The SRVs are also used to relieve pressure for the postulated anticipated transient without scram (ATWS) event.

4.1.2 Design Assumptions

The following assumption with respect to the design is made:

- (1) Actuation logic is provided by two independent and diverse systems.

4.1.3 System Description

4.1.3.1 Hardware Configuration

The automatic depressurization system is composed of ten of eighteen safety/relief valves (SRVs) and eight depressurization valves (DPVs), together with their related instrumentation and control logic. The ten SRVs that are part of the ADS are distributed for each steam line (3-2-2-3). They have 200 mm inlets, 250 mm outlets, are spring-loaded and nitrogen operated. Ten SRVs have the capability of actuating in ADS mode or overpressure protection mode; the remaining SRVs, however, can only actuate in the overpressure protection mode.

Each ADS SRV has an individual discharge line routed to the suppression pool, ending with a submerged steam quencher. This discharge line incorporates redundant vacuum breakers within the drywell pipe run in order to limit re-flood of the line after SRV reclosure, which could result from condensation of residual steam after valve operation. Therefore, the vacuum breakers prevent water hammer and pressure instability conditions in the SRV discharge line.

The DPVs are straight through, squib-actuated, non-reclosing type valves, with a metal diaphragm (seal) which acts as a valve disk and is sheared off by two initiators (squibs) that either individually or jointly are capable of actuating the booster and, subsequently, opening the valve. The DPV has a rated discharge capacity between 862,000 and 1,060,000 kg/hr of steam at an inlet pressure of 7.48 MPa-gauge.

Fig. 4.1-1 shows a simplified diagram of the system.

4.1.3.2 System Operation

Considerable margins exist between the SRV spring setpoint pressure and the reactor operating pressure. The Isolation Condenser System (ICS) is able to keep reactor pressure vessel (RPV) pressure below the SRV spring setpoint during transient events. The depressurization function is requested when the RPV water level 1 is reached.

The automatic depressurization is performed by opening the SRVs and the DPVs. Each DPV has approximately twice the steam flow capability of an SRV. The success criteria for the depressurization function are defined in terms of SRV-equivalent valves, with the simplifying assumption that one DPV is equivalent to two SRVs.

A partial depressurization is considered in order to take into account the possibility of using low-pressure injection systems even if the GDCS cannot yet be made operable. To allow for this, at least five equivalent SRVs must open, that is five SRVs or three DPVs.

In order to depressurize to a point where the GDCS is allowed to operate, at least four equivalent DPVs must open; this can be achieved through the following alternative minimal combinations:

- 6 SRVs must open
- 4 SRVs and 2 DPV must open
- 4 DPVs must open

However, the PRA model currently only credits the last of these alternatives because long term cooling with equalizing lines requires the successful operation of 4 DPVs.

The SRVs are also called into operation during an ATWS event to prevent over pressurization of the RPV and a possible subsequent break. The success criterion for the pressure relief during ATWS is that nine of eighteen SRVs must open. In addition, during this ATWS event, the automatic initiation of ADS (SRVs and DPVs) is inhibited, automatic or manually.

4.1.3.3 Component Location

The SRVs are installed on the main steam lines. Four DPVs are installed on the steam lines, the other four DPVs are on short stub pipes coming off the RPV at approximately (slightly below) the elevation of the main steam lines, from where are branched the Isolation Condenser steam supply lines.

4.1.4 Automatic and Manual Control

The driving concept of the ADS is the ability to actuate SRVs and DPVs even if two electrical or signal divisions are lost. Optical isolators are used for the mixing of the signals and for the voting logic to provide high reliability for the signals coming from the four different divisions. A list of all the control room instrumentation and alarms is reported in Table 4.1-1.

4.1.4.1 Automatic Actuation

The ADS starts automatically upon detection of RPV Low-Low Water Level (level 1) while in the RUN mode.

Each SRV is actuated if any one of two solenoid valves operates, supplying pressurized nitrogen to the valve. The coil of each solenoid valve is connected to a configuration of load drivers that are triggered by the automatic signal.

The DPVs have wire circuits connected with two initiators and one booster. Either individually or jointly, the initiators are capable of actuating the booster and subsequently opening the valve. These firing circuits are connected to the load drivers for the automatic actuation. The configuration of the load drivers and the relays is analogous for the SRVs and the DPVs; they are arranged in such a way that a two-out-of-four actuation logic is obtained.

The ADS logic is automatically initiated when a low reactor water level signal (level 1) is present. If the RPV low water level signal is present concurrently with a high drywell pressure signal, both the main ADS timer and the high drywell pressure bypass timer are initiated. If the RPV low water level signal is present without a concurrent high drywell pressure signal, only the ADS high drywell pressure bypass timer is initiated. Upon the time-out of the ADS high drywell pressure bypass timer and concurrent with a RPV low water level signal, the main ADS timer is initiated, if not already initiated. The main ADS timer continues to completion and times out only in the continued presence of an RPV low water level signal. When the main ADS timer times out, the ADS function is initiated. The depressurization rate is limited to prevent reactor coolant inventory level swells in amounts sufficient to produce liquid carry-over along with steam in the blowdown exhaust flow.

4.1.4.2 Manual Actuation

The ten (10) SRVs that are part of the ADS mode have a third solenoid that gives the operator the possibility of opening the valve from the Control Room. This third solenoid possesses different and independent logic channels than the ones used in the ADS operation. The operator can open an SRV through a pushbutton switch; valves remain open for as long as the pushbutton switch is depressed. With this feature, the operator is able to control normal isolation transients even if, according to the design, no need should arise for such an operation.

The operator can also manually initiate the ADS function from the main control room.

4.1.4.3 Safety Actuation

The SRVs (all 18) also open in the safety mode (i.e., actuated on high RPV pressure) when reactor pressure reaches the spring setpoint value.

4.1.5 System Interfaces

The ADS shares all of the components related to the actuating signals with other safety-related systems. These include RPV water level (level 1) and ATWS inhibit signals. ADS shares four level sensors with the systems actuated on a level 2 signal. The system also requires the 250V DC power supply for the actuation logic and the load drivers.

The SRVs need pressurized nitrogen supply to open and to remain open. This is achieved through accumulators connected to the high pressure, nitrogen supply system (HPNSS), which supplies the valves accumulators. The nitrogen supply in the accumulators is sufficient to keep the SRVs open for time intervals if HPNSS supply is not available. The long-term nitrogen

supply for the SRVs is the nitrogen bottles in the HPNSS. Table 4.1-2 summarizes dependencies on the support systems by division.

4.1.6 System Testing

Testing of SRVs is possible during plant operation. It is assumed that SRV tests are scheduled at 18-months intervals on a staggered tests basis. The SRV safety function setpoints are verified at 24-months intervals for half of the SRVs. Each DPV contains two initiators and one booster. During plant operation, the integrity of the electrical circuits of the initiators is continuously monitored by a low current flowing through the wires. The signal chain, up to the output of the trip logic unit, is tested every three months. Actuating logic devices, such as load drivers, are tested monthly.

4.1.7 System Maintenance

All of the main components are within the drywell, so that no maintenance is planned during plant operation. Maintenance work on SRVs is anticipated during alternate test cycles, or every eight years. During refueling, booster and initiators of squib valves are replaced and test fired; also, an inspection is expected to verify the integrity of the mechanical parts of the DPVs.

Maintenance of logic electronic components is performed on-line if indicated by testing. Since the logic modules are solid state devices, component outages for maintenance are expected to be negligible.

4.1.8 Common Cause Failures

A high degree of redundancy is incorporated in the system design. For that reason, common cause failures (CCFs) turn out to be a potential contributor to system unavailability. The CCFs that have been identified in the system are listed in Table 4.1-5. Only the CCFs of two or three components are considered, as the results of higher combinations are negligible.

4.1.9 Fault Tree Analysis

The ADS fault tree has been evaluated to determine the system unavailability and to identify the components that contribute significantly to system unavailability.

4.1.9.1 Top Event Definitions

The top events list defined for the system is given in Table 4.1-8.

The two main fault trees are shown in the Appendix B.4.1.

4.1.9.2 Fault Tree Description

Two fault trees are developed: the first one represents the opening of four (4) of eight (8) DPVs, while the second one shows the manual opening of five (5) of the ten (10) SRVs with a depressurization function.

4.1.9.3 Human Interaction

In the fault tree relating to the manual opening of SRVs, this opening is initiated by operator action from the Control Room.

There are no local manually operated valves in the ADS system, so no misposition events are considered.

The only specific human action modeled within the DPV fault tree is the backup manual actuation of the ADS (Type C). The backup actuation is initiated in the control room through devices designed to prevent inadvertent actuation.

4.1.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.1-7.

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

The importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.1.11 Model Differences for 72-Hour Operation

No change is necessary for using the model with a 72-hour mission time.

4.1.12 References

4.1.1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.1-1
Control Room Instrumentation and Alarms

| | |
|--|---|
| | Controls |
| | |
| | Displays |
| | |
| | Reactor vessel pressure |
| | Reactor vessel water level |
| | Main steam line pressure |
| | Main condenser vacuum |
| | Main steam line flow |
| | Main steam line radiation (from PRMS) |
| | SRV position |
| | MSIV position |
| | DPV position |
| | Main steam line drain valve position |
| | Position of feedwater motor operated valves within Reactor Building |
| | Feedwater outboard isolation check valve position |
| | Position of feedwater maintenance manual isolation valve |
| | RPV head seal leak detection |
| | Main steam line drain temperature |
| | |
| | Alarms |
| | |
| | RPV head closure seal leakage |
| | Low reactor water level (L1, L2 and L3) |
| | High reactor water level (L8) |
| | High reactor water level (L9) |
| | High SRV discharge line temperature |
| | SRV – Open |
| | MSIV – Closure |
| | DPV – Open |
| | ADS Inhibited/Auto Out of Service |
| | ADS Logic Initiated |
| | Valve travel failure (all feedwater motor-operated valves) |

Table 4.1-2
System Dependency Matrix

| Support System | | | | | | | | | |
|----------------|------|-----------------------|--------|--------|--------|------------------|-----------|-----------|-----------|
| Component | Type | 250 V DC Power Supply | | | | Actuation Signal | | | |
| | | R16-11 | R16-21 | R16-31 | R16-41 | C74 Div 1 | C74 Div 2 | C72 Div 1 | C72 Div 2 |
| F004A | DPV | X | | | X | X | | | X |
| F004B | DPV | | X | X | | | X | X | |
| F004C | DPV | X | | | X | X | | X | |
| F004D | DPV | | X | X | | | X | | X |
| F004E | DPV | X | | | X | X | | | X |
| F004F | DPV | | X | X | | | X | X | |
| F004G | DPV | X | | | X | X | | X | |
| F004H | DPV | | X | X | | | X | | X |

| Support System | | | | | | | | | |
|----------------|------|---------------------------------|----------|----------|---------------------------|-------|-------|-------|------------|
| Component | Type | Uninterruptible AC Power Supply | | | Signal Transmission (C72) | | | | Air Supply |
| | | R13-A1-2 | R13-B1-2 | R13-C1-2 | Div A | Div B | Div C | Div D | HPNSS |
| F006A | SRV | X | | | X | | | | X |
| F006B | SRV | | X | | | X | | | X |
| F006C | SRV | | | X | | | X | | X |
| F006D | SRV | X | | | | | | X | X |
| F006E | SRV | | X | | X | | | | X |
| F006F | SRV | | | X | | X | | | X |
| F006G | SRV | X | | | | | X | | X |
| F006H | SRV | | X | | | | | X | X |
| F006I | SRV | | | X | X | | | | X |
| F006J | SRV | X | | | | X | | | X |

Table 4.1-3
Component Test

| Component | Type | Expected Test Interval |
|--------------------------------|------|---|
| | | |
| SRVs safety function setpoints | SRV | 24-month intervals for half of the SRVs |
| SRV tests | SRV | 18-month intervals on a staggered tests basis |
| Signal chain | DPV | Quarterly |
| Actuating logic devices | DPV | Monthly |
| Initiators and boosters | DPV | 24-months |

Table 4.1-4
Component Maintenance

All of the main components are within the drywell, so that no maintenance is planned during plant operation.

Table 4.1-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|--------------------|----------|-----------------------------|
| B21-SQV-CF-DPVOPEN | 1.50E-05 | CCF OF DPV'S TO OPEN |
| B21-SQV-CF-F004AB | 3.60E-05 | CCF OF DPV A AND B TO OPEN |
| B21-SQV-CF-F004ABC | 9.00E-06 | CCF OF DPV A, B & C TO OPEN |
| B21-SQV-CF-F004ABD | 9.00E-06 | CCF OF DPV A, B & D TO OPEN |
| B21-SQV-CF-F004ABE | 9.00E-06 | CCF OF DPV A, B & E TO OPEN |
| B21-SQV-CF-F004ABF | 9.00E-06 | CCF OF DPV A, B & F TO OPEN |
| B21-SQV-CF-F004ABG | 9.00E-06 | CCF OF DPV A, B & G TO OPEN |
| B21-SQV-CF-F004ABH | 9.00E-06 | CCF OF DPV A, B & H TO OPEN |
| B21-SQV-CF-F004AC | 3.60E-05 | CCF OF DPV A AND C TO OPEN |
| B21-SQV-CF-F004ACD | 9.00E-06 | CCF OF DPV A, C & D TO OPEN |
| B21-SQV-CF-F004ACE | 9.00E-06 | CCF OF DPV A, C & E TO OPEN |
| B21-SQV-CF-F004ACF | 9.00E-06 | CCF OF DPV A, C & F TO OPEN |
| B21-SQV-CF-F004ACG | 9.00E-06 | CCF OF DPV A, C & G TO OPEN |
| B21-SQV-CF-F004ACH | 9.00E-06 | CCF OF DPV A, C & H TO OPEN |
| B21-SQV-CF-F004AD | 3.60E-05 | CCF OF DPV A AND D TO OPEN |
| B21-SQV-CF-F004ADE | 9.00E-06 | CCF OF DPV A, D & E TO OPEN |
| B21-SQV-CF-F004ADF | 9.00E-06 | CCF OF DPV A, D & F TO OPEN |
| B21-SQV-CF-F004ADG | 9.00E-06 | CCF OF DPV A, D & G TO OPEN |
| B21-SQV-CF-F004ADH | 9.00E-06 | CCF OF DPV A, D & H TO OPEN |
| B21-SQV-CF-F004AE | 3.60E-05 | CCF OF DPV A AND E TO OPEN |
| B21-SQV-CF-F004AEF | 9.00E-06 | CCF OF DPV A, E & F TO OPEN |
| B21-SQV-CF-F004AEG | 9.00E-06 | CCF OF DPV A, E & G TO OPEN |
| B21-SQV-CF-F004AEH | 9.00E-06 | CCF OF DPV A, E & H TO OPEN |
| B21-SQV-CF-F004AF | 3.60E-05 | CCF OF DPV A AND F TO OPEN |
| B21-SQV-CF-F004AFG | 9.00E-06 | CCF OF DPV A, F & G TO OPEN |
| B21-SQV-CF-F004AFH | 9.00E-06 | CCF OF DPV A, F & H TO OPEN |
| B21-SQV-CF-F004AG | 3.60E-05 | CCF OF DPV A AND G TO OPEN |
| B21-SQV-CF-F004AGH | 9.00E-06 | CCF OF DPV A, G & H TO OPEN |
| B21-SQV-CF-F004AH | 3.60E-05 | CCF OF DPV A AND H TO OPEN |
| B21-SQV-CF-F004BC | 3.60E-05 | CCF OF DPV B AND C TO OPEN |
| B21-SQV-CF-F004BCD | 9.00E-06 | CCF OF DPV B, C & D TO OPEN |
| B21-SQV-CF-F004BCE | 9.00E-06 | CCF OF DPV B, C & E TO OPEN |
| B21-SQV-CF-F004BCF | 9.00E-06 | CCF OF DPV B, C & F TO OPEN |

Table 4.1-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|--------------------|----------|-----------------------------|
| B21-SQV-CF-F004BCG | 9.00E-06 | CCF OF DPV B, C & G TO OPEN |
| B21-SQV-CF-F004BCH | 9.00E-06 | CCF OF DPV B, C & H TO OPEN |
| B21-SQV-CF-F004BD | 3.60E-05 | CCF OF DPV B AND D TO OPEN |
| B21-SQV-CF-F004BDE | 9.00E-06 | CCF OF DPV B, D & E TO OPEN |
| B21-SQV-CF-F004BDF | 9.00E-06 | CCF OF DPV B, D & F TO OPEN |
| B21-SQV-CF-F004BDG | 9.00E-06 | CCF OF DPV B, D & G TO OPEN |
| B21-SQV-CF-F004BDH | 9.00E-06 | CCF OF DPV B, D & H TO OPEN |
| B21-SQV-CF-F004BE | 3.60E-05 | CCF OF DPV B AND E TO OPEN |
| B21-SQV-CF-F004BEF | 9.00E-06 | CCF OF DPV B, E & F TO OPEN |
| B21-SQV-CF-F004BEG | 9.00E-06 | CCF OF DPV B, E & G TO OPEN |
| B21-SQV-CF-F004BEH | 9.00E-06 | CCF OF DPV B, E & H TO OPEN |
| B21-SQV-CF-F004BF | 3.60E-05 | CCF OF DPV B AND F TO OPEN |
| B21-SQV-CF-F004BFG | 9.00E-06 | CCF OF DPV B, F & G TO OPEN |
| B21-SQV-CF-F004BFH | 9.00E-06 | CCF OF DPV B, F & H TO OPEN |
| B21-SQV-CF-F004BG | 3.60E-05 | CCF OF DPV B AND G TO OPEN |
| B21-SQV-CF-F004BGH | 9.00E-06 | CCF OF DPV B, G & H TO OPEN |
| B21-SQV-CF-F004BH | 3.60E-05 | CCF OF DPV B AND H TO OPEN |
| B21-SQV-CF-F004CD | 3.60E-05 | CCF OF DPV C AND D TO OPEN |
| B21-SQV-CF-F004CDE | 9.00E-06 | CCF OF DPV C, D & E TO OPEN |
| B21-SQV-CF-F004CDF | 9.00E-06 | CCF OF DPV C, D & F TO OPEN |
| B21-SQV-CF-F004CDG | 9.00E-06 | CCF OF DPV C, D & G TO OPEN |
| B21-SQV-CF-F004CDH | 9.00E-06 | CCF OF DPV C, D & H TO OPEN |
| B21-SQV-CF-F004CE | 3.60E-05 | CCF OF DPV C AND E TO OPEN |
| B21-SQV-CF-F004CEF | 9.00E-06 | CCF OF DPV C, E & F TO OPEN |
| B21-SQV-CF-F004CEG | 9.00E-06 | CCF OF DPV C, E & G TO OPEN |
| B21-SQV-CF-F004CEH | 9.00E-06 | CCF OF DPV C, E & H TO OPEN |
| B21-SQV-CF-F004CF | 3.60E-05 | CCF OF DPV C AND F TO OPEN |
| B21-SQV-CF-F004CFG | 9.00E-06 | CCF OF DPV C, F & G TO OPEN |
| B21-SQV-CF-F004CFH | 9.00E-06 | CCF OF DPV C, F & H TO OPEN |
| B21-SQV-CF-F004CG | 3.60E-05 | CCF OF DPV C AND G TO OPEN |
| B21-SQV-CF-F004CGH | 9.00E-06 | CCF OF DPV C, G & H TO OPEN |
| B21-SQV-CF-F004CH | 3.60E-05 | CCF OF DPV C AND H TO OPEN |
| B21-SQV-CF-F004DE | 3.60E-05 | CCF OF DPV D AND E TO OPEN |
| B21-SQV-CF-F004DEF | 9.00E-06 | CCF OF DPV D, E & F TO OPEN |

Table 4.1-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|----------------------|----------|---|
| B21-SQV-CF-F004DEG | 9.00E-06 | CCF OF DPV D, E & G TO OPEN |
| B21-SQV-CF-F004DEH | 9.00E-06 | CCF OF DPV D, E & H TO OPEN |
| B21-SQV-CF-F004DF | 3.60E-05 | CCF OF DPV D AND F TO OPEN |
| B21-SQV-CF-F004DFG | 9.00E-06 | CCF OF DPV D, F & G TO OPEN |
| B21-SQV-CF-F004DFH | 9.00E-06 | CCF OF DPV D, F & H TO OPEN |
| B21-SQV-CF-F004DG | 3.60E-05 | CCF OF DPV D AND G TO OPEN |
| B21-SQV-CF-F004DGH | 9.00E-06 | CCF OF DPV D, G & H TO OPEN |
| B21-SQV-CF-F004DH | 3.60E-05 | CCF OF DPV D AND H TO OPEN |
| B21-SQV-CF-F004EF | 3.60E-05 | CCF OF DPV E AND F TO OPEN |
| B21-SQV-CF-F004EFG | 9.00E-06 | CCF OF DPV E, F & G TO OPEN |
| B21-SQV-CF-F004EFH | 9.00E-06 | CCF OF DPV E, F & H TO OPEN |
| B21-SQV-CF-F004EG | 3.60E-05 | CCF OF DPV E AND G TO OPEN |
| B21-SQV-CF-F004EGH | 9.00E-06 | CCF OF DPV E, G & H TO OPEN |
| B21-SQV-CF-F004EH | 3.60E-05 | CCF OF DPV E AND H TO OPEN |
| B21-SQV-CF-F004FG | 3.60E-05 | CCF OF DPV F AND G TO OPEN |
| B21-SQV-CF-F004FGH | 9.00E-06 | CCF OF DPV F, G & H TO OPEN |
| B21-SQV-CF-F004FH | 3.60E-05 | CCF OF DPV F AND H TO OPEN |
| B21-SQV-CF-F004GH | 3.60E-05 | CCF OF DPV G AND H TO OPEN |
| B21-SRV-CF-6OPEN | 1.90E-04 | CCF TO OPEN 6 SRVs |
| C51-ACT-CF-APRMSTUCK | 2.10E-07 | CCF APRM DETECTORS STUCK AT POWER LEVEL |

Table 4.1-6
List of System Human Errors

| Basic Event | Prob | Description |
|--------------------|-------------|--|
| | | |
| B21-XHE-FO-6OPEN | 1.61E-03 | OPERATOR FAILS TO OPEN 6/10 SRVs |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |

Table 4.1-7
List of System Basic Events

| Basic Event | Prob | Description |
|--------------------|----------|--|
| B21-SQV-CC-F004A | 3.00E-03 | EXPLOSIVE VALVE DPV A FAILS TO OPERATE |
| B21-SQV-CC-F004B | 3.00E-03 | EXPLOSIVE VALVE DPV B FAILS TO OPERATE |
| B21-SQV-CC-F004C | 3.00E-03 | EXPLOSIVE VALVE DPV C FAILS TO OPERATE |
| B21-SQV-CC-F004D | 3.00E-03 | EXPLOSIVE VALVE DPV D FAILS TO OPERATE |
| B21-SQV-CC-F004E | 3.00E-03 | EXPLOSIVE VALVE DPV E FAILS TO OPERATE |
| B21-SQV-CC-F004F | 3.00E-03 | EXPLOSIVE VALVE DPV F FAILS TO OPERATE |
| B21-SQV-CC-F004G | 3.00E-03 | EXPLOSIVE VALVE DPV G FAILS TO OPERATE |
| B21-SQV-CC-F004H | 3.00E-03 | EXPLOSIVE VALVE DPV H FAILS TO OPERATE |
| B21-SQV-CF-DPVOPEN | 1.50E-05 | CCF OF DPV'S TO OPEN |
| B21-SQV-CF-F004AB | 3.60E-05 | CCF OF DPV A AND B TO OPEN |
| B21-SQV-CF-F004ABC | 9.00E-06 | CCF OF DPV A, B & C TO OPEN |
| B21-SQV-CF-F004ABD | 9.00E-06 | CCF OF DPV A, B & D TO OPEN |
| B21-SQV-CF-F004ABE | 9.00E-06 | CCF OF DPV A, B & E TO OPEN |
| B21-SQV-CF-F004ABF | 9.00E-06 | CCF OF DPV A, B & F TO OPEN |
| B21-SQV-CF-F004ABG | 9.00E-06 | CCF OF DPV A, B & G TO OPEN |
| B21-SQV-CF-F004ABH | 9.00E-06 | CCF OF DPV A, B & H TO OPEN |
| B21-SQV-CF-F004AC | 3.60E-05 | CCF OF DPV A AND C TO OPEN |
| B21-SQV-CF-F004ACD | 9.00E-06 | CCF OF DPV A, C & D TO OPEN |
| B21-SQV-CF-F004ACE | 9.00E-06 | CCF OF DPV A, C & E TO OPEN |
| B21-SQV-CF-F004ACF | 9.00E-06 | CCF OF DPV A, C & F TO OPEN |
| B21-SQV-CF-F004ACG | 9.00E-06 | CCF OF DPV A, C & G TO OPEN |
| B21-SQV-CF-F004ACH | 9.00E-06 | CCF OF DPV A, C & H TO OPEN |
| B21-SQV-CF-F004AD | 3.60E-05 | CCF OF DPV A AND D TO OPEN |
| B21-SQV-CF-F004ADE | 9.00E-06 | CCF OF DPV A, D & E TO OPEN |
| B21-SQV-CF-F004ADF | 9.00E-06 | CCF OF DPV A, D & F TO OPEN |
| B21-SQV-CF-F004ADG | 9.00E-06 | CCF OF DPV A, D & G TO OPEN |
| B21-SQV-CF-F004ADH | 9.00E-06 | CCF OF DPV A, D & H TO OPEN |
| B21-SQV-CF-F004AE | 3.60E-05 | CCF OF DPV A AND E TO OPEN |
| B21-SQV-CF-F004AEF | 9.00E-06 | CCF OF DPV A, E & F TO OPEN |
| B21-SQV-CF-F004AEG | 9.00E-06 | CCF OF DPV A, E & G TO OPEN |
| B21-SQV-CF-F004AEH | 9.00E-06 | CCF OF DPV A, E & H TO OPEN |
| B21-SQV-CF-F004AF | 3.60E-05 | CCF OF DPV A AND F TO OPEN |
| B21-SQV-CF-F004AFG | 9.00E-06 | CCF OF DPV A, F & G TO OPEN |

Table 4.1-7
List of System Basic Events

| Basic Event | Prob | Description |
|--------------------|----------|-----------------------------|
| B21-SQV-CF-F004AFH | 9.00E-06 | CCF OF DPV A, F & H TO OPEN |
| B21-SQV-CF-F004AG | 3.60E-05 | CCF OF DPV A AND G TO OPEN |
| B21-SQV-CF-F004AGH | 9.00E-06 | CCF OF DPV A, G & H TO OPEN |
| B21-SQV-CF-F004AH | 3.60E-05 | CCF OF DPV A AND H TO OPEN |
| B21-SQV-CF-F004BC | 3.60E-05 | CCF OF DPV B AND C TO OPEN |
| B21-SQV-CF-F004BCD | 9.00E-06 | CCF OF DPV B, C & D TO OPEN |
| B21-SQV-CF-F004BCE | 9.00E-06 | CCF OF DPV B, C & E TO OPEN |
| B21-SQV-CF-F004BCF | 9.00E-06 | CCF OF DPV B, C & F TO OPEN |
| B21-SQV-CF-F004BCG | 9.00E-06 | CCF OF DPV B, C & G TO OPEN |
| B21-SQV-CF-F004BCH | 9.00E-06 | CCF OF DPV B, C & H TO OPEN |
| B21-SQV-CF-F004BD | 3.60E-05 | CCF OF DPV B AND D TO OPEN |
| B21-SQV-CF-F004BDE | 9.00E-06 | CCF OF DPV B, D & E TO OPEN |
| B21-SQV-CF-F004BDF | 9.00E-06 | CCF OF DPV B, D & F TO OPEN |
| B21-SQV-CF-F004BDG | 9.00E-06 | CCF OF DPV B, D & G TO OPEN |
| B21-SQV-CF-F004BDH | 9.00E-06 | CCF OF DPV B, D & H TO OPEN |
| B21-SQV-CF-F004BE | 3.60E-05 | CCF OF DPV B AND E TO OPEN |
| B21-SQV-CF-F004BEF | 9.00E-06 | CCF OF DPV B, E & F TO OPEN |
| B21-SQV-CF-F004BEG | 9.00E-06 | CCF OF DPV B, E & G TO OPEN |
| B21-SQV-CF-F004BEH | 9.00E-06 | CCF OF DPV B, E & H TO OPEN |
| B21-SQV-CF-F004BF | 3.60E-05 | CCF OF DPV B AND F TO OPEN |
| B21-SQV-CF-F004BFG | 9.00E-06 | CCF OF DPV B, F & G TO OPEN |
| B21-SQV-CF-F004BFH | 9.00E-06 | CCF OF DPV B, F & H TO OPEN |
| B21-SQV-CF-F004BG | 3.60E-05 | CCF OF DPV B AND G TO OPEN |
| B21-SQV-CF-F004BGH | 9.00E-06 | CCF OF DPV B, G & H TO OPEN |
| B21-SQV-CF-F004BH | 3.60E-05 | CCF OF DPV B AND H TO OPEN |
| B21-SQV-CF-F004CD | 3.60E-05 | CCF OF DPV C AND D TO OPEN |
| B21-SQV-CF-F004CDE | 9.00E-06 | CCF OF DPV C, D & E TO OPEN |
| B21-SQV-CF-F004CDF | 9.00E-06 | CCF OF DPV C, D & F TO OPEN |
| B21-SQV-CF-F004CDG | 9.00E-06 | CCF OF DPV C, D & G TO OPEN |
| B21-SQV-CF-F004CDH | 9.00E-06 | CCF OF DPV C, D & H TO OPEN |
| B21-SQV-CF-F004CE | 3.60E-05 | CCF OF DPV C AND E TO OPEN |
| B21-SQV-CF-F004CEF | 9.00E-06 | CCF OF DPV C, E & F TO OPEN |
| B21-SQV-CF-F004CEG | 9.00E-06 | CCF OF DPV C, E & G TO OPEN |
| B21-SQV-CF-F004CEH | 9.00E-06 | CCF OF DPV C, E & H TO OPEN |
| B21-SQV-CF-F004CF | 3.60E-05 | CCF OF DPV C AND F TO OPEN |

Table 4.1-7
List of System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|--|
| B21-SQV-CF-F004CFG | 9.00E-06 | CCF OF DPV C, F & G TO OPEN |
| B21-SQV-CF-F004CFH | 9.00E-06 | CCF OF DPV C, F & H TO OPEN |
| B21-SQV-CF-F004CG | 3.60E-05 | CCF OF DPV C AND G TO OPEN |
| B21-SQV-CF-F004CGH | 9.00E-06 | CCF OF DPV C, G & H TO OPEN |
| B21-SQV-CF-F004CH | 3.60E-05 | CCF OF DPV C AND H TO OPEN |
| B21-SQV-CF-F004DE | 3.60E-05 | CCF OF DPV D AND E TO OPEN |
| B21-SQV-CF-F004DEF | 9.00E-06 | CCF OF DPV D, E & F TO OPEN |
| B21-SQV-CF-F004DEG | 9.00E-06 | CCF OF DPV D, E & G TO OPEN |
| B21-SQV-CF-F004DEH | 9.00E-06 | CCF OF DPV D, E & H TO OPEN |
| B21-SQV-CF-F004DF | 3.60E-05 | CCF OF DPV D AND F TO OPEN |
| B21-SQV-CF-F004DFG | 9.00E-06 | CCF OF DPV D, F & G TO OPEN |
| B21-SQV-CF-F004DFH | 9.00E-06 | CCF OF DPV D, F & H TO OPEN |
| B21-SQV-CF-F004DG | 3.60E-05 | CCF OF DPV D AND G TO OPEN |
| B21-SQV-CF-F004DGH | 9.00E-06 | CCF OF DPV D, G & H TO OPEN |
| B21-SQV-CF-F004DH | 3.60E-05 | CCF OF DPV D AND H TO OPEN |
| B21-SQV-CF-F004EF | 3.60E-05 | CCF OF DPV E AND F TO OPEN |
| B21-SQV-CF-F004EFG | 9.00E-06 | CCF OF DPV E, F & G TO OPEN |
| B21-SQV-CF-F004EFH | 9.00E-06 | CCF OF DPV E, F & H TO OPEN |
| B21-SQV-CF-F004EG | 3.60E-05 | CCF OF DPV E AND G TO OPEN |
| B21-SQV-CF-F004EGH | 9.00E-06 | CCF OF DPV E, G & H TO OPEN |
| B21-SQV-CF-F004EH | 3.60E-05 | CCF OF DPV E AND H TO OPEN |
| B21-SQV-CF-F004FG | 3.60E-05 | CCF OF DPV F AND G TO OPEN |
| B21-SQV-CF-F004FGH | 9.00E-06 | CCF OF DPV F, G & H TO OPEN |
| B21-SQV-CF-F004FH | 3.60E-05 | CCF OF DPV F AND H TO OPEN |
| B21-SQV-CF-F004GH | 3.60E-05 | CCF OF DPV G AND H TO OPEN |
| B21-SRV-CF-6OPEN | 1.90E-04 | CCF TO OPEN 6 SRVs |
| B21-XHE-FO-6OPEN | 1.61E-03 | OPERATOR FAILS TO OPEN 6/10 SRVs |
| C51-ACT-CF-APRMSTUCK | 2.10E-07 | CCF APRM DETECTORS STUCK AT POWER LEVEL |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |

Table 4.1-8
List of System Top Events

| Top Event | Description |
|------------------|---------------------------------------|
| | |
| GB21DPVTOP3 | 5/8 DPVs FAIL TO OPEN |
| GB21-0001-_11 | 6 OUT OF 10 SRV FAIL TO OPEN MANUALLY |

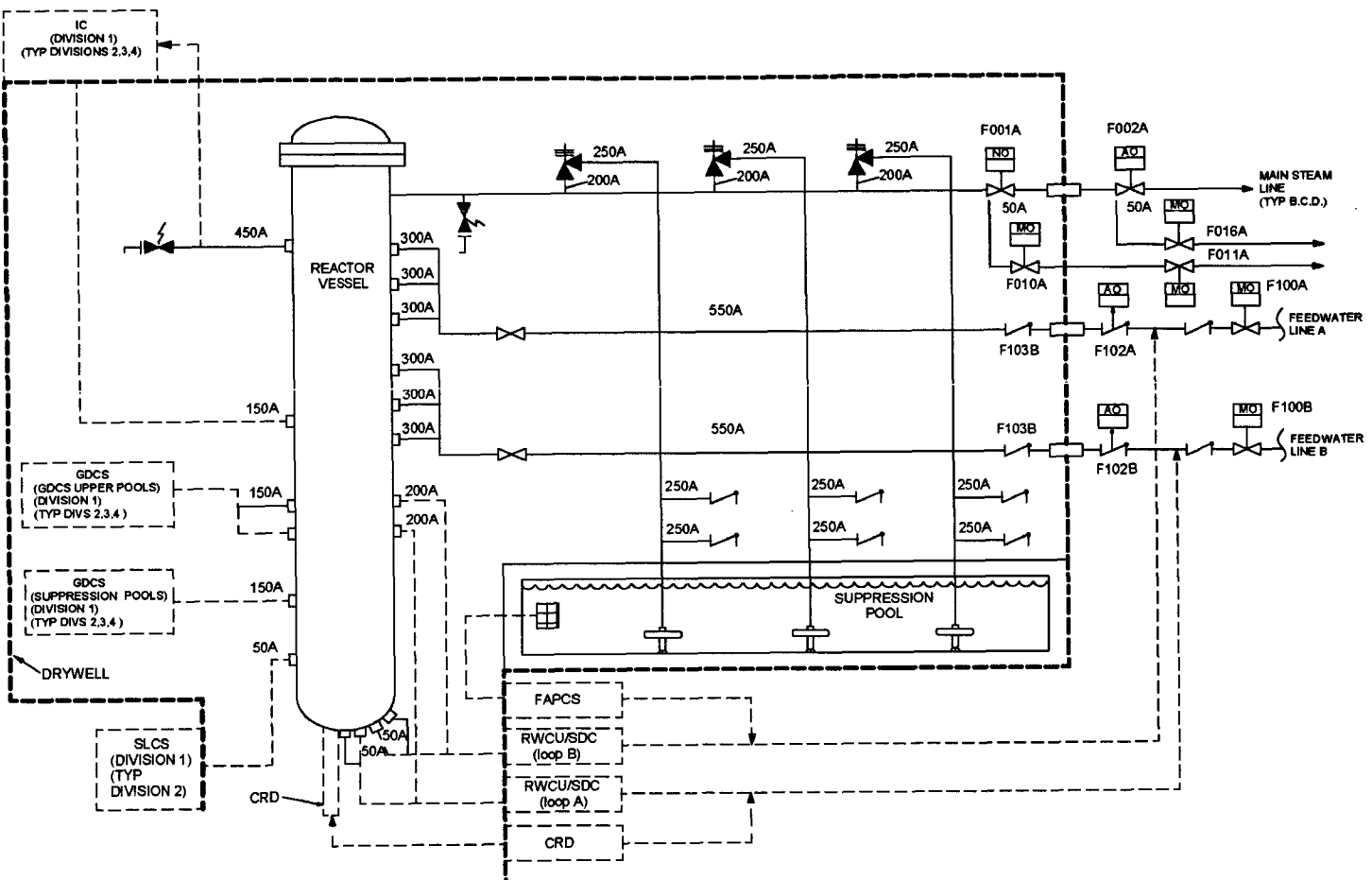


Figure 4.1-1. Simplified Process Diagram for Nuclear Boiler System

4.2 ISOLATION CONDENSER SYSTEM – (B32)

4.2.1 Purpose

The Isolation Condenser System (ICS) limits reactor pressure and temperature within an acceptable range so that safety/relief valve operation is limited and automatic reactor depressurization will not occur when the reactor becomes isolated during power operations.

The ICS is used to transfer decay and residual heat from the reactor after the reactor is shut down and isolated. This function can also be performed by the other Engineered Safety Features (ESFs) of Automatic Depressurization System (ADS) and Gravity Driven Cooling System (GDCS). ICS functions to avoid unnecessary use of these other ESFs for residual heat removal and ICS provides additional liquid inventory upon opening of the condensate return valves to initiate the system. The IC system also provides the means for initial depressurization of the reactor before ADS initiation from a low reactor water level.

The Isolation Condenser System consists of four, high pressure, independent loops, each containing a heat exchanger that condenses steam on the tube side and transfers heat to water in a large Isolation Condenser/Passive Containment Cooling pool (IC/PCC pool) that is vented to the atmosphere.

The units are located in the IC/PCC pool positioned above, and outside, the containment (drywell). The condenser, connected by piping to the reactor pressure vessel, is placed at an elevation above the source of steam (reactor vessel) and, when the steam is condensed, condensate is returned to the vessel via a condensate return pipe.

The steam side connection between the vessel and the IC is normally open and the condensate line is normally closed. This allows the isolation condenser and drain piping to fill with condensate that is maintained at a sub-cooled temperature by the IC/PCC pool water during normal reactor operation.

The isolation condenser is started into operation by draining the condensate to the reactor, thus causing steam from the reactor to fill the tubes that transfer heat to the cooler pool water.

4.2.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) Two limit switches per Main Steam Line are used to generate the MSIV closure signal. Two limit switches, each one belonging to a different MSIV (inboard and outboard) on the same Main Steam Line (MSL), serve as inputs to generate the opening signals to motor-operated F005 valves, when one MSIV in each MSL is closing.
- (2) A diverse initiating signal of motor-operated F005 valves is generated by a high RPV pressure > 7.447 MPa-gauge (1080 psig) for 10 seconds.
- (3) One nitrogen-operated valve (F006) is present in each return condensate line. Each F006 valve is maintained closed by means of the nitrogen flowing through one of two solenoid valves, normally energized, and powered by different sources.

Upon loss of nitrogen or power, the valves open.

Both solenoid valves have to de-energize to open the F006 valve.

The opening of either one of two load drivers in series on the power line reenergizes both solenoid valves.

Different Safety System Logic and Control (SSLC) channels are employed to open the two load drivers.

- (4) Automatic initiation of the ICS F006 valves is also provided by a reactor water level 2 signal.
- (5) The ICS is able to maintain reactor pressure below the safety valve operating pressure with three of four ICs in service following reactor isolation from 100% power and successful scram.

4.2.3 System Description

The systems are assumed to operate as described in Reference 4.2-1 for the ICS.

4.2.3.1 Hardware Configuration

Isolation Condenser System (ICS)

A simplified P&ID is shown in Figure 4.2-1.

The ICS consists of four high pressure, independent loops, each containing a condenser that condenses steam on the tube side and transfers heat to water in a large pool (IC/PCC pool) positioned above and outside the primary containment. The surface of the pool is vented to the atmosphere through a plant stack.

Each ICS condenser is connected by piping to the reactor pressure vessel and is placed at an elevation above the source of steam (RPV). When the steam is condensed, it returns to the vessel via a condensate return pipe.

The steam side connection between the vessel and the ICS condenser is normally open and the condensate line is normally closed. This allows the isolation condenser and drain piping to fill with condensate, which is maintained at a subcooled temperature by the pool water during normal reactor operation.

Portions of the ICS system outside the containment are located in a subcompartment of the safety-related IC/PCC pool to provide protection from missiles, tornadoes and winds. The IC steam supply pipes include flow restrictors and the IC condensate drain pipes diameter is sized such that if an IC piping or tube rupture occurs in the safety-related IC/PCC pool, flow-induced dynamic loads and pressure buildup in the IC/PCC pool would be limited by these design features.

The steam supply line is vertical and feeds two horizontal headers through four pipes. The steam line is properly insulated and enclosed in a guard pipe, which penetrates the containment roof slab, up to the distribution branching point. Guard pipes and special transition fittings are used at the locations where the IC steam supply and condensate return pipes enter the pool at the containment pressure boundary.

Two fail-as-is isolation valves in series (nitrogen-motor-operated F001 and motor-operated F002) are located in the run of steam supply line piping inboard of the containment boundary.

They are used to isolate that part of the ICS that is located outside the containment. Two different valve actuator types are used to assure flow path closure.

Steam is condensed inside vertical tubes and is collected in two lower headers. Two pipes, one from each lower header, take the condensate to the common drain line, which vertically penetrates the containment roof slab.

On the condensate return piping, two fail-as-is isolation valves in series (motor-operated F003 and nitrogen-motor-operated F004) are provided, both located inboard of the containment boundary. They are also used to isolate parts of the ICS outside the containment. Two different valve actuator types are used to assure flow path closure.

A loop seal and a condensate return valve (F006, nitrogen-operated, fail open) are located on the condensate return piping just upstream from the reactor entry point. This valve is closed during normal plant power operations. Because the steam supply line valves are normally open, condensate forms in the ICS condenser and fills up to the steam distributor, above the upper headers.

The isolation condenser operation is initiated by draining the condensate to the reactor, thus causing steam from the reactor to fill the tubes, which transfer heat to the water of the cooler pool. The motor-operated condensate return valve (F005) is opened to commence ICS operation. Whereupon the collected condensate within the condensate return line, drains into the reactor. The steam-water interface in the condenser tube bundle moves downward, below the lower headers, to a point within the main condensate return line. A similar physical process is followed if the valves initially opened are the nitrogen-operated condensate return valves (F006). The fail open nitrogen-operated condensate return bypass valve (F006) opens if the 250 V DC power or nitrogen supply are lost.

A vent line to the suppression pool is provided for both the upper and lower headers in order to remove the non-condensable gases during ICS operation.

Venting is controlled as follows: two normally closed, fail closed, solenoid-operated valves (F009 and F010) are located in the vent line from the lower headers. They can be actuated both automatically and manually; automatically, when RPV pressure is high and either of the condensate return valves is open, and manually, by the control room operator. In case of F009 and/or F010 failure, two bypass motor-operated valves, F011 and F012, (normally closed) allow the operator to vent non-condensable gases.

A vent line from the upper headers equipped with two normally closed, fail closed, solenoid-operated valves (F007 and F008) allows the operator to vent non-condensable gases, if necessary.

During normal plant operation, non-condensable gases are prone to accumulate in the ICS condenser due to hydrogen buildup from water chemistry control additions and air entrained in the feedwater. A purge line is provided to assure that the tubes are not blanketed with non-condensibles when the system is first started.

ICS Pool

The isolation condensers (IC) and the passive containment cooling (PCCS) condensers are located in the same large water pool, positioned above the drywell. The large ICS pool is

partitioned, but both the ICS and PCCS are able to draw water from the entire pool. The air and steam spaces are also common.

The pool subcompartment interconnections are as follows: the ICS and PCCS pool subcompartments are connected to the other pools below the water level by locked open valves, one for each subcompartment. These valves can be closed to isolate and drain the individual partitioned ICS pool for maintenance of the unit. All other pool subcompartments are interconnected below the pool water level.

The remote handwheels on the locked open valves extend above the water level to locations accessible to the operator.

The walls containing the airspace flow path extend above the normal water level. This enhances the flow stability and heat removal capability of the condensers by establishing a flow path for the makeup water through the lower pipes.

4.2.3.2 System Operation

During normal plant operation, the ICS subloop is on ready standby. Both steam supply isolation valves and both isolation valves on the condensate return line are in a normally open position with the condensate level in the ICS condenser extending above the upper headers. The condensate return valves are both closed, and the small vent lines from the ICS condenser top and bottom headers to the suppression pool are also closed.

Steam flow is induced from the steam distributor through the purge line by the pressure differential caused by main steam line flow. The valve status, failure mode, actuator type, pipe size, valve type, and line location are as follows:

| Valve number | Status | Failure mode | Actuator type | Pipe Size mm (inches) | Valve type | Location |
|---------------------|---------------|---------------------|----------------------|------------------------------|-------------------|-------------------|
| F001 | NO | AI | NMO | 350 (14) | Gate | Steam line |
| F002 | NO | AI | MO | 350 (14) | Gate | Steam line |
| F003 | NO | AI | MO | 200 (8) | Gate | Condensate to RPV |
| F004 | NO | AI | NMO | 200 (8) | Gate | Condensate to RPV |
| F005 | NC | AI | MO | 200 (8) | Gate | Condensate to RPV |
| F006 | NC | FO | NO | 200 (8) | Globe | Condensate to RPV |
| F007 | NC | FC | SO | 25 (1) | Globe | Vent line to SP |
| F008 | NC | FC | SO | 25 (1) | Globe | Vent line to SP |
| F009 | NC | FC | SO | 25 (1) | Globe | Vent line to SP |
| F010 | NC | FC | SO | 25 (1) | Globe | Vent line to SP |
| F011 | NC | AI | MO | 25 (1) | Globe | Vent line to SP |
| F012 | NC | AI | MO | 25 (1) | Globe | Vent line to SP |
| F013 | NO | AI | MO | 20(3/4) | Globe | Purge line to MSL |

| | | |
|--------|-----|---------------------------|
| Legend | NO | = normally open |
| | NC | = normally closed |
| | AI | = as is |
| | FO | = fail open |
| | FC | = fail closed |
| | MO | = motor operated |
| | NMO | = nitrogen motor operated |
| | SO | = solenoid operated |
| | NO | = nitrogen operated |

During refueling, the ICS condenser is isolated from the reactor; all isolation valves (F001 through F004) and all vent valves (F007 through F012) are closed.

During plant operation, when one of the ICS initiation signals (as described in Section 4.2.4) opens condensate return valve F005 or F006 with a delay of 30 seconds, operation of the ICS starts.

If, during ICS operation and after the initial transient, the RPV pressure increases above 7.516 MPa-gauge, the bottom vent valves, F009 and F010, automatically open. When the RPV pressure decreases below 7.447 MPa-gauge (reset value), and after a time delay these two valves close.

4.2.3.3 Component Location

The ICS condensers are located in a large pool positioned above, and outside, the ESBWR primary containment.

The large pool is partitioned between the ICS and PCCS condensers, but each is able to draw water from the entire pool; the air and steam space is also open to both.

The ICS condensers are extensions of the reactor coolant pressure boundary and are not isolated from it.

The piping and valves are all within the drywell. The vent valves (F007 through F012) are located in a vertical pipe run near the top of the drywell.

The radiation monitors used to detect steam leakage to the pool are installed in each IC pool exhaust passage to the atmosphere to detect IC leakage.

4.2.4 Automatic and Manual Control

The initiation signals actuate all four ICS loops at the same time by opening the F005 valves (A, B, C and D). The initiation signals are described as follows:

- a. The "reactor mode switch is in run" and at least one of the inboard or outboard MSIVs position switch are less than 92% open on the four Main Steam Lines.

There are two MSIVs on each main steam line. The logic is one-out-of-two limit switches of the MSIVs on the same line plus one-out-of-two limit switches of the MSIVs on the other lines. All MSLs must be closed for ICS actuation.

- b. RPV pressure (with logic two-out-of-four) > 7.447 MPa-gauge for 10 seconds or more.
- c. Operator Manual Initiation.

Other initiation signals actuate all four ICS loops at the same time by opening the F006 valves (A, B, C, D). These other initiation signals are as follows:

- a. Reactor water level below Level 2.
- b. Operator Manual Initiation.

The condensate return valve (F006) opens automatically upon a loss of the two electrical power divisions, or on the loss of nitrogen supply, or manually, by operator action.

Initiation of the ICS automatically generates the automatic opening of valves (A, B, C, and D) for F001 through F004.

The automatic valves interconnecting the upper containment pools have a manual backup. The signal is from the water level in the pools. This gives the operators a chance to makeup with demineralized water and thus prevent opening of the valves.

Automatic actuation for the vent valves (F009 and F010) is provided by a high RPV pressure (above system actuation value) and either of the condensate return valves not fully closed (with a time delay to avoid the vents opening during the initial transient). The valves close preventing loss of inventory when the RPV pressure decreases below the 7.447 MPa-gauge reset value.

The Leak Detection and Isolation System (LD&IS) isolates each IC loop individually on high pool radiation or on high flow (as measured by high differential pressure) in the steam supply line or the condensate return line. (See Paragraph 4.1 for detailed system description).

- Four radiation sensors are installed in each IC-HX pool exhaust passages that vent air and coolant vapor to the environment. Detection of a low-level leak (radiation level above background, logic two-out-of-four) initiates an alarm. Detection of a high radiation level (exceeding site boundary limits, logic two-out-of-four) isolates the leaking isolation condenser automatically (closure of isolation valves F001 through F004). The high radiation may be due to a leak from any condenser tube and a subsequent release of noble gas to the air above the pool.
- Four redundant sets of differential Pressure Transmitters (dPT) on the steam line and another four sets on the condensate return line are used to detect a possible LOCA. A high dPT signal coming from two-out-of-four dPTs on the same line (steam or condensate) results in alarms to the operator and automatic closure of all isolation valves, rendering the affected IC inoperable.

Closure of the isolation valves (F001 through F004) and an alarm is automatic on the following signals coming from their own loop (logic two-out-of-four):

- high mass flow in the ICS steam supply line or
- high mass flow in the ICS condensate return line or

- high radiation in the IC-HX pool steam flow path

The operator cannot override the high radiation signals from the IC atmospheric vents and high differential pressure IC isolation signals.

A temperature element is provided downstream from the valves in each vent line to confirm vent-valve function and in the condensate return line, downstream from isolation valve F004.

The instrument and alarm signals reported in the control room are given in Table 4.2-1.

4.2.5 System Interfaces

Support Systems

The condensate return valve (MO F005) is a motor-operated, double disk wedge gate type valve, and designed to fail as-is upon loss of essential 480 V AC power. The condensate return valve (MO F005, A, B, C or D) for each ICS condenser is supplied as follows:

| | |
|------|----------------------------------|
| IC A | Essential 480 V AC power, Div. 1 |
| IC B | Essential 480 V AC power, Div. 2 |
| IC C | Essential 480 V AC power, Div. 3 |
| IC D | Essential 480 V AC power, Div. 4 |

The inboard isolation valves (NMO-F001 and F004) on any ICS loop are supplied by essential 250 V DC power, Div. 2 and the High Pressure Nitrogen Gas Supply System. Nitrogen charged pneumatic accumulators supply the valves during abnormal (i.e. emergency) conditions.

The outboard isolation valves (MO-F002 and F003, A, B, C, and D) for any ICS loop are supplied by essential 250 V DC power, Div. 1.

Valve F006 (A, B, C, and D) interfaces with the SSLC which provides the initiation signals for valve opening. Table 4.2-2 reports the System Dependency Matrix for valves F006 (A, B, C, D).

The Fire Protection System is able to provide makeup water to the IC/PCC pool in the long term and it is included in the model.

4.2.6 System Testing

During normal plant operations, a quarterly surveillance test of the normally-closed valves, F005 and F006, on the ICS condensate line to the RPV is performed. The test procedure of the condensate return valve starts after the condensate return line isolation valves, F003 and F004, are closed. This avoids subjecting the condenser to unnecessary thermal heat-up and cooldown cycles. Isolation valves on the steam supply line (i.e., F001 and F002) remain open to avoid ICS depressurization. The control room operator via remote manual switches that actuate the isolation valves and the condensate return valves performs the test. Their status light verifies the opening and closure of the valves. The procedure is as follows:

- Close valves F003 and F004
- Open fully and then close valves F005/F006
- Re-open isolation valves F003 and F004 to put the ICS on stand-by condition

The isolation valves (F001, F002, F003, and F004) are tested quarterly, one at a time. If an automatic system actuation signal occurs during the test, all the valves are aligned automatically to permit the ICS to start operation.

Each vent valve (F007 through F012) is tested quarterly. These valves, in series, are opened one at a time during normal plant operation. A permissive allows the operator to open one vent valve if the other one in series is closed.

Purge line root valve F013 is tested quarterly.

Table 4.2-3 shows the test program for the system.

4.2.7 System Maintenance

Preventive maintenance actions are not expected to be performed during normal plant operation for the ICS.

Corrective maintenance for tube plugging following the detection of a tube leak is performed during refueling. After closing the isolation valves to or from the condenser, plugging of the leaking tube is performed by personnel operating from the refueling floor. Maintenance is performed from the upper and lower ends of the condensers after removal of the header covers. A remotely operated tool is used.

The pool water in the isolation condenser subcompartment is removable without emptying the entire IC/PCC pool. The individual partitioned ICS pool is isolated by closing the locked open valve interconnecting it to the other pools and emptying the compartment using a portable pump. This water is discharged into the common pool. To replenish the pool, the normally locked open valve is reopened and the water refills the pool.

Table 4.2-4 shows the test program for the system.

4.2.8 Common Cause Failures

Common cause failures within the ICS are summarized in Table 4.2-5 including the CCF of logic.

4.2.9 Fault Tree Analysis

The ICS fault trees are evaluated to determine the system unavailabilities and to identify the components that contribute significantly.

4.2.9.1 Top Event Definitions

The ICS is analyzed for the following events using the same fault tree (GB32TOP2):

- All Transients, to avoid Level 1 without RCPB leakage and to prevent lifting SRVs
- ATWS events

Table 4.2-8 reports the list of ICS top events.

4.2.9.2 Fault Tree Description

The fault tree is developed in Appendix B.4.2.

4.2.9.3 Human Interactions

Table 4.2-6 summarizes the human errors considered in the present analysis.

4.2.10 Results of Fault Tree Analysis

The fault tree results provide quantitative values of systems unavailability and the importance of specific components to that total.

The definition of each basic event is reported in Table 4.2-7.

The quantification of top events for ICS includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

The importance measures obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.2.11 Model Differences for 72-Hour Operation

The IC pools provide sufficient cooling water for 24 hours. To maintain the pool cooling function beyond 24 hours, it is necessary to interconnect IC/PCC and Reactor Well pools through the MOVs provided (F72HA, F72HB, F72HC, F72HD). This is an automatic action initiated on low pool level. (See Figure 4.2-2) Alternatively, the lost water can be resupplied via a manual cross tie from the Fire Protection Systems.

4.2.12 References

4.2-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.2-1
Control Room Instrumentation and Alarms

| |
|--|
| Controls |
| |
| Displays |
| (6) Remote operable valves status lights |
| (7) A vent line element downstream of the valves to confirm vent function (these elements send signals to a temperature recorder in the control room). |
| Alarms |
| (8) High mass flow (dPT) in steam mass supply |
| (9) High mass flow (dPT) in condensate return line to RPV |
| (10) High radiation in the pool vents |
| (11) Remote operable valves computer alarms |

Table 4.2-2
System Dependency Matrix

| | | Support Systems | | | |
|-----------|------|-----------------|-------|-------|-------|
| | | Power Supply | | | |
| | | R13 480 V AC | | | |
| Component | TYPE | DIV 1 | DIV 2 | DIV 3 | DIV 4 |
| F005A | MOV | X | | | |
| F005B | MOV | | X | | |
| F005C | MOV | | | X | |
| F005D | MOV | | | | X |

Table 4.2-3
Component Test

| COMPONENT | TYPE | Expected test interval | Components aligned away from emergency position without automatic return logic |
|------------------|-------------|-------------------------------|---|
| F005A/B/C/D | MOV | Quarterly | None |
| F006A/B/C/D | NMO | Quarterly | None |
| F001A/B/C/D | NMO | Quarterly | None |
| F002A/B/C/D | MOV | Quarterly | None |
| F003A/B/C/D | MOV | Quarterly | None |
| F004A/B/C/D | NMO | Quarterly | None |

Table 4.2-4
Component Maintenance

No preventive actions are expected to be performed during normal plant operation.

Corrective maintenance of ICS condenser tube plugging following tube leak detection can be performed during refueling.

If there is considerable damage to some component part of the ICS or PCCS, each module of the unit is easily removable, after cutting the feed, drain and vent lines.

Table 4.2-5**List of System Common Cause Failures**

| Basic Event | Prob | Description |
|-----------------------|-------------|--|
| B21-LT_-CF-N001ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-LT_-CF-N010ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 10A/B/C/D |
| B21-PT_-CF-02ALLNO | 3.00E-08 | CCF OF RPV PRESSURE TRANSMITTERS |
| B32-ACV-CF-2ICABCD | 1.55E-05 | CCF TO OPEN 2/4 ACV VALVES TRAINS A, B, C, D |
| B32-LT_-CF-LT72HA | 7.20E-08 | CCF 3 OUT OF FOUR A LEVEL TRANSMITTERS |
| B32-MOV-CF-2ICABCD | 8.08E-06 | CCF TO OPEN 2/4 MOV VALVES TRAINS A, B, C, D |
| B32-MOV-CF-F005A/B | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & B |
| B32-MOV-CF-F005A/C | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & C |
| B32-MOV-CF-F005A/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & D |
| B32-MOV-CF-F005B/C | 1.44E-05 | CCF TO OPERATE BETWEEN F005 B & C |
| B32-MOV-CF-F005B/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 B & D |
| B32-MOV-CF-F005C/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 C & D |
| B32-MOV-CF-F72HADOPEN | 2.00E-04 | CCF VALVES F72HA AND D TO OPEN |
| B32-MOV-CF-F72HBCOPEN | 2.00E-04 | CCF VALVES F72HB AND C TO OPEN |
| B32-PDT-CF-3ICAHIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER A SPURIOUS ACTUATION |
| B32-PDT-CF-3ICBHIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER B SPURIOUS ACTUATION |
| B32-PDT-CF-3ICC HIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER C SPURIOUS ACTUATION |
| B32-PDT-CF-3ICD HIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER D SPURIOUS ACTUATION |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV1/2/3/4 |
| C74-VLU-CF-ALL | 3.12E-06 | CCF OF VOTER LOGIC UNITS |
| D11-ACT-CF-3ICAHIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-A SPURIOUS ACTUATION |
| D11-ACT-CF-3ICBHIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-B SPURIOUS ACTUATION |
| D11-ACT-CF-3ICC HIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-C SPURIOUS ACTUATION |
| D11-ACT-CF-3ICD HIGH | 1.50E-07 | CCF ¾ RADIATION MONITORS ICD-D SPURIOUS ACTUATION |
| H23-EMS-CF-ALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-EMS-CF-ALL | 9.00E-07 | CCF OLF REMOTE MULTIPLEXING UNITS TO OPERATE |

Table 4.2-6
List of System Human Errors

| Basic Event | Prob | Description |
|--------------------|-------------|---|
| B32-XHE-FO-F72HA | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HA |
| B32-XHE-FO-F72HB | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HB |
| B32-XHE-FO-F72HC | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HC |
| B32-XHE-FO-F72HD | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HD |
| XXX-XHE-FO-ICPCCS | 1.61E-03 | OPERATOR FAILS TO RECOGNIZE NEED OF MAKE UP TO IC/PCCS POOLS |

| Table 4.2-7 List of System Basic Events | | |
|--|----------|---|
| Basic Event | Prob | Description |
| B21-ACV-OO-F001A | 2.00E-03 | NOV F001A FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F001B | 2.00E-03 | NOV F001B FAILS TO OPEARTE TO DEENER. POSITION. |
| B21-ACV-OO-F001C | 2.00E-03 | NOV F001C FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F001D | 2.00E-03 | NOV F001D FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F002A | 2.00E-03 | AOV F002A FAILS TO OPERATE TO DEENER. POSTION. |
| B21-ACV-OO-F002B | 2.00E-03 | AOV F002B FAILS TO OPERATE. TO DENER. POSTION. |
| B21-ACV-OO-F002C | 2.00E-03 | AOV F002C FAILS TO OPERATE TO DEENER. POSTION. |
| B21-ACV-OO-F002D | 2.00E-03 | AOV F002D FAILS TO OPERATE TO DEENER. POSTION. |
| B21-LT_-CF-N001ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-LT_-CF-N010ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 10A/B/C/D |
| B21-LT_-NO-N001A | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1A (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001B | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1B (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001C | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1C (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001D | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1D (LEVEL 1&2) FAILS |
| B21-OR_-PG-01A | 1.44E-05 | ORIFICE INSTR. LINE 1A FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01B | 1.44E-05 | ORIFICE INSTR. LINE 1B FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01C | 1.44E-05 | ORIFICE INSTR. LINE 1C FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01D | 1.44E-05 | ORIFICE INSTR. LINE 1D FAILS TO REMAIN OPEN (PLUG) |
| B21-PT_-CF-02ALLNO | 3.00E-08 | CCF OF RPV PRESSURE TRANSMITTERS |
| B21-PT_-NO-0002A | 1.15E-05 | PRESSURE TRANSMITTER 0002A FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B21-PT_-NO-0002B | 1.15E-05 | PRESSURE TRANSMITTER 0002B FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B21-PT_-NO-0002C | 1.15E-05 | PRESSURE TRANSMITTER 0002C FAILS TO RESPOND TO CHANGE IN PRESSURE |

| Table 4.2-7 List of System Basic Events | | |
|--|----------|---|
| Basic Event | Prob | Description |
| B21-PT_-NO-0002D | 1.15E-05 | PRESSURE TRANSMITTER 0002D FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B32-ACV-CC-F006A | 2.00E-03 | AIR OPERATED VALVE F006A FAILS TO OP. TO DEENERG. POSIT. |
| B32-ACV-CC-F006B | 2.00E-03 | AIR OPERATED VALVE F006B FAILS TO OP. TO DEENERG. POSIT. |
| B32-ACV-CC-F006C | 2.00E-03 | AIR OPERATED VALVE F006A FAILS TO OP. TO DEENERG. POSIT. |
| B32-ACV-CC-F006D | 2.00E-03 | AIR OPERATED VALVE F006D FAILS TO OP. TO DEENERG. POSIT. |
| B32-ACV-CF-2ICABCD | 1.55E-05 | CCF TO OPEN 2/4 ACV VALVES TRAINS A, B, C, D |
| B32-LT_-CF-LT72HA | 7.20E-08 | CCF 3 OUT OF FOUR A LEVEL TRANSMITTERS |
| B32-LT_-NO-F72HADIV1 | 2.40E-05 | LEVEL TRANSMITTER DIV 1 POOL A FAILS |
| B32-LT_-NO-F72HADIV2 | 2.40E-05 | LEVEL TRANSMITTER DIV 2 POOL A FAILS |
| B32-LT_-NO-F72HADIV3 | 2.40E-05 | LEVEL TRANSMITTER DIV 3 POOL A FAILS |
| B32-LT_-NO-F72HADIV4 | 2.40E-05 | LEVEL TRANSMITTER DIV 4 POOL A FAILS |
| B32-LT_-NO-F72HBDIV1 | 2.40E-05 | LEVEL TRANSMITTER DIV 1 POOL B FAILS |
| B32-LT_-NO-F72HBDIV2 | 2.40E-05 | LEVEL TRANSMITTER DIV 2 POOL B FAILS |
| B32-LT_-NO-F72HBDIV3 | 2.40E-05 | LEVEL TRANSMITTER DIV 3 POOL B FAILS |
| B32-LT_-NO-F72HBDIV4 | 2.40E-05 | LEVEL TRANSMITTER DIV 4 POOL B FAILS |
| B32-LT_-NO-LT72HB | 2.40E-05 | CCF 3 OUT OF FOUR B LEVEL TRANSMITTERS |
| B32-MOV-CC-F005A | 4.00E-03 | MOTOR OPER. VALVE F005A FAILS TO OPEN |
| B32-MOV-CC-F005B | 4.00E-03 | MOTOR OPER. VALVE F005B FAILS TO OPEN |
| B32-MOV-CC-F005C | 4.00E-03 | MOTOR OPER. VALVE F005C FAILS TO OPEN |
| B32-MOV-CC-F005D | 4.00E-03 | MOTOR OPER. VALVE F005A FAILS TO OPEN |
| B32-MOV-CC-F72HA | 4.00E-03 | MOTOR OPERATED VALVE F72HA FAILS TO OPEN |
| B32-MOV-CC-F72HB | 4.00E-03 | VALVE F72HB FAILS TO OPEN |
| B32-MOV-CC-F72HC | 4.00E-03 | MOTOR OPERATED VALVE FAILS TO OPEN |
| B32-MOV-CC-F72HD | 4.00E-03 | MOTOR OPERATED VALVE FAILS TO OPEN |
| B32-MOV-CF-2ICABCD | 8.08E-06 | CCF TO OPEN 2/4 MOV VALVES TRAINS A, B, C, D |
| B32-MOV-CF-F005A/B | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & B |
| B32-MOV-CF-F005A/C | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & C |
| B32-MOV-CF-F005A/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 A & D |
| B32-MOV-CF-F005B/C | 1.44E-05 | CCF TO OPERATE BETWEEN F005 B & C |
| B32-MOV-CF-F005B/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 B & D |
| B32-MOV-CF-F005C/D | 1.44E-05 | CCF TO OPERATE BETWEEN F005 C & D |
| B32-MOV-CF-F72HADOPEN | 2.00E-04 | CCF VALVES F72HA AND D TO OPEN |
| B32-MOV-CF-F72HBCOPEN | 2.00E-04 | CCF VALVES F72HB AND C TO OPEN |

| Table 4.2-7 | | |
|------------------------------------|-------------|---|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| B32-MOV-OC-F001A | 3.36E-06 | MOTOR OPER. VALVE F001A FAILS TO REMAIN OPEN |
| B32-MOV-OC-F001B | 3.36E-06 | MOTOR OPER. VALVE F001B FAILS TO REMAIN OPEN |
| B32-MOV-OC-F001C | 3.36E-06 | MOTOR OPER. VALVE F001C FAILS TO REMAIN OPEN |
| B32-MOV-OC-F001D | 3.36E-06 | MOTOR OPER. VALVE F001D FAILS TO REMAIN OPEN |
| B32-MOV-OC-F002A | 3.36E-06 | MOTOR OPER. VALVE F002A FAILS TO REMAIN OPEN |
| B32-MOV-OC-F002B | 3.36E-06 | MOTOR OPER. VALVE F002B FAILS TO REMAIN OPEN |
| B32-MOV-OC-F002C | 3.36E-06 | MOTOR OPER. VALVE F002C FAILS TO REMAIN OPEN |
| B32-MOV-OC-F002D | 3.36E-06 | MOTOR OPER. VALVE F002D FAILS TO REMAIN OPEN |
| B32-MOV-OC-F003A | 3.36E-06 | MOTOR OPER. VALVE F003A FAILS TO REMAIN OPEN |
| B32-MOV-OC-F003B | 3.36E-06 | MOTOR OPER. VALVE F003B FAILS TO REMAIN OPEN |
| B32-MOV-OC-F003C | 3.36E-06 | MOTOR OPER. VALVE F003C FAILS TO REMAIN OPEN |
| B32-MOV-OC-F003D | 3.36E-06 | MOTOR OPER. VALVE F003D FAILS TO REMAIN OPEN |
| B32-MOV-OC-F004A | 3.36E-06 | MOTOR OPER. VALVE F004A FAILS TO REMAIN OPEN |
| B32-MOV-OC-F004B | 3.36E-06 | MOTOR OPER. VALVE F004B FAILS TO REMAIN OPEN |
| B32-MOV-OC-F004C | 3.36E-06 | MOTOR OPER. VALVE F004C FAILS TO REMAIN OPEN |
| B32-MOV-OC-F004D | 3.36E-06 | MOTOR OPER. VALVE F004D FAILS TO REMAIN OPEN |
| B32-OR_-PG-F72HADIV1 | 1.44E-05 | ORIFICE INSTR. LINE DIV 1 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HADIV2 | 1.44E-05 | ORIFICE INSTR. LINE DIV 2 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HADIV3 | 1.44E-05 | ORIFICE INSTR. LINE DIV 3 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HADIV4 | 1.44E-05 | ORIFICE INSTR. LINE DIV 4 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HBDIV1 | 1.44E-05 | ORIFICE INSTR. LINE DIV 1 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HBDIV2 | 1.44E-05 | ORIFICE INSTR. LINE DIV 2 FAILS TO REMAIN OPEN (PLUG) |

| Table 4.2-7 | | |
|------------------------------------|-------------|--|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| B32-OR_-PG-F72HBDIV3 | 1.44E-05 | ORIFICE INSTR. LINE DIV 3 FAILS TO REMAIN OPEN (PLUG) |
| B32-OR_-PG-F72HBDIV4 | 1.44E-05 | ORIFICE INSTR. LINE DIV 4 FAILS TO REMAIN OPEN (PLUG) |
| B32-PDT-CF-3ICAHIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER A SPURIOUS ACTUATION |
| B32-PDT-CF-3ICBHIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER B SPURIOUS ACTUATION |
| B32-PDT-CF-3ICC HIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER C SPURIOUS ACTUATION |
| B32-PDT-CF-3ICD HIGH | 9.71E-06 | CCF 3/4 PDT'S ISOLATION CONDENSER D SPURIOUS ACTUATION |
| B32-RMU-FC-F72HB1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| B32-RMU-FC-F72HB2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| B32-RMU-FC-F72HC1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| B32-RMU-FC-F72HC2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| B32-SYS-TM-ICA | 4.16E-02 | IC "A" UNAVAILABLE [# 7] |
| B32-XHE-FO-F72HA | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HA |
| B32-XHE-FO-F72HB | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HB |
| B32-XHE-FO-F72HC | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HC |
| B32-XHE-FO-F72HD | 1.61E-03 | OPERATOR FAILS TO OPEN VALVE F72HD |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV1/2/3/4 |
| C74-DTM-FC-DIV1 | 6.00E-04 | DTM OF SSLC DIV 1 FAILS TO TRIP |
| C74-DTM-FC-DIV2 | 6.00E-04 | DTM OF SSLC DIV. 2 FAILS TO TRIP |
| C74-DTM-FC-DIV3 | 6.00E-04 | DTM OF SSLC DIV. 3 FAILS TO TRIP |
| C74-DTM-FC-DIV4 | 6.00E-04 | DTM OF SSLC DIV. 4 FAILS TO TRIP |
| C74-VLU-CF-ALL | 3.12E-06 | CCF OF VOTER LOGIC UNITS |
| D11-ACT-CF-3ICAHIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-A SPURIOUS ACTUATION |
| D11-ACT-CF-3ICBHIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-B SPURIOUS ACTUATION |
| D11-ACT-CF-3ICC HIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-C SPURIOUS ACTUATION |
| D11-ACT-CF-3ICD HIGH | 1.50E-07 | CCF 3/4 RADIATION MONITORS IC-D SPURIOUS ACTUATION |
| H23-EMS-CF-ALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-EMS-FC-DIV1 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 1 FAILS TO FUNCTION |

| Table 4.2-7 | | |
|------------------------------------|-------------|--|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| H23-EMS-FC-DIV2 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 2 FAILS TO FUNCTION |
| H23-EMS-FC-DIV3 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 3 FAILS TO FUNCTION |
| H23-EMS-FC-DIV4 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 4 FAILS TO FUNCTION |
| H23-RMU-CF-ALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-FC-0002A | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002A FAILS TO FUNCTION |
| H23-RMU-FC-0002B | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002B FAILS TO FUNCTION |
| H23-RMU-FC-0002C | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002C FAILS TO FUNCTION |
| H23-RMU-FC-0002D | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002D FAILS TO FUNCTION |
| H23-RMU-FC-F72HA1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-F72HA2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-F72HADIV1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV1 FAILS TO FUNCTION |
| H23-RMU-FC-F72HADIV2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 2 FAILS TO FUNCTION |
| H23-RMU-FC-F72HADIV3 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 3 FAILS TO FUNCTION |
| H23-RMU-FC-F72HADIV4 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 4 FAILS TO FUNCTION |
| H23-RMU-FC-F72HBDIV1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 1 FAILURE |
| H23-RMU-FC-F72HBDIV2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 2 FAILURE |
| H23-RMU-FC-F72HBDIV3 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 3 FAILURE |
| H23-RMU-FC-F72HBDIV4 | 3.00E-04 | REMOTE MULTIPLEXING UNIT DIV 4 FAILURE |
| H23-RMU-FC-F72HD1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-F72HD2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| XXX-POL-RP-IC/PCC | 3.00E-07 | LOSS OF IC/PCC POOL WATER DUE TO BREAK DURING ACCIDENT |
| XXX-XHE-FO-ICPCCS | 1.61E-03 | OPERATOR FAILS TO RECOGNIZE NEED OF MAKE UP TO IC/PCCS POOLS |

Table 4.2-8
List of System Top Events

| Top Event | Description |
|------------------|---|
| GC32TOP2 | 2 OUT OF 4 IC's FAIL TO RETURN WATER TO RPV IN TRANSIENTS |

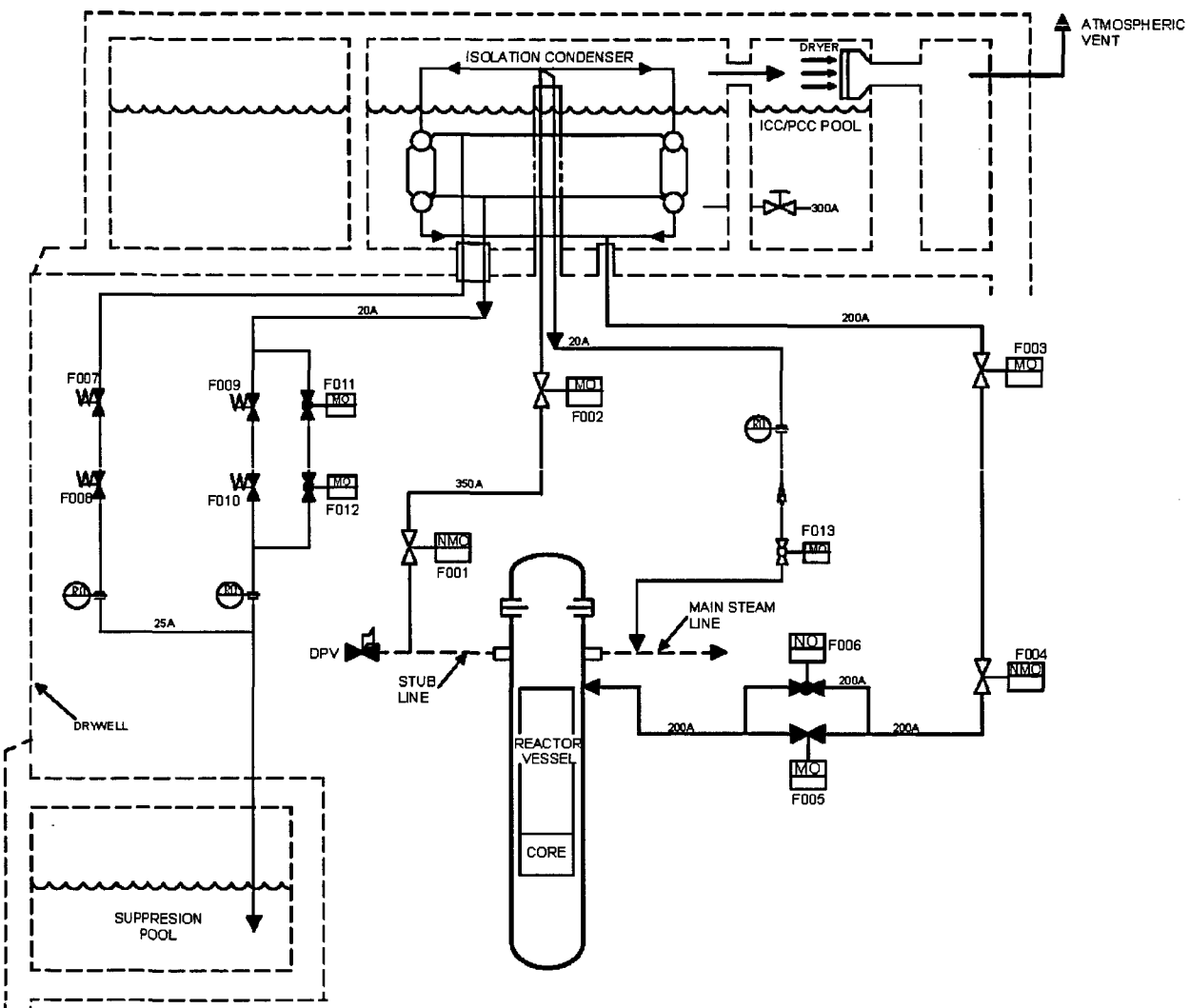


Figure 4.2-1. Simplified Process Flow Diagram for Isolation Condenser System

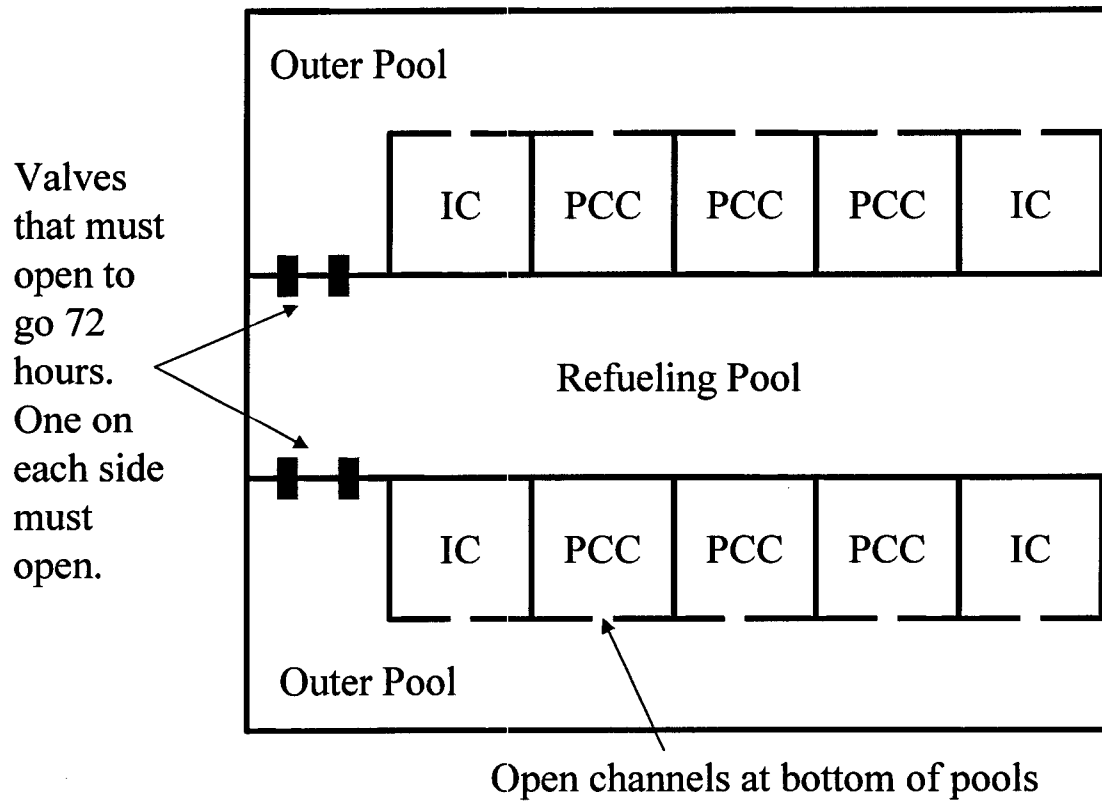


Figure 4.2-2. Schematic of the IC/PCC Pools and Interconnections

4.3 CONTROL ROD DRIVE SYSTEM - (C12)

4.3.1 Purpose

The purpose of this section is focused on the CRDS function to provide high-pressure water injection to the Reactor Pressure Vessel (RPV) in the event of loss of feedwater or other events resulting in low RPV water level. This injection function is initiated automatically by a RPV water level 2 signal.

4.3.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) Failure of either the A or B flow control loops resulting in a full-open condition of valves F020A or B, results in a trip of both CRD pumps.
- (2) Strainers D001A and B do not have sufficient flow capacity for CRDS in the water injection mode, and the opening of bypass valves F014A or B is required. Opening either F014A or B valves is sufficient to supply the total flow required by pumps C001A and B.
- (3) CRD trains are rotated on a monthly basis.
- (4) Human actuation time values are assumed consistent with ESBWR calculations.

4.3.3 System Description

The CRD System is composed of three major elements: (1) the electro-hydraulic fine motion control rod drive (FMCRD) mechanisms, (2) the hydraulic control units (HCU), and (3) the control rod drive hydraulic system (CRDHS). Figure 4.3-1 provides a general outline of the CRD System.

The CRDHS portion of CRDS supplies clean, demineralized water that is regulated and distributed to provide charging of the HCU scram accumulators and purge water flow to the FMCRDs during normal operation. The CRDHS is also the source of pressurized water for purging the Reactor Water Cleanup / Shutdown Cooling (RWCU/SDC) System pumps and Nuclear Boiler System (NBS) reactor water level reference leg instrument lines. Additionally, the CRDHS provides high pressure makeup water to the reactor during events in which the feedwater system is unable to maintain normal reactor water level. This makeup water is supplied to the reactor via a bypass line off the CRD pump discharge header that connects to the feedwater inlet piping via the RWCU/SDC return piping.

In normal plant operation, the CRDS pumps supply high-pressure water to charge the HCU accumulators, to the Fine Motion Control Rod Drive (FMCRD), to RWCU/SDC pumps, and to the reactor water level reference leg instrument lines. Upon receipt of a reactor water level 2 signal, the mode of operation changes to the RPV injection mode.

4.3.3.1 Hardware Configuration

A simplified diagram of the CRDS water injection system is shown on Figure 4.3-1. The condensate storage tank supplies the suction header of the CRDS water injection system. During normal plant operation, the CRDS water injection mode is in standby, and the CRDS is operating

in the purging HCU-charging mode using water filtered through one of the pump-suction filters, D001A and B. Upon receipt of a reactor water level 2 signal, valves F014A and B, in the redundant suction CRDS header, open to provide condensate water directly to the suction of pumps C001A and B. Under normal operating conditions, condensate water is supplied to the pump suctions through a pipe that is connected downstream of the condensate polisher. In an alternative mode, CRDS can draw condensate water directly from the condensate storage tank through a normally open manual valve (F01T) and a check valve (CV01T). The CRD system is arranged so that water is supplied from the source of higher pressure, without the need for any active change in valve positions. During losses of offsite power (when condensate pumps are unavailable), the water source is provided from the CST.

High-pressure water is discharged from each of the two pumps into a discharge header. The discharge header supplies water to the HCU accumulators, the FMCRDs, and the Reactor Water Cleanup System/Shutdown Cooling (RWCU/SDC) pumps for purge water. Other pipe runs, from each pump, supply water to the RPV through the normally closed valves, F020A and B, and to the RWCU/Shutdown Cooling (SDC) discharge piping and valves. A test line is supplied to provide means of testing the components of this run during plant operation.

In the water injection mode, the CRDS pump discharge is controlled by each of two flow control valves (F020A and B). The flow control valves are controlled by flow control loops made up of flow sensors N019A and B that provide the control input signals to the valves.

The two CRDS pumps are motor-driven, multi-stage, high-pressure, centrifugal pumps. The pump motors are driven by 6.9 kV AC power and the Reactor Component Cooling System (RCCS) water cools the pumps. All motor-operated system valves are powered by 480 V AC power.

4.3.3.2 System Operation

During normal plant operation, the CRDS is aligned with one of the two pump continuously operating to pressurize the system with water from the condensate and feedwater system and/or condensate storage tank (CST), and the associated control valves F010A or B, regulating the purge water flow to FMCRDs. During this condition, the other CRD pump is shutdown and in standby and the associated control valve is closed. Upon receipt of a reactor water level 2 signal, the CRDS shifts to the water injection mode of operation. The standby pump starts, the purge water header valve F012 and the charging water header valve F030 closes, and valves F014 A and B and F020A and B open to provide suction from condensate and a discharge path to the RPV.

The systems available for each of the required functions (identified in Section 3.2) and their success criteria are summarized in that Table 3.2-1 and discussed in detail in Section 3 of this ESBWR PRA report.

An additional requirement in the analysis is that no room cooling for the CRDS compartment is required for the first 24 hours of the accident.

4.3.3.3 System Location

All CRDS equipment, with the exception of piping from the condensate system and to the RWCU/SDC system, is located in the CRDS hydraulic compartment adjacent to the HCU's in the reactor building.

4.3.4 Automatic and Manual Control

Automatic Actuation. The CRDS is automatically aligned and initiated for water injection upon receipt of a reactor water level 2 signal. The following actions occur:

- (1) Suction valves F014 A and B open,
- (2) Injection-control valves F020A and B open,
- (3) The standby pump starts, and
- (4) The charging water header isolation valve F030 and purge water header isolation valve F012 close.

The water level 2 initiating signal is supplied by the CRDS logic. Four level transmitters on four different divisions generate the water level 2 signal. The logic for the signal transmission is two-out-of-four.

A high reactor water level 8 signal closes the flow control valves F020A and B to stop the flow to the reactor in order to prevent flooding of the main steam lines.

For accident sequences with loss of offsite power:

- The running CRD pump stops
- Circuit breakers must reclose to allow restart of the pumps
- All needed power is supplied by the on-site diesel generators

Initiation of the CRDS automatically generates the automatic opening and closing of valves F023 and F024, respectively.

Manual Actuation. As a backup to automatic initiation, the operator has the capability to manually initiate the water injection function in the event of failure of automatic initiation. For transients and small Loss-of-Coolant Accident (LOCA) events, the operator would have at least 20 minutes available for initiation of CRDS water injection to avoid automatic depressurization at L-1. Following depressurization at L-1, there would be an additional 45 minutes available before the Peak Cladding Temperature (PCT) limit would be reached. For medium and large LOCA events, the operator would have at least 45 minutes available to initiate CRDS water injection.

Instrumentation and Alarms. The instrumentation and alarm signals provided in the main control room for the CRDS water injection function are listed in Table 4.3-1.

4.3.5 System Interfaces

Support Systems. The water injection function of the CRDS interfaces with several support systems, including AC power and the Reactor Component Cooling Water System (RCCW). The CRD pump motors are driven by 6.9 kV AC power, and the system valves are operated by 480 V

AC power. The CRD pump room coolers are cooled by RCCW water. These specific dependency relationships are shown in Table 4.3-2.

Shared Components. The water injection function of the CRDS uses the same pumps (C001A and B) and water source (condensate) as the purging HCU-charging function. The automatic initiation signal uses the same reactor water level 2 signal and some of the same instrumentation and logic components as are used by other systems.

The CRDS injects high-pressure water into the RWCUS/SDCS discharge line, and shares with it and feedwater line B several check valves and isolation valves.

Spurious signal from GDCS pump stop logic trips CRDS pumps is modeled in the fault, this interface is described as follows. When in the high pressure makeup mode of operation, the CRD pumps are automatically tripped to terminate CRD System flow on receipt of low water level signals from two out of three GDCS pools. This logic is interlocked with a sealed-in low reactor water level 1 signal so that the pump trip can occur only during an accident condition, and not during normal operation.

4.3.6 System Testing

All standby system components for the CRDS water injection function are tested quarterly, including functional testing of the flow control loops. A test line to the CST from downstream of the two flow-control valves (F020A and B), together with an isolation valve on the vessel injection line provides a means of testing each of the flow control functions without injecting water into the RPV. A modulating valve in the test line allows variation of the line resistance to simulate different reactor pressures so that the flow control function can be tested over its full range of operation. If an initiation signal is received while the system is in test, the system automatically reverts to the water injection mode.

4.3.7 System Maintenance

All components of the CRDS water injection function can be maintained while the plant is in normal operation, subject to technical specification requirements and limitations. All normally open manual isolation valves are subject to being left in the closed position following maintenance. The system test isolation valve is not considered subject to misalignment following test since it is opened on receipt of an actuation signal.

4.3.8 Common Cause Failures

The following common cause component failures within the CRDS were considered in the present analysis:

Common cause failures within the CRD are summarized in Table 4.3-5 including the CCF of logic.

4.3.9 Fault Tree Analysis

Fault trees have been developed for the CRDS water injection functions for transient, LOCA, and loss of offsite-power initiating events using a success criterion of both CRDS pumps operating. Fault trees have also been constructed for a single pump (one-out-of-two) success,

with and without offsite power available. The success criteria is discussed in Section 3 of this PRA and summary table is provided in Table 3.3-1.

4.3.9.1 Top Event Definitions

Two top events are presented for the CRDS fault tree analysis:

- (1) GC12TOP1 - System Fails to Provide Inventory Makeup (failure of both CRD supplies) (TRAN with leakages).
- (2) GC12TOP2 - System Fails to Provide Inventory Makeup (failure of one CRD supplies) (LOCA & TRAN).

4.3.9.2 Fault Tree Description

The fault tree for GC12TOP1 is presented in Appendix B.4.3 and represents the probability that both loops of the CRD water injection function are unavailable if called upon in the event of a transient event with leakage. This function is used to support successful IC operation.

The fault tree for GC12TOP2 is presented in Appendix B.4.3 and represents the probability that either loop of the CRD water injection function is unavailable if called on in the event of LOCA or TRANSIENT requiring both CRD pumps to operate.

4.3.9.3 Human Interactions

Two types of human errors have been considered in developing the fault trees:

- Human errors in leaving normally open manual valves in closed position following maintenance.
- Human error in recovery following failure to receive an automatic initiation signal on a water level 2 condition. For this situation, the operator should manually initiate the CRDS water injection function.

These human errors are identified in Table 4.3-6.

4.3.10 Results of Fault Tree Analysis

The fault tree results provide quantitative values of system unavailability and of the importance of specific components to that total.

The definition of each basic event is reported in Table A.3-7.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This qualification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.3.11 References

4.3-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.3-1
Control Room Instrumentation and Alarms

| Controls |
|--|
| |
| Displays |
| (1) CRD pump operating status (run/stop) |
| (2) CRD pump auxiliary lube oil pump status (run/stop) |
| (3) Scram valve position |
| (4) Flow control valve position |
| (5) Pump suction pressure |
| (6) Pump discharge pressure |
| (7) System flow rate |
| (8) Charging water header pressure |
| (9) Reactor/purge water header differential pressure |
| Alarms |
| (1) Low pump lube oil pressure |
| (2) High pump lube oil temperature |
| (3) Low pump suction pressure |
| (4) High pump suction filter differential pressure |
| (5) High drive water filter differential pressure |
| (6) High scram air header pressure |
| (7) Low scram air header pressure |
| (8) HCU trouble alarm (low HCU accumulator pressure or high accumulator water level) |
| (9) Low charging header pressure |
| (10) FMCRD control rod separation |

Table 4.3-2
System Dependencies Matrix

| | | Support System | | | | RCCWS |
|-----------|----------|----------------|-------|---------------|-------|-------|
| | | Power Supply | | | | |
| | | R12 480 V AC | | R11 6.9 KV AC | | |
| | | DIV A | DIV B | DIV A | DIV A | |
| Component | Type | DIV A | DIV B | DIV A | DIV A | |
| F014A | MO | X | | | | |
| F014B | MO | | X | | | |
| F020A | MO | X | | | | |
| F020B | MO | | X | | | |
| F023A | MO | X | | | | |
| F023B | MO | | X | | | |
| F024A | MO | X | | | | |
| F024B | MO | | X | | | |
| C001A | Pump | | | X | | |
| C001B | Pump | | | | X | |
| C001A | Oil Pump | | | X | | X |
| C001B | Oil Pump | | | | X | X |

Table 4.3-3
Component Test

All components are tested quarterly.

Table 4.3-4
Component Maintenance

All components of the CRDS water injection function can be maintained while the plant is in normal operation, subject to technical specification requirements and limitations.

Table 4.3-5
System Common Cause Failures

| Basic Event | Prob | Description |
|-----------------------|-------------|--|
| B21-LT_-CF-DPSWR | 1.20E-07 | CCF OF DPS WR LEVEL TRANSMITTERS |
| B21-LT_-CF-L8NO | 1.20E-07 | CCF LEVEL TRANSMITTERS L8 (FAILURE TO RESPOND TO CHANGE) |
| B21-LT_-CF-L9SPU1 | 1.20E-07 | SPURIOUS L9 SIGNAL CCF |
| C12-MCB-CF-CLOSE | 9.29E-05 | CIRCUIT BREAKER CCF TO CLOSE |
| C12-MOV-CF-OPEN | 1.78E-04 | CCF MOV TO OPEN |
| C12-MPC-CR-C001AB | 3.57E-06 | CCF OF CRD PUMPS TO RUN |
| C12-MPC-CS-C001A/B | 2.05E-04 | CCF PUMPS TO START |
| C12-PT_-CF-N001A/BLOW | 3.00E-08 | CCF PRESSURE TRANSMITTERS IN SUCTION LINES FAIL LOW |
| C74-DTM-CF-DIDALL | 5.50E-05 | COMMON CAUSE FAILURE 3/4 DTM DID LOGIC |
| E50-LT_-CF-N005ABCLOW | 2.40E-07 | CCF 2/3 LEVEL TRANSMITTERS E50-N005A/B/C LOW |
| H23-EMS-CF-DIDALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |

Table 4.3-6
System Human Errors

| Basic Event | Prob | Description |
|--------------------|-------------|----------------------------|
| C12-XHE-FO-LEVEL2 | 3.22E-02 | MANUAL ACTUATION FAILURE |
| C12-XHE-MH-F003B | 1.20E-02 | MISPOSITION OF VALVE F003B |
| C12-XHE-MH-F013A | 4.80E-02 | MISPOSITION OF VALVE F013A |
| C12-XHE-MH-F013B | 4.80E-02 | MISPOSITION OF VALVE F013B |
| C12-XHE-MH-F015A | 4.80E-02 | MISPOSITION OF VALVE F015A |
| C12-XHE-MH-F015B | 4.80E-02 | MISPOSITION OF VALVE F015B |
| C12-XHE-MH-F018A | 1.20E-02 | MISPOSITION OF VALVE F018A |
| C12-XHE-MH-F018B | 1.20E-02 | MISPOSITION OF VALVE F018B |
| C12-XHE-MH-F021A | 1.20E-02 | MISPOSITION OF VALVE F021A |
| C12-XHE-MH-F021B | 1.20E-02 | MISPOSITION OF VALVE F021B |
| P30-XHE-MH-F015 | 4.80E-02 | MISPOSITION OF VALVE F01T |

Table 4.3-7
System Basic Events

| Basic Event | Prob | Description |
|-----------------------|----------|--|
| B21-LT_-CF-DPSWR | 1.20E-07 | CCF OF DPS WR LEVEL TRANSMITTERS |
| B21-LT_-CF-L8NO | 1.20E-07 | CCF LEVEL TRANSMITTERS L8 (FAILURE TO RESPOND TO CHANGE) |
| B21-LT_-CF-L9SPU1 | 1.20E-07 | SPURIOUS L9 SIGNAL CCF |
| B21-UV_-CC-F102A | 1.60E-03 | CHECK VALVE F102A IN FEEDWATER LINE A FAILS TO OPEN |
| B21-UV_-CC-F103A | 1.60E-03 | CHECK VALVE F103A IN FEEDWATER LINE A FAILS TO OPEN |
| C12-MCB-CF-CLOSE | 9.29E-05 | CIRCUIT BREAKER CCF TO CLOSE |
| C12-MCB-CO-C001A | 1.20E-05 | CIRCUIT BREAKER FOR C001A & OIL PUMP A OPENS SPURIOUSLY |
| C12-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER FOR C001A & OIL PUMP A FAILS TO CLOSE |
| C12-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER FOR C001B & AUX OIL PMP B FAILS TO CLOSE |
| C12-MOV-CC-F014A | 4.00E-03 | MOTOR OPER. VALVE F014A FAILS TO OPEN |
| C12-MOV-CC-F014B | 4.00E-03 | MOTOR OPER. VALVE F014B FAILS TO OPEN |
| C12-MOV-CC-F020A | 4.00E-03 | MOTOR OPER. VALVE F020A FAILS TO OPEN |
| C12-MOV-CC-F020B | 4.00E-03 | MOTOR OPER. VALVE F020B FAILS TO OPEN |
| C12-MOV-CF-OPEN | 1.78E-04 | CCF MOV TO OPEN |
| C12-MOV-FC-F020A | 3.13E-03 | FLOW CONTROL A FAILS WIDE OPEN |
| C12-MOV-FC-F020B | 3.13E-03 | FLOW CONTROL B FAILS WIDE OPEN |
| C12-MPC-CR-C001AB | 3.57E-06 | CCF OF CRD PUMPS TO RUN |
| C12-MPC-CS-C001A/B | 2.05E-04 | CCF PUMPS TO START |
| C12-MPC-FR-C001A | 5.76E-05 | MOTOR-DRIVEN PUMP C001A FAILS TO RUN, GIVEN START |
| C12-MPC-FR-C001B | 5.76E-05 | MOTOR-DRIVEN PUMP C001B FAILS TO RUN, GIVEN START |
| C12-MPC-FS-C001A | 2.40E-03 | MOTOR-DRIVEN PUMP C001A FAILS TO START |
| C12-MPC-FS-C001B | 2.40E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO START |
| C12-MP_-CS-C001A/BOIL | 2.05E-04 | CCF AUX. OIL PUMPS TO START |
| C12-MP_-FS-C001AOIL | 2.40E-03 | MOTOR-DRIVEN AUX. OIL PUMP FOR C001A FAILS TO RESTART |
| C12-MP_-FS-C001BOIL | 2.40E-03 | MOTOR-DRIVEN AUX. OIL PUMP FOR C001B FAILS TO START |
| C12-OR_-PG-D007A | 6.48E-04 | ORIFICE D007A FAILS TO REMAIN OPEN (PLUG) |
| C12-OR_-PG-D007B | 6.48E-04 | ORIFICE D007B FAILS TO REMAIN OPEN (PLUG) |
| C12-PT_-CF-N001A/BLOW | 3.00E-08 | CCF PRESSURE TRANSMITTERS IN SUCTION LINES FAIL |

Table 4.3-7
System Basic Events

| Basic Event | Prob | Description |
|-----------------------|----------|---|
| | | LOW |
| C12-PT_-LO-N004A | 7.92E-06 | PRESSURE TRANSMITTER IN SUCTION LINE TRAIN A FAILS LOW |
| C12-PT_-LO-N004B | 7.92E-06 | PRESSURE TRANSMITTER IN SUCTION LINE TRAIN B FAILS LOW |
| C12-SYS-TM-TRAINB | 3.00E-03 | TRAIN B IN MAINTENANCE |
| C12-UV_-CC-F022 | 1.60E-03 | CHECK VALVE F022 FAILS TO OPEN |
| C12-XHE-FO-LEVEL2 | 3.22E-02 | MANUAL ACTUATION FAILURE |
| C12-XHE-MH-F003B | 1.20E-02 | MISPOSITION OF VALVE F003B |
| C12-XHE-MH-F013A | 4.80E-02 | MISPOSITION OF VALVE F013A |
| C12-XHE-MH-F013B | 4.80E-02 | MISPOSITION OF VALVE F013B |
| C12-XHE-MH-F015A | 4.80E-02 | MISPOSITION OF VALVE F015A |
| C12-XHE-MH-F015B | 4.80E-02 | MISPOSITION OF VALVE F015B |
| C12-XHE-MH-F018A | 1.20E-02 | MISPOSITION OF VALVE F018A |
| C12-XHE-MH-F018B | 1.20E-02 | MISPOSITION OF VALVE F018B |
| C12-XHE-MH-F021A | 1.20E-02 | MISPOSITION OF VALVE F021A |
| C12-XHE-MH-F021B | 1.20E-02 | MISPOSITION OF VALVE F021B |
| C74-DTM-CF-DIDALL | 5.50E-05 | COMMON CAUSE FAILURE 3/4 DTM DID LOGIC |
| E50-LT_-CF-N005ABCLOW | 2.40E-07 | CCF 2/3 LEVEL TRANSMITTERS E50-N005A/B/C LOW |
| E50-LT_-LO-N005A | 1.22E-05 | LEVEL TRANSMITTER E50-N005 DRIFTS LOW |
| E50-LT_-LO-N005B | 1.22E-05 | LEVEL TRANSMITTER E50-N005B DRIFTS LOW |
| E50-LT_-LO-N005C | 1.22E-05 | LEVEL TRANSMITTER E50-N005C DRIFTS LOW |
| H23-EMS-CF-DIDALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| H23-RMU-FC-C12001 | 3.00E-04 | RMU 001 CRD SYSTEM C.R.MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-C12002 | 3.00E-04 | RMU 002 CRD SYSTEM C.R.MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-C12003 | 3.00E-04 | RMU 003 CRD SYSTEM C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| P30-TNK-RP-A001 | 2.40E-06 | CONDENSATE STORAGE TANK LEAKS CATASTROPHICALLY |
| P30-UV_-CC-CV01T | 2.00E-04 | CHECK VALVE CV01T FAILS TO OPEN |
| P30-XHE-MH-F015 | 4.80E-02 | MISPOSITION OF VALVE F01T |

Table 4.3-8
System Top Events

| Top Event | Description |
|------------------|------------------------------------|
| GC12TOP1 | LOSS OF FLOW FROM BOTH CRD PUMPS |
| GC12TOP2 | LOSS OF FLOW FROM EITHER CRD PUMPS |

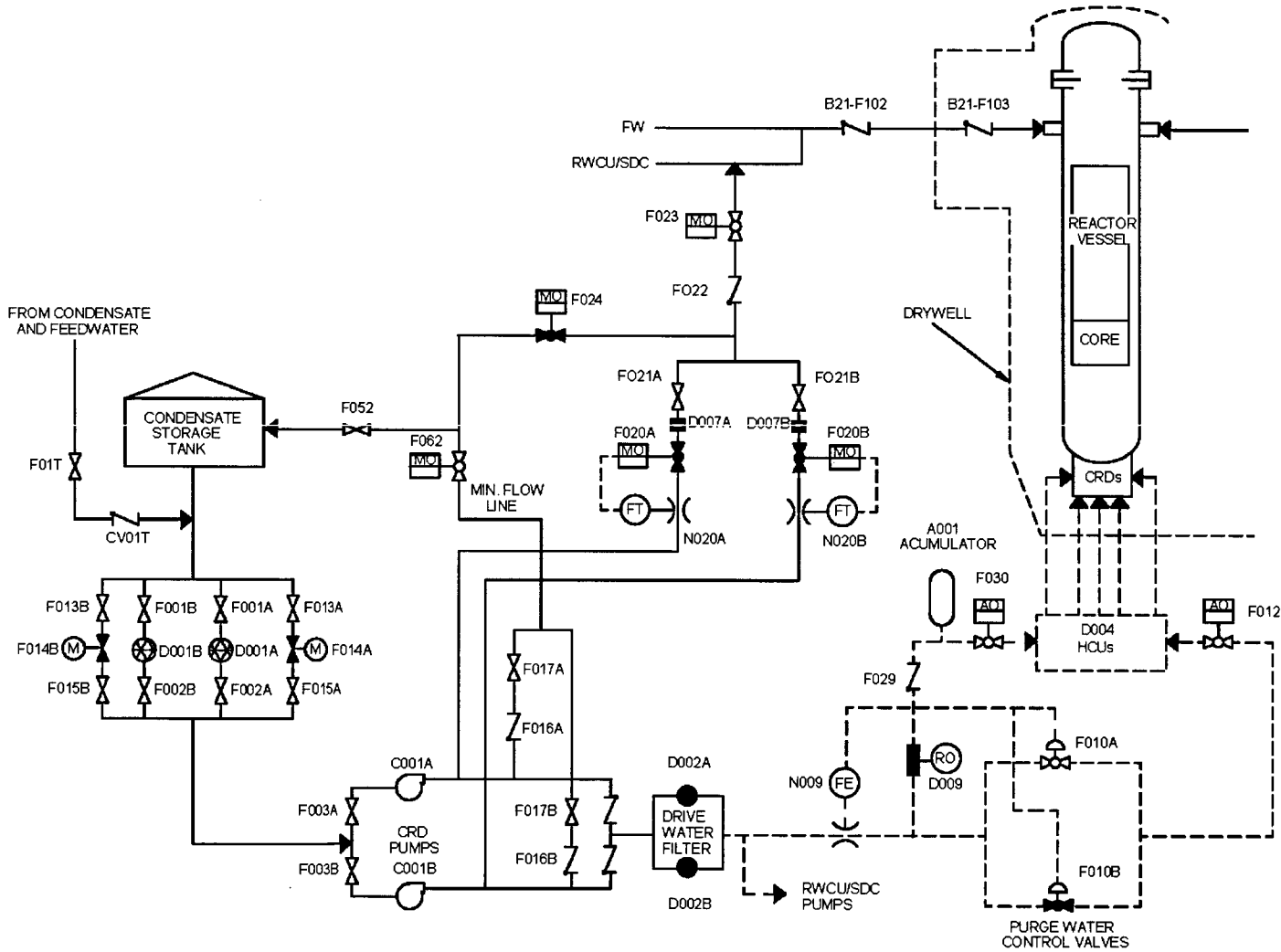


Figure 4.3-1. Simplified Process Flow Diagram for Control Rod Drive System

4.4 STANDBY LIQUID CONTROL SYSTEM - (C41)

4.4.1 Purpose

The purpose of the SLCS is to provide a backup function to the Control Rod Drives for reactivity control to assure automatic reactor shutdown from full power to subcritical given an anticipated transient without scram (ATWS).

Reactor shutdown is achieved by injection of a solution of neutron absorber sodium pentaborate inside the reactor pressure vessel (RPV), where it rapidly becomes mixed with the primary reactor coolant. No control rod movement is required. The SLCS is initiated automatically by either high reactor pressure or low reactor water level (Level 2) signals and the average power range monitor (APRM) not downscale. The operator from the control room may also initiate it manually.

4.4.2 Design Assumptions

The following design assumptions (DA) have been used for the model:

- (1) Neither heat tracing in the discharge lines nor ventilation dependencies are necessary.
- (2) The success criterion applied to the SLCS is that both loops are required to fulfill the safety function related to control of reactivity to prevent core damage. While the boron from a single tank is sufficient to bring the plant to hot shutdown, there are concerns about sufficient mixing in the core. Localized core damage is assumed to occur if only a single tank injects into the RPV. The localized damage releases fission products into the RPV, but does not challenge the overall coolability of the core.
- (3) No maintenance is expected to occur during power operation that makes either train of the SLCS unavailable.
- (4) Initially, each loop of the SLCS must be on Standby mode, with the explosive injection valves closed.
- (5) For purposes of the system boundary with respect to the model, the system includes:
 - Instrumentation and control system, except for the transmission of signals (acquisition, logic and actuation) of the squib valves
 - Sodium Pentaborate Solution (SPBS) preparation and makeup system
 - High-pressure nitrogen makeup system
 - Accumulator tank vent line valves
 - SPBS injection system, consisting of accumulator and valves corresponding to the injection lines

4.4.3 System Description

The SLCS consists of two loops, each one consisting of a high-pressure accumulator tank containing an enriched sodium pentaborate solution; a main discharge line containing a pair of parallel squib-actuated injection valves; an in-RPV supply header that connects to several

sparger tubes, each located inside the core bypass region; a system connections to a high-pressure, nitrogen gas supply, and associated valves, piping, local instrumentation, and control.

4.4.3.1 Hardware Configuration

A simplified diagram of the SLCS is shown on Figure 4.4-1. The sodium pentaborate solution is stored in the accumulator tanks, A001A and A001B. The discharge is through an 80 mm (3 inch nominal) line containing a locked-open isolation valve, F001A and B. Downstream of the isolation valve, a normally open, motor-operated valve (F002A and B) closes on a low-level signal from the tank to prevent nitrogen injection into the RPV. Downstream of F002A and B, the injection line divides into two parallel lines, each of which contains a normally closed explosive (squib) valve (F003A and C for loop A and F003B and D for loop B). These valves receive an open signal given the ATWS logic is satisfied. The discharge of the squib valves is routed to the RPV through two check isolation valves (F004A and B and F005A and B) and a normally open manual valve (F006A and B) inside the drywell. The position of this valve is indicated in the control room.

Pressure in the accumulator tank is maintained between 15.2 and 15.5 MPa (2200 and 2250 psi) from high-pressure cryogenic nitrogen gasification supply system. Pressure makeup is supplied through a motor-operated globe valve (F029A and B), and check valve (F028A and B). Two normally closed valves (a motor-operated valve, F014, and a solenoid valve, F013) control nitrogen makeup based on accumulator tank pressure. These charging valves (F029A and B) are operated manually, but they automatically close if the system is initiated. Accumulator tank pressure is monitored in the control room and pressure makeup can be initiated manually.

A keylock switch is provided in the control room, which operates two series high pressure air operated diaphragm valves (F507A and B and F508A and B) to vent the accumulator tank. These depressurization valves are actuated manually and automatically to prevent nitrogen from getting into the RPV.

4.4.3.2 System Operation

During normal plant operation, the system is in standby with the sodium pentaborate solution in the accumulator tanks and the tanks pressure maintained automatically between 15.2 and 15.5 MPa-gauge and a level between 2.148 and 2.188 mm. All valves in the tank discharge line are open except the four squib valves and check valves.

In the event of an ATWS, the four squib valves are fired open given an initiation signal when ATWS logic is satisfied. The SLC system automatically initiated with one of the following conditions:

- a. High reactor vessel dome pressure and APRM not downscale for 3 minutes, or
- b. Low reactor vessel water level (Level 2) and APRM not downscale for 3 minutes, or
- c. APRM not downscale for 3 minutes following manual initiation of ARI/FMCRD run.

Note that one SLC tank is sufficient to prevent global core damage. TRACG based calculations have confirmed 1 SLC is adequate to get the reactor to hot shutdown. The squib valves can also be fired from the control room by depressing two switches simultaneously.

Following firing of the squib valves, the sodium pentaborate solution is injected into the RPV. When each tank reaches a set low value, local instrumentation closes the respective motor-operated valves F002A and B to prevent the injection of nitrogen into the RPV. The same signal that closes valve F002A and B also opens the diverse accumulator depressurization valves, F507A and B and F508A and B after a delay.

For successful operation of the system to prevent core damage, the injection of both tanks through at least one of the two ignition squib valves is necessary.

4.4.3.3 System Location

The SLCS equipment is located in a corner room outside the drywell at elevation 22,500 of the Reactor Building.

4.4.4 Automatic and Manual Control

Automatic Actuation. The SLCS is automatically initiated for sodium pentaborate injection upon receipt of an ATWS signal. The ATWS signal is supplied by the SLCS two-out-of-four logic. The ATWS signal is generated by either a reactor water level 2 or a reactor high pressure signal together with an "APRM not downscale" permissive plus a timer. Four instruments on four different divisions generate the initiating signals, and are hard-wired to dedicated analog trip modules (ATM). The outputs from the logic are hard-wired to the squib-valve firing circuits.

When the SPBS level in the accumulator tanks have reached low level, redundant accumulator level measurement instrumentation using 2-out-of-4 logic closes the shutoff valve F002A (F002B). To provide a diverse means of preventing nitrogen injection into the RPV, the nitrogen depressurization valves F507A and B, and F508A and B, are also actuated by the low SPBS level signal after a delay.

These actions prevent further discharge of effluent and thus prevent nitrogen injection into the RPV where it could adversely affect the performance of the isolation condenser system.

For accident sequences with loss of preferred power, all needed power is supplied by 250 V DC battery supply.

Manual Actuation. As a backup to automatic actuation, the operator has the capability to manually initiate the boron injection from the main control room by simultaneously depressing two switches. If automatic feedwater runback occurs successfully, the operator has a very long time available to insert boron. In the event of failure of feedwater runback, the operator's time window is much shorter (approximately 10 minutes) and therefore not credited.

Instrumentation and Alarms. The instrumentation and alarm signals provided in the main control room for the SLCS function are listed in Table 4.4-1.

4.4.5 System Interfaces

Support Systems. The only support systems required by the SLCS and associated instrumentation are the 250 V DC power supply and the nitrogen supply system. The system dependency on DC power is listed in Table 4.4-2. System operation is not affected by nitrogen supply unavailability as long as the accumulator remains within the required operational pressure band.

4.4.6 System Testing

The SLC system is designed with passive and redundant features such that minimum in-service testing is required at times other than during normally scheduled plant refueling outages. Assurance that SLC system design conditions are maintained during plant operation is accomplished by critical parameter alarms and periodic surveillances. A small, trickle current circuit is used at each squib charge, and automatic testing is done (typically every 30 minutes) to verify circuit continuity for receiving firing signals.

The SLCS is tested during plant refueling shutdowns. The system fault tree analysis is based on a test interval of two years for all SLCS equipment except valve F010, which are tested every 90 days. Generally, initiating instrumentation is also tested every 90 days. Valves not critical to system initiation including F002A and B are periodically tested (quarterly).

4.4.7 System Maintenance

No specific maintenance activities are planned or foreseen on the SLCS during normal plant operation. If necessary, all components of the SLCS can be maintained while the plant is in normal operation, subject to technical specification requirements and limitations. Mispositioning of valves F001A, F001B, F006A, and F006B is not considered an error because they are checked periodically. All normally-open manual isolation valves are subject to being left in the closed position following maintenance. The probability of this is included in the analysis.

4.4.8 Common-Cause Failures

The common-cause failures of concern in the SLCS equipment are common-cause failure of the redundant squib valves and transmitters to operate within each loop. This is included in the analysis. Common cause failures of VLU, DTM and RMU components are also considered in the model.

4.4.9 Fault Tree Analysis

The only fault tree developed for the SLCS is for a transient initiating event with failure to scram, namely ATWS. Instrumentation and logic for SLCS initiation are presented herein as separate fault trees, one tree for each squib valve initiation.

4.4.9.1 Top Event Definitions

One top event is developed for the SLCS fault tree analysis:

- GC41-0001-_1 – Long term boration fails during ATWS

4.4.9.2 Fault Tree Description

The fault tree for GC41-0001-_1 is presented in Appendix B.4.4, representing the probability that the SLCS fails to inject boron into the RPV following an ATWS. This tree includes the probability that both squib valves of each system loop fail to fire, either because of valve failure or failure to receive firing signals.

In the SLCS fault tree analysis, it is assumed that the accumulator tank pressure requires recharging at some time during the assumed two-year interval. It is further assumed that if no

corrective actions were taken, the SLCS would fail to inject sufficient boron in the event of an ATWS. This combination of assumptions may be conservative.

4.4.9.3 Human Interactions

Three types of human errors have been considered in developing the fault trees:

- Human errors in leaving normally-open manual valves in closed position following maintenance.
- Human error in failing to provide manual initiation of SLCS in the event of failure of automatic initiation.
- Human error in failing to provide manual initiation of pressure makeup to the accumulator, or to take other action, in the event of need for makeup and failure of automatic makeup.

These human errors are listed in Table 4.4-6.

4.4.10 Results of Fault Tree Analysis

The fault tree results provide quantitative values of system unavailability and of the contribution of specific components to that total.

The definition of each basic event is reported in Table 4.4-7.

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This qualification process enables the checking of the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow the identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.4.11 Model Differences for 72-Hour Operation

The model is not different for a 72-hour mission time because the time required to inject the sodium pentaborate solution is very short.

4.4.12 References

- 4.4-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.4-1
Control Room Instrumentation and Alarms

| Controls |
|---|
| Remote manual switches for shut-off valves F002A (F002B), F029A (F029B) and trace heating of the accumulator vent lines (MCR panels) |
| Two key-locked switches actuating squib valves F003A/C (F003B/D) (MCR panels) |
| Accumulator depressurization solenoid valves (MCR panels) |
| On/off remote manual switch for the nitrogen vaporizer B001, piston pump C001, and nitrogen gas exhaust solenoid valve F034 (Local Panels & Racks) |
| On/off remote manual switches for a mixer, piston pump C003, drum heating, mixing pump C002, and trace heating for accumulator vent lines (Local Panels & Racks) |
| Drum temperature control (Local Panels & Racks) |
| Displays |
| Accumulator level monitoring (MCR panels and Local Panels & Racks) |
| Accumulator pressure monitoring (MCR panels) |
| Nitrogen gas temperature monitoring (MCR panels and Local Panels & Racks) |
| Position lights indicating open/close for squib valves F003A/C (F003B/D), shut-off valves F002A (F002B) and F029A (F029B) and penetration check valves F004A (F004B) and F005A (F005B) (MCR panels) |
| On/off position lights for the depressurization valves vent line trace heating (MCR panels) |
| Pressure indication for auxiliary high pressure air system of depressurization valves (MCR panels) |
| The temperature of the SLC system rooms shall be monitored by the reactor building HVAC (MCR panels) |
| Nitrogen vaporizer inlet and outlet temperature indications (Local Panels & Racks) |
| Level indication of the nitrogen trap vessel A004 (Local Panels & Racks) |
| Pump C001 pressure monitoring (Local Panels & Racks) |
| Open/closed indication for the valve F007A (F007B) connecting the suction line of the mixing pump to the injection line (Local Panels & Racks) |
| On/off indicator for the nitrogen vaporizer B001, piston pump C001, and nitrogen gas exhaust solenoid valve F034 (Local Panels & Racks) |
| On/off indicators for a mixer, piston pump C003, drum heating, mixing pump C002, and trace heating for accumulator vent lines (Local Panels & Racks) |
| Drum temperature monitor (Local Panels & Racks) |
| Alarms |
| Accumulator level alarms (MCR panels and Local Panels & Racks) |
| Accumulator pressure alarms (MCR panels) |
| Nitrogen gas temperature alarms (MCR panels and Local Panels & Racks) |
| Pressure indication and alarms for auxiliary high pressure air system of depressurization valves (MCR panels) |
| Pump C001 cool discharge temperature and high temperature alarm (Local Panels & Racks) |
| Pump C001 high pressure alarm (Local Panels & Racks) |
| Drum temperature control alarms (Local Panels & Racks) |

Table 4.4-2
System Dependencies Matrix

| | | Support System | | | |
|-----------|------|----------------|-------|-------|------|
| | | Power Supply | | | |
| | | R16 250 V DC | | | |
| Component | Type | DIV 1 | DIV 2 | DIV 3 | DIV4 |
| F003A | SQV | X | | | X |
| F003B | SQV | X | | | X |
| F003C | SQV | | X | X | |
| F003D | SQV | | X | X | |

Table 4.4-3
Component Tests

| Component | Type | Expected Test Interval |
|------------------|-------------|-------------------------------|
| F003A/B/C/D | SQV | 30 minutes (1) |
| F002A/B | MOV | Quarterly |
| F004A/B | UV | During plant outages |
| F005A/B | UV | During plant outages |
| F006A/B | BV | Quarterly |
| F013 | SOV | Quarterly |
| F014 | MOV | Quarterly |
| F029A/B | MOV | Quarterly |
| F028A/B | UV | Quarterly |
| F507A/B | ACV | Quarterly |
| F508A/B | ACV | Quarterly |
| F001A/B | BV | During plant outages |

(1): A small, trickle current circuit shall be used to verify circuit continuity for receiving firing signals

Table 4.4-4
Component Maintenance

The SLC system and components shall be designed such that they can be maintained with relative ease and minimum maintenance time.

Table 4.4-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|--------------------------|-------------|---|
| B21-LT_-CF-ATWS | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-PT_-CF-ATWS | 3.00E-08 | CCF 3 OUT OF 4 RPV PRESSURE TRANSMITTERS |
| C41-LT_-CF- N001TALOW | 2.40E-07 | CCF 2 OUT OF 4 OF LEVEL TRANSMITTERS LOW |
| C41-LT_-CF- N001TBLOW | 2.40E-07 | CCF 2 OUT OF 4 LEVEL TRANSMITTERS LOW |
| C41-SQV-CF-F003AC | 1.50E-04 | CCF TO OPERATE OF SQUIB VALVES ON SLCS-A |
| C41-SQV-CF-F003BD | 1.50E-04 | CCF TO OPERATE OF SQUIB VALVES ON SLCS-B |
| C51-ACT-CF-SRNM | 2.98E-04 | CCF OF SRNM CORE FLUX CHANNELS |
| C51-VLU-CF-SRNM | 7.48E-06 | CCF 3/4 OF SRNM 2/4 VLU |
| C74-DTM-CF- ATWSALL | 1.30E-05 | CCF 3 OUT OF 4 DIGITAL TRIP MODULES |
| C74-VLU-CF-ATWS | 3.12E-06 | CCF 3/4 VOTING LOGIC UNIT ATWS DIVISIONS |
| C74-VLU-CF-SLCS | 7.48E-06 | CCF 2/2 VOTING CARD SLCS LOGIC |
| H23-EMS-CF- ATWSALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-RMU-CF-ALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-CF- ATWSALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |

Table 4.4-6
List of System Human Errors

| Basic Event | Prob | Description |
|----------------------|-------------|--|
| C41-XHE-FO-INI | 1.77E-02 | MANUAL ACTUATION FAILURE |
| C41-XHE-FO-INISLCS | 1.77E-01 | OPERATOR FAILS TO MANUALLY INITIATE SLCS (SHORT TIME) |
| C41-XHE-FO-OPENF002A | 2.69E-01 | OPERATOR FAILS TO OPEN VALVE F002A (AFTER INADV.CLOS.) |
| C41-XHE-FO-OPENF002B | 2.69E-01 | OPERATOR FAILS TO OPEN VALVE F002B (AFTER INADV.CLOS.) |
| C41-XHE-MH-F002A | 4.03E-03 | MISPOSITION OF VALVE F002A |
| C41-XHE-MH-F002B | 4.03E-03 | MISPOSITION OF VALVE F002B |

Table 4.4-7
List of System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|---|
| B21-LT_-CF-ATWS | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-LT_-NO-ATWS1A | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1A (LEVEL 1&2) FAILS |
| B21-LT_-NO-ATWS1B | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1B (LEVEL 1&2) FAILS |
| B21-LT_-NO-ATWS1C | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1C (LEVEL 1&2) FAILS |
| B21-LT_-NO-ATWS1D | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1D (LEVEL 1&2) FAILS |
| B21-OR_-PG-01A | 1.44E-05 | ORIFICE INSTR. LINE 1A FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01B | 1.44E-05 | ORIFICE INSTR. LINE 1B FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01C | 1.44E-05 | ORIFICE INSTR. LINE 1C FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01D | 1.44E-05 | ORIFICE INSTR. LINE 1D FAILS TO REMAIN OPEN (PLUG) |
| B21-PT_-CF-ATWS | 3.00E-08 | CCF 3 OUT OF 4 RPV PRESSURE TRANSMITTERS |
| B21-PT_-NO-ATWS2A | 1.15E-05 | PRESSURE TRANSMITTER 0002A FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B21-PT_-NO-ATWS2B | 1.15E-05 | PRESSURE TRANSMITTER 0002B FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B21-PT_-NO-ATWS2C | 1.15E-05 | PRESSURE TRANSMITTER 0002C FAILS TO RESPOND TO CHANGE IN PRESSURE |
| B21-PT_-NO-ATWS2D | 1.15E-05 | PRESSURE TRANSMITTER 0002D FAILS TO RESPOND TO CHANGE IN PRESSURE |
| C41-LT_-CF-N001TALOW | 2.40E-07 | CCF 2 OUT OF 3 OF LEVEL TRANSMITTERS LOW |
| C41-LT_-CF-N001TBLOW | 2.40E-07 | CCF 2 OUT OF 3 LEVEL TRANSMITTERS LOW |
| C41-LT_-LO-N001AA | 1.22E-05 | LEVEL TRANSMITTER A TANK A FAILS LOW |
| C41-LT_-LO-N001AB | 1.22E-05 | LEVEL TRANSMITTER A TANK B FAILS LOW |
| C41-LT_-LO-N001BA | 1.22E-05 | LEVEL TRANSMITTER B TANK A FAILS LOW |
| C41-LT_-LO-N001BB | 1.22E-05 | LEVEL TRANSMITTER B TANK B FAILS LOW |
| C41-LT_-LO-N001CA | 1.22E-05 | LEVEL TRANSMITTER C TANK A FAILS LOW |
| C41-LT_-LO-N001CB | 1.22E-05 | LEVEL TRANSMITTER C TANK B FAILS LOW |
| C41-MOV-OC-F002A | 3.36E-06 | MOTOR OPER. VALVE F002A FAILS TO REMAIN OPEN |
| C41-MOV-OC-F002B | 3.36E-06 | MOTOR OPER. VALVE F002-B FAILS TO REMAIN OPEN |
| C41-SQV-CC-F003A | 3.00E-03 | EXPLOSIVE VALVE F003A FAILS TO OPERATE |
| C41-SQV-CC-F003B | 3.00E-03 | EXPLOSIVE VALVE F003B FAILS TO OPERATE |
| C41-SQV-CC-F003C | 3.00E-03 | EXPLOSIVE VALVE F003C FAILS TO OPERATE |
| C41-SQV-CC-F003D | 3.00E-03 | EXPLOSIVE VALVE F003D FAILS TO OPERATE |
| C41-SQV-CF-F003AC | 1.50E-04 | CCF TO OPERATE OF SQUIB VALVES ON SLCS-A |
| C41-SQV-CF-F003BD | 1.50E-04 | CCF TO OPERATE OF SQUIB VALVES ON SLCS-B |
| C41-SYS-FF-MAKEUP | 1.00E-01 | INVENTORY MAKE-UP BORATION FAILURE |
| C41-UV_-CC-F004A | 1.60E-03 | CHECK VALVE F004A FAILS TO OPEN |

Table 4.4-7
List of System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|--|
| C41-UV_-CC-F004B | 1.60E-03 | CHECK VALVE F004B FAILS TO OPEN |
| C41-UV_-CC-F005A | 1.60E-03 | CHECK VALVE F005A FAILS TO OPEN |
| C41-UV_-CC-F005B | 1.60E-03 | CHECK VALVE F005B FAILS TO OPEN |
| C41-XHE-FO-INI | 1.77E-02 | MANUAL ACTUATION FAILURE |
| C41-XHE-FO-INISLCS | 1.77E-01 | OPERATOR FAILS TO MANUALLY INITIATE SLCS (SHORT TIME) |
| C41-XHE-FO-OPENF002A | 2.69E-01 | OPERATOR FAILS TO OPEN VALVE F002A (AFTER INADV.CLOS.) |
| C41-XHE-FO-OPENF002B | 2.69E-01 | OPERATOR FAILS TO OPEN VALVE F002B (AFTER INADV.CLOS.) |
| C41-XHE-MH-F002A | 4.03E-03 | MISPOSITION OF VALVE F002A |
| C41-XHE-MH-F002B | 4.03E-03 | MISPOSITION OF VALVE F002B |
| C51-ACT-CF-SRNM | 2.98E-04 | CCF OF SRNM CORE FLUX CHANNELS |
| C51-ACT-LO-CHASRNM | 1.90E-02 | SRNM CHANNEL A FAILS |
| C51-ACT-LO-CHBSRNM | 1.90E-02 | SRNM CHANNEL B FAILS |
| C51-ACT-LO-CHCSRNM | 1.90E-02 | SRNM CHANNEL C FAILS |
| C51-ACT-LO-CHDSRNM | 1.90E-02 | SRNM CHANNEL D FAILS |
| C51-ACT-LO-CHESRNM | 1.90E-02 | SRNM CHANNEL E FAILS |
| C51-ACT-LO-CHFSRNM | 1.90E-02 | SRNM CHANNEL F FAILS |
| C51-ACT-LO-CHGSRNM | 1.90E-02 | SRNM CHANNEL G FAILS |
| C51-ACT-LO-CHHSRNM | 1.90E-02 | SRNM CHANNEL H FAILS |
| C51-ACT-LO-CHJSRNM | 1.90E-02 | SRNM CHANNEL J FAILS |
| C51-ACT-NO-CHL | 4.32E-05 | SRNM CHANNEL L FAILS |
| C51-VLU-CF-SRNM | 7.48E-06 | CCF 3/4 OF SRNM 2/4 VLU |
| C51-VLU-FC-DIV1 | 6.24E-04 | SRNM DIV 1 2/4 VLU FAILS |
| C51-VLU-FC-DIV2 | 6.24E-04 | SRNM DIV 2 2/4 VLU FAILS |
| C51-VLU-FC-DIV3 | 6.24E-04 | SRNM DIV 3 2/4 VLU FAILS |
| C51-VLU-FC-DIV4 | 6.24E-04 | SRNM DIV 4 2/4 VLU FAILS |
| C74-DTM-CF-ATWSALL | 1.30E-05 | CCF 3 OUT OF 4 DIGITAL TRIP MODULES |
| C74-DTM-FC-DIV1ATWS | 6.00E-04 | DIGITAL TRIP MODULE FAILS TO FUNCTION (ATWS) |
| C74-DTM-FC-DIV2ATWS | 6.00E-04 | DIGITAL TRIP MODULE FAILS TO FUNCTION (ATWS) |
| C74-DTM-FC-DIV3ATWS | 6.00E-04 | DIGITAL TRIP MODULE FAILS TO FUNCTION (ATWS) |
| C74-DTM-FC-DIV4ATWS | 6.00E-04 | DIGITAL TRIP MODULE FAILS TO FUNCTION |
| C74-VLU-CF-ATWS | 3.12E-06 | CCF 3/4 VOTING LOGIC UNIT ATWS DIVISIONS |
| C74-VLU-CF-SLCS | 7.48E-06 | CCF 2/2 VOTING CARD SLCS LOGIC |
| C74-VLU-FC-ATWS1 | 6.24E-04 | ATWS DIV 1 VOTING LOGIC UNIT FAILS |
| C74-VLU-FC-ATWS2 | 6.24E-04 | ATWS DIV 2 VOTING LOGIC UNIT FAILS |
| C74-VLU-FC-ATWS3 | 6.24E-04 | ATWS DIV 3 VOTING LOGIC UNIT FAILS |

Table 4.4-7
List of System Basic Events

| Basic Event | Prob | Description |
|--------------------|----------|---|
| C74-VLU-FC-ATWS4 | 6.24E-04 | ATWS DIV 4 VOTING LOGIC UNIT FAILS |
| C74-VLU-FC-SLCS1 | 6.24E-04 | 2/4 VOTING LOGIC CARD FOR SLCS DIV 1 |
| C74-VLU-FC-SLCS2 | 6.24E-04 | 2/4 VOTING LOGIC CARD FOR SLCS DIV 2 |
| H23-EMS-CF-ATWSALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-RMU-CF-ALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-CF-ATWSALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-FC-001C41 | 3.00E-04 | RMU 001 CRD SYSTEM C.R. MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-002C41 | 3.00E-04 | RMU 002 CRD SYSTEM C.R. MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-003C41 | 3.00E-04 | RMU 003 CRD SYSTEM C.R. MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-ATWS1A | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ATWS1B | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ATWS1C | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ATWS1D | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ATWS2A | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002A FAILS TO FUNCTION |
| H23-RMU-FC-ATWS2B | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002B FAILS TO FUNCTION |
| H23-RMU-FC-ATWS2C | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002C FAILS TO FUNCTION |
| H23-RMU-FC-ATWS2D | 3.00E-04 | REMOTE MULTIPLEXING UNIT 0002D FAILS TO FUNCTION |

Table 4.4-8
List of System Top Events

| Top Event | Description |
|-------------|--------------------------------------|
| GC41-0001_1 | Long Term Boration Fails During ATWS |

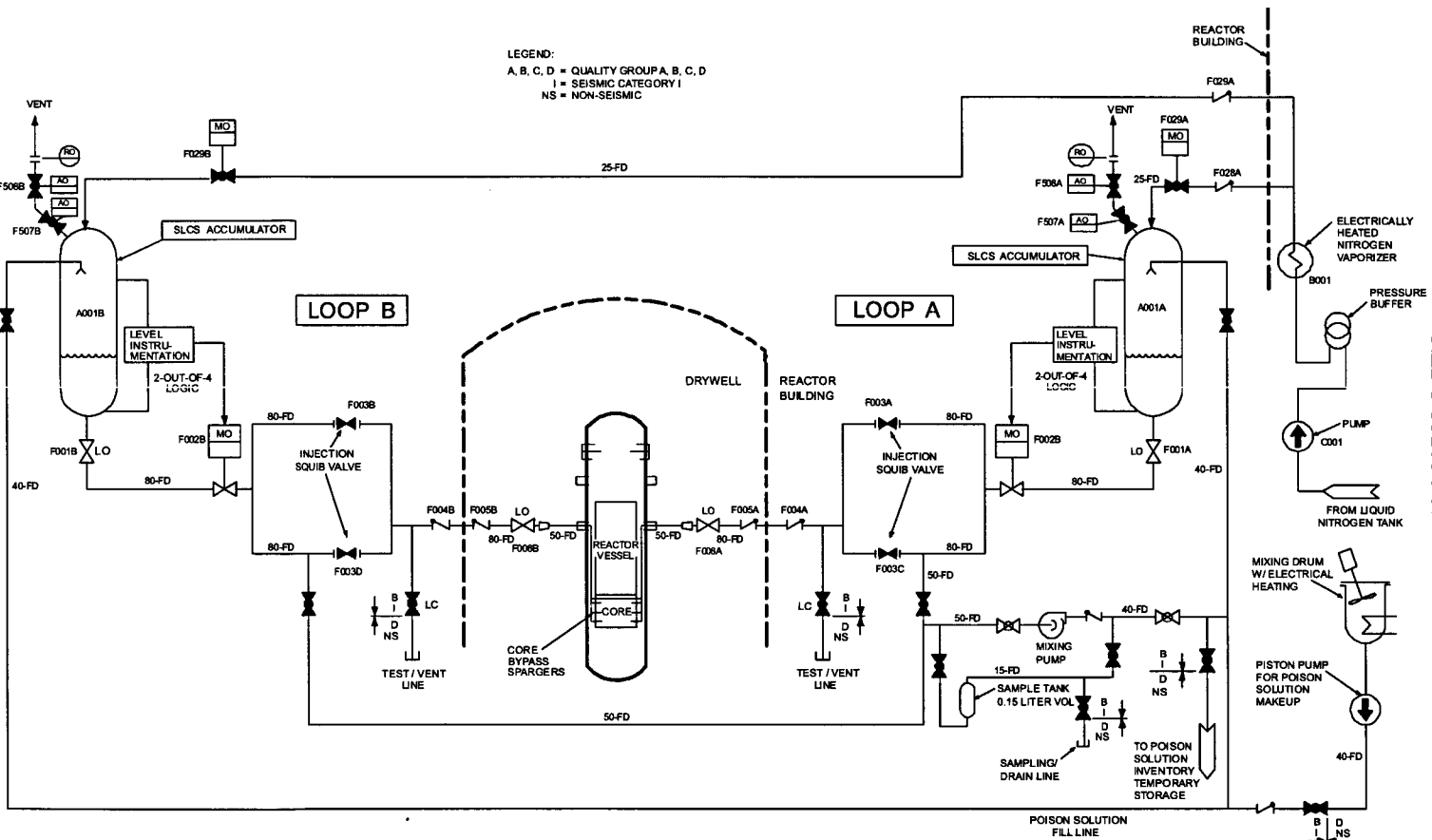


Figure 4.4-1. Simplified Process Flow Diagram for Standby Liquid Control System

4.5 INSTRUMENTATION, LOGIC AND CONTROL SYSTEM – (C62, C63, C71 and C74)

4.5.1 Purpose

The ESBWR Instrumentation, Logic and Control (I&C) System provides a centralized set of equipment for implementing safety and non-safety related logic functions.

4.5.2 Design Assumptions

The following design assumptions (DA) have been used for the model:

- (1) The ESBWR Instrumentation, Logic and Control (I&C) System is made up of the following six independent subsystems:
 - SSLC/ECCS subsystem (Class 1E)
 - SSLC/RTIF - SCRAM subsystem (Class 1E)
 - SSLC/RTIF – ATWS subsystem (Class 1E)
 - DPS subsystem (Non Class 1E)
 - NE RTNSS subsystem (Non class 1E)
 - NE BOP subsystem (Non class 1E)
- (2) The SSLC/ECCS subsystem processes the automatic and manual signals that act on systems B32, E50 (GDCS and Equalizing Lines), P54, B21 (ADS signal) and T10 (IC and RWCU isolation).
- (3) The SSLC/RTIF - SCRAM subsystem processes the automatic and manual signals that act on system T10 (MSIVs isolation), as well as the feedwater runback and inhibition ADS signals.
- (4) The SSLC/RTIF – ATWS subsystem processes the automatic and manual signals that act on system C41, as well as the feedwater runback and ADS inhibits signals.
- (5) The DPS subsystem processes the automatic and manual signals that act on systems B21 (ADS signal) and E50 (GDCS).
- (6) The NE RTNSS subsystem processes the automatic and manual signals that act on systems C12, G21, G31, P21, P41, P52, R21 and U43.
- (7) The NE BOP subsystem processes the automatic and manual signals that act on systems N21, N71, P22, P30, and P51.
- (8) Each of these subsystems is comprised of the corresponding part of the E-DCIS or NE-DCIS that provides redundant and distributed control and instrumentation data communications networks to support the monitoring and control of interfacing plant systems and the logic channels (microprocessor-based, software-controlled, processing modules) located in the cabinets of the Main Control Room area.
- (9) The SSLC/ECCS, SSLC/RTIF – SCRAM, SSLC/RTIF – ATWS and DPS subsystems are composed of four independent and separated instrumentation divisions I, II, III, and IV

(logic channels), while the NE RTNSS and NE BOP subsystems are made up of three independent and separated instrumentation divisions A, B and C (logic channels).

- (10) The structure of the logic channels of the SSLC/ECCS, SSLC/RTIF – ATWS, DPS, NE RTNSS and NE BOP subsystems are identical.
- (11) Each one of divisions I, II, III and IV of the SSLC/ECCS, SSLC/RTIF and DPS subsystems is fed electrical power from divisions I, II, III and IV Class 1E of the Uninterruptible Power Supply System (R13).
- (12) Each one of the divisions A, B and C of subsystems NE RTNSS and NE BOP is powered from trains A, B and C Non Class 1E of the Uninterruptible Power Supply System (R13).

4.5.3 System Description

The Instrumentation, Logic and Control (I&C) System analysis described here refers to the system configuration shown in Figure 4.5-1.

The I&C system is designed to accomplish both essential and non-essential functions.

The essential functions are those designed for the following:

- To lead to an automatic reactor scram when plant conditions reach safety-related limits
- To provide automatic and manual actuation of essential safety systems, thus limiting the consequences of accidents
- To collect plant data and alert the operator to any abnormal plant conditions so that corrective action can be initiated to prevent a reactor scram

The non-essential functions are designed for the following:

- To provide for the automatic or manual control of the Balance of Plant (BOP) systems from the control room
- To provide manual and automatic actuation of all non-safety systems

Some basic characteristics of ESBWR I&C are reported in Reference 4.5-1.

4.5.3.1 Hardware Configuration

The logic and control functions are basically accomplished by the following systems:

- Safety System Logic & Control (SSLC)
- Essential Distributed Control and Instrumentation System (E-DCIS)
- Non-Essential Distributed Control and Instrumentation System (NE-DECIS)

The SSLC enables both the safety and non-safety systems to perform their functions and is composed of the following independent subsystems:

- SSLC/ECCS subsystem (Class 1E)
- SSLC/RTIF - SCRAM subsystem (Class 1E)
- SSLC/RTIF – ATWS subsystem (Class 1E)

- DPS subsystem (Non Class 1E)
- NE RTNSS subsystem (Non class 1E)
- NE BOP subsystem (Non class 1E)

SSLC resides in independent and separated instrumentation divisions and integrates the control logic in each division into microprocessor-based, software-controlled, processing modules located in divisional cabinets in the Main Control Room MCRB area.

Each division performs functionally identical logical operations on the process sensor signals to initiate protective action. Many sensor signals are shared by the control logic of multiple safety systems.

The main equipment in each SSLC/ECCS, DPS, NE RTNSS, and NE BOP division and the functions performed are described below:

- Digital Trip Module (DTM). Sensor channel trip decisions (comparison of input variables to programmed setpoints).
- Voter Logic Unit (VLU). The Voter Logic trains in each division are redundant but not independent modules. Each of the redundant pairs of VLUs receives the trip status from the DTMs in all four divisions and performs 2-out-of-4 and 3-out-of-4 logic to determine the actuation status for each system function. VLUs are located in main control room area SSLC/ESF cabinet (per division).
- Safety System Logic Unit (SLU). SLUs are located in each RMU. The SLUs receive actuation signals from Voter Logic Units and combines actuation signals with interlock logic permissives that implement system functional logic as defined on the logic diagrams of the affected system. Output logic is 2-out-of-2 to the device actuators. Each VLU/SLU train is defined in Plant Technical Specification as a Logic Channel. The SLUs also provides outputs, including annunciator outputs from built-in self diagnostic functions, to the operator displays and to the plant computer.
- Bypass Unit (BPU). Division-of-sensors bypass and division maintenance bypass functions that permit on-line calibration, test, and repair without causing containment isolation, or degrade ECCS or other ESF operation. Located in SSLC Test Cabinet in main control room area (per division).
- Manual switches in Control Room.

The equipment in each RTIF division is as follows:

- Digital Trip Module (DTM) that compares individual monitored variable values with trip setpoint values and for each variable sends a separate, discrete (trip/no trip) output signal to Trip Logic Units (TLUs) in all four divisions of trip logic.
- Trip Logic Unit (TLU) that performs the initiation logic, normally checking for 2-out-of-4 coincidence of trip conditions in any set of instrument channel signals coming from the four division DTMs.
- Bypass Unit (BPU) that performs bypass and interlock logic for the channel sensors bypass and the division trip logic unit bypass. It sends a separate bypass signal for all four channels to the trip logic unit in the same division for channel sensors bypass.

- Output Logic Unit (OLU) that performs division trip, seal-in, reset and trip test function. Each output logic unit receives bypass inputs from the BPU, trip inputs from the TLU of the same division and various manual inputs from switches within the same division and provides discrete trip outputs to the trip actuators in the same division.

The E-DCIS and the NE-DCIS, using fiber optic main transmission lines, provide distributed control and instrumentation data communication networks to support the monitoring and control of interfacing plant systems.

The E-DCIS has independent and separated data multiplexing divisions. Each E-DCIS division contains a redundant fiber optic network that comprises of RMUs, fiber optic signal transmission path, and isolated digital interfaces, etc. The network connects divisional Class 1E VDUs, SSLC Cabinets, Test Cabinet, and several RMUs located in each of the equipment rooms in different buildings. The network also interfaces with NE-DCIS through isolated digital interfaces. E-DCIS acquires, formats, and transmits plant sensor data and control commands within the time response requirements of the safety-related systems with which it interfaces.

The Non-Essential Distributed Control and Instrumentation System (NE-DCIS) has independent and separated data multiplexing divisions and provides redundant and distributed control and instrumentation data communications networks to support the monitoring and control of interfacing plant systems. Nonsafety-related data are processed through the NE-DCIS.

4.5.3.2 System Operation

The I&C evaluates safety system trip conditions continuously in all modes of plant operation; i.e., Run, Startup, Hot Shutdown, Cold Shutdown, and Refueling and itself operates in two modes that affect all safety systems simultaneously: Normal and Bypass.

During normal operation, the systems supported by the I&C are maintained in a standby condition and are ready to function on demand when an accident or transient event occurs. The following tasks are performed repetitively in all plant modes on data received during each scan of E-DCIS or from other sensor data sources.

The first stage of I&C operation is the comparison of the measured parameters to their setpoints. Digital or digitized input signals from E-DCIS or NE-DCIS provide the value or status of critical plant process parameters to the DTMs contained in each division cabinets. The input signals for each division are acquired from separate and independent sensor channels. At the DTMs, the value of each monitored parameter is compared to a reference setpoint stored in digital memory. For each parameter that exceeds the setpoint limit, the DTM declares a sensor channel trip. The trips are sent to coincident VLU (SSLC/ECCS, DPS, NE RTNSS, and NE BOP subsystems) or TLU (RTIF subsystems) within their own division and, by isolated interconnection, to the other subsystem divisions.

The trip outputs from the DTM are compared with the trip outputs from the DTMs of the other subsystem divisions. This function is performed, for subsystems SSLC/ECCS, DPS, NE RTNSS, and NE BOP, in parallel VLUs (one pair per system processed) and, for subsystem RTIF, in the TLU.

The VLU pairs (VLU1 and VLU2) are redundant, with a 2-out-of-2-confirmation configuration at the SLUs within the RMU. Dual SLUs with identical logic functions are connected in parallel

and simultaneously process the same sensor trip or contact closure inputs from the VLUs or other input sources.

Trip outputs from each VLU are serially multiplexed on separate communication links of E-DCIS or NE-DCIS to the SLUs in the RMUs, where a 2-out-of-2 voter determines if there is agreement between the two channels. Comparisons for voting are performed in one of the two parallel SLUs, designated as SLU A. Agreement results in processing of control and interlock signals in the identical logic of SLUs A and B, which communicate across the card file backplane. Correct performance of SLUs A and B results in output of control signals for equipment initiation or nuclear system isolation as required. Control signals to component actuators are hardwired from contact closures in the RMUs of E-DCIS or NE-DCIS to the appropriate motor control centers, switchgear, or solenoids.

Disagreement between the two channels inhibits the control output.

This arrangement prevents inadvertent equipment initiation or primary containment vessel isolation due to hardware or software failure in a single SLU. The SLUs provide device interlock logic and formulate the protective action initiation commands that are transmitted to the local area equipment actuators.

The TLU sends the trip signal to the OLU within its own division. The OLU performs division trip.

4.5.3.3 Components Location

The SSLC cabinets are located in the MCRB area.

The E-DCIS and the NE-DCIS fiber optic networks connect SSLC cabinets with the RMUs located in the equipment rooms of the different buildings.

4.5.4 Automatic and Manual Control

The following automatic and manual signals are processed through the I&C system:

Isolation Condenser System (ICS)

The B32-F005A/B/C/D and B32-F006A/B/C/D open automatically by the following signals:

- (1) High reactor pressure sensed by B21-PT-002A/B/C/D pressure transmitters (if the high pressure remains for more than 10 seconds).

OR

- (2) Reactor water level 2 sensed by B21-LT-N001A/B/C/D level transmitters.

OR

- (3) MSIV closure, i.e., closure of all Main Steam Lines (MSL), sensed by one limit switch per valve.

The valves (B32-F72HA/B/C/D), that provide cross connection of the IC and PCC pools, to allow 72 hours of cooling by the ICS and PCCS heat exchangers, open automatically on the following signals:

- (1) Low level in the pool A sensed by B32-LT-F72HA1/2/3/4 level transmitters (valves B32-F72HA/D)

OR

- (2) Low level in the pool B sensed by B32-LT-F72HB1/2/3/4 level transmitters (valve B32-F72HB/C)

Operator actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

These signals are processed through the SSLC/ECCS subsystem.

Automatic Depressurization System (ADS)

The DPVs and SRVs are actuated automatically by the following signal:

- Reactor water level 1 sensed by B21-LT-N001A/B/C/D and B21-LT-DPSWRA/B/C/D water level transmitters.

Operator actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is processed through the SSLC/EECS and DPS subsystems.

Gravity Driven Cooling System (GDCS)

The GDCS is actuated automatically by the following signal:

- Reactor water level 1 sensed by B21-LT-N001A/B/C/D and B21-LT-DPSWRA/B/C/D water level transmitters.

Operator actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is processed through the SSLC/ECCS and DPS subsystems.

High Pressure Nitrogen Supply System (HPNSS)

The essential nitrogen bottles are aligned automatically by the following signal.

- Low pressure of nitrogen sensed by P54-PT-002/003 pressure transmitters.

Operator actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is processed through the SSLC/ECCS subsystem.

Main Steam Isolation Valve (MSIV)

The MSIV close automatically by the following signals:

- Reactor water level 2 sensed by B21-LT-N001A/B/C/D level transmitters.

Operator actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is processed through the SSLC/RTIF-SCRAM subsystem.

Leak Detection and Isolation System (LD&IS)

The LD&IS is employed to isolate either the entire containment or a single system according to the signal received.

- c. ICS isolation in case of LOCA outside containment. In that case, the line break is isolated automatically by the following signals:

- High differential pressure between steam supply line and condensate return line of the corresponding loop sensed by B32-PDT-N001/2/3/4A, B, C or D pressure differential transmitters.

OR

- High radiation in the corresponding pool exhaust sensed by D11-ACT-1/2/3/4A, B, C or D radiation monitors.

- d. RWCU isolation in case of LOCA outside containment. In that case, the line break is isolated automatically by the following signals:

- Reactor water level 2 sensed by B21-LT-N001A/B/C/D level transmitters.

OR

- High flow in the corresponding RWCU train sensed by C21-FT-N001A/B/C/D (A) for train A or by C21-FT-N001A/B/C/D (B) for train B flow transmitters.

OR

- High temperature in the steam tunnel sensed by C21-TT-N003A/B/C/D temperature transmitters.

Human actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

These signals are processed through the SSLC/ECCS and NE RTNSS subsystems.

Standby Liquid Control System (SLCS)

The SLCS is assumed to be automatically actuated by the following signals:

- (1) High reactor pressure sensed by narrow range B21-PT-ATWSA/B/C/D

OR

Reactor water level 2 sensed by wide range water B21-LT-ATWSA/B/C/D

AND

- (2) Permissive of “APRM Not Downscale” signal sensed by SRNM flux detectors

Human actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

These signals are processed through the SLCS/RTIF-ATWS subsystem.

ADS Inhibit

The ADS Inhibit that precludes DPV opening is assumed to be automatically actuated by the following signals:

- (1) High reactor pressure sensed by B21 PT-ATWSA/B/C/D

OR

Reactor water level 2 sensed by wide range water B21-LT-ATWSA/B/C/D

AND

- (2) Permissive of "APRM Not Downscale" signal sensed by SRNM flux detectors

These signals are processed through the SSLC/RTIF-ATWS subsystem.

Feedwater Runback (FWRB)

The FWRB is actuated automatically by the following signals:

- High reactor pressure sensed by narrow range B21-PT-ATWSA/B/C/D with permissive of "APRM Not Downscale" signal sensed by the Source Range Neutron Monitor (SRNM) flux detectors.

These signals are processed through the SSLC/RTIF-ATWS subsystem.

Control Rod Drive System (CRDS)

The CRDS is automatically initiated in its injection mode by the following signal:

- Reactor water level 2 sensed by B21-LT-DPSWRA/B/C/D level transmitters.

Human actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is generated in Class 1E sensor channels and processed through the NE RTNSS subsystem.

Fuel and Auxiliary Pools Cooling System (FAPCS)

The FAPCS is automatically initiated in its suppression pool cooling mode by the following signal:

- High temperature in the suppression pool sensed by T62-TT-001/2/3/4 temperature transmitters.

Human actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

This signal is processed through the NE RTNSS subsystem.

Standby On-Site AC Power Supply (R21)

The DGs automatically start by the following signal:

- Low voltage in bus A2 sensed by the undervoltage relay 10A11 (DGA).
- Low voltage in bus B2 sensed by the undervoltage relay 10A21 (DGB).

The startup of any one of the diesel generators leads to the automatic startup of the corresponding pumps of systems P21 and P41 that provide cooling to their respective diesel generators.

Human actions are possible as backup to these automatic actuation signals. Control switches in the main control room are used for the manual backup.

These signals are processed through the NE RTNSS subsystem.

Besides the automatic actions and the manual support indicated above, the I&C system processes manual actions from the Control Room that allows the alignment and operation of the different systems.

4.5.5 System Interfaces

The I&C system dependencies are included in the dependency matrix given in Table 4.5-2.

4.5.6 System Testing

The continuous self-diagnostics of E-DCIS and NE-DCIS automatically detect most data transmission errors and hardware failures at the card level. A comprehensive, off-line, network performance test confirms that the data transmission capability is as intended. The test provides confidence that data error rates are within specified limits, signal quality is within specifications, and the network is capable of handling the required throughput.

Most SSLC surveillance testing is included as part of surveillance testing for its supported systems, since SSLC contains the control logic for these systems. However, the following surveillance tests are unique to SSLC, since they involve functions shared by all interfacing systems:

- **Sensor Channel (Instrument Channel) Check** - This check is a visual comparison, on the MCR main control console, of the parameter indicated in one sensor channel to a similar parameter in a different sensor channel. Since redundant sets of sensors measure the same process, the indications should be reasonably close. This check is performed every shift.
- **Divisional Functional Test** - The test is performed by replacing the process signal with a test signal generated by the SSLC Surveillance Test Controller located within each divisional SSLC Test cabinet. The test signal, which can simulate the full range of an analog or digital process signal, is injected at the RMU input of E-DCIS or NE-DCIS to test the DTMs. The as-found trip setpoints are confirmed to be within their allowable values. Test signals are also injected at the SLUs to check trip logic and interlock logic response. However, VLU logic is not tested, since inputs from multiple divisions would be required. This test, performed quarterly, supplements the continuous self-diagnostic checks within each SSLC controller.
- **Comprehensive Functional Test** - This test, which is performed during an outage, verifies overall SSLC system function, computer component function, software and hardware interactions, response times, and error handling in four divisions. Successful completion of these tests establishes operability of sensor channels (instrument channels), logic channels (trip logic), and output channels (trip actuators). This end-to-end test

injects test signals simultaneously in the four divisions at the RMU inputs and thus checks the voting logic units (VLU).

- **Sensor Channel Calibration** - A sensor channel calibration or channel calibration is a complete check of the channel from the sensor output through any intervening devices and to the trip output (e.g., RMU or other signal conditioner, multiplexing network, DTM). This test verifies that a channel responds to the measured parameter within the necessary range and accuracy. This calibration is performed during outages.

4.5.7 System Maintenance

No regular maintenance is required for E-DCIS or NE-DCIS logic components. Continuous self-diagnostics in each component monitor the state of the system and each module. Fault conditions that result in detected failures are alarmed in the MCR and are traceable to the lowest replaceable module.

To SSLC components, maintenance is performed on-line by using the SSLC bypass provisions to remove components from service in one division at a time. A division-of-sensors bypass permits a DTM to be serviced, while a train bypass permits a VLU to be serviced. Sufficient channel redundancy is always maintained while the system is in bypass.

Therefore, no maintenance unavailability is considered.

4.5.8 Common-Cause Failures

The common cause failures have been grouped in Table 4.5-5.

4.5.9 Fault Tree Analysis

In relation to the functions that the Containment System has to carry out, various fault trees have been developed that allow determining the system unavailability and identifying components that contribute significantly to system unavailability.

The functions required of the system are the following:

- Signal transmission through division A of subsystem NE RTNSS.
- Signal transmission through division B of subsystem NE RTNSS.
- Signal transmission through division C of subsystem NE RTNSS.
- Signal transmission through division A of subsystem NE BOP.
- Signal transmission through division B of subsystem NE BOP.
- Signal transmission through division C of subsystem NE BOP.
- Signal transmission through division I of subsystem SSLC/ECCS.
- Signal transmission through division II of subsystem SSLC/ECCS.
- Signal transmission through division III of subsystem SSLC/ECCS.
- Signal transmission through division IV of subsystem SSLC/ECCS.
- Generation of automatic depressurization signal division I SSLC/ECCS subsystem.

- Generation of automatic depressurization signal division II SSLC/ECCS subsystem.
- Signal transmission through division I of subsystem DPS.
- Signal transmission through division II of subsystem DPS.
- Signal transmission through division III of subsystem DPS.
- Signal transmission through division IV of subsystem DPS.
- Generation of automatic depressurization signal division I DPS subsystem.
- Generation of automatic depressurization signal division II DPS subsystem.
- Inhibition of automatic depressurization signal in case of ATWS.
- Generation of feedwater runback signal in case of ATWS.

4.5.9.1 Top Event Definitions

- Gate GC62-0001-_1 represents the failure to transmit a signal through division A of subsystem NE RTNSS.
- Gate GC62-0001-_2 represents the failure to transmit a signal through division B of subsystem NE RTNSS.
- Gate GC62-0001-_3 represents the failure to transmit a signal through division C of subsystem NE RTNSS.
- Gate GC62-0001-_4 represents the failure to transmit a signal through division A of subsystem NE BOP.
- Gate GC62-0001-_5 represents the failure to transmit a signal through division B of subsystem NE BOP.
- Gate GC62-0001-_6 represents the failure to transmit a signal through division C of subsystem NE BOP.
- Gate GC74-0001-_1 represents the failure to transmit a signal through division I of subsystem SSLC/ECCS.
- Gate GC74-0001-_2 represents the failure to transmit a signal through division II of subsystem SSLC/ECCS.
- Gate GC74-0001-_3 represents the failure to transmit a signal through division III of subsystem SSLC/ECCS.
- Gate GC74-0001-_4 represents the failure to transmit a signal through division IV of subsystem SSLC/ECCS.
- Gate GC74-0001-_5 represents the failure to generate the automatic depressurization signal division I SSLC/ECCS subsystem.
- Gate GC74-0001-_6 represents the failure to generate the automatic depressurization signal division II SSLC/ECCS subsystem.

- Gate GC72-0001-_1 represents the failure to transmit a signal through division I of subsystem DPS.
- Gate GC72-0001-_2 represents the failure to transmit a signal through division II of subsystem DPS.
- Gate GC72-0001-_3 represents the failure to transmit a signal through division III of subsystem DPS.
- Gate GC72-0001-_4 represents the failure to transmit a signal through division IV of subsystem DPS.
- Gate GC72-0001-_5 represents the failure to generate the automatic depressurization signal division I DPS subsystem.
- Gate GC72-0001-_6 represents the failure to generate the automatic depressurization signal division II DPS subsystem.
- Gate GC74-0001-_11 represents the failure to inhibit the automatic depressurization signal in case of ATWS.

4.5.9.2 Model Assumptions

The following assumptions with respect to the design are made:

- (1) System limits: In general and for the purpose of the PRA, the I&C system includes the components through which the logic signals run which are located between the RMUs that receive the signals of the sensor channels and send them to the DTMs and the RMUs that transmit the actuation signals related to the different components of the plant, including the RMUs. The sensor channels related to the generation of the ADS Actuation, Inhibit ADS, and Feedwater Runback signals are included within the I&C system.
- (2) Within the I&C system, no generic models have been developed for signal transmission across the divisions of the SSLC/RTIF subsystem. Specific models have been built and included within the models of the systems that use said signals. These models are the following:
 - SLC actuation signal (C41 system model)
 - MSIV isolation signal (T10 Containment system models)
 - Inhibition ADS signal (I&C system models)
 - Feedwater Runback signal (I&C system models)
- (3) Specific models have been developed for LOOP, and for the startup and connection signal of the DGs due to undervoltage in busbars. Models have been developed for the signal transmission through the NE RTNSS subsystem that depends only on electrical supply from batteries. The models has been developed in this manner to avoid circular logic. These models are included within the AC system models (R11).

4.5.9.3 Human Actions

The human actions considered in the fault trees are the following:

B21-XHE-FO-ADS represents the operator's failure to recognize that the automatic initiation of ADS has failed and manually initiate it.

XXX-XHE-FO-DEPRESS represents the operator's failure to recognize the need for depressurization.

Human actions in the fault trees are connected by the house-event XHOSMAN that represents operator unavailability to carry out manual actions.

4.5.10 Results of Fault Tree Analysis

The fault tree results provide quantitative values of system unavailability and of the importance of specific components to that total.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

The importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.5.11 Model Differences for 72-Hour Operation

The long-term Instrumentation, Logic and Control System operation is similar to that during the first 24 hours following a transient or LOCA. There are no fault tree model changes necessary to extend the operation to 72 hours.

4.5.12 References

4.5-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.5-1
Control Room Instrumentation and Alarms

| Main Control Room Panel Controls |
|--|
| The data transmission and processing functions require no operator intervention are provided |
| Main Control Console Displays |
| Division I of sensors in bypass |
| Division II of sensors in bypass |
| Division III of sensors in bypass |
| Division IV of sensors in bypass |
| ESF loop 1 in bypass |
| ESF loop 2 in bypass |
| ESF loop 3 in bypass |
| ESF loop 4 in bypass |
| SSLC Division I controller inoperative (DTM, VLU or SLU) |
| SSLC Division II controller inoperative (DTM, VLU or SLU) |
| SSLC Division III controller inoperative (DTM, VLU or SLU) |
| SSLC Division IV controller inoperative (DTM, VLU or SLU) |
| NIM, CIM or BTM Division I inoperative |
| NIM, CIM or BTM Division II inoperative |
| NIM, CIM or BTM Division III inoperative |
| NIM, CIM or BTM Division IV inoperative |
| E-DCIS Division I diagnostic displays |
| E-DCIS Division II diagnostic displays |
| E-DCIS Division III diagnostic displays |
| E-DCIS Division IV diagnostic displays |
| NE-DCIS diagnostic displays |
| Main Control Room Alarms |
| SSLC/ESF in bypass |
| SSLC/ESF system trouble |
| E-DCIS Division I trouble |
| E-DCIS Division II trouble |
| E-DCIS Division III trouble |
| E-DCIS Division IV trouble |
| NE-DCIS alarms |

Table 4.5-2
System Dependencies Matrix

| COMPONENT | R13 (Class 1E) | R13 (NO 1E) |
|------------------------|---------------------------|------------------------|
| SSLC/ECCS Division I | DIV 1 | |
| SSLC/ECCS Division II | DIV 2 | |
| SSLC/ECCS Division III | DIV 3 | |
| SSLC/ECCS Division IV | DIV 4 | |
| SSLC/RTIF Division I | DIV 1 | |
| SSLC/RTIF Division II | DIV 2 | |
| SSLC/RTIF Division III | DIV 3 | |
| SSLC/RTIF Division IV | DIV 4 | |
| DPS Division I | DIV 1 | |
| DPS Division II | DIV 2 | |
| DPS Division III | DIV 3 | |
| DPS Division IV | DIV 4 | |
| NE RTNSS Division A | | DIV 1 |
| NE RTNSS Division B | | DIV 2 |
| NE RTNSS Division C | | DIV 3 |
| NE BOP Division A | | DIV 1 |
| NE BOP Division B | | DIV 2 |
| NE BOP Division C | | DIV 3 |

Table 4.5-3
Component Tests
(See Section 4.5.6)

Table 4.5-4
Component Maintenance

N/A

Table 4.5-5
Common Cause Failure

| Basic Event | Probability | Description |
|------------------------|--------------------|---|
| B21-LT_-CF-DPSWR | 1.20E-07 | CCF OF DPS WR LEVEL TRANSMITTERS |
| B21-LT_-CF-N001A/B/C/D | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 10A/B/C/D |
| C51-ACT-CF-1PRM | 2.98E-04 | CCF APRM NEUTRON CHANNELS |
| C51-ACT-CF-APRMSTUCK | 2.10E-07 | CCF APRM DETECTORS STUCK AT POWER LEVEL |
| C51-ACT-CF-SRNM | 2.98E-04 | CCF OF SRNM CORE FLUX CHANNELS |
| C51-VLU-CF-1PRM | 9.30E-07 | PRNM DIV I 2/4 MODULES FAILS |
| C51-VLU-CF-2PRM | 9.30E-07 | PRNM DIV II 2/4 MODULES FAILS |
| C51-VLU-CF-APRM | 9.30E-07 | CCF 2/4 MODULES APRM |
| C62-BYP-CF-N1EALL | 1.50E-05 | CCF OF BYPASS UNITS (N1E) |
| C62-VLU-CF-DIDALL | 3.12E-05 | CCF OF VOTER LOGIC UNITS |
| C62-VLU-CF-N1EALL | 3.12E-05 | CCF OF VOTER LOGIC UNITS |
| C72-BYP-CF-DPSALL | 1.50E-06 | CCF OF BYPASS UNITS |
| C72-DTM-CF-DPSALL | 1.20E-05 | CCF 3/4 DTM OF DPS DIV 1/2/3/4 |
| C72-VLU-CF-DPSALL | 3.12E-06 | CCF OF VOTER LOGIC UNITS |
| C74-BYP-CF-ALL | 1.50E-06 | CCF OF BYPASS UNITS |
| C74-BYP-CF-DIDALL | 9.00E-07 | CCF OF BYPASS UNITS (DID) |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV 1/2/3/4 |
| C74-VLU-CF-ALL | 3.12E-06 | CCF OF VOTER LOGIC UNITS |
| C74-VLU-CF-ATWS | 3.12E-06 | CCF 3/4 VOTING LOGIC UNIT ATWS |
| H23-EMS-CF-ALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-EMS-CF-DIDALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-EMS-CF-DPSALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| H23-EMS-CF-N1EALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-RMU-CF-ALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| H23-RMU-CF-DPSALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-CF-N1EALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (NO 1E) TO OPERATE |

Table 4.5-6
Human Errors

| Basic Event | Probability | Description |
|--------------------|-------------|--|
| B21-XHE-FO-ADS | 1.77E-02 | OPERATOR FAILS TO RECOGNIZE NEED OR ACTUATE ADS |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |

Table 4.5-7
List of System Basic Events

| Basic Event | Probability | Description |
|--------------------|-------------|--|
| B21-LT_-NO-DPSWRA | 2.40E-05 | DPS WIDE RANGE LEVEL TRANSMITTER A (LEVEL 1&2) FAILS |
| B21-LT_-NO-DPSWRB | 2.40E-05 | DPS WIDE RANGE LEVEL TRANSMITTER B (LEVEL 1&2) FAILS |
| B21-LT_-NO-DPSWRC | 2.40E-05 | DPS WIDE RANGE LEVEL TRANSMITTER C (LEVEL 1&2) FAILS |
| B21-LT_-NO-DPSWRD | 2.40E-05 | DPS WIDE RANGE LEVEL TRANSMITTER D (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001A | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1A (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001B | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1B (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001C | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1C (LEVEL 1&2) FAILS |
| B21-LT_-NO-N001D | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 1D (LEVEL 1&2) FAILS |
| B21-OR_-PG-01A | 1.44E-05 | ORIFICE INSTR. LINE 1A FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01B | 1.44E-05 | ORIFICE INSTR. LINE 1B FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01C | 1.44E-05 | ORIFICE INSTR. LINE 1C FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-01D | 1.44E-05 | ORIFICE INSTR. LINE 1D FAILS TO REMAIN OPEN (PLUG) |
| C31-VLU-FC-RUNBACK | 7.80E-05 | C31 SYSTEM VOTER LOGIC UNIT FAILS |
| C51-ACT-LO-CHASRNM | 1.90E-02 | SRNM CHANNEL A FAILS |
| C51-ACT-LO-CHBSRNM | 1.90E-02 | SRNM CHANNEL B FAILS |
| C51-ACT-LO-CHCSRNM | 1.90E-02 | SRNM CHANNEL C FAILS |
| C51-ACT-LO-CHDSRNM | 1.90E-02 | SRNM CHANNEL D FAILS |
| C51-VLU-FC-1APRM | 7.80E-05 | APRM DIV I 2/4 MODULE FAILS |
| C51-VLU-FC-2APRM | 7.80E-05 | APRM DIV II 2/4 MODULE FAILS |
| C51-VLU-FC-3APRM | 7.80E-05 | APRM DIV III 2/4 MODULE FAILS |

Table 4.5-7
List of System Basic Events

| Basic Event | Probability | Description |
|---------------------|-------------|--|
| C51-VLU-FC-4APRM | 7.80E-05 | APRM DIV IV 2/4 MODULE FAILS |
| C62-BYP-FC-DIVBN1E | 3.00E-04 | BYPASS UNIT DIV B (N1E) FAILS TO FUNCTION |
| C62-BYP-FC-DIVCN1E | 3.00E-04 | BYPASS UNIT DIV C (N1E) FAILS TO FUNCTION |
| C62-VLU-FC-ESF1ADID | 6.24E-04 | 1ST DIV A ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF1BDID | 6.24E-04 | 1ST DIV B ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF1BN1E | 6.24E-04 | 1ST DIV B N1E ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF1CDID | 6.24E-04 | 1ST DIV C (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF1CN1E | 6.24E-04 | 1ST DIV C N1E ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2ADID | 6.24E-04 | 2ND DIV A (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2BDID | 6.24E-04 | 2ND DIV B (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2BN1E | 6.24E-04 | 2ND DIV B (N1E) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2CDID | 6.24E-04 | 2ND DIV C (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2CN1E | 6.24E-04 | 2ND DIV C (N1E) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-BYP-FC-DPSDIV1 | 3.00E-04 | BYPASS UNIT DIV 1 FAILS TO FUNCTION |
| C72-BYP-FC-DPSDIV2 | 3.00E-04 | BYPASS UNIT DIV 1 FAILS TO FUNCTION |
| C72-BYP-FC-DPSDIV3 | 3.00E-04 | BYPASS UNIT DIVIII FAILS TO FUNCTION |
| C72-BYP-FC-DPSDIV4 | 3.00E-04 | BYPASS UNIT DIVIV FAILS TO FUNCTION |
| C72-DTM-FC-DPSDIV1 | 6.00E-04 | DTM OF DPS DIV 1 FAILS TO TRIP |
| C72-DTM-FC-DPSDIV2 | 6.00E-04 | DTM OF DPV DIV 2 FAILS TO TRIP |
| C72-DTM-FC-DPSDIV3 | 6.00E-04 | DTM OF DPS DIV 3 FAILS TO TRIP |
| C72-DTM-FC-DPSDIV4 | 6.00E-04 | DTM OF DPS DIV 4 FAILS TO TRIP |
| C72-VLU-FC-DPSESF11 | 6.24E-04 | 1ST DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF12 | 6.24E-04 | 1ST DIV 1 ESF VOTER LOGIC UNIT FAILS TO |

Table 4.5-7
List of System Basic Events

| Basic Event | Probability | Description |
|---------------------|-------------|--|
| | | FUNCTION |
| C72-VLU-FC-DPSESF13 | 6.24E-04 | 1ST DIV III ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF14 | 6.24E-04 | 1ST DIV IV ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF21 | 6.24E-04 | 2ND DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF22 | 6.24E-04 | 2ND DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF23 | 6.24E-04 | 2ND DIV III ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C72-VLU-FC-DPSESF24 | 6.24E-04 | 2ND DIV IV ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-BYP-FC-DIV1 | 3.00E-04 | BYPASS UNIT DIV 1 FAILS TO FUNCTION |
| C74-BYP-FC-DIV2 | 3.00E-04 | BYPASS UNIT DIV 1 FAILS TO FUNCTION |
| C74-BYP-FC-DIV3 | 3.00E-04 | BYPASS UNIT DIVIII FAILS TO FUNCTION |
| C74-BYP-FC-DIV4 | 3.00E-04 | BYPASS UNIT DIVIV FAILS TO FUNCTION |
| C74-BYP-FC-DIVADID | 3.00E-04 | BYPASS UNIT DIV A (DID) FAILS TO FUNCTION |
| C74-BYP-FC-DIVBDID | 3.00E-04 | BYPASS UNIT DIV B (DID) FAILS TO FUNCTION |
| C74-BYP-FC-DIVCDID | 3.00E-04 | BYPASS UNIT DIV C (DID) FAILS TO FUNCTION |
| C74-DTM-FC-DIV1 | 6.00E-04 | DTM OF SSLC DIV 1 FAILS TO TRIP |
| C74-DTM-FC-DIV2 | 6.00E-04 | DTM OF SSLC DIV 2 FAILS TO TRIP |
| C74-DTM-FC-DIV3 | 6.00E-04 | DTM OF SSLC DIV 3 FAILS TO TRIP |
| C74-DTM-FC-DIV4 | 6.00E-04 | DTM OF SSLC DIV 4 FAILS TO TRIP |
| C74-VLU-FC-ESF11 | 6.24E-04 | 1ST DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF12 | 6.24E-04 | 1ST DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF13 | 6.24E-04 | 1ST DIV III ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF14 | 6.24E-04 | 1ST DIV IV ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF21 | 6.24E-04 | 2ND DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |

Table 4.5-7
List of System Basic Events

| Basic Event | Probability | Description |
|--------------------|-------------|---|
| C74-VLU-FC-ESF22 | 6.24E-04 | 2ND DIV 1 ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF23 | 6.24E-04 | 2ND DIV III ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-VLU-FC-ESF24 | 6.24E-04 | 2ND DIV IV ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| H23-EMS-FC-DIV1 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 1 FAILS TO FUNCTION |
| H23-EMS-FC-DIV2 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 2 FAILS TO FUNCTION |
| H23-EMS-FC-DIV3 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 3 FAILS TO FUNCTION |
| H23-EMS-FC-DIV4 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 4 FAILS TO FUNCTION |
| H23-EMS-FC-DIVADID | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV A (DID) FAILS TO FUNCTION |
| H23-EMS-FC-DIVBDID | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV B (DID) FAILS TO FUNCTION |
| H23-EMS-FC-DIVBN1E | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV B (N1E) FAILS TO FUNCTION |
| H23-EMS-FC-DIVCDID | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV C (DID) FAILS TO FUNCTION |
| H23-EMS-FC-DIVCN1E | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV C (N1E) FAILS TO FUNCTION |
| H23-EMS-FC-DPSDIV1 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DPS DIV 1 FAILS TO FUNCTION |
| H23-EMS-FC-DPSDIV2 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DPS DIV 2 FAILS TO FUNCTION |
| H23-EMS-FC-DPSDIV3 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DPS DIV 3 FAILS TO FUNCTION |
| H23-EMS-FC-DPSDIV4 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DPS DIV 4 FAILS TO FUNCTION |
| H23-RMU-FC-1ADS | 3.00E-04 | RMU 001 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-2ADS | 3.00E-04 | RMU 002 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-DIV1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |

Table 4.5-7
List of System Basic Events

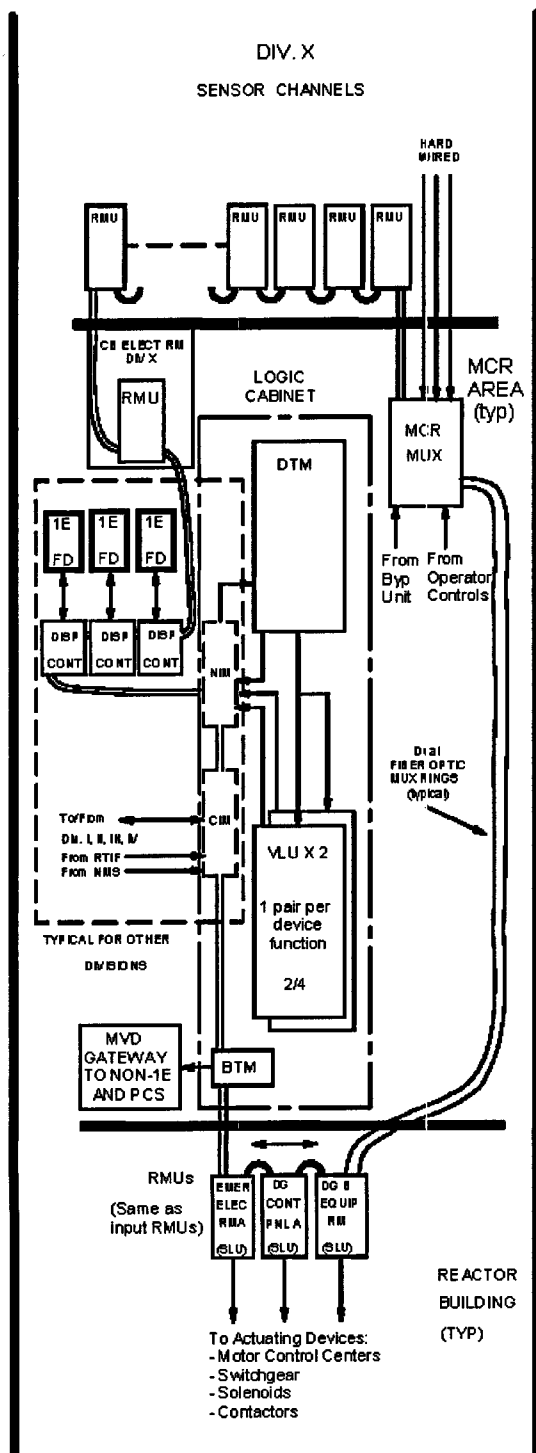
| Basic Event | Probability | Description |
|---------------------|-------------|--|
| H23-RMU-FC-DIV2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DIV3 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DIV4 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPS1ADS | 3.00E-04 | RMU 001 FOR MANUAL DPS ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-DPS2ADS | 3.00E-04 | RMU 002 FOR MANUAL DPS ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-DPSDIV1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSDIV2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSDIV3 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSDIV4 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF11 | 3.00E-04 | 1ST DIV II ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF12 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF13 | 3.00E-04 | 1ST DIV III ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF14 | 3.00E-04 | 1ST DIV IV ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF21 | 3.00E-04 | 2ND DIV I ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF22 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF23 | 3.00E-04 | 2ND DIVIII ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-DPSESF24 | 3.00E-04 | 2ND DIVIII ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF11 | 3.00E-04 | 1ST DIV II ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF12 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |

Table 4.5-7
List of System Basic Events

| Basic Event | Probability | Description |
|---------------------|-------------|--|
| H23-RMU-FC-ESF13 | 3.00E-04 | 1ST DIV III ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF14 | 3.00E-04 | 1ST DIV IV ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1ADID | 3.00E-04 | 1ST DIV A (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1BDID | 3.00E-04 | 1ST DIV B (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1BN1E | 3.00E-04 | 1ST DIV B (N1E) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1CDID | 3.00E-04 | 1ST DIV C (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1CN1E | 3.00E-04 | 1ST DIV C (N1E) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF21 | 3.00E-04 | 2ND DIV 1 ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF22 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF23 | 3.00E-04 | 2ND DIV III ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF24 | 3.00E-04 | 2ND DIV III ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2ADID | 3.00E-04 | 2ND DIV A (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2BDID | 3.00E-04 | 2ND DIV B (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2BN1E | 3.00E-04 | 2ND DIV B (N1E) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2CDID | 3.00E-04 | 2ND DIV C DID ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2CN1E | 3.00E-04 | 2ND DIV C (N1E) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |

Table 4.5-8
List of System Top Events

| TOP EVENT | Description |
|------------------|--|
| GC31-0001-_1 | FEEDWATER RUNBACK SIGNAL FAILURE |
| GC62-0001-_1 | DIV A (NE RTNSS) SIGNAL TRANSMISSION FAILURE |
| GC62-0001-_2 | DIV B (NE RTNSS) SIGNAL TRANSMISSION FAILURE |
| GC62-0001-_3 | DIV C (NE RTNSS) SIGNAL TRANSMISSION FAILURE |
| GC62-0001-_4 | DIV A (NE BOP) SIGNAL TRANSMISSION FAILURE |
| GC62-0001-_5 | DIV B (NE BOP) SIGNAL TRANSMISSION FAILURE |
| GC62-0001-_6 | DIV C (NE BOP) SIGNAL TRANSMISSION FAILURE |
| GC72-0001-_1 | DPS DIV I SIGNAL TRANSMISSION FAILURE |
| GC72-0001-_2 | DPS DIV II SIGNAL TRANSMISSION FAILURE |
| GC72-0001-_3 | DPS DIV III SIGNAL TRANSMISSION FAILURE |
| GC72-0001-_4 | DPS DIV IV SIGNAL TRANSMISSION FAILURE |
| GC72-0001-_5 | LOSS OF DPS DIV 1 DPV & SRV ACTUATION SIGNAL |
| GC72-0001-_6 | LOSS OF DPS DIV 2 DPV & SRV ACTUATION SIGNAL |
| GC74-0001-_1 | SSLC DIV I SIGNAL TRANSMISSION FAILURE |
| GC74-0001-_2 | SSLC DIV II SIGNAL TRANSMISSION FAILURE |
| GC74-0001-_3 | SSLC DIV III SIGNAL TRANSMISSION FAILURE |
| GC74-0001-_4 | SSLC DIV IV SIGNAL TRANSMISSION FAILURE |
| GC74-0001-_5 | LOSS OF DIV 1 DPV & SRV ACTUATION SIGNAL |
| GC74-0001-_6 | LOSS OF DIV 2 DPV & SRV ACTUATION SIGNAL |
| GC74-0001-_11 | ADS INHIBIT FAILURE |



DIV. X TYPICAL FOR :

- DIVISIONS I, II, III, IV of ESF subsystem.
- DIVISIONS A, B, C of DID subsystem.
- DIVISIONS A, B, C of NO 1E subsystem

Figure 4.5-1 (1/2). Simplified Diagram of Instrument, Logic and Control System

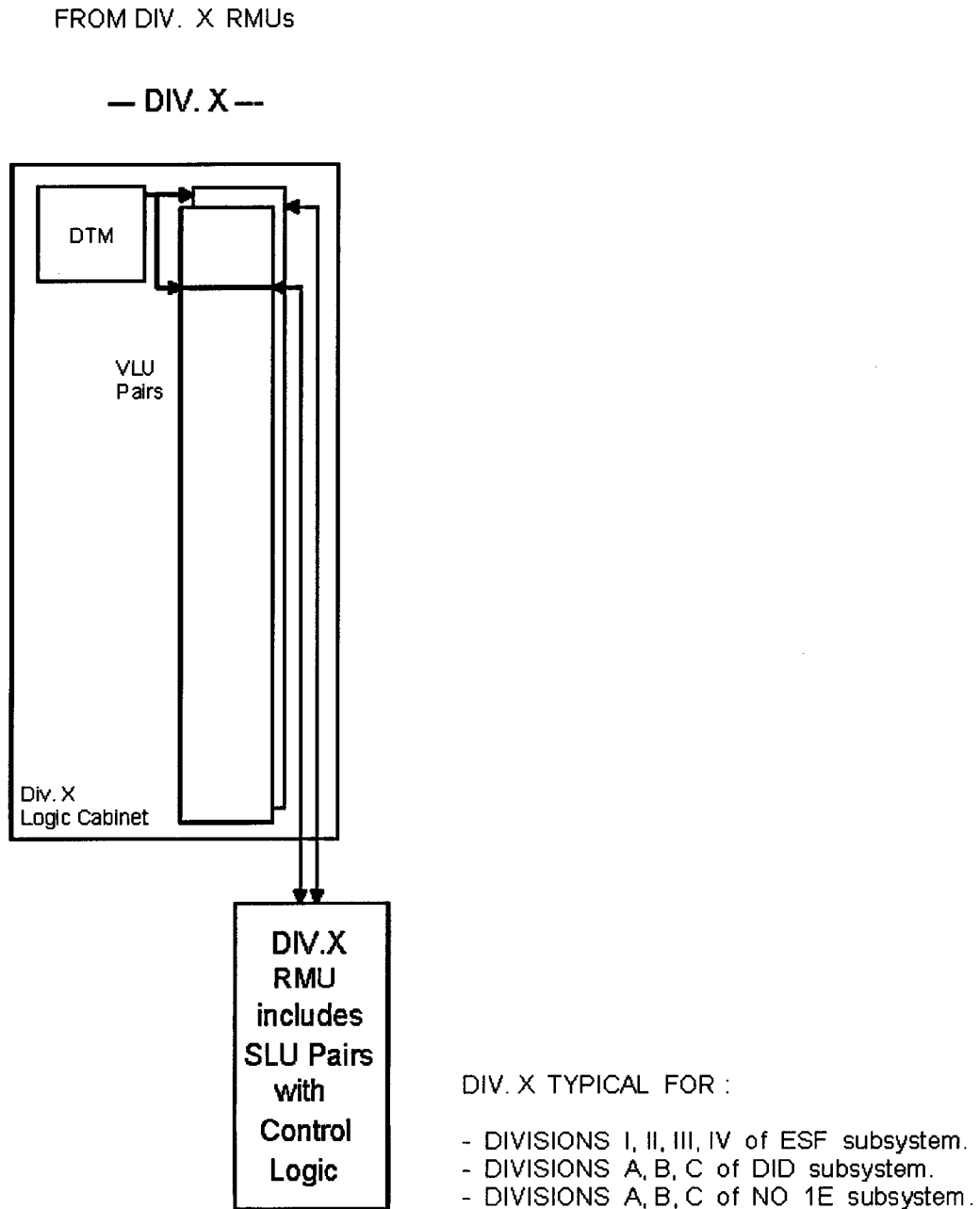


Figure 4.5-1 (2/2). Simplified Diagram of Instrument, Logic And Control System

4.6 GRAVITY-DRIVEN COOLING SYSTEM - (E50)

4.6.1 Purpose

The primary purpose of the Gravity-Driven Cooling System (GDCS) is to passively provide emergency core cooling after any event that threatens the reactor coolant inventory. Once the Nuclear Boiler System (NBS) has been depressurized via the Automatic Depressurization System (ADS), the GDCS is capable of passively injecting large volumes of water into the depressurized reactor pressure vessel (RPV) to keep the fuel covered over both short and long term time frames following system initiation.

Another function is to provide water to the lower drywell in case of a severe accident that causes RPV breach. This latter function has not been modeled in the Level 1 PRA.

In addition, the GDCS pools collect the condensate from the PCCS exchangers. GDCS includes pool water inventory, pool injection lines (main, branch secondary lines and deluge lines), suppression pool (S/P) equalizer lines, including strainers and pool piping penetrations.

The GDCS is not credited for unisolated breaks outside the containment.

4.6.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) There are three elevated GDCS pools of water at drywell pressure.
- (2) One 200 mm (8-inch) line (with a maintenance valve F004) carries water from each of two GDCS pools (Pools A and C), and two independent 200 mm (8-inch) lines carry water from the third GDCS pool (Pool B). These lines have a branch line that discharges into the drywell situated in the 8" pool discharge line between the manually operated valve nearest the pool (F004) and the check valve (F003). This line then branches into 3, each one with an explosive valve (F009). All three lines discharge into the upper drywell.
- (3) Each of four 200 mm (8-inch) lines divides into two 150 mm (6-inch) lines. These six-inch lines each join the reactor pressure vessel at a nozzle.
- (4) Each of the 150 mm (6-inch) lines has a squib valve (F002), a check valve (F003), a maintenance valve (F001), and small lines to allow testing of the check valves and flow paths.
- (5) There are four 150 mm (6-inch) divisional lines (equalizing lines) to carry long-term water from the suppression pool (SP) to the RPV, and each of these lines has two maintenance valves (F005, F008), a squib valve (F006), a check valve (F007), and small lines to allow testing of check valves and flow paths.
- (6) The manual valves of the RPV discharge lines and the connection to the SP are locked open.

Information regarding alarms and instrumentation is preliminary.

4.6.3 System Description

The GDCS consists of a short-term cooling subsystem and a long-term cooling subsystem, both of which provide cooling water under force of gravity to replace water inventory lost during a loss-of-coolant accident (LOCA) event and subsequent decay heat boil-off. A GDCS deluge subsystem discharges in the lower part of the upper drywell region water from GDCS for severe accident conditions.

The success criteria used in the model is summarized as follows:

- In the short term, injection is necessary through at least two of the eight GDCS 150 mm lines, and the discharge of 2 of the 3 pools.
- In the long term, at least one of the equalizing lines is to be operating to fulfill the function.

4.6.3.1 Hardware Configuration

A simplified diagram of the GDCS is shown in Figure 4.6-1. According to the design assumptions, the GDCS consists of two independent systems, the short term and the long term cooling systems. The short-term system supplies gravity-driven flow to eight separate nozzles on the vessel with suction flow from three separate GDCS pools. The long-term system supplies gravity-driven flow to four other nozzles with suction flow from the suppression pool (SP) through the equalizing lines.

Each division of the GDCS injection system consists of one 200 mm (8-inch) nominal diameter pipe (with a block valve) exiting from the GDCS pool and branching into two 150 mm (6-inch) paths, each containing one check valve, one squib valve, and one block valve. Each division of the long-term system consists of one 150 mm (6-inch) equalizing line with two block valves, a check valve, and a squib valve.

The twelve RPV injection line nozzles all contain integral flow limiters with a venturi-like shape and a minimum throat diameter of 75 mm (three inches) for short-term injection lines and 60 mm (two inches) for long-term injection lines. On each injection line, there is a locked open, manually operated maintenance valve located near the vessel nozzle and another such valve located near the water source.

In each of the 150 mm (6-inch) injection lines, the check valve is located upstream of the squib-actuated injection valve. The squib valve is designed to withstand reactor pressure without leakage during operation but, once actuated, it provides a permanent open flow path from the water source to the vessel. The check valves prevent gross reverse back flow to the pools after the squib valves are actuated if the vessel pressure is still higher than the pool pressure plus its gravity head. Once the vessel pressure has decayed below pool pressure, the differential pressure opens the check valve and allows water to begin flowing into the vessel.

The check valves are designed such that they remain partially open when zero pressure difference exists across the valve to limit core-cladding temperature. This is to minimize the potential for sticking in the closed position during long periods of non-use. Test connection lines are provided to provide a means of testing for both closing and opening operation.

The drywell-GDCS airspace has a mesh screen or equivalent to prevent the entry of debris material that might be carried into the pool, such as during a large break LOCA. A thermally

actuated deluge valve (Valves F009 on Figure 4.6-1) is connected to each of the four GDCS 200 mm (8-inch) downcomer lines in order to provide a means to flood the lower drywell cavity in the event of a core melt sequence which causes failure of the lower vessel head and allows the molten debris to reach the cavity floor.

4.6.3.2 System Operation

The GDCS begins operation following an RPV level 1 water level signal. The level 1 signal starts two sets of timers, two 150-second timers for initiation of the short-term water injection lines and two 30 minutes timers for initiation of the long-term injection lines. At 150 seconds after level 1, the eight squib valves (F002A-H) open to allow water flow from the GDCS pools to the RPV. This water flow maintains the RPV water level high enough to maintain the core submerged.

The long-term portion of GDCS begins operation 30 minutes after a level 1 signal, subject to the RPV transient water being no higher than one meter above the TAF, with opening of the four squib valves (F006A-D) on the four Suppression Pool (SP) to RPV equalizing lines. Therefore, the Level 1 RPV water level signal coupled with completion of the 30-minute timer gives the permissive for the Equalization line squib valves to fire when the RPV level reaches 1m above the Top of Active Fuel (Level 0.5). The small elevation difference between the SP and the RPV is sufficient to allow SP water to maintain RPV water level 30 minutes after shutdown.

The success criteria applied to the GDCS are that a sufficient number of GDCS lines must function in the short term and that a sufficient number of GDCS pools or SP equalizing lines are required for the long term.

In the short term, following a LOCA, GDCS lines from the GDCS pools are used to recover and maintain RPV level. In the medium and long term, the SP equalizing lines perform the RPV inventory control function while making up the following inventory losses:

- LOCA above the core:

Evaporation losses to the drywell (Most evaporation is returned to the RPV through the isolation condensers or the passive containment cooling system (PCCS). Some moisture is vented to the SP from the PCCS.)

- Vessel bottom break

Medium term:

Evaporation losses to the drywell

Break flow losses

Long term:

Evaporation losses to the drywell

In the event that 7 of 8 GDCS pool lines fail, the SP equalizing lines alone are not sufficient to prevent core damage in the short term because of low flow due to the small available driving head. Even if the GDCS pool lines correctly operate, SP lines are necessary to maintain RPV level in the long term for some accident sequences.

Passive breaks of GDCS lines are not considered, other than as a specific LOCA initiating event, because GDCS lines are designed for full system pressure. No break due to high pressure is expected when squib valves are fired open.

4.6.3.3 Component Location

Basic components of the GDCS are located within the primary containment. The GDCS pools, piping and valves are all located in the drywell. The suppression pool is on the outer periphery of the drywell within the containment envelope. The logic elements that provide controls for the actuation of GDCS squib valves are outside primary containment and located in the corresponding four corners of the reactor building floors. Batteries that provide DC power to actuate the squib valves are located outside the containment.

The GDCS pools are located within the drywell at an elevation significantly above the top of the active core. The suppression pool water level is a few feet above the active core.

4.6.4 Automatic and Manual Control

Actuation of the GDCS is performed automatically, without the need for operator action. A backup to automatic actuation is the ability of operators to actuate the GDCS manually, if necessary.

Automatic Actuation. Automatic actuation of the GDCS is provided by an RPV level 1 water level signal (level 1). System logic then actuates the eight squib valves (F002A-H) installed in the 150 mm lines from GDCS Pools A, B, and C after a 150 sec time delay. The level 1 signal also actuates the ADS to reduce RPV pressure if other permissive logic is satisfied. (See ATWS discussion of ADS inhibit.) Also, after a longer time delay (30 min), the four squib valves (F006A-D) mounted on the SP equalizing lines are actuated, if the RPV transient water level is also less than one meter above the TAF.

The level 1 signals coming from Div. I, II, III, and IV level indicators (belonging to the Nuclear Boiler System) generate the GDCS logic divisional level 1 output signals that are sealed in for 30 minutes. Each divisional level 1 signal is passed to four different logic timers that initiate whenever two-out-of-four signals are sealed in.

The GDCS squib valves are identified in four different logic groups and two different time delay types (Table 4.6-1A). Two DC power divisions and two logic divisions are supplied for each valve to open. Each of the eight GDCS 150 mm line squib valves is similarly actuated.

Manual Actuation. The plant operator can actuate the GDCS manually if automatic actuation fails. Manual actuation, Short-Term and Long-Term (Equalizing), is interlocked to a low RPV pressure signal. Deluge squibs to flood drywell are interlocked with high-high Drywell pressure. The manual actuation circuit has four channels, one for each division. On the actuation circuitry for Type 2 valves (SP equalizing line valves), 30-minutes time delay logic is replicated to reproduce the sequence of the automatic actuation. To actuate the GDCS, the operator has to push two buttons at the same time. A signal coming from each button closes one switch for each division. If the pressure interlock switch is closed (low pressure conditions are present in the primary system), the signal remains sealed in for 30 minutes. This manual actuation process immediately fires open the valves located on the eight GDCS lines and, after 30 minutes, if the

RPV level has dropped below Level 0.5 (1 m above the TAF), actuates the SP equalizing line valves.

Instrumentation and Alarms. The instrument and alarm signals reported in the control room for the GDCS are given in Table 4.6-1B.

4.6.5 System Interfaces

The GDCS is a passive system that interfaces with two support systems and shares components with the RPV water level instrumentation and the other systems using the suppression pool. It only requires the following systems to be able to carry out its function:

- DC power supply (essential power)
- Safety System Logic and Control (SSLC)

Squib valves are fired by DC power. Dependency of each valve on specific DC power supply divisions and logic arrangement for each valve are shown in Table 4.6-2. The fault tree for loss of DC power is modeled in Section 4.17.

SSLC Divisional signals provide the automatic and manual initiating signal. Loss of the uninterruptible AC power supply system (R13) is modeled in Section 4.15.

4.6.6 System Testing

General testing of GDCS equipment is performed during refueling. Table 4.6-3 shows the test program for the system. The only valves directly operated for testing are the normally closed isolation valves on the test lines. Flow through the test lines is used to open and close check valves and to show that there is no obstruction of the RPV nozzles. Valve realignment following testing is controlled administratively. A non-intrusive magnetically coupled torque-motor together with disc position sensor are used on the check valve to test and monitor position. The torque motor and sensor provide the capability for in-service surveillance testing. With regards to model assumptions, it is assumed that this test is performed quarterly.

No unavailability events during power operation due to tests or maintenance are postulated, as there are considered to be enough administrative controls to make their contribution negligible.

4.6.7 System Maintenance

Maintenance of GDCS components is performed during refueling, every two years. There is no maintenance of components during plant operation.

4.6.8 Common Cause Failures

The GDCS provides a high degree of redundancy. That redundancy increases the relative importance of common cause failure (CCF) in overall system unavailability. The mechanical components that are more exposed to CCF events involving just the GDCS are the following:

- Orifices in injection and equalizing lines
- Squib valves
- SP suction line strainers

Common cause failures within the system, including the CCF of the logic, are summarized in Table 4.6-5.

Squib valve failure is mainly affected by failure of initiators. Common environmental conditions expose initiators to CCF due to:

- High temperature in the drywell
- Aging
- Irradiation
- Humidity
- Jet impingement

The squib valve CCFs that would affect the various modeled functions are postulated in the fault tree.

Suction line strainers plugging as a CCF can occur after a LOCA when debris is discharged into the SP via drywell to wetwell vents.

The common cause failures related to the instrumentation and logic which affect the system appear in the last part of Table 4.6-5.

4.6.9 Fault Tree Analysis

The GDACS failure tree has been evaluated to determine the system unavailability and which components contribute significantly to system unavailability.

4.6.9.1 Top Event Definitions

A TOP event has been modeled which represents short-term and long-term coolant supply. The contributions to the failure (with their corresponding gates) are the following:

- Two-of-eight GDACS lines needed, short term (failure in 7 of the 8 lines) - GE50TOP1
- Two-of-three GDACS pools needed (failure in 2 of the 3 GDACS pools), long term - GE50TOP2
- One equalizing line needed (failure in the 4 equalizing lines), long term - GE50TOP3
- One-of-three GDACS pools fails to discharge - GE50TOP4
- Three-of-three GDACS pools fail to discharge - GE50TOP5
- Two-of-three GDACS pools fail to discharge and four-of-four equalizing lines fail to open - GE50TOP6

4.6.9.2 Model Assumptions

The possibility that the LOCA occurs in one of the discharge lines in pool B, the most limiting case, is being considered, because if it happened, the discharge of this pool would not be effective, affecting 4 discharge lines.

It is assumed that check valve tests are performed quarterly (See Table 4.6-3).

The catastrophic failures of the GDCS pools are not considered, because none of them lead to the failure of the functions and their combination with other failures is highly improbable.

The Type C human actions included in the model are included in the Emergency Operating Procedures or in other Operating Procedures.

4.6.9.3 Fault Tree Description

The fault tree for GE50TOP is developed in Appendix B.4.6.

Among the dominant contributors to failure in the long term is the failure probability that none of the three equalizing lines from the SP function for long term core cooling following a level 1 signal.

4.6.9.4 Human Interactions

Human errors appear in Table 4.6-6. In the failure tree, this error is connected in a logic OR gate with a house event to allow for its inclusion in the reliability analysis. The contribution to GDCS unavailability due to Locked-open Maintenance Valves mispositioning can be disregarded considering that:

- They are maneuvered only during special maintenance occasions.
- They are located inside the drywell and are locked open.
- Their status indication is reported in the control room.
- Administrative procedures assist operators in verifying valve status before each start-up.

Human actions in support of automatic actuations must be covered in the corresponding procedures.

4.6.10 Results of Fault Tree Analysis

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

A summary of basic events included in the system fault tree is reported in Table 4.6-7.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.6.11 Model Differences for 72-hour Operation

The long term GDCS operation with the core cooling water provided from the GDCS pools or through one of four suppression pool equalizer lines continues beyond 24 hours. There are no model changes necessary to extend operation to 72 hours.

4.6.12 References

4.6-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.6-1A
GDCS Squib Valve Grouping

| | <u>Pool A</u> | <u>Pool B</u> | <u>Pool C</u> | <u>SP</u> |
|--------|----------------------|----------------------|----------------------|------------------|
| TYPE 1 | F002A | F002B | F002D | |
| | F002E | F002F | F002H | |
| | | F002C | | |
| | | F002G | | |
| TYPE 2 | | | | F006A |
| | | | | F006B |
| | | | | F006C |
| | | | | F006D |

Note:

Type 1 valves actuate 150 seconds after Level 1 signal, and Type 2 valves actuate 30 minutes after Level 1 signal provided Level 0.5 is achieved (See 4.6.3.2).

Table 4.6-1B
Control Room Instrumentation And Alarms

| | |
|---|--|
| Controls | |
| Squib valves actuation | |
| Displays | |
| GDCS Pool Level | |
| SP Pool Level | |
| Locked-open manually operated maintenance valve status (F001A-H, F004A-D, F005A-D, F008A-D) | |
| Squib Valve Status (F002A-H, F006A-D) | |
| Alarms | |
| GDCS Pool Level | |
| SP Pool Level | |

Table 4.6-2
System Dependency Matrix

| | | Support System | | | |
|-----------|-------------|-----------------|-------|-------|-------|
| | | Power Actuation | | | |
| | | 250 V DC (R16) | | | |
| Component | Type | DIV 1 | DIV 2 | DIV 3 | DIV 4 |
| F002A | Squib valve | X | | | X |
| F002B | Squib valve | | X | X | |
| F002C | Squib valve | X | | | X |
| F002D | Squib valve | | X | X | |
| F002E | Squib valve | X | | | X |
| F002F | Squib valve | | X | X | |
| F002G | Squib valve | X | | | X |
| F002H | Squib valve | | X | X | |
| F006A | Squib valve | X | | | X |
| F006B | Squib valve | | X | X | |
| F006C | Squib valve | X | | | X |
| F006D | Squib valve | | X | X | |
| F009A | Squib valve | X | | | X |
| F009B | Squib valve | | X | X | |
| F009C | Squib valve | X | | | X |
| F009D | Squib valve | | X | X | |
| F009E | Squib valve | X | | | X |
| F009F | Squib valve | | X | X | |
| F009G | Squib valve | X | | | X |
| F009H | Squib valve | | X | X | |
| F009I | Squib valve | X | | | X |
| F009J | Squib valve | | X | X | |
| F009K | Squib valve | X | | | X |
| F009L | Squib valve | | X | X | |

Table 4.6-2
System Dependency Matrix

| Component | Type | Support System | | | |
|-----------|-------------|----------------|-------|-----------|-------|
| | | SSLC Signal | | DPS (C72) | |
| | | DIV 1 | DIV 2 | DIV 3 | DIV 4 |
| F002A | Squib valve | X | | | X |
| F002B | Squib valve | | X | X | |
| F002C | Squib valve | X | | | X |
| F002D | Squib valve | | X | X | |
| F002E | Squib valve | X | | | X |
| F002F | Squib valve | | X | X | |
| F002G | Squib valve | X | | | X |
| F002H | Squib valve | | X | X | |
| F009A | Squib valve | X | | | X |
| F009B | Squib valve | | X | X | |
| F009C | Squib valve | X | | | X |
| F009D | Squib valve | | X | X | |
| F009E | Squib valve | X | | | X |
| F009F | Squib valve | | X | X | |
| F009G | Squib valve | X | | | X |
| F009H | Squib valve | | X | X | |
| F009I | Squib valve | X | | | X |
| F009J | Squib valve | | X | X | |
| F009K | Squib valve | X | | | X |
| F009L | Squib valve | | X | X | |

| Component | Type | Support System | | | |
|-----------|-------------|----------------|-------|-------|-------|
| | | SSLC Signal | | | |
| | | DIV 1 | DIV 2 | DIV 3 | DIV 4 |
| F006A | Squib valve | X | | | X |
| F006B | Squib valve | | X | X | |
| F006C | Squib valve | X | | | X |
| F006D | Squib valve | | X | X | |

Table 4.6-3
Component Test

| Component Type | Expected Test Interval | Components aligned away from emergency position without automatic return logic |
|---|-------------------------------|---|
| Check valves (F003A-H) | Quarterly | Test Valves |
| Check Valves (F007A-D) | Quarterly | Test Valves |
| Maintenance valves (F001A-H, F004A-D, F005A-D, F008A-D) | 24 Months | None |

Note: See Section 4.6.6

Table 4.6-4
Component Maintenance

Maintenance of the GDCS components is performed during refueling, every two years. There is no maintenance of components during plant operation.

Therefore, no unavailability events during power operation due to tests or maintenance are postulated in the model, as there are considered to be sufficient administrative controls to make their contribution negligible.

Table 4.6-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|---------------------------|----------|--|
| B21-LT_-CF-N001ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-LT_-CF-N012ABCD | 7.00E-08 | CCF 3/4 LEVEL TRANSMITTERS N012A/B/C/D |
| B21-OR_-CF-N012ABCD | 4.00E-08 | CCF 3/4 ORIFICES LINES 12 A/B/C/D (PLUG) |
| B21-PT_-CF-02HIGH | 3.00E-08 | CCF OF PRESSURE RPV TRANSMITTERS HIGH |
| C51-ACT-CF-APRMSTUCK | 2.10E-07 | CCF APRM DETECTORS STUCK AT POWER LEVEL |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV1/2/3/4 |
| E50-OR_-CF-4EQUA | 4.00E-08 | CCF OF 4 ORIFICES EQUALIZING LINES (PLUG) |
| E50-OR_-CF-4PLUG | 7.00E-08 | CCF OF 4 OR MORE ORIFICES TO PLUG |
| E50-OR_-CF-7PLUG | 7.00E-08 | CCF OF 7 ORIFICES TO PLUG |
| E50-OR_-CF-D001AE | 7.20E-07 | CCF 2/2 ORIFICES LINES A AND E (PLUG) |
| E50-OR_-CF-D001BCFG | 7.00E-08 | CCF 4/4 ORIFICES LINES B, C, F AND G (PLUG) |
| E50-OR_-CF-D001DH | 7.20E-07 | CCF 2/2 ORIFICES LINES D AND H (PLUG) |
| E50-OR_-CF-PLUGALL | 7.20E-08 | CCF OF ALL ORIFICES TO PLUG |
| E50-SQV-CF-EQALLOPEN | 3.00E-05 | CCF OF ALL 4 SQUIB VALVES TO OPEN |
| E50-SQV-CF-F002A/2E | 3.60E-05 | CCF OF SQUIB VALVES F002A/ F002E |
| E50-SQV-CF-F002B/2C/2F/2G | 1.50E-05 | CCF OF SQUIB VALVES F002B/ F002C/ F002F/ F002G |
| E50-SQV-CF-F002D/2H | 3.60E-05 | CCF OF SQUIB VALVES F002D/ F002H |
| E50-SQV-CF-GDCS7OPEN | 1.50E-05 | CCF OF 7 SQUIB VALVES IN GDCS LINES TO OPEN |
| E50-SQV-CF-OPENALL | 1.50E-05 | CCF OF ALL SQUIB VALVES TO OPEN |
| E50-STR-CF-SPPLUG | 3.75E-04 | CCF FILTER/STRAINER IN PSP TO PLUG |

Table 4.6-6
List of System Human Errors

| Basic Event | Prob | Description |
|--------------------|-------------|--|
| E50-XHE-FO-EQU | 1.61E-03 | OPERATOR FAILS TO ACTUATE GDCS |
| E50-XHE-FO-GDCS | 1.61E-03 | OPERATOR FAILS TO ACTUATE GDCS |
| XXX-XHE-FO-RPVLDE | 1.61E-02 | OP. FAILS TO RECOG. OR CHECK THE RPV DECREASING LEVEL |

Table 4.6-7
Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|---|
| B21-LT_-CF-N001ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-LT_-CF-N012ABCD | 7.00E-08 | CCF 3/4 LEVEL TRANSMITTERS N012A/B/C/D |
| B21-LT_-NO-N012A | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 12A FAILS |
| B21-LT_-NO-N012B | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 12B FAILS |
| B21-LT_-NO-N012C | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 12C FAILS |
| B21-LT_-NO-N012D | 2.40E-05 | WIDE RANGE LEVEL TRANSMITTER 12D FAILS |
| B21-OR_-CF-N012ABCD | 4.00E-08 | CCF 3/4 ORIFICES LINES 12 A/B/C/D (PLUG) |
| B21-OR_-PG-12A | 1.44E-05 | ORIFICE INSTR. LINE 12A FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-12B | 1.44E-05 | ORIFICE INSTR. LINE 12B FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-12C | 1.44E-05 | ORIFICE INSTR. LINE 12C FAILS TO REMAIN OPEN (PLUG) |
| B21-OR_-PG-12D | 1.44E-05 | ORIFICE INSTR. LINE 12D FAILS TO REMAIN OPEN (PLUG) |
| B21-PT_-CF-02HIGH | 3.00E-08 | CCF OF PRESSURE RPV TRANSMITTERS HIGH |
| C51-ACT-CF-APRMSTUCK | 2.10E-07 | CCF APRM DETECTORS STUCK AT POWER LEVEL |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV1/2/3/4 |
| C74-DTM-FC-DIV1 | 6.00E-04 | DTM OF SSLC DIV 1 FAILS TO TRIP |
| C74-DTM-FC-DIV2 | 6.00E-04 | DTM OF SSLC DIV 2 FAILS TO TRIP |
| C74-DTM-FC-DIV3 | 6.00E-04 | DTM OF SSLC DIV 3 FAILS TO TRIP |
| C74-DTM-FC-DIV4 | 6.00E-04 | DTM OF SSLC DIV 4 FAILS TO TRIP |
| C74-SLU-CF-DLTD | 1.08E-05 | CCFs TO OPERATE OF DISCRETE LOGIC TIME DELAYS |
| C74-SLU-FC-LMU12A | 9.00E-04 | LMU 12A FAILURE |
| C74-SLU-FC-LMU12B | 9.00E-04 | LMU 12B FAILURE |
| C74-SLU-FC-LMU12C | 9.00E-04 | LMU 12C FAILURE |
| C74-SLU-FC-LMU12D | 9.00E-04 | LMU 12D FAILURE |
| E50-OR_-CF-4EQUA | 4.00E-08 | CCF OF 4 ORIFICES EQUALIZING LINES (PLUG) |
| E50-OR_-CF-4PLUG | 7.00E-08 | CCF OF 4 OR MORE ORIFICES TO PLUG |
| E50-OR_-CF-7PLUG | 7.00E-08 | CCF OF 7 ORIFICES TO PLUG |
| E50-OR_-CF-D001AE | 7.20E-07 | CCF 2/2 ORIFICES LINES A AND E (PLUG) |
| E50-OR_-CF-D001BCFG | 7.00E-08 | CCF 4/4 ORIFICES LINES B, C, F AND G (PLUG) |
| E50-OR_-CF-D001DH | 7.20E-07 | CCF 2/2 ORIFICES LINES D AND H (PLUG) |
| E50-OR_-CF-PLUGALL | 7.20E-08 | CCF OF ALL ORIFICES TO PLUG |
| E50-OR_-PG-D001A | 1.44E-05 | ORIFICE 001A FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001B | 1.44E-05 | ORIFICE 001B FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001C | 1.44E-05 | ORIFICE 001C FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001D | 1.44E-05 | ORIFICE 001D FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001E | 1.44E-05 | ORIFICE 001E FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001F | 1.44E-05 | ORIFICE 001F FAILS TO REMAIN OPEN (PLUG) [# 5] |

Table 4.6-7
Basic Events

| Basic Event | Prob | Description |
|---------------------------|----------|--|
| E50-OR_-PG-D001G | 1.44E-05 | ORIFICE 001G FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D001H | 1.44E-05 | ORIFICE 001H FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D002A | 1.44E-05 | ORIFICE 002A FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D002B | 1.44E-05 | ORIFICE 002B FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D002C | 1.44E-05 | ORIFICE 002C FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-OR_-PG-D002D | 1.44E-05 | ORIFICE 002D FAILS TO REMAIN OPEN (PLUG) [# 5] |
| E50-POL-RP-POOLA | 3.00E-07 | GDCS POOL A LEAKS CATASTROPHICALLY |
| E50-POL-RP-POOLB | 3.00E-07 | GDCS POOL B LEAKS CATASTROPHICALLY |
| E50-POL-RP-POOLC | 3.00E-07 | GDCS POOL C LEAKS CATASTROPHICALLY |
| E50-SQV-CC-F002A | 3.00E-03 | SQUIB VALVE F002A FAILS TO OPERATE |
| E50-SQV-CC-F002B | 3.00E-03 | SQUIB VALVE F002B FAILS TO OPERATE |
| E50-SQV-CC-F002C | 3.00E-03 | SQUIB VALVE F002C FAILS TO OPERATE |
| E50-SQV-CC-F002D | 3.00E-03 | SQUIB VALVE F002D FAILS TO OPERATE |
| E50-SQV-CC-F002E | 3.00E-03 | SQUIB VALVE F002E FAILS TO OPERATE |
| E50-SQV-CC-F002F | 3.00E-03 | SQUIB VALVE F002F FAILS TO OPERATE |
| E50-SQV-CC-F002G | 3.00E-03 | SQUIB VALVE F002G FAILS TO OPERATE |
| E50-SQV-CC-F002H | 3.00E-03 | SQUIB VALVE F002H FAILS TO OPERATE |
| E50-SQV-CC-F006A | 6.00E-03 | SQUIB VALVE F006A FAILS TO OPERATE IN EXTREME CONDITIONS |
| E50-SQV-CC-F006B | 6.00E-03 | SQUIB VALVE F006B FAILS TO OPERATE IN EXTREME CONDITIONS |
| E50-SQV-CC-F006C | 6.00E-03 | SQUIB VALVE F006C FAILS TO OPERATE IN EXTREME CONDITIONS |
| E50-SQV-CC-F006D | 6.00E-03 | SQUIB VALVE F006D FAILS TO OPERATE IN EXTREME CONDITIONS |
| E50-SQV-CF-4OPEN | 1.50E-05 | CCF OF 4 OR MORE SQUIB VALVES TO OPEN |
| E50-SQV-CF-EQUALLOPEN | 3.00E-05 | CCF OF ALL 4 SQUIB VALVES TO OPEN |
| E50-SQV-CF-F002A/2E | 3.60E-05 | CCF OF SQUIB VALVES F002A/ F002E |
| E50-SQV-CF-F002B/2C/2F/2G | 1.50E-05 | CCF OF SQUIB VALVES F002B/ F002C/ F002F/ F002G |
| E50-SQV-CF-F002D/2H | 3.60E-05 | CCF OF SQUIB VALVES F002D/ F002H |
| E50-SQV-CF-GDCS7OPEN | 1.50E-05 | CCF OF 7 SQUIB VALVES IN GDCS LINES TO OPEN |
| E50-SQV-CF-OPENALL | 1.50E-05 | CCF OF ALL SQUIB VALVES TO OPEN |
| E50-SQV-CO-F009A | 3.50E-03 | SQUIB DELUGE VALVE F009A SPUR. OPENING [#7] |
| E50-SQV-CO-F009B | 3.50E-03 | SQUIB DELUGE VALVE F009B SPUR. OPENING [#7] |
| E50-SQV-CO-F009C | 3.50E-03 | SQUIB DELUGE VALVE F009C SPUR. OPENING [#7] |
| E50-SQV-CO-F009D | 3.50E-03 | SQUIB DELUGE VALVE F009D SPUR. OPENING [#7] |
| E50-SQV-CO-F009E | 3.50E-03 | SQUIB DELUGE VALVE F009E SPUR. OPENING [#7] |
| E50-SQV-CO-F009F | 3.50E-03 | SQUIB DELUGE VALVE F009F SPUR. OPENING [#7] |
| E50-SQV-CO-F009G | 3.50E-03 | SQUIB DELUGE VALVE F009G SPUR. OPENING [#7] |
| E50-SQV-CO-F009H | 3.50E-03 | SQUIB DELUGE VALVE F009H SPUR. OPENING [#7] |
| E50-SQV-CO-F009I | 3.50E-03 | SQUIB DELUGE VALVE F009I SPUR. OPENING [#7] |
| E50-SQV-CO-F009J | 3.50E-03 | SQUIB DELUGE VALVE F009J SPUR. OPENING [#7] |
| E50-SQV-CO-F009K | 3.50E-03 | SQUIB DELUGE VALVE F009K SPUR. OPENING [#7] |

| Table 4.6-7 | | |
|---------------------|-------------|--|
| Basic Events | | |
| Basic Event | Prob | Description |
| E50-SQV-CO-F009L | 3.50E-03 | SQUIB DELUGE VALVE F009L SPUR. OPENING [#7] |
| E50-STR-CF-SPPLUG | 3.75E-04 | CCF FILTER/STRAINER IN PSP TO PLUG |
| E50-STR-PG-K001A | 1.07E-02 | STRAINER/FILTER K001A PLUGS DURING OPERATION |
| E50-STR-PG-K001B | 1.07E-02 | STRAINER/FILTER K001B PLUGS DURING OPERATION |
| E50-STR-PG-K001C | 1.07E-02 | STRAINER/FILTER K001C PLUGS DURING OPERATION |
| E50-STR-PG-K001D | 1.07E-02 | STRAINER/FILTER K001D PLUGS DURING OPERATION |
| E50-SYS-FF-MSLOCA | 3.91E-01 | PROBABILITY OF MEDIUM STEAM LOCA IN GDCS LINES |
| E50-SYS-FF-SSLOCA | 3.90E-01 | PROBABILITY OF SSLOCA IN GDCS LINES |
| E50-UV_-OC-F003A | 1.75E-03 | CHECK VALVE F003A FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003B | 1.75E-03 | CHECK VALVE F003B FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003C | 1.75E-03 | CHECK VALVE F003C FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003D | 1.75E-03 | CHECK VALVE F003D FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003E | 1.75E-03 | CHECK VALVE F003E FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003F | 1.75E-03 | CHECK VALVE F003F FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003G | 1.75E-03 | CHECK VALVE F003G FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F003H | 1.75E-03 | CHECK VALVE F003H FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F007A | 1.75E-03 | CHECK VALVE F007A FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F007B | 1.75E-03 | CHECK VALVE F007B FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F007C | 1.75E-03 | CHECK VALVE F007C FAILS TO REMAIN OPEN OR PLUG |
| E50-UV_-OC-F007D | 1.75E-03 | CHECK VALVE F007D FAILS TO REMAIN OPEN OR PLUG |
| E50-XHE-FO-EQU | 1.61E-03 | OPERATOR FAILS TO ACTUATE GDCS |
| E50-XHE-FO-GDCS | 1.61E-03 | OPERATOR FAILS TO ACTUATE GDCS |
| H23-RMU-FC-CRF0061 | 3.00E-04 | RMU DIV 1 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-CRF0062 | 3.00E-04 | RMU DIV 2 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-CRF0063 | 3.00E-04 | RMU DIV 3 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |

| Table 4.6-7 | | |
|------------------------|-------------|---|
| Basic Events | | |
| Basic Event | Prob | Description |
| H23-RMU-FC-CRF0064 | 3.00E-04 | RMU DIV 4 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-DPSE503 | 3.00E-04 | RMU DIV 3 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-DPSE504 | 3.00E-04 | RMU DIV 4 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-E50DIV1 | 3.00E-04 | RMU DIV 1 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-E50DIV2 | 3.00E-04 | RMU DIV 2 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| T23-POL-RP-SP | 3.00E-07 | SUPPRESSION POOL LEAKS CATASTROPHICALLY |
| XXX-SYS-FF-SCRAMFOLLOW | 7.80E-05 | LOSS OF SCRAM FOLLOW SIGNAL. (2/4 RELAY LOGIC FAILURE |
| XXX-XHE-FO-RPVLDE | 1.61E-02 | OP. FAILS TO RECOG. OR CHECK THE RPV DECREASING LEVEL |

Table 4.6-8
List of System Top Events

| Top Event Name | Description |
|-----------------------|---|
| GE50TOP | 2/8 LINES, 2/3 GDCS POOLS AND 1/4 EQUALIZING LINES |
| GE50TOP1 | 7/8 LINES FAIL TO INJECT INTO RPV |
| GE50TOP2 | 2/3 GDCS POOLS FAIL TO DISCHARGE |
| GE50TOP3 | 4/4 EQUALIZING LINES FAIL TO INJECT INTO RPV |
| GE50TOP4 | 1/3 GDCS POOLS FAILS TO DISCHARGE |
| GE50TOP5 | 3/3 GDCS POOLS FAIL TO DISCHARGE |
| GE50TOP6 | 2/3 GDCS POOLS FAIL TO DISCHARGE AND 4/4 EQUALIZING LINES FAILS TO OPEN |

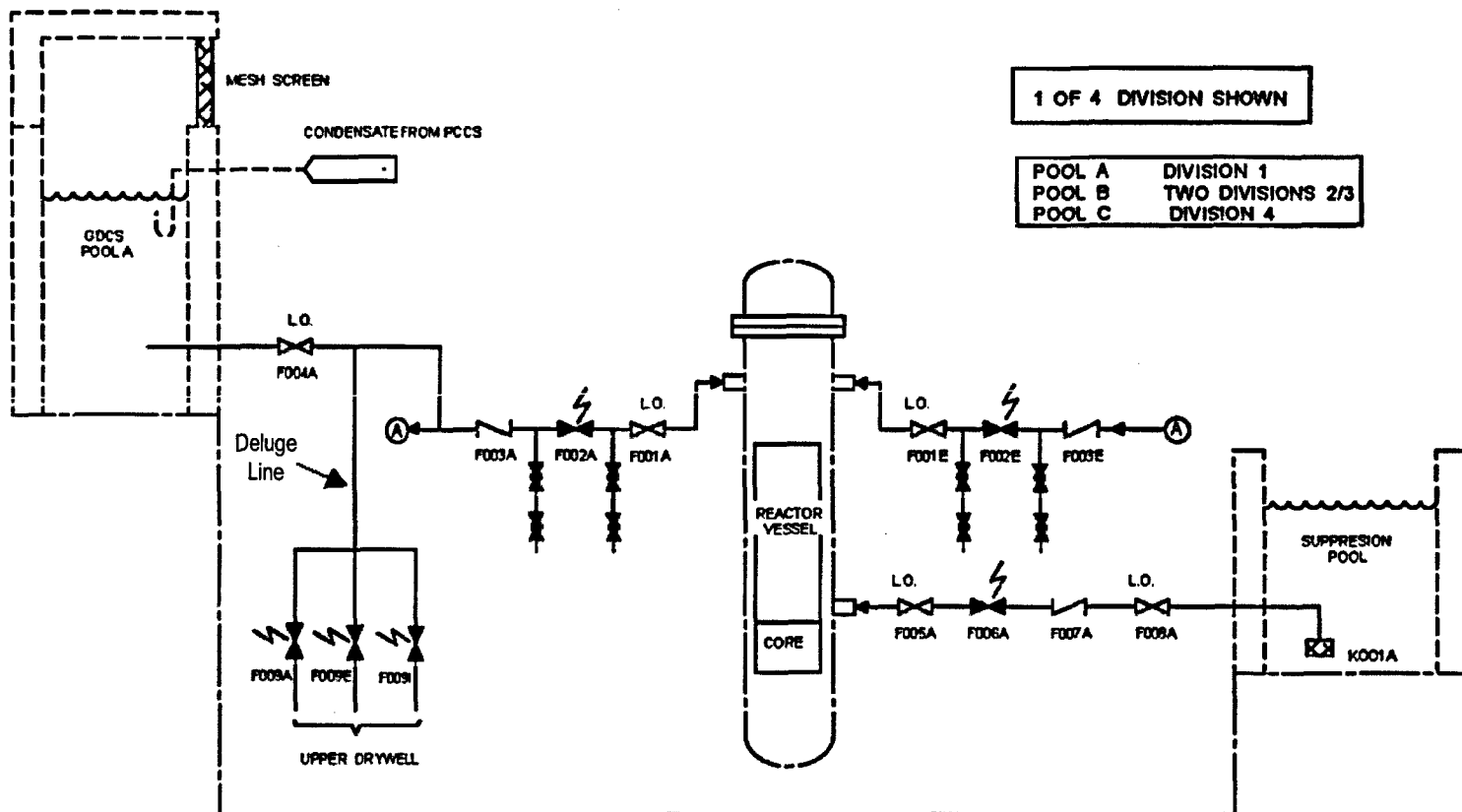


Figure 4.6-1. Simplified Process Flow Diagram for GDCS System

4.7 FUEL AND AUXILIARY POOLS COOLING SYSTEM - (G21) AND FIRE PROTECTION SYSTEM - (U43)

4.7.1 Purpose

For its non-safety related functions the Fuel and Auxiliary Pools Cooling System (FAPCS) is designed to perform pool water cooling, cleanup, and distribution to the following pools (operation in the FAPC mode) in the reactor building and auxiliary fuel building:

- e. Spent Fuel Storage Pool
- f. Cask Loading and Transfer Pool
- g. Inclined Fuel Transfer System Lower Pool
- h. Equipment Storage Pools
- i. Reactor Well
- j. Isolation Condenser and Passive Containment Cooling (IC/PCC) Pool
- k. Gravity Driven Cooling System (GDCS) Pools
- l. Suppression Pool

The FAPCS is also designated as a back-up to provide makeup water from the suppression pool to the reactor pressure vessel (RPV) through one of the main feedwater lines after the reactor has been depressurized down to approximately 5 bar (74 psi), under an accident condition, (operation in low pressure coolant injection - LPCI mode). This enables, in conjunction with the safety-relief valves in the nuclear boiler system, that the FAPCS is capable of providing backup shutdown cooling water.

The FAPCS is also used to drain the reactor well to permit removal and replacement of the drywell head and the Reactor Pressure Vessel (RPV) head during outages, and to refill the reactor well to support refueling and ultimately power generation operations.

For its safety-related functions, the FAPCS is also able to provide cooling water during the long term using a pipe connection to convey water for post loss-of-coolant accident (LOCA) recovery after 72 hours to the IC/PCC pool. It is also capable to convey water using passive-type pipe connections to the spent fuel storage pool at the end of 72 hours post-LOCA period.

For containment isolation purposes, the portions of the FAPCS that penetrate the containment are also considered to be safety-related.

The functions included in the PRA model are:

- Suppression Pool cooling mode.
- Low pressure coolant injection (LPCI mode).

It is possible to inject water through the FAPCS into the vessel from the Fire Protection System (FPS) through the line corresponding to the LPCI mode. In addition, using this system, it is possible to supply water to the upper pools (IC/PCC pool) from the FPS. Both connections are represented in the simplified diagram (Figure 4.7-1(A)). The fault tree corresponding to the

failure of these supplies, both to the vessel and to the IC/PCC pool, is also included within this system description and the associated fault tree models.

4.7.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) The FAPCS automatically actuates in the suppression pool cooling (SPC) mode on a suppression pool high temperature signal (see Section 4.7.4), whenever the system has not been previously aligned for low pressure coolant injection mode.
- (2) The loss of pneumatic supplies do not fail FACPS or FPS. It is assumed that the necessary measures will be taken to make air operated valves not dependent from the Instrument Air System on emergency situation.
- (3) The electrical supplies are provided according to System Dependence Matrix shown in Table 4.7-2.
- (4) The GDCS and suppression pools are instrumented and alarmed such that inadvertent draining of a pool would be immediately obvious to the crew and pool level would be restored.
- (5) The FAPCS and FPS injection capability is adequate to provide adequate core cooling for transients given successful DPV or ADS valve operation, even if containment pressure is at the ultimate containment pressure.
- (6) Information regarding to alarms and instrumentation is preliminary.

4.7.3 System Description

4.7.3.1 Hardware Configuration

Figure 4.7-1(A) shows a simplified diagram of the FAPCS for initial configuration. When the FAPCS operates in the cooling mode, water is drawn from the pools listed in 4.7.1 "a" through "e" using surface level skimmers. It is drained to skimmer surge tanks.

Figure 4.7-1(B) depicts the cooling by the RCCWS of the heat exchangers included in the FAPCS model.

A line leading from the bottom of the skimmer surge tanks A and B conveys water to be cooled or cleaned to the inlet end of a dual-train of series-arranged components. Each train consists of the following components:

- A 100 percent circulating water pump with a check valve in the discharge piping;
- A shell-and-tube-type heat exchanger on the downstream side of the pump discharge check valve (cooling flow is provided by the reactor component cooling water system);
- A filter/demineralizer unit located downstream of the heat exchanger with an inlet motor-operated valve (MOV); a parallel line containing an MOV is provided to bypass the filter/demineralizer.

Both the inlets and outlets of the trains are connected by MOVs F011A/B, F003A/B, F014A/B, and F008A/B, respectively. This arrangement allows each train to take suction flow from

different sources, and allows the train discharge flow to be routed to alternative common headers depending on whether the water flow is routed to the pools listed in 4.7.1 “a” through “e”, or to other possible uses or pools (IC pools, gravity driven cooling system (GDCS) pools, suppression pools, or RWCU/SDC discharge line).

When the FAPCS operates in the LPCI or SPC mode, water is drawn from the suppression pool by means of a line emerging through the inner pool wall into the primary containment and then penetrating through the containment wall. Two air-operated gate valves (F321 and F322) located immediately outside the primary containment are provided on the line for containment isolation purposes.

The downstream piping from the outboard valves merges with the common line conveying water from the IC pools or the GDCS pools to the upstream end of the cooling/cleanup trains. A nitrogen-operated valve inside containment and a motor-operated valve outside containment isolate GDCS pools suction. (Not Modeled).

A common discharge header from the cooling/cleanup trains splits into several lines. In turn, one of those lines splits into multiple branches before penetrating the primary containment. One branch, leading to the suppression pool, contains the motor-operated gate valve, F306, outside the primary containment and the check valve, F307, inside. This branch is aligned for flow when the FAPCS operates in the SPC mode. Another branch, returning water to the GDCS, has a motor-operated gate valve outside the containment and a check valve inside. A second line discharges water to the RWCU/SDC which sends it to the feedwater system to be injected into the RPV when the FAPCS operates in the LPCI mode. This line is provided with an air-operated gate valve, F332, and an air-operated testable check valve, F333, downstream of it. Finally, an air-operated gate valve can isolate a third line returning water to the IC pool.

It is possible to inject water through the FAPCS into the vessel from the Fire Protection System (FPS), through the line corresponding to the LPCI mode. In addition, using this system, it is possible to supply water to the upper pools (IC/PCC pool) from the FPS. Both connections are represented in the simplified diagram (Figure 4.7-1(A)). The fault tree corresponding to the failure of these supplies, both to the vessel and to the IC/PCC pool, is also included within this system description and the associated fault tree models.

4.7.3.2 System Operation

The two modeled operating modes are the following:

- **Suppression Pool Cooling and Cleanup Mode.** Cooling and cleanup of the suppression pool is performed by drawing pool water through a line penetrating the containment and reaches the inlet of the cooling/cleanup trains. From the outlet of these, the processed water returns to a discharge point near the bottom of the suppression pool
- **Low Pressure Coolant Injection (LPCI) Mode.** Under accident conditions, once the reactor has been depressurized to a pressure that is below the nominal shutoff discharge head from the FAPCS pumps, logical processing enables the opening of several valves that automatically align the FAPCS to draw coolant from the suppression pool and pump this coolant into RWCU/SDC piping that leads into one of the two main feedwater lines, thus achieving a low pressure coolant injection.

During normal plant conditions, the FAPCS operates with one pump (C001A) running and the other in standby, to continuously cool, clean, and clarify the water of the fuel storage pool.

During or following a seismic event, the pool water treatment subsystem is bypassed and the cleaning function is not performed.

During other emergency conditions, the standby train can be either manually actuated to work in the LPCI mode, or automatically actuated on suppression pool water high temperature signal to work in the SPC mode. At the same time, the other train continues to operate in the FAPC mode. Table 4.7-1A lists the relevant valves of the system shown in Figure 4.7-1 and lists their status for each system operating mode, assuming train A is working and train B is in standby during normal plant conditions.

4.7.3.3 System Location

The basic components of the FAPCS are all within the fuel and reactor buildings except nitrogen-operated isolation valve F323 (GDCS pools suction) which is located inside the primary containment.

4.7.4 Automatic and Manual Control

Actuation of the FAPCS is performed manually when the system is required to operate in the LPCI mode and automatically (on suppression pool high temperature signal) when operating in the SPC mode.

If automatic actuation fails, the operator is expected to manually actuate the system in the SPC mode.

The signals that trip the FAPCS pumps are the following:

- Low suction pressure
- High/low discharge flow

Position lights for all relevant valves are provided in the control room. Instrument and alarm signals reported in the control room for the FAPCS are given in Table 4.7-1B.

The fault tree system top events are shown in Table 4.7-8.

4.7.5 System Interfaces

The FAPCS components depend on electrical, fluid, and nitrogen/air-support systems as indicated by the dependence matrix given in Table 4.7-2.

Upon loss of electric power, each system's motor-operated valve (MOV) fails "as is", and on loss of air supply, each nitrogen/air-operated valve (N/AOV) fails closed.

The FAPCS shares check valves F102A and F103A of the FDW line A with the RWCUs/SDCS.

4.7.6 System Testing

The FAPCS is non-safety related; therefore, no Technical Specifications apply. However, the following assumptions have been made:

- MOV F303, F306, AOVs F321, F322, F332, all have isolation functions and are tested quarterly.
- The MOVs and the pump of each train are tested quarterly before the operator calls the standby train into operation and the other one is stopped.
- All remaining components are tested every 24 months during refueling.
- Monthly rotation between trains has been assumed.

4.7.7 System Maintenance

Maintenance during normal plant operation is only expected for the components found failed during a test.

For all other components, scheduled maintenance is carried out during reactor refueling.

4.7.8 Common Cause Failures

Common cause failures within the system are summarized in Table 4.7-5.

4.7.9 Fault Tree Analysis

4.7.9.1 Top Event Definitions

The list of the top events used in the PRA is shown in Table 4.7-8.

Four top events have been defined for the FAPCS.

- (1) GG21-0001-_1 for Both FAPCS trains fail SPC operation mode actuation.
- (2) GG21-0001-_4 for Both FAPCS trains fail LPCI operation mode.
- (3) GG21-0001-_5 for Both FAPCS trains LPCI operation mode after ADS.
- (4) GG21-0001-_6 for Both FAPCS trains fail SPC operation mode actuation after ADS.

For Fire Protection System functions three tops have been developed:

- (1) GU43-0001-_1 for U43 fails to inject water into RPV.
- (2) GU43-0001-_2 for Water make up to the IC/PCCS pools from U43 (FPS) fails
- (3) GU43-0001-_3 for U43 fails to inject water into RPV after ADS.

4.7.9.2 Model Assumptions

- Monthly rotation between trains is assumed.
- Failures of branch lines relevant to no-modeled functions are not taken into account.
- Failures of manual valves in the closed position (misposition) are not taken into account, except if single failures affect system function.
- FAPCS includes suction piping from the pools to the inlet of the Cooling/Cleanup Trains, including suction strainers, and piping extending from the downstream end of the Cooling/Cleanup Trains to their discharge ends in each pool, including discharge

diffusers. However, FAPCS excludes the nitrogen and air supply lines, and any equipment inside the pools not attached to FAPCS piping.

- Initially, Train A is considered to be operating in FAPCS mode and train B, on standby. The alignment of each train in each operating mode is shown in Table 4.7-1A.
- The system modeled is shown in the simplified diagram, based on P&ID 107E6299, Rev 1, and corresponds to the SBWR.
- The Type C human actions included in the model are included in the Emergency Operating Procedures or in other operating procedures.
- If a loss of power supply occurs, the circuit breaker of the pump in operation must be closed once power supply is reestablished.
- The correct operation of one of the trains, with its corresponding heat exchanger, is necessary in both SPC and LPCI operating modes, along with the proper alignment of the corresponding intake and discharge.
- For modeling purposes, failure of the automatic actuation signals in the cooling mode of the SP is included in the system limits.
- Manual valve F320 in the SP intake and manual valve F334 in the discharge to FDWA may both be subjected to maintenance, and therefore, failure due to misposition is considered.
- In the RCCWS part of heat exchanger cooling included in the model, the possibility of failure due to incorrect position of the manual valves is not considered, because it is assumed that alarms and indications in the Control Room warn the operators of improper closing of a valve before putting into operation the corresponding train.

4.7.9.3 Fault Tree Description

The following main fault trees are shown in the Appendix B.4.7. The list of the top event gates is shown in Table 4.7-8.

Four fault trees have been developed for the FAPCS:

- (1) Fault tree with top event GG21-0001-_1.
- (2) Fault tree with top event GG21-0001-_4.
- (3) Fault tree with top event GG21-0001-_5.
- (4) Fault tree with top event GG21-0001-_6.

For Fire Protection System functions, three fault trees have been developed:

- (1) GU43-0001-_1 for U43 fails to inject water into RPV.
- (2) GU43-0001-_2 for Water make up to the IC/PCCS pools from U43 (FPS) fails
- (3) GU43-0001-_3 for U43 fails to inject water into RPV after ADS.

4.7.9.4 Human Interactions

A summary of the human errors included as basic events in the system fault trees is reported in Table 4.7-6.

In the fault trees, each one of these errors is connected in a logic OR with a House Event for purposes of sensitivity analysis.

4.7.10 Results of Fault Tree Analysis

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

A summary of the basic events included in the system fault tree are reported in Table 4.7-7.

The importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.7.11 Model Differences for 72-Hour Operation

Operation of the FAPCS beyond 24 hours presents no unique problems. There are no changes to the model for operation to 72 hours other than the mission time.

4.7.12 References

4.7-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.7-1A
Status of the FAPCS Relevant Valves for Each System Operating Mode

| Valve | Operating Modes ^{(1) (2)} | | |
|--------------------|------------------------------------|--------|--------|
| | FAPCS | LPCI | SPC |
| MOV F003A | NO | NO | NO |
| MOV F003B | NC | NC | NC |
| MOV F005A | NO | NO | NO |
| MOV F005B | NC | NC | NC |
| MOV F008A | NO | NO | NO |
| MOV F008B | NC | NC | NC |
| MOV F011A | NC | NC | NC |
| MOV F011B | NC | Opened | Opened |
| MOV F013A | NC | NC | NC |
| MOV F013B | NC | Opened | Opened |
| MOV F014A | NC | NC | NC |
| MOV F0014B | NC | Opened | Opened |
| Isolation AOV F321 | NC | Opened | Opened |
| Isolation AOV F322 | NC | Opened | Opened |
| Isolation MOV F306 | NC | NC | Opened |
| AOV F332 | NC | Opened | NC |

Notes:

- (1) Assumes Train A is operating and Train B is in standby during normal plant operation
- (2) AOV = Air Operated Valve
 MOV = Motor Operated Valve
 NC = Normally Closed
 NO = Normally Open

Table 4.7-1B
Control Room Instrumentation and Alarms

| Controls | |
|--|--|
| IC/PCC pool water level | |
| Make-up water supply to the skimmer surge tank | |
| Displays | |
| Differential pressure between suction and discharge of each pump | |
| Water flow in each train | |
| Temperature upstream and downstream of each heat exchanger | |
| Temperature in the Suppression Pool | |
| Suppression pool water level | |
| IC/PCC pool water level | |
| Spent fuel storage pool water level | |
| Skimmer surge tank level | |
| Alarms | |
| Water flow in each train | |
| Temperature in the Suppression Pool | |
| Low and high suppression pool water level | |
| Low and high skimmer surge tank water level | |
| Low and high IC/PCC pool water level | |
| Low and spent fuel storage pool water level | |

Table 4.7-2
System Dependency Matrix

| | | Support System ⁽¹⁾ | | | | |
|-----------|------|-------------------------------|-------|-------|-------|-----|
| | | Power Supply | | | | IAS |
| | | 480 V AC (R13• 1E) | | | | |
| | | DIV 1 | DIV 2 | DIV 3 | DIV 4 | |
| Component | Type | DIV 1 | DIV 2 | DIV 3 | DIV 4 | |
| F306 | MOV | X | | | | |
| F321 | AOV | X | | | | X |
| F322 | AOV | | X | | | X |
| F332 | AOV | X | | | | X |

| | | Support System | | |
|-----------|------|-----------------|------------------|-------|
| | | Power Supply | | RCCWS |
| Component | Type | 6.9 kV AC (R11) | 480 V AC (R12FB) | |
| C001A | Pump | X | | X |
| C001B | Pump | X | | X |
| F0011A | MOV | | A | |
| F011B | MOV | | B | |
| F014A | MOV | | A | |
| F014B | MOV | | B | |
| F013A | MOV | | A | |
| F013B | MOV | | B | |
| F005A | MOV | | A | |
| F005B | MOV | | B | |
| F003A | MOV | | A | |
| F003B | MOV | | B | |
| F008A | MOV | | A | |
| F008B | MOV | | B | |
| N077A | MOV | | A | |
| N077B | MOV | | B | |
| F051A | MOV | | A | |
| F051B | MOV | | B | |
| F055A | MOV | | A | |
| F055B | MOV | | B | |

- ⁽¹⁾ The FPS used as an RPV injection source or as make up to the IC/PCC pools is considered to have no support system dependencies that interface with the remainder of the plant. Therefore, no dependency matrix is required.

Table 4.7-3
Component Test

| Component Type | Expected Test Interval | Components aligned away from emergency position without automatic return logic |
|---|-------------------------------|---|
| Pumps | Quarterly | None |
| Active valves | Quarterly | None |
| Maintenance valves | 24 Months | None |
| Note: Monthly rotation between trains has been assumed. | | |

Table 4.7-4
Component Maintenance

Maintenance during normal plant operation is only expected for the components found failed during a test. For all other components, scheduled maintenance is carried out during reactor refueling (expected maintenance interval: 24 months).

Table 4.7-5A
List of Common Cause Failures for FAPCS

| Event Name | Unavailability | Description |
|-------------------------|-----------------------|--|
| B21-LT_-CF-N001ABCDLOW | 6.00E-08 | CCF LEVEL TRANSMITTER L2 FAIL LOW |
| G21-ACV-CF-SUCTION | 1.33E-05 | CCF OPEN ACV SUCTION PUMPS LINE |
| G21-FT_-CF-N014A/BLOW | 8.20E-07 | CCF DISCHARGE FLOW TRANSMITTERS |
| G21-HX_-CF-PLUG/LEAK | 1.19E-06 | CCF H/Xs TO PLUG/LEAK |
| G21-MCB-CF-C001A/BCLOSE | 9.29E-05 | CCF CIRCUIT BREAKERS TO CLOSE |
| G21-MOV-CF-CLOSEA | 6.06E-05 | CCF TO CLOSE MOV TRAIN A |
| G21-MOV-CF-OPENA/B | 8.64E-06 | CCF TO OPEN MOV TRAINS A AND B |
| G21-MP_-CR-C001A/B | 2.23E-04 | CCF PUMPS TO RUN |
| G21-MP_-CS-C001A/B | 2.56E-04 | CCF PUMPS TO START |
| G21-PT_-CF-N002A/BLOW | 5.94E-06 | CCF SUCTION PRESSURE TRANSMITTERS |
| G21-STR-CF-SPPLUG | 1.01E-03 | CCF FILTER/STRAINER IN PSP TO PLUG |
| H23-EMS-CF-ALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |
| T62-TT_-CF-SPALL | 4.00E-08 | CCF OF SUPPRESSION POOL TEMP. TRANSMITTERS |

Table 4.7-5B
List of Common Cause Failures for Fps

No common cause failures have been defined for Fire Protection System.

Table 4.7-6A
List of Human Errors for FAPCS

| Event Name | Unavailability | Description |
|---------------------|-----------------------|---|
| G21-XHE-FO-LPCI | 1.61E-03 | OPERATOR FAILS TO ALIGN AND ACTUATE FAPCS IN LPCI MODE |
| G21-XHE-FO-LPCIADS | 1.61E-02 | OPERATOR FAILS TO ALIGN AND ACTUATE FAPCS IN LPCI MODE |
| G21-XHE-FO-SPC | 1.61E-03 | OPERATOR FAILS TO MANUALLY INI. FAPCS IN SPC MODE #3, #4 |
| G21-XHE-FO-SPCADS | 1.61E-02 | OPERATOR FAILS TO MANUALLY INI. FAPCS IN SPC MODE AFTER ADS |
| G21-XHE-MH-F308 | 0.00E+00 | MISPOSITION OF VALVE F308 |
| G21-XHE-MH-F320 | 0.00E+00 | MISPOSITION OF VALVE F320 |
| G21-XHE-MH-F334 | 4.80E-02 | MISPOSITION OF VALVE F334 |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |
| XXX-XHE-FO-LPMAKEUP | 1.61E-01 | OP. FAILS TO RECOG. NEED FOR LOW PRESS. MAKEUP (LOCA) |
| XXX-XHE-FO-SPC | 1.61E-02 | OPERATOR FAILS TO RECOG. NEED FOR SPC |

Table 4.7-6B
List of Human Errors for FPS

| Event Name | Unavailability | Description |
|---------------------|-------------------------|--|
| G21-XHE-MH-F334 | 4.80E-02 | MISPOSITION OF VALVE F334 |
| U43-XHE-FO-LPCI | 1.61E-03 | OPERATOR FAILS TO ACTUATE U43 IN LPCI MODE |
| U43-XHE-FO-LPCIADS | 1.61E-02 | OPERATOR FAILS TO ACTUATE U43 IN LPCI MODE AFTER ADS |
| U43-XHE-FO-MAKEUP | 1.61E-02 | OPERATOR FAILS TO ACTUATE U43 IN MAKE UP MODE |
| U43-XHE-FO-YARD | 5.32E-02 ⁽¹⁾ | OPERATOR FAILS TO MAKE UP FROM YARD AREA |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |
| XXX-XHE-FO-ICPCCS | 1.61E-03 | OPERATOR FAILS TO RECOGNIZE NEED OF MAKE UP TO IC/PCCS POOLS |
| XXX-XHE-FO-LPMAKEUP | 1.61E-01 | OP. FAILS TO RECOG. NEED FOR LOW PRESS. MAKEUP (LOCA) |

⁽¹⁾ The value currently used in the model and reported in Section 6 is 1.77E-03. The revised HEP of 5.32E-02 is the subject of sensitivity calculations.

Table 4.7-7A
List of Basic Events for FAPCS

| Basic Event | Prob | Description |
|-------------------------|----------|---|
| B21-LT_-CF-NO01ABCDLOW | 6.00E-08 | CCF LEVEL TRANSMITTER L2 FAIL LOW |
| B21-UV_-CC-F102A | 1.60E-03 | CHECK VALVE F102A IN FEEDWATER LINE A FAILS TO OPEN |
| B21-UV_-CC-F103A | 1.60E-03 | CHECK VALVE F103A IN FEEDWATER LINE A FAILS TO OPEN |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV 1/2/3/4 |
| C74-DTM-FC-DIV1 | 6.00E-04 | DTM OF SSLC DIV 1 FAILS TO TRIP |
| C74-DTM-FC-DIV2 | 6.00E-04 | DTM OF SSLC DIV 2 FAILS TO TRIP |
| C74-DTM-FC-DIV3 | 6.00E-04 | DTM OF SSLC DIV 3 FAILS TO TRIP |
| C74-DTM-FC-DIV4 | 6.00E-04 | DTM OF SSLC DIV 4 FAILS TO TRIP |
| G21-ACV-CC-F321 | 2.00E-03 | AOV F321 FAILS TO OPERATE TO NOT DEENERG.POS. |
| G21-ACV-CC-F322 | 2.00E-03 | AOV F322 FAILS TO OPERATE TO NOT DEENERG.POS. |
| G21-ACV-CC-F332 | 2.00E-03 | AOV F332 FAILS TO OPERATE TO NOT DEENERG.POS. |
| G21-ACV-CF-SUCTION | 1.33E-05 | CCF OPEN ACV SUCTION PUMPS LINE |
| G21-ACV-TM-F321/F322 | 8.00E-04 | AOVs F321 OR F322 IN MAINTENANCE OR TEST |
| G21-FT_-CF-N014A/BLOW | 8.20E-07 | CCF DISCHARGE FLOW TRANSMITTERS |
| G21-FT_-LO-N014A | 4.08E-05 | DISCHARGE FLOW TRANSMITTER FAILS LOW |
| G21-FT_-LO-N014B | 6.12E-04 | DISCHARGE FLOW TRANSMITTER FAILS LOW |
| G21-HX_-CF-PLUG/LEAK | 1.19E-06 | CCF H/Xs TO PLUG/LEAK |
| G21-HX_-LK-B001A | 2.40E-05 | HEAT EXCHANGER B001A FAILS WHILE OPERATING |
| G21-HX_-LK-B001B | 3.60E-04 | HEAT EXCHANGER B001B FAILS WHILE OPERATING |
| G21-MCB-CF-C001A/BCLOSE | 9.29E-05 | CCF CIRCUIT BREAKERS TO CLOSE |
| G21-MCB-CO-C001A | 1.20E-05 | CIRCUIT BREAKER OF PUMP C001A OPENS SPURIOUSLY |
| G21-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER FAILS TO CLOSE |
| G21-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER OF PUMP C001B FAILS TO CLOSE |
| G21-MOV-CC-F011A | 4.00E-03 | MOTOR OPER. VALVE F011A FAILS TO OPEN |
| G21-MOV-CC-F011B | 4.00E-03 | MOTOR OPER. VALVE F011B FAILS TO OPEN |
| G21-MOV-CC-F013A | 4.00E-03 | MOTOR OPER. VALVE F013A FAILS TO OPEN |
| G21-MOV-CC-F013B | 4.00E-03 | MOTOR OPER. VALVE F013 FAILS TO OPEN |
| G21-MOV-CC-F014A | 4.00E-03 | MOTOR OPER. VALVE F014A FAILS TO OPEN |
| G21-MOV-CC-F014B | 4.00E-03 | MOTOR OPER. VALVE F014B FAILS TO OPEN |

Table 4.7-7A
List of Basic Events for FAPCS

| Basic Event | Prob | Description |
|-----------------------|----------|---|
| G21-MOV-CC-F306 | 4.00E-03 | MOTOR OPER. VALVE F306 FAILS TO OPEN |
| G21-MOV-CF-CLOSEA | 6.06E-05 | CCF TO CLOSE MOV TRAIN A |
| G21-MOV-CF-OPENA/B | 8.64E-06 | CCF TO OPEN MOV TRAINS A AND B |
| G21-MOV-OO-F003A | 4.00E-03 | MOTOR OPERATED VALVE F003A FAILS TO CLOSE |
| G21-MOV-OO-F008A | 4.00E-03 | MOTOR OPER. VALVE F008A FAILS TO CLOSE |
| G21-MOV-TM-F306 | 8.00E-04 | MOV F306 IN MAINTENANCE OR TEST |
| G21-MP_-CR-C001A/B | 2.23E-04 | CCF PUMPS TO RUN |
| G21-MP_-CS-C001A/B | 2.56E-04 | CCF PUMPS TO START |
| G21-MP_-FR-C001A | 3.59E-03 | MOTOR-DRIVEN PUMP C001A FAILS TO RUN, GIVEN START |
| G21-MP_-FR-C001B | 3.59E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO RUN, GIVEN START |
| G21-MP_-FS-C001A | 3.00E-03 | PUMP C001A FAILS TO START |
| G21-MP_-FS-C001B | 3.00E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO START |
| G21-PT_-CF-N002A/BLOW | 5.94E-06 | CCF SUCTION PRESSURE TRANSMITTERS |
| G21-PT_-LO-N002A | 7.92E-06 | SUCTION PRESSURE TRANSMITTER FAILS LOW |
| G21-PT_-LO-N002B | 1.19E-04 | SUCTION PRESSURE TRANSMITTER FAILS LOW |
| G21-STR-CF-SPPLUG | 1.01E-03 | CCF FILTER/STRAINER IN PSP TO PLUG |
| G21-SYS-TM-TRAINB | 3.00E-03 | TRAIN B IN MAINTENANCE |
| G21-UV_-CC-F004B | 2.00E-04 | CHECK VALVE F004B FAILS TO OPEN |
| G21-UV_-CC-F307 | 1.60E-03 | CHECK VALVE F307 FAILS TO OPEN |
| G21-UV_-CC-F331 | 1.60E-03 | CHECK VALVE F331 FAILS TO OPEN |
| G21-UV_-CC-F333 | 1.60E-03 | CHECK VALVE F333 FAILS TO OPEN |
| G21-UV_-CC-F348 | 1.60E-03 | CHECK VALVE F348 FAILS TO OPEN |
| G21-UV_-TM-F332/333 | 8.00E-04 | MAINTENANCE FOR CV F332 OR CV F333 |
| G21-XHE-FO-LPCI | 1.61E-03 | OPERATOR FAILS TO ALIGN AND ACTUATE FAPCS IN LPCI MODE |
| G21-XHE-FO-LPCIADS | 1.61E-02 | OPERATOR FAILS TO ALIGN AND ACTUATE FAPCS IN LPCI MODE |
| G21-XHE-FO-SPC | 1.61E-03 | OPERATOR FAILS TO MANUALLY INI. FAPCS IN SPC MODE #3, #4 |
| G21-XHE-FO-SPCADS | 1.61E-02 | OPERATOR FAILS TO MANUALLY INI. FAPCS IN SPC MODE AFTER ADS |
| G21-XHE-MH-F308 | 0.00E+00 | MISPOSITION OF VALVE F308 |
| G21-XHE-MH-F320 | 0.00E+00 | MISPOSITION OF VALVE F320 |
| G21-XHE-MH-F334 | 4.80E-02 | MISPOSITION OF VALVE F334 |
| H23-EMS-CF-ALL | 1.80E-06 | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 |

Table 4.7-7A
List of Basic Events for FAPCS

| Basic Event | Prob | Description |
|---------------------|----------|--|
| H23-EMS-FC-DIV1 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 1 FAILS TO FUNCTION |
| H23-EMS-FC-DIV2 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 2 FAILS TO FUNCTION |
| H23-EMS-FC-DIV3 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 3 FAILS TO FUNCTION |
| H23-EMS-FC-DIV4 | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV 4 FAILS TO FUNCTION |
| H23-RMU-CF-ALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE |
| H23-RMU-FC-1G21 | 3.00E-04 | RMU 001 FOR FAPCS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-2G21 | 3.00E-04 | RMU 002FOR FAPCS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-LMU9A | 3.00E-04 | LMU 9A FAILURE |
| H23-RMU-FC-LMU9B | 3.00E-04 | LMU 9B FAILURE |
| H23-RMU-FC-LMU9C | 3.00E-04 | LMU 9C FAILURE |
| H23-RMU-FC-LMU9D | 3.00E-04 | LMU 9D FAILURE |
| P21-MOV-CC-F051B | 4.00E-03 | MOTOR OPERATED VALVE P21-F051B FAILS TO OPEN |
| P21-MOV-CC-F055B | 4.00E-03 | MOTOR OPERATED VALVE P21-F055B FAILS TO OPEN |
| P21-MOV-CO-N077B | 1.80E-04 | MOTOR OPERATED VALVE P21-N077A FAILS TO REMAIN CLOSED |
| T23-POL-RP-SP | 3.00E-07 | SUPPRESSION POOL LEAKS CATASTROPHICALLY |
| T62-TT_-CF-SPALL | 4.00E-08 | CCF OF SUPPRESSION POOL TEMP. TRANSMITTERS |
| T62-TT_-NO-001 | 8.40E-06 | SP TEMPERATURE TRANSMITTER 001 (HI TEMP.) FAILS |
| T62-TT_-NO-002 | 8.40E-06 | SP TEMPERATURE TRANSMITTER 002 (HI TEMP.) FAILS |
| T62-TT_-NO-003 | 8.40E-06 | SP TEMPERATURE TRANSMITTER 003 (HI TEMP.) FAILS |
| T62-TT_-NO-004 | 8.40E-06 | SP TEMPERATURE TRANSMITTER 004 (HI TEMP.) FAILS |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |
| XXX-XHE-FO-LPMAKEUP | 1.61E-01 | OP. FAILS TO RECOG. NEED FOR LOW PRESS. MAKEUP (LOCA) |
| XXX-XHE-FO-SPC | 1.61E-02 | OPERATOR FAILS TO RECOG. NEED FOR SPC |

Table 4.7-7B
List of Basic Events for FPS

| Basic Event | Prob | Description |
|---------------------|----------|--|
| B21-UV_-CC-F102A | 1.60E-03 | CHECK VALVE F102A IN FEEDWATER LINE A FAILS TO OPEN |
| B21-UV_-CC-F103A | 1.60E-03 | CHECK VALVE F103A IN FEEDWATER LINE A FAILS TO OPEN |
| G21-ACV-CC-F332 | 2.00E-03 | AOV F332 FAILS TO OPERATE TO NOT DEENERG.POS. |
| G21-ACV-CC-U43 | 2.00E-03 | AIR OPERATED VALVE FAILS TO OPEN |
| G21-UV_-CC-F333 | 1.60E-03 | CHECK VALVE F333 FAILS TO OPEN |
| G21-UV_-CC-U43 | 2.00E-04 | CHECK VALVE FAILS TO OPEN |
| G21-UV_-OC-F331 | 2.16E-04 | CHECK VALVE F331 FAILS TO CLOSE |
| G21-UV_-TM-F332/333 | 8.00E-04 | MAINTENANCE FOR CV F332 OR CV F333 |
| G21-XHE-MH-F334 | 4.80E-02 | MISPOSITION OF VALVE F334 |
| H23-RMU-FC-1MAKEUP | 3.00E-04 | RMU 001 FOR U43 C.R MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-1U43LPCI | 3.00E-04 | RMU 001 FOR U43 C.R. MANUAL ACTUATES. FAILS TO OPERATE |
| H23-RMU-FC-2MAKEUP | 3.00E-04 | RMU 002 FOR U43 C.R. MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-2U43LPCI | 3.00E-04 | RMU 002 FOR U43 C.R. MANUAL ACTUATES. FAILS TO OPERATE |
| U43-SYS-FF-LPCI | 2.40E-02 | U43 HARDWARE FAILURES |
| U43-SYS-FF-YARD | 2.00E-03 | HARDWARE FAILURES IN YARD AREA |
| U43-XHE-FO-LPCI | 1.61E-03 | OPERATOR FAILS TO ACTUATE U43 IN LPCI MODE |
| U43-XHE-FO-LPCIADS | 1.61E-02 | OPERATOR FAILS TO ACTUATE U43 IN LPCI MODE AFTER ADS |
| U43-XHE-FO-MAKEUP | 1.61E-02 | OPERATOR FAILS TO ACTUATE U43 IN MAKE UP MODE |
| U43-XHE-FO-YARD | 5.32E-02 | OPERATOR FAILS TO MAKE UP FROM YARD AREA |
| XXX-XHE-FO-DEPRESS | 1.61E-01 | OPERATOR FAILS TO RECOGNIZE NEED OF DEPRESSURIZATION |
| XXX-XHE-FO-ICPCCS | 1.61E-03 | OPERATOR FAILS TO RECOGNIZE NEED OF MAKE UP TO IC/PCCS POOLS |
| XXX-XHE-FO-LPMAKEUP | 1.61E-01 | OP. FAILS TO RECOG. NEED FOR LOW PRESS. MAKEUP (LOCA) |

Table 4.7-8
List of System Top Events

| Event Name | Event Tree Functional Node | Description |
|-------------------|-----------------------------------|---|
| GG21-0001-_4 | VLF | BOTH FAPCS TRAINS FAIL LPCI OP. MODE (INCLUDES DEPENDENT HEP FOR DEPRESSURIZATION) |
| GG21-0001-_1 | WL | BOTH FAPCS TRAINS FAIL SPC OP. MODE ACTUATION (USED FOR ATWS SEQUENCES) INCLUDES CREDIT FOR AUTO INITIATION OF THE STDBY LOOP (LOOP B) |
| GG21-0001-_5 | VLFL | BOTH FAPCS TRAINS FAIL LPCI OP. MODE AFTER ADS (NO DEPENDENT HEP) |
| GG21-0001-_6 | WLL | BOTH FAPCS TRAINS FAIL SPC OP. MODE ACTUATION AFTER ADS (USED FOR NON-ATWS SEQUENCES) DOES NOT INCLUDE AUTOMATIC INITIATION |
| GU43-0001-_1 | VF | U43 FAILS TO INJECT WATER INTO RPV |
| GU43-0001-_2 | WT | WATER MAKE UP TO THE IC/PCCS POOLS FROM U43 (FPS) FAILS |
| GU43-0001-_3 | VFL | U43 FAILS TO INJECT WATER INTO RPV AFTER ADS |

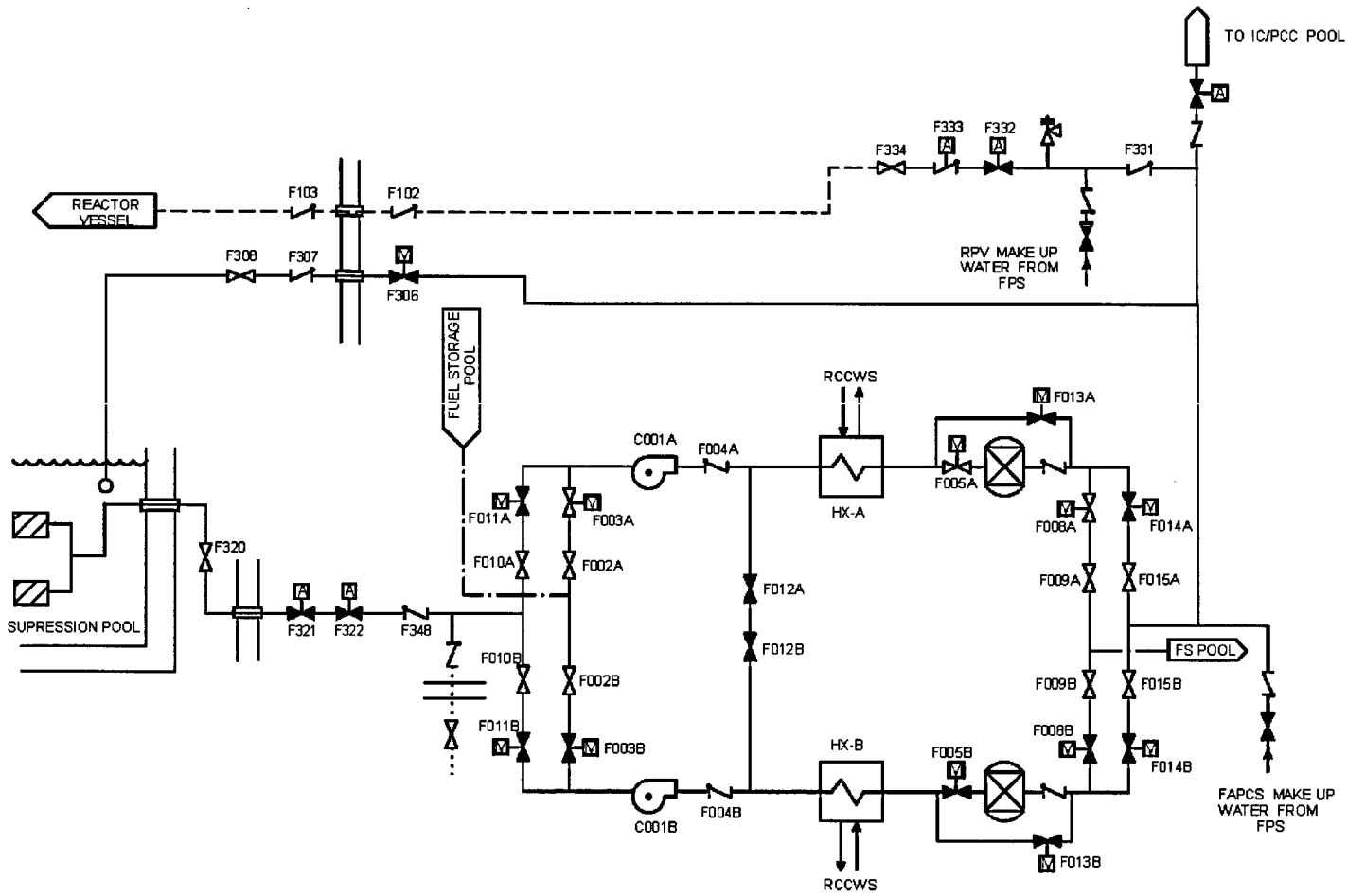


Figure 4.7-1(A). Simplified Process Flow Diagram for FAPCS System

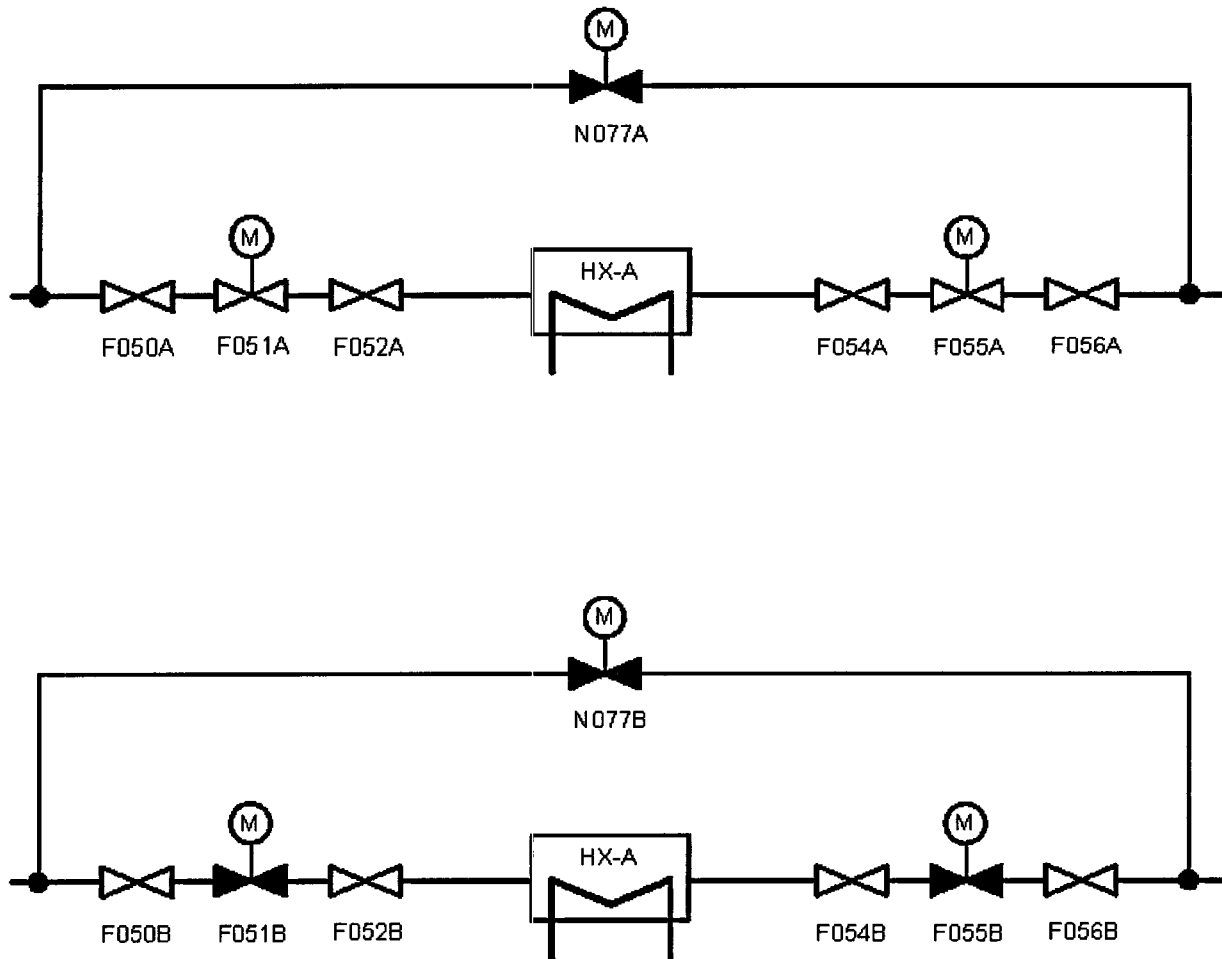


Figure 4.7-1(B). RCCWS-FAPCS Diagram

4.8 REACTOR WATER CLEANUP/SHUTDOWN COOLING SYSTEM – (G31)

4.8.1 Purpose

The Reactor Water Cleanup and Shutdown Cooling System (RWCU/SDCS) is designed:

- To maintain reactor water quality within specified limits during reactor operation (operation in RWCU mode);
- To maintain reactor pressure vessel (RPV) water level during startup or refueling and provide residual heat removal;
- To provide high-pressure cooling of the primary coolant during reactor hot standby condition in order to limit the heat transferred to the isolation condenser (IC) pools (operation in SDC mode) and to bring the plant to its cold shutdown state.

4.8.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) A third air-operated isolation valve is included in each train, outside containment (F3A and F3B). It is assumed that these valves are open during normal operation in both trains, in A in operation and also in B, supposing that the other two (F005B and F006B) fulfill the isolation function.
- (2) It has been assumed that the relevant measures will be taken so that the air-operated valves do not depend on the Instrument Air System in an emergency situation.

4.8.3 System Description

4.8.3.1 Hardware Configuration

Figure 4.8-1(A) shows a simplified diagram of the RWCU/SDCS.

This system consists of two independent and redundant trains of series-arranged components, each one with one regenerative heat exchanger, one non-regenerative heat exchanger, one recirculation water pump driven by adjustable speed drive, and one filter/demineralizer. Reactor coolant in each train is drawn from two points on the reactor:

- From the RPV, via the nozzles located at the top of the core chimney (main suction line);
- From small bottom head nozzles (drain line).

Reactor coolant passes first through the tube side of the regenerative heat exchanger and next through the tube side of the non-regenerative heat exchanger where it is further cooled by secondary coolant supplied by the reactor component cooling water system (RCCWS). The part of the RCCWS (HXs cooling) that has been considered as part of the RWCU/SDC for the purpose of the fault tree model is shown in Figure 4.8-1(B).

Next, the process coolant passes through the RWCU recirculation pump where its pressure is raised to above feedwater inlet pressure levels. Pump discharge next passes into the water treatment units, where it becomes demineralized via the mixed-bed type demineralizer.

Purified cool water is then recycled back through the regenerative heat exchanger and returned to the RPV via the feedwater (FW) piping.

Downstream of the water treatment unit, process flow can be totally bypassed around the regenerative heat exchanger and can also be diverted to the main condenser or to radwaste.

4.8.3.2 System Operation

During normal plant conditions, the RWCU/SDCS operates in the RWCU mode with one pump running at reduced speed (delivering a flow rate of about one percent of the normal feedwater flow) and the other one in standby.

One of the water treatment units is working and the regenerative heat exchanger is not bypassed.

During emergency plant conditions, the system is used in the SDC mode to relieve the Isolation Condenser System (ICS) load, that is, the standby train is aligned, and its pump is started.

The pump speed flow is adjusted automatically to ensure that the heat rejection rate to the RCCWS is maintained within the RCCW limitation and the RCCWS flow rate to each non-regenerative heat exchanger is increased to the maximum. The flow returning to the RPV is bypassed around the regenerative heat exchangers.

4.8.3.3 System Location

Basic components of the RWCU/SDCS are all within the reactor building except valves located inside the containment.

4.8.4 Automatic and Manual Control

Actuation of the RWCU/SDCS in the SDC mode is performed manually.

Position lights for all relevant valves are provided in the control room.

Instrumentation for flow, pressure, temperature, NPSH available, and conductivity are recorded in the main control room, along with suitable alarms.

4.8.5 System Interfaces

The RWCU/SDCS components depend on electrical, fluid, air, and nitrogen support systems such as those indicated by the dependence matrix given in Table 4.8-2.

In case of loss of electric power, MOVs fail as is. In case of loss of nitrogen supply, each NOV fails closed.

Each RWCU/SDCS train injects into a Feedwater line upstream of the containment isolation valves.

4.8.6 System Testing

The RWCU/SDCS is non-safety related; no Technical Specifications exist for it. However, it is assumed that all active train components are tested quarterly before the periodic change in train operation when the standby pump is manually actuated and the other one is stopped. Anticipated testing is described in Table 4.8-3. All remaining components are assumed to be tested every 24 months during the plant shutdown for refueling.

4.8.7 System Maintenance

Maintenance during normal plant operation is only expected to be the unscheduled maintenance of the components in each train found failed during a test. For all other components, scheduled maintenance is carried out during reactor refueling.

4.8.8 Instrumentation and Alarms

The RWCU/SDC System instrumentation for flow, pressure, temperature, NPSH available and conductivity are recorded or indicated in the main control room, along with suitable alarms. The parameters with controls, displays, or alarms in the main control room are shown in Table 4.8-1.

4.8.9 Common Cause Failures

Common Cause Failure (CCF) events are identified within the system and are listed in Table 4.8-5.

4.8.10 Fault Tree Analysis

4.8.10.1 Top Event Definitions

The list of the top events defined for the RWCU/SDCS is shown in Table 4.8-8.

The following top event has been defined for the system:

- (1) GG31TOP for loss of RWCU/SDC taking into account loss-of-coolant accident (LOCA) cases.

4.8.10.2 Model Assumptions

The following assumptions with respect to the design are made:

- (1) It is assumed that the trains are rotated on a monthly basis.
- (2) Failures of manual valves in the closed position (misposition) are not taken into account, except if single failure affects system functions.
- (3) The Type C human actions included in the model are included in the Emergency Operating Procedures or in other operating procedures.

4.8.10.3 Fault Tree Description

The fault tree is shown in annex A.

4.8.10.4 Human Interactions

Human errors are shown in Table 4.8-6. On the fault trees, these errors are connected in a logic OR with a house event for purposes of sensitivity analysis.

4.8.11 Results of Fault Tree Analysis

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

A Summary of the basic events included in the system fault tree and reported in Table 4.8-7.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.8.12 Model Differences for 72-Hour Operation

Operation of the RWCU/SDCS beyond 24 hours presents no unique problems. There are no changes to model for 72-hour operation other than the mission time.

4.8.13 References

4.8-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.8-1
Control Room Instrumentation And Alarms

| Controls |
|--|
| Flow controls valves on RWCU/SDC System suction line from the mid-vessel RPV nozzle and in the bottom drain line |
| Pump speed |
| Position of the RHX shell-side bypass and inlet valves |
| Demineralizer automated filling, draining and resin transfer operations |
| Automatic pressure drop control for demineralizer system |
| Displays |
| Position lights for all relevant valves |
| Process flow, pressure and temperature |
| Flow of the RHX tube side and shell side |
| NPSH available |
| Conductivity of demineralizer influent and effluent |
| Position of the isolation valves |
| RWCU/SDC return to feedwater |
| Demineralizer automated filling, draining and resin transfer operations |
| Alarms |
| High pump cooling water temperature |
| High pump motor winding temperature |
| High pump room temperature |
| Low pump suction flow |
| Containment isolation valves F006A, B and F005A, B not fully open during all modes of operation |
| Containment isolation valves F007A and B not fully open during low pressure shutdown |
| Reactor water high-high temperature at NRHX tube outlet |
| Conductivity in excess of water quality requirements |
| Differential pressure measurements with alarm function for demineralizer and resin trap |

Table 4.8-2
System Dependency Matrix

| Component | | Type | Support System | | | | HPNSS |
|-----------|-----|------|--------------------|-------|-------|-------|-------|
| | | | Power Supply | | | | |
| | | | 480 V AC (R13- 1E) | | | | |
| | | | DIV 1 | DIV 2 | DIV 3 | DIV 4 | |
| F006A | NOV | | | | X | X | |
| F006B | NOV | | | X | | X | |
| F007A | AOV | X | | | | | |
| F007B | AOV | | X | | | | |
| F3A | AOV | X | | | | | |
| F3B | AOV | | X | | | | |

| Component | | Support System | | |
|-----------|------|-----------------|----------------|-------|
| | | Power supply | | RCCWS |
| | | 6.9 kV AC (R11) | 480 V AC (R12) | |
| C001A | Pump | X | | X |
| C001B | Pump | X | | X |
| F009A | MOV | | X | |
| F009B | MOV | | X | |
| F020A | MOV | | X | |
| F020B | MOV | | X | |
| F030A | MOV | | X | |
| F030B | MOV | | X | |
| F036A | MOV | | X | |
| F036B | MOV | | X | |

Table 4.8-3
Component Test

| Component Type | Expected Test Interval | Components aligned away from emergency position without automatic return logic |
|-----------------------|-------------------------------|---|
| Pumps | Quarterly | None |
| Active valves | Quarterly | None |
| Maintenance valves | 24 Months | None |

Note: A monthly train rotation is assumed.

Table 4.8-4
Component Maintenance

Maintenance during normal plant operation is only expected to be the unscheduled maintenance of the components in each train found failed during a test. For all other components, scheduled maintenance is carried out during reactor refueling (expected maintenance interval: 24 months).

Table 4.8-5**List of System Common Cause Failures**

| Basic Event | Prob | Description |
|------------------------|-------------|--|
| B21-LT_-CF-N001ABCDLOW | 6.00E-08 | CCF LEVEL TRANSMITTERS (L2) FAIL LOW |
| G31-ACV-CF-DEENERGA/B | 3.64E-05 | AOV/NOV SPURIOUS TRANSF. TO DEENERG. POSITION |
| G31-FT_-CF-N002ABLOW | 3.06E-05 | CCF SUCTION FLOW TRANSMITTERS FAIL LOW |
| G31-MCB-CF-CLOSE | 7.62E-05 | CCF CIRCUIT BREAKER TO CLOSE |
| G31-MOV-CF-F018/62/63A | 6.06E-05 | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018A, F062A AND F063A TO CLOSE |
| G31-MOV-CF-F018/62/63B | 6.06E-05 | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018B, F062B AND F063B TO CLOSE |
| G31-MOV-CF-OPENA | 2.11E-05 | MOV _s (3) TRAIN A CCF TO OPEN |
| G31-MOV-CF-OPENA/B | 8.64E-06 | CCF MOV TO OPEN |
| G31-MOV-CF-OPENB | 2.11E-05 | MOV _s (3) TRAIN B CCF TO OPEN |
| G31-MP_-CR-RUN | 1.49E-05 | CCF OF RWCU PUMPS TO RUN |
| G31-MP_-CS-C001A/B | 1.96E-04 | CCF TO RESTART PUMPS |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| T64-TT_-CF-HIGH | 2.28E-06 | CCF TEMPERATURE TRANSMITTERS IN STEAM TUNNEL FAIL HIGH |

Table 4.8-6
List of System Human Errors

| Basic Event | Prob | Description |
|--------------------|-------------|---|
| G31-XHE-FO-SDC | 1.77E-02 | OPERATOR FAILS TO ACTUATE SDC MODE |
| G31-XHE-FO-SDCMSL | 3.22E-02 | OPERATOR FAILS TO ACTUATE SDC MODE |
| G31-XHE-FO-SDCSLCS | 1.77E-01 | OPERATOR FAILS TO ACTUATE SDC MODE AFTER SLCS ACTUATION |

| Table 4.8-7 | | |
|------------------------------------|-------------|--|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| B21-LT_-CF-N001ABCDLOW | 6.00E-08 | CCF LEVEL TRANSMITTERS (L2) FAIL LOW |
| B21-UV_-CC-F102A | 1.60E-03 | CHECK VALVE F102A IN FEEDWATER LINE A FAILS TO OPEN |
| B21-UV_-CC-F102B | 1.60E-03 | CHECK VALVE #1 IN FEEDWATER LINE B FAILS TO REOPEN |
| B21-UV_-CC-F103A | 1.60E-03 | CHECK VALVE F103A IN FEEDWATER LINE A FAILS TO OPEN |
| B21-UV_-CC-F103B | 1.60E-03 | CHECK VALVE #2 IN FEEDWATER LINE B FAILS TO REOPEN |
| G31-ACV-CC-F007A | 2.00E-03 | AIR OPER. VALVE F007A FAILS TO OPEN |
| G31-ACV-CC-F007B | 2.00E-03 | MOTOR OPER. VALVE F007B FAILS TO OPEN |
| G31-ACV-CF-DEENERGA/B | 3.64E-05 | AOV/NOV SPURIOUS TRANSF. TO DEENERG. POSITION |
| G31-ACV-OC-F006A | 3.60E-06 | AIR OPERATED VALVE F006A SPUR. TRANSF. TO DEENERG. POSIT. |
| G31-ACV-OC-F006B | 5.40E-05 | AIR OPERATED VALVE F006B SPUR. TRANSF. TO DEENERG. POSIT. |
| G31-ACV-OC-F3A | 5.40E-05 | AIR OPERATED VALVE F3A CLOSSES SPURIOUSLY |
| G31-ACV-OC-F3B | 5.40E-05 | AIR OPERATED VALVE F3B CLOSSES SPURIOUSLY |
| G31-FT_-CF-N002ABLOW | 3.06E-05 | CCF SUCTION FLOW TRANSMITTERS FAIL LOW |
| G31-FT_-LO-N002A | 4.08E-05 | FLOW TRANSMITTER N002A FAILS LOW |
| G31-FT_-LO-N002B | 6.12E-04 | SUCTION FLOW TRANSMITTER N002B FAILS LOW |
| G31-HX_-LK-B002A | 2.40E-05 | HEAT EXCHANGER A FOR RWCU/SDCS FAILS WHILE OPERATING |
| G31-HX_-LK-B002B | 3.60E-04 | HEAT EXCHANGER B FOR RWCU/SDCS FAILS WHILE OPERATING |
| G31-MCB-CF-CLOSE | 7.62E-05 | CCF CIRCUIT BREAKER TO CLOSE |
| G31-MCB-CO-C001A | 1.20E-05 | CIRCUIT BREAKER OF PUMP A OPENS SPURIOUSLY |
| G31-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER OF PUMP A FAILS TO RECLOSE |
| G31-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER OF PUMP B FAILS TO CLOSE |
| G31-MOV-CC-F009A | 4.00E-03 | MOTOR OPER. VALVE F009A FAILS TO OPEN |
| G31-MOV-CC-F009B | 4.00E-03 | MOTOR OPER. VALVE F009B FAILS TO OPEN |
| G31-MOV-CC-F020A | 4.00E-03 | MOTOR OPER. VALVE F020A FAILS TO OPEN |
| G31-MOV-CC-F020B | 4.00E-03 | MOTOR OPER. VALVE F020B FAILS TO OPEN |
| G31-MOV-CC-F030A | 4.00E-03 | MOTOR OPER. VALVE F030A FAILS TO OPEN |
| G31-MOV-CC-F030B | 4.00E-03 | AIR OPER. VALVE F030B FAILS TO OPEN |
| G31-MOV-CF-F018/62/63A | 6.06E-05 | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018A, F062A AND F063A TO CLOSE |
| G31-MOV-CF-F018/62/63B | 6.06E-05 | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018B, F062B AND F063B TO CLOSE |
| G31-MOV-CF-OPENA | 2.11E-05 | MOV's (3) TRAIN A CCF TO OPEN |
| G31-MOV-CF-OPENA/B | 8.64E-06 | CCF MOV TO OPEN |
| G31-MOV-CF-OPENB | 2.11E-05 | MOV's (3) TRAIN B CCF TO OPEN |
| G31-MP_-CR-RUN | 1.49E-05 | CCF OF RWCU PUMPS TO RUN |

| Table 4.8-7 | | |
|-----------------------------|----------|--|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| G31-MP_-CS-C001A/B | 1.96E-04 | CCF TO RESTART PUMPS |
| G31-MP_-FR-C001A | 2.40E-04 | MOTOR-DRIVEN PUMP A FAILS TO RUN, GIVEN START |
| G31-MP_-FR-C001B | 2.40E-04 | MOTOR-DRIVEN PUMP B FAILS TO RUN, GIVEN START |
| G31-MP_-FS-C001A | 2.30E-03 | MOTOR-DRIVEN PUMP A FAILS TO RESTART |
| G31-MP_-FS-C001B | 2.30E-03 | MOTOR-DRIVEN PUMP B FAILS TO START |
| G31-OR_-PG-RWA | 1.44E-05 | ORIFICE A FAILS TO REMAIN OPEN (PLUG) |
| G31-OR_-PG-RWB | 2.16E-04 | ORIFICE B FAILS TO REMAIN OPEN (PLUG) |
| G31-SYS-TM-B | 3.00E-03 | RWCU/SDCS TRAIN B IN MAINTENANCE OR OUT OF SERVICE |
| G31-UV_-CC-F017B | 2.00E-04 | CHECK VALVE F017B FAILS TO OPEN |
| G31-UV_-CC-F035B | 1.60E-03 | CHECK VALVE F035B FAILS TO OPEN |
| G31-UV_-CC-F094B | 1.60E-03 | CHECK VALVE F094B FAILS TO OPEN |
| G31-XHE-FO-SDC | 1.77E-02 | OPERATOR FAILS TO ACTUATE SDC MODE |
| G31-XHE-FO-SDCMSL | 3.22E-02 | OPERATOR FAILS TO ACTUATE SDC MODE |
| G31-XHE-FO-SDCSLCS | 1.77E-01 | OPERATOR FAILS TO ACTUATE SDC MODE AFTER SLCS ACTUATION |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| H23-RMU-FC-001G31 | 3.00E-04 | RMU 001 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-002G31 | 3.00E-04 | RMU 002 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| N21-SYS-FF-FWAL | 5.00E-01 | PROBABILITY OF LSLOCA IN FW TRAIN A PIPE INSIDE CONTAINMENT |
| N21-SYS-FF-FWBL | 5.00E-01 | PROBABILITY OF LSLOCA IN FW TRAIN B PIPE INSIDE CONTAINMENT |
| P21-ACV-CO-F042A | 4.80E-07 | AIR OPERATED VALVE P21-F042A SPURIOUSLY TRANSFER TO ENERGIZED POSITION |
| P21-ACV-CO-F042B | 7.20E-06 | AIR OPERATED VALVE P21-F042B SPURIOUSLY TRANSFER TO ENERGIZED POSITION |
| P21-MOV-CC-F039B | 4.00E-03 | MOTOR OPERATED VALVE P21-F039B FAILS TO OPEN |
| P21-MOV-CC-F040B | 4.00E-03 | MOTOR OPERATED VALVE P21-F040B FAILS TO OPEN |
| P21-MOV-OC-F039A | 3.36E-06 | MOTOR OPERATED VALVE P21-F039 FAILS TO REMAIN OPEN |
| P21-MOV-OC-F040A | 3.36E-06 | MOTOR OPERATED VALVE P21-F040A FAILS TO REMAIN OPEN |
| T64-TT_-CF-HIGH | 2.28E-06 | CFE TEMPERATURE TRANSMITTERS IN STEAM TUNNEL FAIL HIGH |

Table 4.8-8
List of System Top Events

| Top Event | Description |
|-----------|------------------|
| G31TOP | LOSS OF RWCU/SDC |

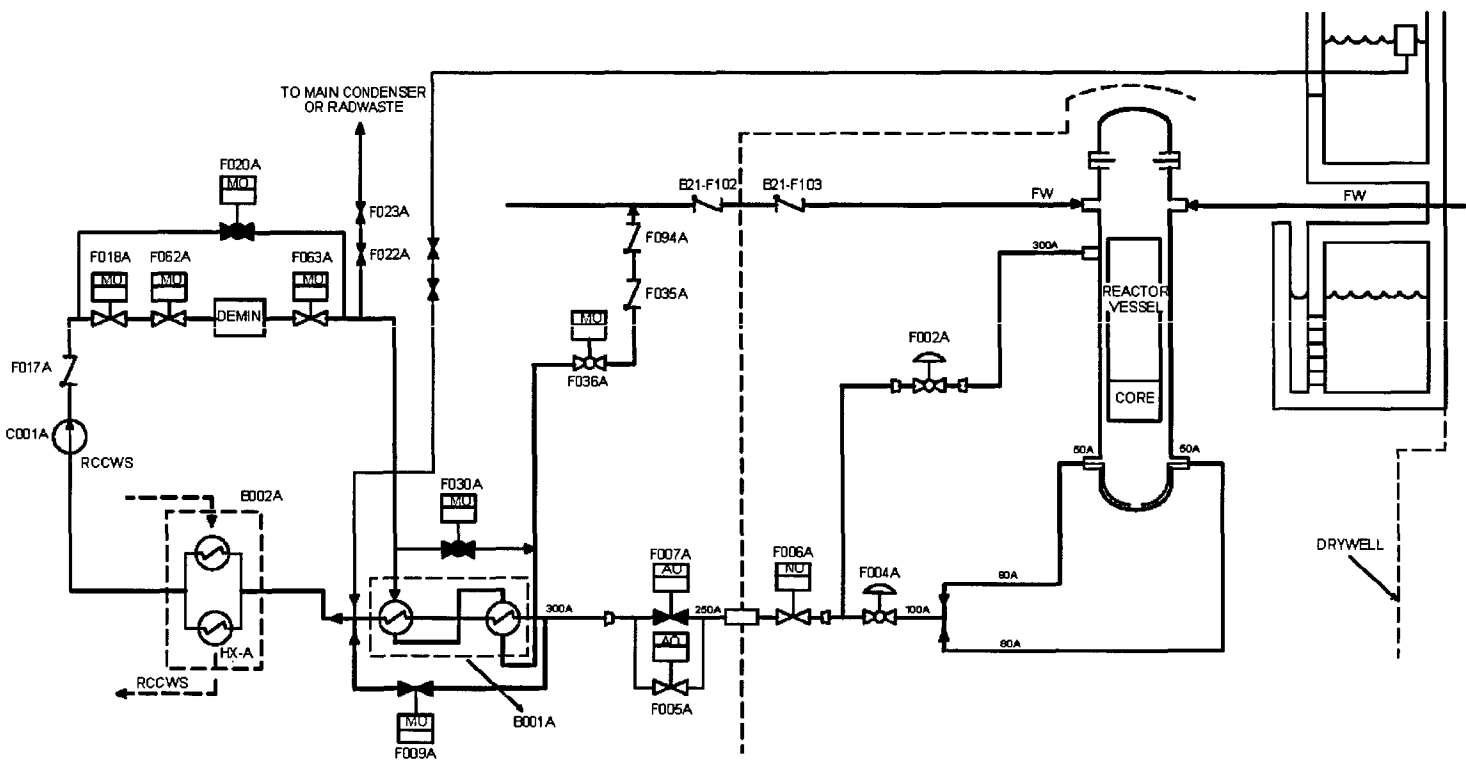


Figure 4.8-1(A). Simplified Process Flow Diagram for RWCU/SDC System

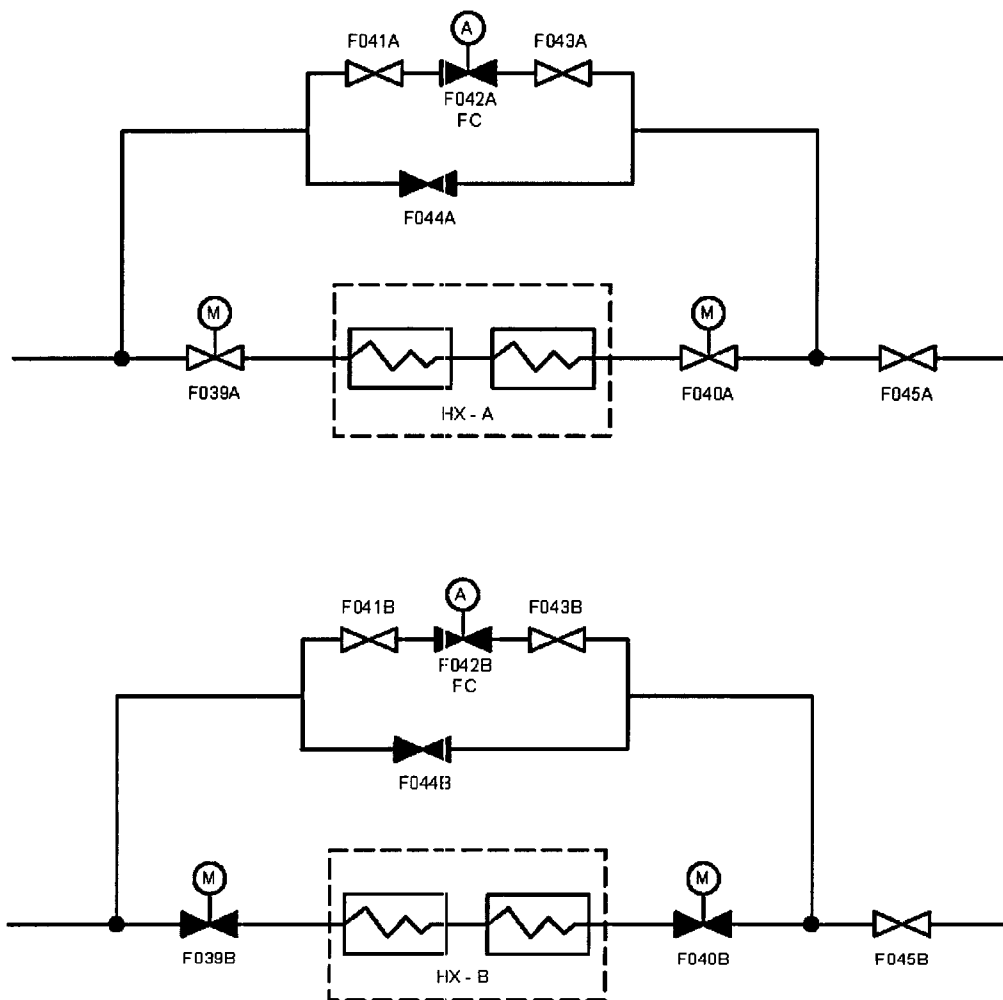


Figure 4.8-1(B). RCCWS/SDCS Simplified Process Flow Diagram

4.9 FEEDWATER AND CONDENSATE SYSTEM - (N21 and P22)

4.9.1 Purpose

This section includes the following parts of the plant: feedwater and condensate system, condenser hotwell makeup, main condenser and Circulating Water System (CIRC), steam bypass, and the Turbine Component Cooling Water system (TCCWS).

The purpose of the feedwater and condensate system is to receive condensate from the condenser hotwell and deliver feedwater to the reactor at the required flow rate, pressure, and temperature, both during normal operation and after shutdown.

The main condenser is the steam cycle heat sink, which receives and condenses the steam coming from the turbine or the steam bypass valves. It is utilized as a heat sink in the initial phase of reactor cooldown during a normal plant shutdown.

Condenser hotwell makeup from the Condensate Storage & Transfer System (CS&TS) and condensate reject to the Condensate Storage & Transfer System tank (CST) are provided to maintain condenser hotwell level. The makeup water to the CST is made through the demineralized water system.

The turbine bypass system (TBS) provides the capability to discharge steam from the main steam lines directly to the condenser.

The TCCW system provides cooling water to all turbine island auxiliary equipment.

4.9.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) Based on the detail of design available, assumptions have been made in order to construct a model corresponding to the level of analysis generally adopted for this study.
- (2) Valves F018 and F016 have been considered air operated valves, that fail to remain open on a loss of air supply.

4.9.3 System Description

4.9.3.1 Hardware Configuration

The feedwater and condensate system include the following main components:

- Four adjustable speed (33 to 45% each) motor-driven feedwater pumps.
- Four feedwater recirculation lines, one for each pump.
- Bypass lines for the feedwater pumps and the high-pressure heaters.
- Parallel strings, seven stages, low (four parallel strings) and high (two parallel strings) pressure feedwater heaters.
- Condensate filters and demineralizers.
- Four (33 to 37% each) percent motor-driven condensate pumps.

- One condensate recirculation line to the condenser.
- One condensate line to the CS&TS.
- One makeup line from the CST to the hotwell.

The TCCW system is composed of three 50 percent pumps, two 100 percent heat exchangers, a surge tank, and the related valves.

The current configuration for the CIRC has four 25 percent operating pumps. The turbine bypass system (TBS) has a 110 percent capacity, which allows full load reject capability without causing a reactor scram or opening any of the safety relief valves, and opens on a signal of high pressure in the steam line.

Figures 4.9-1, 2, 3, and 4 show simplified diagrams of the feedwater and condensate (Trains A and B), TCCWS, and circulating water systems, respectively, with the main components indicated.

The part of the Plant Service Water system that has been considered as part of the TCCWS for the purpose of the model is highlighted in Figure 4.9-5.

4.9.3.2 System Operation

During normal operation, three feedwater pumps and three condensate pumps are normally running at 33 percent flow. Upon a signal of low flow on the feedwater discharge line, the pump speed increases.

After a shutdown, two cases are possible:

- The system is still able to inject water at high pressure in the RPV with at least one feedwater pump and one condensate pump. If the Power Conversion System (PCS) is not available, the condenser makeup is also needed.
- The feedwater is not able to inject. In this case the operator can align the condensate pumps to provide the cooling water at low pressure with makeup from the condenser hotwell after opening the recirculation line. If the Power Conversion System (PCS) is not available, the condenser makeup is also needed.

A bypass is provided around the feedwater pumps to allow feedwater to be supplied to the reactor during startup or when the reactor is depressurized by using only the condensate pumps.

The condensate pumps take suction from the condenser hotwell and discharge into one common header, which feeds the filters, demineralizers, and auxiliary coolers. The recirculation line, downstream of the auxiliary condensers, allows the condensate pump flow to return to the main condenser, thus avoiding a condensate pump trip when the feedwater pumps stop.

The condenser hotwell level is maintained by a set of two parallel modulating air-operated valves (one for normal and one for emergency conditions). The set allows water to flow down by gravity from the CS&TS to the hotwell. Another set of valves allows for pumping water back to the CS&TS using the drain pump condenser.

Direct contact FW heater – Low pressure heater stage No. 5 is combined with a large horizontal storage tank (feedwater tank). The direct contact FW heater receives the condensate from the outlet of the low-pressure close contact heat up strings. Heating steam flows to the direct contact

FW heater to raise the temperature of the condensate to saturation level. Non-condensibles are vented through an orifice and valve assembly to the main condenser.

4.9.3.3 Component Location

All the main components and most of the piping related to this system are located within the non-safety related turbine building. Pumps of the CIRC are located in pump intake structure of the cooling tower basin.

4.9.4 Automatic and Manual Control

4.9.4.1 Automatic Control

Under normal operating conditions, system operation is automatic. The feedwater flow is regulated by the Feedwater Control system that utilizes measurements of steam flow, feedwater flow, and reactor level to regulate the feedwater pump speed. See also Section 7.7.3 of the Design Control Document, Reference 4.9-1.

After a scram, two of the three running pumps are automatically stopped in order to maintain the reactor level below Level 8.

A runback of all the feedwater pumps demand to zero is initiated upon the condition of high reactor water, Level 8. A trip of main feedwater pumps and condensate pumps is generated on high reactor water Level 9. The signal is derived from the narrow range level sensors.

An automatic and redundant level control system controls the level in all FW heaters, MS/RH drain tanks, direct contact feedwater heater and the condenser hotwells.

4.9.4.2 Manual Control

If manual recovery of automatic actuation failures is feasible, it is considered.

4.9.5 System Interfaces

Tables 4.9-2A, B and C show the dependency matrices for the feedwater and condensate, circulating water systems and TCCW, respectively.

The supporting systems are the 13.8 kV AC and 6.9 kV AC that supplies power to the pumps, the 480 V AC for MOV actuation, and the 125 V DC non-1E for the logic and the Instrument Air System (IAS) for air-operated valves.

The TCCW is a supporting system for the Feedwater and the Condensate systems, and needs the Plant Service Water System (PSWS) for cooling as well as the electrical power for the pumps (13.8 kV, 6.9 kV AC), the MOVs (480 V AC) and the logic (125 V DC).

4.9.6 System Testing

Monthly rotation of trains is assumed in Condensate, Feedwater and TCCWS.

The components on the bypass lines and the main lines (motor-operated isolation valves) are assumed to be not testable during operation.

Table 4.9-3 lists the components subject to test and their expected frequency of test.

4.9.7 System Maintenance

Maintenance of one branch at a time is allowed during normal operation for the feedwater, condensate, and TCCW systems.

4.9.8 Common Cause Failures

The possible types of common cause failures (CCFs) within the system are summarized in Table 4.9-5.

4.9.9 Fault Tree Analysis

4.9.9.1 Top Event Definitions

The following fault trees were developed:

GN21-0050-_1: PCS failure

GN21-0051-_1: High-pressure injection with feedwater fails.

GP22-0001-_1_: Loss of cooling from TCCWS System.

GN71-0001-_1: Circulating water system fails to provide condensate cooling.

4.9.9.2 Modeling Assumptions

- The system includes the condenser hotwell as well as its supply system from the condensate storage tank (included) and the turbine bypass, and the condensate and feedwater system.
- The CIRC and TCCWS are also modeled, including the cooling part of the heat exchangers corresponding to the PSWS indicated in Figure 4.9-5.
- The failure of the valves associated with the pumps operating during normal operation is not considered.
- Misalignment failures are not considered for valves associated with the pump that is in standby
- Monthly rotation of the standby train for Condensate, Feedwater and TCCWS is assumed.
- Type C human actions are included in the Emergency Operating Procedures or in other operating procedures.
- The failures of the main steam valves are considered as encompassed in the special event N21-SYS-FF-BYPASS corresponding to the failure of the turbine bypass.
- Failures of the condenser makeup system valves are considered as the cause of failure of the function.
- The failure due to maintenance of one condensate branch and of one feedwater branch is considered, without possibility of coinciding with another branch in maintenance, because the other three would be in operation if the plant were at 100% power.

(Operation at reduced power levels for maintenance on multiple trains is considered to produce negligible unavailability.)

4.9.9.3 Fault Tree Description

- Top event GN21-0050-_1 models the PCS failure. To achieve PCS success, operation of the turbine bypass is required for water supply, as well as the operation of at least one condensate pump and one feedwater pump for injection to the vessel. Condenser cooling by the CIRC is also required. However, condenser makeup is not required in this case, because the condenser is operating within the cycle.
- Top event GN21-0051-_1 models failure of high-pressure injection. Success of this function requires operation of at least one condensate pump and one feedwater pump for injection, as well as condenser water supply by gravity from the CS&TS.
- Top event GN71-0001-_1 models the circulating water system failure to provide condenser cooling. For this function to be successful, operation of at least one of the circulating water pumps is required.
- Top event GP22-0001-_1, an auxiliary system, evaluates the unavailability of the TCCW system; it is not common to any other front line system. For this function to be successful, operation of at least two of the three pumps is required, as well as operation of one of the two heat exchangers.

4.9.9.4 Human Interaction

The human interactions and their associated error probabilities that are considered in the analysis are the following as shown in Tables 4-9.6A, -6B, and -6C. No credit is taken for FW operation after runback for shutdown or ATWS conditions.

4.9.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.9-7.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This qualification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.9.11 Model Differences for 72-Hours Operation

The only change necessary for the model is the adjustment of the mission time to 72 hours.

4.9.12 References

- 4.9-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.9-1
Control Room Instrumentation and Alarms

| Controls | |
|--|--|
| Condensate and Feedwater flow | |
| Condenser hotwell level | |
| Automatic FW pump recirculation flow | |
| | |
| Displays | |
| Steam flow | |
| Feedwater flow | |
| Reactor water level | |
| Hotwell level | |
| CST level | |
| FW pump suction and discharge pressure | |
| | |
| Alarms | |
| ---- | |

Table 4.9-2A
Condensate and Feedwater Dependency Matrix

| | | Support System | | | | |
|-----------|------------|----------------|--------|--------|--------|-------|
| | | Power Supply | | | | |
| | | 13.8 kV AC | | | | |
| Component | Type | Bus A1 | Bus B1 | Bus C1 | Bus D1 | TCCWS |
| N21C001 | Cond. Pump | X | | | | X |
| N21C002 | Cond. Pump | | X | | | X |
| N21C003 | Cond. Pump | | | X | | X |
| N21C004 | Cond. Pump | | | | X | X |
| N21C010 | FW Pump | X | | | | X |
| N21C020 | FW Pump | | X | | | X |
| N21C030 | FW Pump | | | X | | X |
| N21C040 | FW Pump | | | | X | X |

| | | Support System | | | | |
|-----------|------|-----------------------------|----------|----------|--------------------|-----|
| | | Power Supply | | | 480 V AC Non-1E | IAS |
| | | Uninterruptible 208/120 VAC | | | | |
| Component | Type | R13-A2-1 | R13-B2-1 | R13-C1-3 | | |
| F008 | MOV | | | | X | |
| F023 | AOV | X | | | | X |
| F026 | AOV | | X | | | X |
| F016 | AOV | | | X | | X |
| F061 | MOV | | | X | | |

Table 4.9-2B
Circulating Water System Dependency Matrix

| | | Support System | | | | TCCWS |
|-----------|------|----------------|--------|--------|--------|-------|
| | | Power Supply | | | | |
| | | 13.8 kV AC | | | | |
| Component | Type | Bus A1 | Bus B1 | Bus C1 | Bus D1 | |
| C001 | Pump | X | | | | X |
| C002 | Pump | | X | | | X |
| C003 | Pump | | | X | | X |
| C004 | Pump | | | | X | X |

Table 4.9-2C
Turbine Component Cooling Water System Dependency Matrix

| | | Support System | | |
|-----------|----------------|----------------|--------|-----|
| | | Power Supply | | PSW |
| | | 6.9 kV AC | | |
| | | Bus A3 | Bus B3 | |
| Component | Type | | | |
| C001 | Pump | X | | |
| C002 | Pump | | X | |
| C003 | Pump | | X | |
| B001 | Heat exchanger | | | X |
| B002 | Heat exchanger | | | X |

Table 4.9-3
Component Test

| Component Type | Expected Test Interval | Components aligned away from emergency position without automatic return logic |
|-----------------------|-------------------------------|---|
| F061 | 2 years | - |
| F016 | 2 years | - |
| F023 | 2 years | - |
| F026 | 2 years | - |
| F008 | 1 month | - |
| Transmitters | 2 years | - |

Note: monthly rotation between trains in Feedwater, Condensate and TCCWS has been assumed.

Table 4.9-4
Component Maintenance

Maintenance of one branch at time is allowed during normal operation for the feedwater, condensate and TCCW systems. The failure due to maintenance of one condensate branch and of one feedwater branch is considered, without possibility of coinciding with another branch in maintenance, because the other three would be in operation if the plant were at 100% power. (Operation at reduced power levels for maintenance on multiple trains is considered to produce negligible unavailability.).

Table 4.9-5A
Common Cause Failures for Condensate and Feedwater System

| Event Name | Unavailability | Description |
|--------------------|-----------------------|--|
| B21-LT_-CF-L8NO | 1.20E-07 | CCF LEVEL TRANSMITTERS L8 (FAILURE TO RESPOND TO CHANGE) |
| B21-LT_-CF-L9SPU1 | 1.20E-07 | SPURIOUS L9 SIGNAL CCF |
| C62-DTM-CF-N1EALL | 5.50E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| H23-RMU-CF-N1EALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (NO 1E) TO OPERATE |
| N21-ACV-CF-MKUPA/B | 1.08E-04 | CCF TO OPERATE AOVs TRAINS A & B |
| N21-LT_-CF-FWTKLOW | 6.00E-08 | CCF OF LEVEL TRANSMITTERS (LOW) IN FW STORAGE TANK |
| N21-LT_-CF-FWTKNO | 4.38E-05 | CCF FEEDWATER STORAGE TANK LEVEL TRANSMITTERS |
| N21-LT_-CF-HWLOW | 6.00E-08 | CCF HOTWELL LEVEL TRANSMITTERS TO STOP PUMPS |
| N21-LT_-CF-HWNO | 4.38E-05 | CCF HOTWELL LEVEL TRANSMITTERS |
| N21-MPF-CR-RUN | 3.00E-06 | FEEDWATER PUMPS CCF TO RUN |
| N21-MP_-CR-COND | 3.00E-06 | CONDENSATE PUMPS CCF TO RUN |

Table 4.9-5B
Common Cause Failures For Circulating Water System

| Event Name | Unavailability | Description |
|--------------------|----------------|--------------------------|
| N71-MP_-CR-PUMP | 7.91E-06 | CCF PUMPS CWS TO RUN |
| N71-STR-CF-CWS | 1.07E-05 | CCF STRAINER TO RUN |
| XXX-MP_-CR-SWS/CWS | 7.91E-06 | CCF PUMPS TO RUN SWS/CWS |

Table 4.9-5C**Common Cause Failures For Turbine Component Cooling Water System**

| Event Name | Unavailability | Description |
|--------------------|-----------------------|--|
| C62-DTM-CF-N1EALL | 5.5E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| P22-MCB-CF-2ALL | 9.29E-05 | CCF 2 OUT OF 3 CIRCUIT BEAKERS FAIL TO CLOSE |
| P22-MP_-CR-C001ABC | 2.40E-06 | CCF TO RUN 2 PUMPS |
| P22-MP_-CS-2ALL | 2.60E-05 | CCF 2 OUT OF 3 PUMPS FAIL TO START |
| P22-PT_-CF-N001 | 1.72E-06 | CCF 2/3 DISCHARGE PRESSURE TRANSMITTERS |
| P22-PT_-CF-N002LOW | 3.00E-08 | CCF SUCTION PRESSURE 2/ 3 TRANSMITTERS |
| P22-TT_-CF-N003LOW | 2.20E-07 | CCF 2/ 3 TEMPERATURE TRANSMITTERS |

Table 4.9-6A
Human Errors For Feedwater And Condensate System

| Event Name | Unavailability | Description |
|----------------------|----------------|---|
| N21-XHE-FO-CONDPUMPD | 2.69E-01 | OPERATOR FAILS TO START CONDENSATE PUMP D |
| N21-XHE-FO-FWPUMPD | 2.69E-01 | OPERATOR FAILS TO START FEEDWATER PUMP D |

Table 4.9-6B
Human Errors for Circulating Water System

No human errors have been considered

Table 4.9-6C

Human Errors for Turbine Component Cooling Water System

| Event Name | Unavailability | Description |
|----------------|----------------|------------------------------|
| P22-XHE-FO-HXB | 2.69E-01 | OPERATOR FAILS TO ALIGN HX B |

Table 4.9-7A
Basic Events for Feedwater and Condensate System

| Event Name | Unavailability | Description |
|-------------------|----------------|---|
| B21-LT_-CF-L8NO | 1.20E-07 | CCF LEVEL TRANSMITTERS L8 (FAILURE TO RESPOND TO CHANGE) |
| B21-LT_-CF-L9SPUI | 1.20E-07 | SPURIOUS L9 SIGNAL CCF |
| B21-LT_-NO-LVL8A | 2.40E-05 | LEVEL TRANSMITTER A (RX LVL 8) FAILS TO RESPOND TO CHANGE |
| B21-LT_-NO-LVL8B | 2.40E-05 | LEVEL TRANSMITTER B (RX LVL 8) FAILS TO RESPOND TO CHANGE |
| B21-LT_-NO-LVL8C | 2.40E-05 | LEVEL TRANSMITTER C (RX LVL 8) FAILS TO RESPOND TO CHANGE |
| C62-DTM-CF-N1EALL | 5.5E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| C62-DTM-FC-N1EA | 9.00E-04 | DIGITAL TRIP MODULE TRAIN A NO 1E FAILS |
| C62-DTM-FC-N1EB | 9.00E-04 | DIGITAL TRIP MODULE TRAIN B NO 1E FAILS |
| C62-DTM-FC-N1EC | 9.00E-04 | DIGITAL TRIP MODULE TRAIN C NO 1E FAILS |
| H23-RMU-CF-N1EALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (NO 1E) TO OPERATE |
| H23-RMU-FC-1CONDD | 3.00E-04 | RMU 001 FOR COND PUMP D MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-1FWD | 3.00E-04 | RMU 001 FOR FW PUMP D MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-2CONDD | 3.00E-04 | RMU 002 FOR COND PUMP D MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-2FWD | 3.00E-04 | RMU 002 FOR FW PUMP D MANUAL ACTUATION FAILS TO OPERATE |
| H23-RMU-FC-FWTKA | 3.00E-04 | RMU LEVEL TRANSMITTER A FW TANK FAILS |
| H23-RMU-FC-FWTKB | 3.00E-04 | RMU LEVEL TRANSMITTER B FW TANK FAILS |
| H23-RMU-FC-FWTKC | 3.00E-04 | RMU LEVEL TRANSMITTER C FW TANK FAILS |
| H23-RMU-FC-HWA | 3.00E-04 | RMU LEVEL TRANSMITTER FW TANK A FAILS |
| H23-RMU-FC-HWB | 3.00E-04 | RMU LEVEL TRANSMITTER FW TANK B FAILS |
| H23-RMU-FC-HWC | 3.00E-04 | RMU LEVEL TRANSMITTER FW TANK C FAILS |
| H23-RMU-FC-LVL8A | 3.00E-04 | RMU LEVEL TRANSMITTER A (RX LVL 8) FAILS TO FUNCTION |
| H23-RMU-FC-LVL8B | 3.00E-04 | RMU LEVEL TRANSMITTER B (RX LVL 8) FAILS TO FUNCTION |
| H23-RMU-FC-LVL8C | 3.00E-04 | RMU LEVEL TRANSMITTER C (RX LVL 8) FAILS TO FUNCTION |
| N21-ACV-CC-F023 | 1.58E-02 | AIR OPERATED VALVE N21-F023 FAILS TO OPEN |
| N21-ACV-CC-F026 | 1.58E-02 | AIR OPERATED VALVE N21-F026 FAILS TO OPEN |

Table 4.9-7A
Basic Events for Feedwater and Condensate System

| Event Name | Unavailability | Description |
|--------------------|----------------|--|
| N21-ACV-CF-MKUPA/B | 1.08E-04 | CCF TO OPERATE AOVs TRAINS A & B |
| N21-ACV-OC-F016 | 1.31E-03 | AIR OPERATED VALVE N21-F016 FAILS TO REMAIN OPEN |
| N21-ACV-OC-F018 | 1.31E-03 | AIR OPERATED VALVE N21-F018 FAILS TO REMAIN OPEN |
| N21-ACV-TM-F026 | 8.00E-04 | VALVE N21-F026 UNAVAILABLE DUE TO TEST OR MAINTENANCE |
| N21-LT_-CF-FWTKLOW | 6.00E-08 | CCF OF LEVEL TRANSMITTERS (LOW) IN FW STORAGE TANK |
| N21-LT_-CF-FWTKNO | 4.38E-05 | CCF FEEDWATER STORAGE TANK LEVEL TRANSMITTERS |
| N21-LT_-CF-HWLOW | 6.00E-08 | CCF HOTWELL LEVEL TRANSMITTERS TO STOP PUMPS |
| N21-LT_-CF-HWNO | 4.38E-05 | CCF HOTWELL LEVEL TRANSMITTERS |
| N21-LT_-LO-FWTKA | 1.22E-05 | LEVEL TRANSMITTER TRAIN A FAILS LOW |
| N21-LT_-LO-FWTKB | 1.22E-05 | LEVEL TRANSMITTER TRAIN B FAILS LOW |
| N21-LT_-LO-FWTKC | 1.22E-05 | LEVEL TRANSMITTER TRAIN C FAILS LOW |
| N21-LT_-LO-HWA | 1.22E-05 | LEVEL TRANSMITTER TRAIN A FAILS LOW |
| N21-LT_-LO-HWB | 1.22E-05 | LEVEL TRANSMITTER TRAIN B FAILS LOW |
| N21-LT_-LO-HWC | 1.22E-05 | LEVEL TRANSMITTER TRAIN C FAILS LOW |
| N21-LT_-NO-FWTKA | 8.71E-03 | LEVEL TRANSMITTER TRAIN A FAILS |
| N21-LT_-NO-FWTKB | 8.71E-03 | LEVEL TRANSMITTER TRAIN B FAILS |
| N21-LT_-NO-FWTKC | 8.71E-03 | LEVEL TRANSMITTER TRAIN C FAILS |
| N21-LT_-NO-HWA | 8.71E-03 | LEVEL TRANSMITTER TRAIN A FAILS |
| N21-LT_-NO-HWB | 8.71E-03 | LEVEL TRANSMITTER TRAIN B FAILS |
| N21-LT_-NO-HWC | 8.71E-03 | LEVEL TRANSMITTER TRAIN C FAILS |
| N21-MCB-CO-CONDA | 1.20E-05 | CIRCUIT BREAKER FOR CONDENSATE PUMP A OPENS SPURIOUSLY |
| N21-MCB-CO-CONDB | 1.20E-05 | CIRCUIT BREAKER FOR CONDENSATE PUMP B OPENS SPURIOUSLY |
| N21-MCB-CO-CONDC | 1.20E-05 | CIRCUIT BREAKER FOR CONDENSATE PUMP C OPENS SPURIOUSLY |
| N21-MCB-CO-FWA | 1.20E-05 | CIRCUIT BREAKER FOR FW PUMP A OPENS SPURIOUSLY |
| N21-MCB-CO-FWB | 1.20E-05 | CIRCUIT BREAKER FOR FW PUMP B OPENS SPURIOUSLY |
| N21-MCB-CO-FWC | 1.20E-05 | CIRCUIT BREAKER FOR FW PUMP C OPENS SPURIOUSLY |
| N21-MCB-CO-FWD | 1.20E-05 | CIRCUIT BREAKER FOR FW PUMP D OPENS SPURIOUSLY |

Table 4.9-7A
Basic Events for Feedwater and Condensate System

| Event Name | Unavailability | Description |
|-----------------------|----------------|--|
| N21-MCB-OO-CONDD | 1.00E-03 | CIRCUIT BREAKER FOR CONDENSATE PUMP D FAILS TO CLOSE |
| N21-MCB-OO-FWD | 1.00E-03 | CIRCUIT BREAKER TO FEEDWATER PUMP D FAILS TO CLOSE |
| N21-MOV-CC-F008 | 4.00E-03 | MOTOR OPERATED VALVE F008 FAILS TO OPEN |
| N21-MOV-CC-F061 | 3.13E-02 | MOTOR OPERATED VALVE N21-F061 FAILS TO OPEN |
| N21-MPF-CR-RUN | 3.00E-06 | FEEDWATER PUMPS CCF TO RUN |
| N21-MPF-FR-C001A | 6.00E-04 | MOTOR-DRIVEN FDWTR PUMP A FAILS TO RUN, GIVEN START |
| N21-MPF-FR-C001B | 6.00E-04 | MOTOR-DRIVEN FDWTR PUMP B FAILS TO RUN, GIVEN START |
| N21-MPF-FR-C001C | 6.00E-04 | MOTOR-DRIVEN FDWTR PUMP C FAILS TO RUN, GIVEN START |
| N21-MPF-FR-C001D | 6.00E-04 | MOTOR-DRIVEN FDWTR PUMP D FAILS TO RUN, GIVEN START |
| N21-MPF-FS-C001D | 2.00E-03 | MOTOR-DRIVEN FEEDWATER PUMP D FAILS TO RESTART |
| N21-MPF-TM-C001D | 2.00E-03 | FEEDWATER PUMP BRANCH D IN MAINTENANCE |
| N21-MP_-CR-COND | 3.00E-06 | CONDENSATE PUMPS CCF TO RUN |
| N21-MP_-FR-CONDA | 6.00E-04 | MOTOR-DRIVEN COND. PUMP A FAILS TO RUN, GIVEN START |
| N21-MP_-FR-CONDB | 6.00E-04 | MOTOR-DRIVEN COND PUMP B FAILS TO RUN, GIVEN START |
| N21-MP_-FR-CONDC | 6.00E-04 | MOTOR-DRIVEN COND PUMP C FAILS TO RUN, GIVEN START |
| N21-MP_-FR-CONDD | 6.00E-04 | MOTOR-DRIVEN PUMP D FAILS TO RUN, GIVEN START |
| N21-MP_-FS-CONDD | 2.00E-03 | MOTOR-DRIVEN CONDENSATE PUMP D FAILS TO START |
| N21-MP_-TM-CONDD | 3.00E-03 | FEEDWATER PUMP BRANCH D IN MAINTENANCE |
| N21-OR_-PG-FWCPRECIRD | 1.44E-05 | ORIFICE FOR FWP D RECIRC FAILS TO REMAIN OPEN (PLUG) |
| N21-STR-PG-FL001A | 2.40E-04 | STRAINER/FILTER FOR PMP A PLUGS DURING OPERATION |
| N21-STR-PG-FL001B | 2.40E-04 | STRAINER/FILTER FOR PMP B PLUGS DURING OPERATION |
| N21-STR-PG-FL001C | 2.40E-04 | STRAINER/FILTER FOR PMP C PLUGS DURING OPERATION |
| N21-STR-PG-FL001D | 3.59E-03 | STRAINER/FILTER FOR PMP D PLUGS DURING OPERATION |

Table 4.9-7A**Basic Events for Feedwater and Condensate System**

| Event Name | Unavailability | Description |
|----------------------|-----------------------|---|
| N21-SYS-FF-BYPASS | 1.00E-02 | TURBINE BYPASS FAILS |
| N21-UV_-CC-F008D | 2.00E-04 | CHECK VALVE F008D FOR CND PUMP D FAILS TO OPEN |
| N21-UV_-CC-F021D | 2.00E-04 | CHECK VALVE F021D FOR FWP D FAILS TO OPEN |
| N21-XHE-FO-CONDPUMPD | 2.69E-01 | OPERATOR FAILS TO START CONDENSATE PUMP D |
| N21-XHE-FO-FWPUMPD | 2.69E-01 | OPERATOR FAILS TO START FEEDWATER PUMP D |
| P30-TNK-RP-A001 | 2.40E-06 | CONDENSATE STORAGE TANK LEAKS CATASTROPHICALLY |

Table 4.9-7B
Basic Events For Circulating Water System

| Event Name | Unavailability | Description |
|--------------------|----------------|--|
| N71-MCB-CO-PUMPA | 1.20E-05 | CIRCUIT BREAKER FOR CIRC PUMP A OPENS SPURIOUSLY |
| N71-MCB-CO-PUMPB | 1.20E-05 | CIRCUIT BREAKER FOR CIRC PUMP B OPENS SPURIOUSLY |
| N71-MCB-CO-PUMPC | 1.20E-05 | CIRCUIT BREAKER FOR PUMP C OPENS SPURIOUSLY |
| N71-MCB-CO-PUMPD | 1.20E-05 | CIRCUIT BREAKER FOR PUMP D OPENS SPURIOUSLY |
| N71-MP_-CR-PUMP | 7.91E-06 | CCF PUMPS CWS TO RUN |
| N71-MP_-FR-C001A | 6.00E-04 | MOTOR-DRIVEN PUMP A FOR CIRC FAILS TO RUN, GIVEN START |
| N71-MP_-FR-C001B | 6.00E-04 | MOTOR-DRIVEN PUMP B FOR CIRC FAILS TO RUN, GIVEN START |
| N71-MP_-FR-C001C | 6.00E-04 | MOTOR-DRIVEN PUMP C FOR CIRC FAILS TO RUN, GIVEN START |
| N71-MP_-FR-C001D | 6.00E-04 | MOTOR-DRIVEN PUMP D FOR CIRC WATER SYS FAILS TO RUN |
| N71-STR-CF-CWS | 1.07E-05 | CCF STRAINER TO RUN |
| XXX-MP_-CR-SWS/CWS | 7.91E-06 | CCF PUMPS TO RUN SWS/CWS |

Table 4.9-7C
Basic Events for Turbine Component Cooling Water System

| Event Name | Unavailability | Description |
|---------------------|-----------------------|--|
| C62-DTM-CF-N1EALL | 5.5E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| C62-DTM-FC-N1EA | 9.00E-04 | DIGITAL TRIP MODULE TRAIN A NO 1E FAILS |
| C62-DTM-FC-N1EB | 9.00E-04 | DIGITAL TRIP MODULE TRAIN B NO 1E FAILS |
| C62-DTM-FC-N1EC | 9.00E-04 | DIGITAL TRIP MODULE TRAIN C NO 1E FAILS |
| H23-RMU-FC-1HXB | 3.00E-04 | RMU 001 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-2HXB | 3.00E-04 | RMU 002 FOR MANUAL ACTUATION IN CONTROL ROOM FAILS |
| H23-RMU-FC-P22N001A | 3.00E-04 | RMU PRESS TANSMITTER A FAILS |
| H23-RMU-FC-P22N001B | 3.00E-04 | RMU PRESS TANSMITTER B FAILS |
| H23-RMU-FC-P22N001C | 3.00E-04 | RMU PRESS TANSMITTER C FAILS |
| H23-RMU-FC-P22N002A | 3.00E-04 | RMU PRESS TRANSMITTER A FAILS |
| H23-RMU-FC-P22N002B | 3.00E-04 | RMU PRESS TRANSMITTER B FAILS |
| H23-RMU-FC-P22N002C | 3.00E-04 | RMU PRESS TRANSMITTER C FAILS |
| H23-RMU-FC-P22N003A | 3.00E-04 | RMU TRANSMITTER N003A FAILS |
| H23-RMU-FC-P22N003B | 3.00E-04 | RMU TRANSMITTER N003B FAILS |
| H23-RMU-FC-P22N003C | 3.00E-04 | RMU TRANSMITTER N003C FAILS |
| P22-ACV-FT-BYPASS | 2.00E-03 | HEAT EXCHANGERS BYPASS VALVE FAILS TO REGULATE |
| P22-HX_-PG-B001A | 5.28E-05 | HEAT EXCHANGER A FAILS |
| P22-HX_-PG-B001B | 7.92E-04 | HEAT EXCHANGER B FAILS |
| P22-HX_-TM-B001B | 2.20E-03 | HEAT EXCHANGER B IN MAINTENANCE |
| P22-MCB-CF-2ALL | 9.29E-05 | CCF 2 OUT OF 3 CIRCUIT BEAKERS FAIL TO CLOSE |
| P22-MCB-CO-C001A | 1.20E-05 | CIRCUIT BREAKER FOR TCCW PUMP A OPENS SPURIOUSLY |
| P22-MCB-CO-C001B | 1.20E-05 | CIRCUIT BREAKER FOR TCCW PUMP B OPENS SPURIOUSLY |
| P22-MCB-OO-C001A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER (PUMP A) FAILS TO CLOSE |
| P22-MCB-OO-C001B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P22-MCB-OO-C001C | 1.00E-03 | CIRCUIT BREAKER FOR TCCW PUMP C FAILS TO CLOSE |
| P22-MOV-CC-HXB | 4.00E-03 | MOV OF HX B FAILS TO OPEN |
| P22-MP_-CR-C001ABC | 2.40E-06 | CCF TO RUN 2 PUMPS |
| P22-MP_-CS-2ALL | 2.60E-05 | CCF 2 OUT OF 3 PUMPS FAIL TO START |

Table 4.9-7C

Basic Events for Turbine Component Cooling Water System

| Event Name | Unavailability | Description |
|--------------------|----------------|---|
| P22-MP_-FR-C001A | 1.20E-04 | MOTOR-DRIVEN PUMP A FOR TCCWS FAILS TO RUN, GIVEN START |
| P22-MP_-FR-C001B | 1.20E-04 | MOTOR-DRIVEN PUMP B FOR TCCWS FAILS TO RUN, GIVEN START |
| P22-MP_-FR-C001C | 1.20E-04 | MOTOR-DRIVEN PUMP C FOR TCCWS FAILS TO RUN, GIVEN START |
| P22-MP_-FS-C001A | 1.30E-03 | MOTOR-DRIVEN PUMP (ALL TYPES) FAILS TO START |
| P22-MP_-FS-C001B | 1.30E-03 | MOTOR-DRIVEN PUMP (ALL TYPES) FAILS TO START |
| P22-MP_-FS-C001C | 1.30E-03 | MOTOR-DRIVEN PUMP C FOR TCCWS FAILS TO START |
| P22-MP_-TM-C001C | 2.00E-03 | TCCW PUMP C IN MAINTENANCE |
| P22-PT_-CF-N001 | 1.72E-06 | CCF 2/3 DISCHARGE PRESSURE TRANSMITTERS |
| P22-PT_-CF-N002LOW | 3.00E-08 | CCF SUCTION PRESSURE 2/ 3 TRANSMITTERS |
| P22-PT_-LO-N002A | 7.92E-06 | PRESSURE TRANSMITTER N002A FAILS |
| P22-PT_-LO-N002B | 7.92E-06 | PRESSURE TRANSMITTER N002B FAILS |
| P22-PT_-LO-N002C | 1.19E-04 | PRESSURE TRANSMITTER N002C FAILS |
| P22-PT_-NO-N001A | 3.46E-04 | PRESSURE TRANSMITTER NOO1A FAILS |
| P22-PT_-NO-N001B | 3.46E-04 | PRESSURE TRANSMITTER NOO1B FAILS |
| P22-PT_-NO-N001C | 3.46E-04 | PRESSURE TRANSMITTER NOO1C FAILS |
| P22-TNK-RP-A001 | 2.40E-06 | TCCW SURGE TANK LEAKS CATASTROPHICALLY |
| P22-TT_-CF-N003LOW | 2.20E-07 | CCF 2/ 3 TEMPERATURE TRANSMITTERS |
| P22-TT_-LO-N003A | 4.56E-05 | TEMPERATURE TRANSMITTER N003A FAILS |
| P22-TT_-LO-N003B | 4.56E-05 | TEMPERATURE TRANSMITTER N003B FAILS |
| P22-TT_-LO-N003C | 4.56E-05 | TEMPERATURE TRANSMITTER N003C FAILS |
| P22-UV_-CC-F034C | 2.00E-04 | CHECK VALVE F034 C FAILS TO OPEN |
| P22-UV_-OO-F034A | 1.00E-03 | CHECK VALVE F034A FAILS TO CLOSE |
| P22-UV_-OO-F034B | 1.00E-03 | CHECK VALVE F034B FAILS TO CLOSE |
| P22-XHE-FO-HXB | 2.69E-01 | OPERATOR FAILS TO ALIGN HX B |
| P41-ACV-OC-F000A | 1.08E-04 | VALVE P22-F000A FAILS TO REMAIN OPEN |
| P41-ACV-OC-F000B | 1.08E-04 | VALVE P22-F000B FAILS TO REMAIN OPEN |
| P41-ACV-OC-F001A | 1.08E-04 | VALVE P22-F001A FAILS TO REMAIN OPEN |
| P41-ACV-OC-F001B | 1.08E-04 | VALVE P22-F001B FAILS TO REMAIN OPEN |

Table 4.9-8
List of System Top Events

| Top Event Name | Description |
|-----------------------|---|
| GN21-0050-_1 | PCS FAILURE |
| GN21-0051-_1 | HIGH PRESSURE INJECTION WITH FEEDWATER FAILS |
| GP22-0001-_1 | LOSS OF COOLING FROM TCCW SYSTEM |
| GN71-0001-_1 | CIRCULATING WATER SYSTEM FAILS TO PROVIDE COND. COOL. |

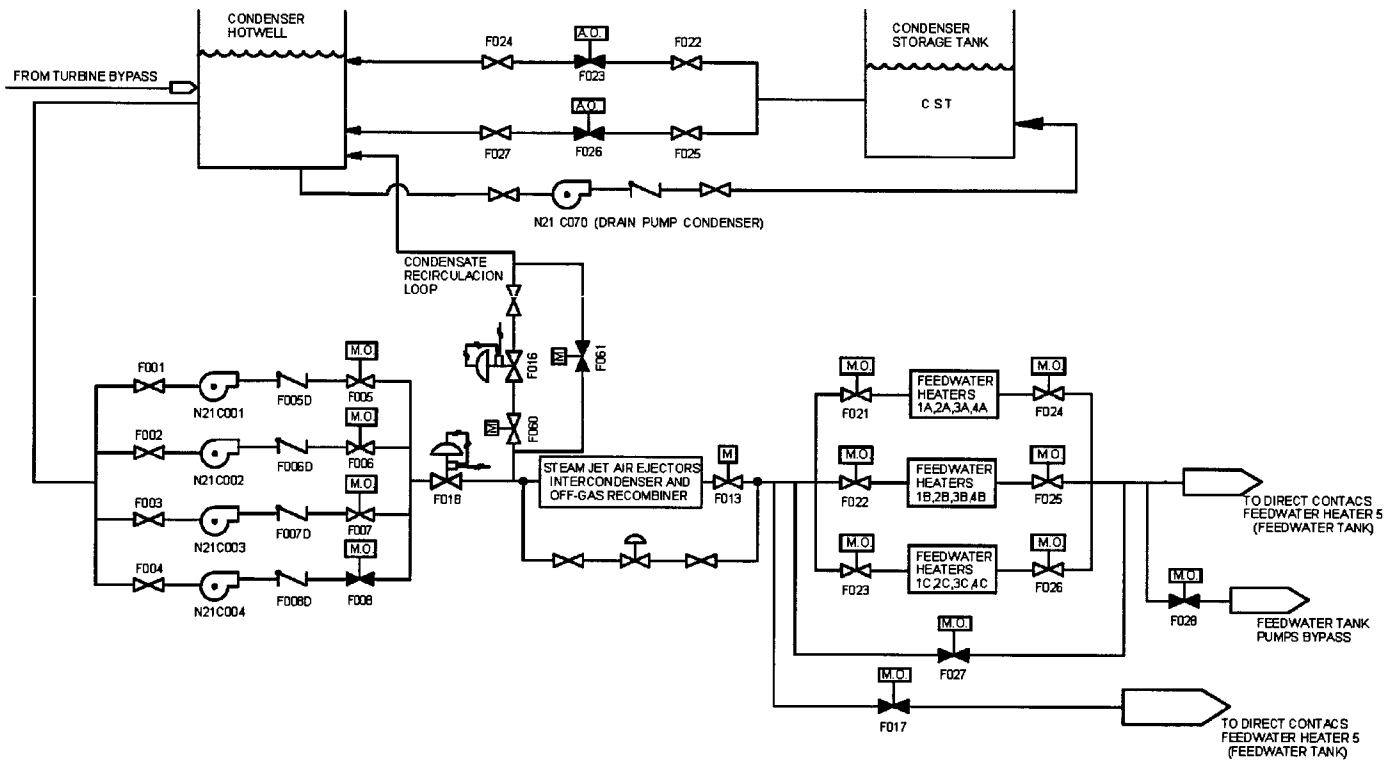


Figure 4.9-1. Condensate and Feedwater System (A) Process Flow Diagram

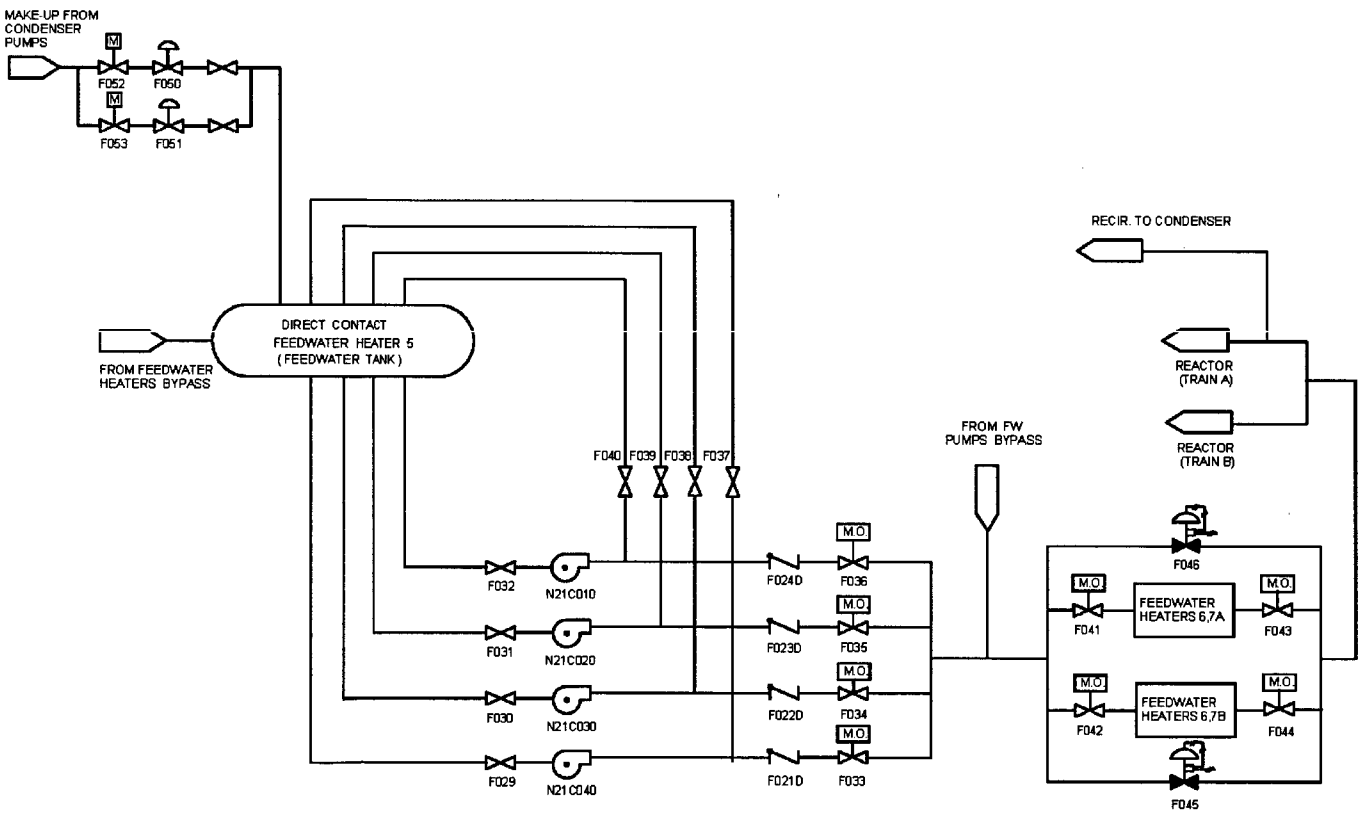


Figure 4.9-2. Condensate and Feedwater System (B) Process Flow Diagram

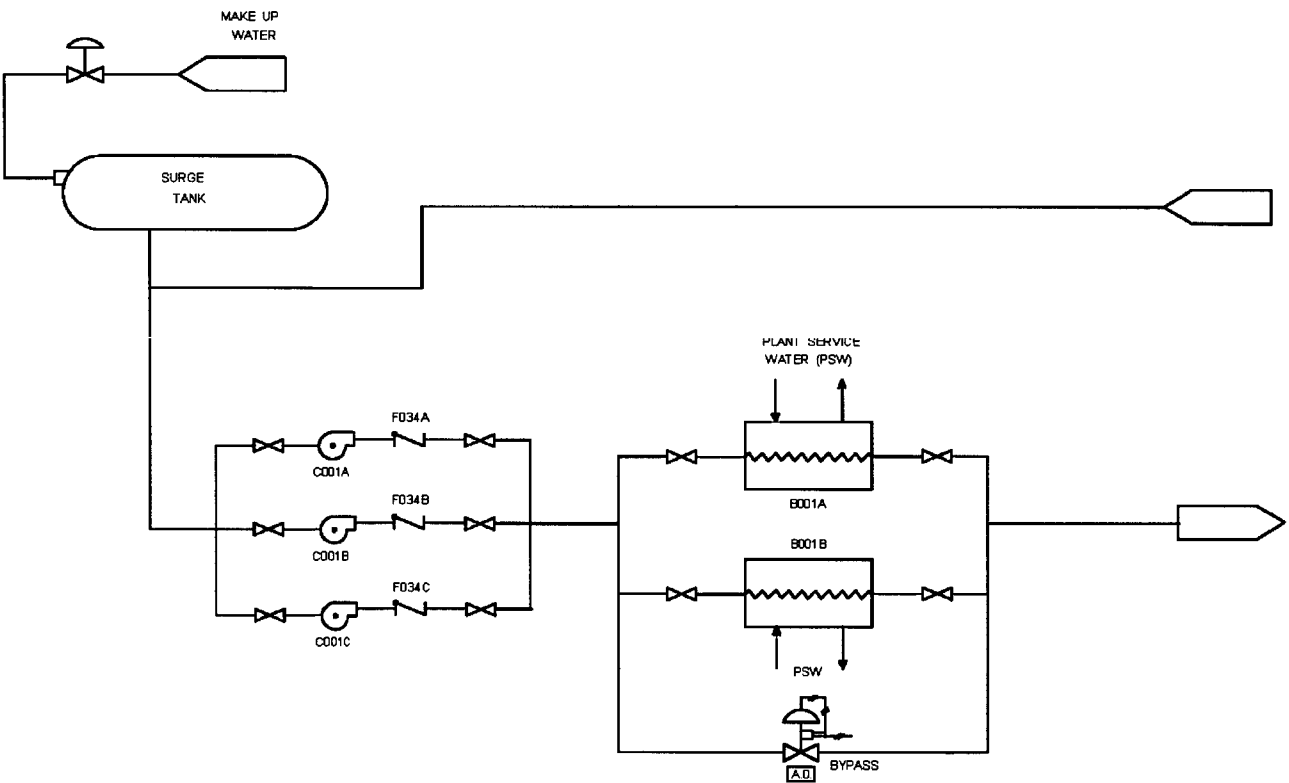


Figure 4.9-3. Turbine Component Cooling Water System Process Flow Diagram

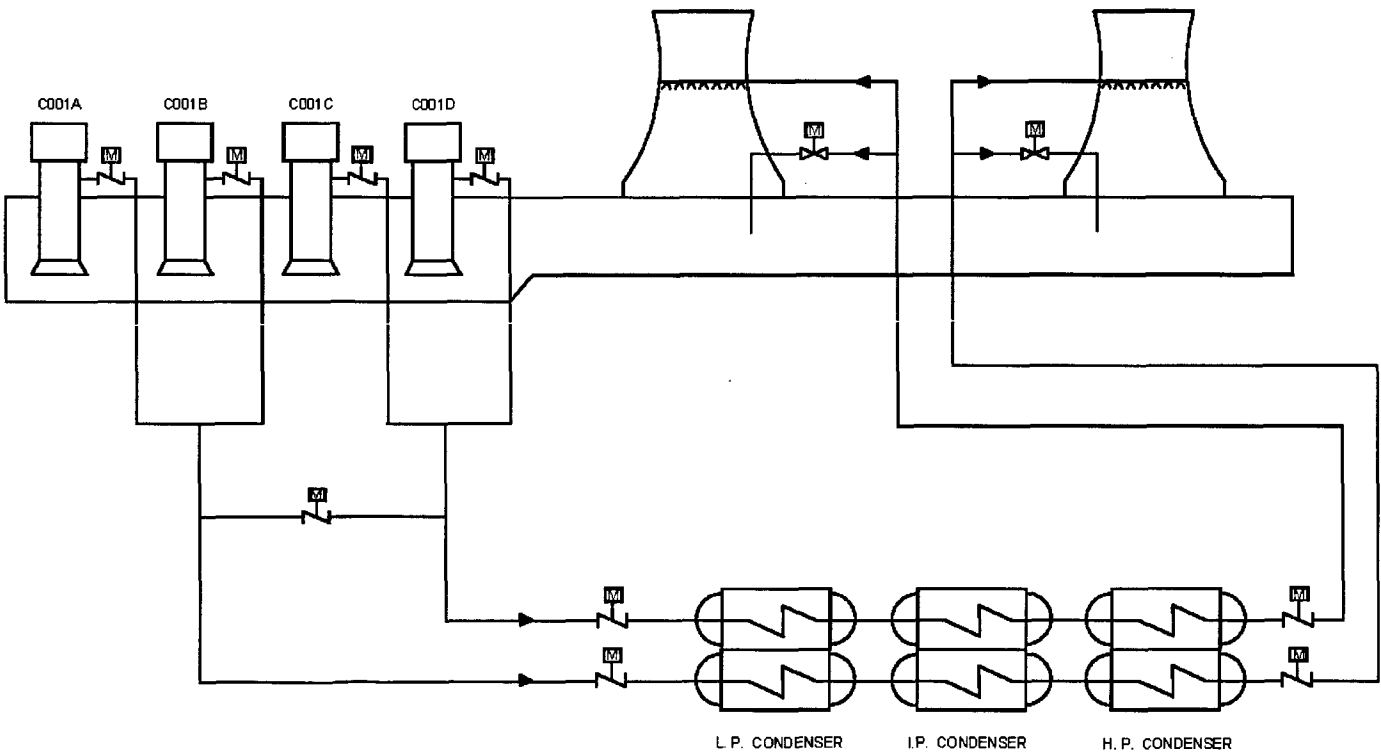
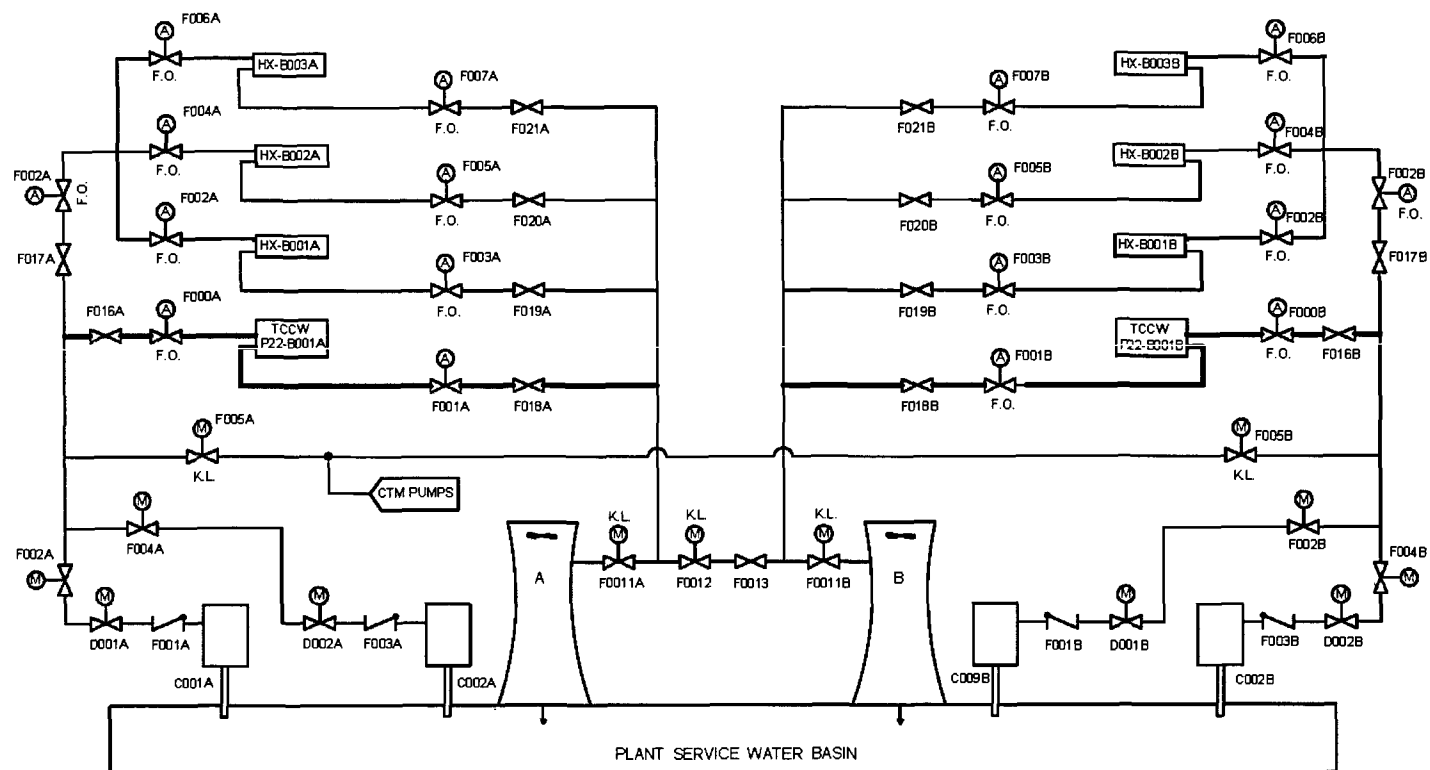


Figure 4.9-4. Circulating Water System Process Flow Diagram

Figure 4.9-5. TCCW-PSW System



Notes :
 F.O. : Fail Open
 K.L. : Key Locked

4.10 REACTOR COMPONENT COOLING WATER SYSTEM – (P21)

4.10.1 Purpose

RCCWS (P21) removes heat from systems in the drywell and Reactor Building that require cooling water during normal operation, shutdown, and hot standby, and rejects this heat to the environment using the PSW system.

Reactor Component Cooling Water System has no safety-related functions.

4.10.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) The volume of water in the tanks required by Technical Specification are sufficient for water to be supplied to the pumps for 72 hours and therefore, the make up of the inventory in the surge tank using the Fire Protection System is not necessary.
- (2) All the components of train A are powered from division I and those of train B from division II.
- (3) The connection valves have the following uninterrupted power supply: valves P21-F010 and P21-F032 are powered from division A and valves P21-F013 and P21-F027 are powered from an uninterrupted power supply from division B.
- (4) The relevant measures are taken so that the air-operated valves do not depend on the Instrument Air System in an emergency situation.
- (5) In case one of the pumps fails, another pump from the same train receives a startup signal. This signal transmission within the division, is included in the model. It is also considered that the pumps can be started from the control room.
- (6) In shutdown mode, 2 pumps and 2 heat exchangers are sufficient to cool the necessary loads. In the case of a LOPP, another heat exchanger and pump are required to be started to replace the train that has lost power.
- (7) Room cooling is not required for first 24 hours of the accident.
- (8) All loads needed for the PRA are specified/required to be cooled with at least n+1 redundancy.

4.10.3 System Description

The RCCWS analysis described here refers to the system configuration shown in the diagram of Figure 4.10.1.

4.10.3.1 Hardware Configuration

RCCWS is comprised of two interconnected divisions of closed cooling water. Each division consists of 3 pumps and 3 heat exchangers connected in parallel to a common discharge header/piping system that provides cooling water to designated coolers located in the reactor and turbine buildings.

Both trains have a common suction line from the RCCWS Surge Tank. There are manual isolation valves on the suction and discharge lines of each for maintenance.

The discharge of the pumps and the exchangers of the 2 trains are interconnected. This means that any combination of pumps and heat exchangers can fulfill the success criteria by delivering cooling water to any combination of loads.

The flow rate for each heat exchanger train is regulated by the bypass valves (P21-F022A/B) and the exchanger discharge valves (P21-F025A/B). Both valves are pneumatic. The flow through these valves regulates the temperature of the cold leg water supply temperature. As described below, it is assumed that in case of instrument air failure, air operated valves change or maintains their position in order to guarantee cooling flow. Because of this, it is assumed that heat exchanger discharge valves open, heat exchanger bypass valves close, and cross connection valves fail as is, given a loss of pneumatic supply.

The pumps are all sized for an output of 1250 m³/hr, and the heat exchangers have a maximum design flow rate of 1250 m³/hr.

4.10.3.2 System Operation

During normal power operation the system operates with 2 pumps and 2 heat exchangers in operation.

RCCWS cooling water is supplied to two cooling water trains during normal operation with two pumps in service and the other four pumps shall be in standby. Two heat exchangers are functioning during normal operation and the other four heat exchangers are in standby. The cross-tie valves are open during normal operation to allow operating flexibility. Major users of RCCWS water (CWS, FAPCS, and RWCU/SDC) have redundant heat exchangers on each train, and only one of these trains is in use during normal operating conditions. The heat exchangers for major users not in use have the cooling water supply automatically isolated to reduce wear on the unused heat exchanger, and reduce the required cooling water flow rate during normal operation.

In the event that the cooling water trains need to be separated, the cross-tie valves are closed and two pumps and two heat exchangers are in use in each train. If the cross-tie valves need to be closed, the CWS, FAPCS, and RWCU/SDC heat exchangers that are not required to be in operation are bypassed. The separation of trains is considered an abnormal operating condition. Opening the bypass line valves for CWS, FAPCS, and RWCU/SDC is required to keep the RCCWS pumps within their operating ranges.

During normal power operation, flow through the RWCU/SDC heat exchanger is regulated with a bypass flow control valve. This valve shall be controlled by RWCU/SDC, not RCCWS. Control of this bypass valve by RWCU/SDC shall prevent overcooling of the reactor coolant.

In shutdown mode, the configuration of the system varies depending on the initiating event that caused the scram.

- Non-LOPP events: Initially pumps 1A and 1B are operating. The two remaining pumps from each train are in standby. The water passes through the 6 HXs, which are in line, and the bypass valve regulates the flow based on the outlet temperature. This

configuration is maintained when the trip occurs. In case of an operation failure in a pump, another one from the same train automatically initiates.

All the interconnection valves of the trains are open. Valves F022A/B and valves F025A/B are also regulated by a signal from the same temperature transmitter.

- LOPP event: One pump from each train is operating when the event initiates. Consequently, the pumps trip and the 2 trains isolate by the closing of the pneumatic connection valves. The diesel generators initiate and once the power on buses is reestablished, the RCCW pumps re-initiate. The success criterion in this case requires the operation of 3 pumps for the same train, with interconnection valve openings that guarantee the delivery of the cooling flow to the consumers, whichever configuration of loads is in operation.

During shutdown/refueling RCCWS remains available for cooling associated with plant maintenance.

4.10.3.3 Components Location

The RCCWS components are located within the turbine building.

4.10.4 Automatic and Manual Control

In case of the failure of one working pump, one standby pump is automatically actuated. Manual initiation of the standby pump is also possible. Status indication for valves and pumps are expected to be in the control room.

Manual actuation of pneumatic and motorized valves can be carried out from the control room.

The flow rate through the heat exchangers is adjusted by the temperature regulation in the outlet of the heat exchangers, which acts on the bypass valves and the pneumatic valves of the heat exchanger discharge.

4.10.5 System Interfaces

The RCCWS dependencies are included in the dependency matrix given in Table 4.10-2.

4.10.6 System Testing

The RCCWS is a non-safety related system and therefore no Technical Specifications exist for it.

However, a periodic change in train operation is assumed so that the relevant train components are checked quarterly, as indicated in Table 4.10-3.

4.10.7 System Maintenance

Given the redundancy of the RCCWS system design, it is possible to carry out maintenance on pumps and/or heat exchangers during power operation.

For modeling purposes, it has been considered that maintenance can be carried out in both trains simultaneously, as long as it does not affect more than one pump or exchanger in each train.

Maintenance unavailabilities are assigned to train 2 pumps and heat exchangers as indicated in Table 4.10-4.

4.10.8 Common-Cause Failures

The common cause failures have been grouped in Table 4.10-5.

4.10.9 Fault Tree Analysis

- A fault tree is developed, whose TOP gate GP21-0001, which represents the system failures, is “Insufficient Cooling flow from RCCWS”. The fault tree is shown in Appendix B.4.10.
- The TOP groups together two mutually exclusive situations:
 - Gate GP21-0001-_2 represents the system failures in normal operation and shutdown cooling.
 - Gate GP21-0001-_3 represents the system failures after a loss of preferred power (LOPP).

4.10.9.1 Top Event Definitions

- A fault tree for insufficient cooling flow from RCCWS is developed, whose TOP gate GP21-0001-_1 represents the system failure.
- The TOP includes two mutually exclusive situations:
 - Gate GP21-0001-_2 represents the system failures in normal operation and shutdown cooling. The success criterion of this part of the tree is to supply cooling water through two heat exchangers with two pumps.
 - Gate GP21-0001-_3 represents the system failures after a loss of preferred power (LOPP). In this case, the success criterion is to supply cooling water from 3 pumps in the same train. In the case of a LOPP, the house-event XHSLOSP is assigned a value of 1, therefore gate GP21-0001-_3 is cancelled. If the opposite occurs, the gate to be cancelled is GP21-0001-_3.

4.10.9.2 Model Assumptions

- System limits: For the purpose of the PRA, the RCCWS system includes suction lines from the RCCWS surge tank, pumps, cross-ties, heat exchangers, cooling lines and associated valves from PSWS and discharge lines up to RWCU/SDC, FAPCS, and CRD connections.
- Gate GP21-0001-_2 represents the system failures in normal operation and shutdown cooling. The success criterion of this part of the tree is the supply through two heat exchangers with two pumps. It is considered that initially this system is operating with one pump and one heat exchanger of every train. To guarantee cooling for consumers, independently of the train which the loads to be cooled belong to, and of the configuration of the pumps and heat exchangers, it has been considered that at least one of the ties between the trains downstream from the heat exchangers should be open.
- Gate GP21-0001-_3 represents the system failures after a loss of preferred power (LOPP). In this case the success criterion is the supply through 3 pumps. It is considered that as a consequence of the LOPP the pumps trip and the trains are isolated from each

other due to the closing of the air operated interconnection valves. The model outlines the necessity for the startup of 3 pumps in the same train. The interconnection of the trains in the part of the heat exchanger discharge which guarantees the cooling of the consumers independently of the train they belong to and the starting configuration of the pumps.

- Manual valve failures caused by mispositioning are not considered failures because the system is normally in operation, mis-positions would therefore be immediately detectable.
- RCCW is a closed circuit and the function of the tanks is to maintain the pressure in the circuit. The loss of water supply from tanks is not considered to be the cause of a system failure, because a leak would be detectable during normal operation (alarms and level indicators). Catastrophic failures of the tanks or lines are considered to be improbable compared to active component failures, and therefore, are not considered in the model.
- The possibility of obstruction in the pump suction filters is not considered because it is a clean water system, it operates in all plant modes, it is subjected to periodic tests and the periodic rotation of trains, and it is a passive failure, which makes it less probable than active failures.
- It is not considered that the motorized valves could be closed and de-energized after being manipulated because it is considered that there are indications in the control room, opening signals for the corresponding pump startup and probably once the valve manipulations have been completed, tests are performed on the flow or train rotation and therefore their erroneous position would be detected immediately.
- During power operation, a pump or heat exchanger from each train may be under maintenance. In the model, maintenance events are assigned to pumps and heat exchangers 2A and 2B.
- The two interconnecting lines of the discharge from the heat exchanger allows cooling supply flow to the consumers of any train for any combination of pumps and heat exchangers that meet the system success criteria.
- Failures in the return line valves from the consumer heat exchangers to the pump suction are not considered to cause a system failure.
- In case one of the pumps fails, another pump from the same train receives an initiation signal. It is considered that this signal is generated directly from the running pump circuit breaker. It is also considered that the pumps can be started from the control room. This human action is represented by the basic event P21-XHE-FO-STDBYPUMP.
- In case of a LOPP, standby pumps receive starting signals directly from DGs starting signal. These signals are represented under gates GR21-0025-_1 and GR21-0025-_2.
- In case of a LOPP with a malfunction of the startup of one pump in operation, the manual startup is not considered. The possibility of the manual startup is considered if the automatic startup failure of one pump in standby occurs.

4.10.9.3 Human actions

One human action is considered in the fault tree:

P21-XHE-FO-STDBYPUMP represents the operator's failure to manually start the standby pump when automatic actuation fails.

Human actions in the fault trees are connected by the house-event XHOSMAN that represents operator unavailability to carry out manual actions.

4.10.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.10-7.

The fault tree results provide quantitative values of system unavailability and of the importance of specific components to that total.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This qualification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

The importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.10.11 Model Differences for 72-Hour Operation

The long-term RCCWS operation is similar to that during the first 24 hours following a transient or LOCA. There are no fault tree model changes necessary to extend the operation to 72 hours.

4.10.12 References

4.10-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.10-1
Control Room Instrumentation and Alarms

| Main Control Room Panel Controls |
|---|
| (1) RCCWS pump(s) start/stop and pump selection controls |
| (2) RCCWS heat exchanger(s) outlet valve open/close controls |
| (3) RCCWS supply temperature control valve(s) controls |
| (4) FAPC heat exchanger(s) inlet and outlet valves open/close controls |
| (5) CWS chiller cooling water outlet block valve controls |
| (6) Containment isolation valves open/close controls |
| (7) Non-safety-related supply MOV and AOV open/close controls |
| (8) RWCU/SDC heat exchanger(s) inlet valve open/close controls |
| (9) Makeup water valve open/close control for MWS to RCCWS surge tank(s) |
| CWS backup DCS supply valves open/close. |
| Main Control Console Displays |
| (1) RCCWS supply pressure |
| (2) RCCWS supply flow |
| (3) RCCWS supply/return temperature |
| (4) Major users pressure, temperature, and flow at inlet/outlet |
| (5) RCCWS surge tank(s) low and high level |
| (6) Pump start/stop status |
| (7) High water level in Turbine Building for indication of pipe break or large leak |
| (8) RCCWS heat exchanger(s) outlet valve open/close status |
| (9) CWS chiller outlet block valve position status |
| (10) Non-safety supply MOV and AOV open/close status |
| (11) RWCU/SDC heat exchanger(s) inlet valve open/close status |
| (12) FAPC heat exchanger(s) inlet and outlet valve open/close status |
| (13) Containment isolation valves open/close status |
| (14) Surge Tank(s) makeup isolation valve from MWS open/close status |
| RCCWS pump NPSH available |
| Main Control Room Panel Controls |
| (1) RCCWS supply pressure low |
| (2) RCCWS supply flow low |
| (3) RCCWS supply temperature low and high |
| (4) RCCWS surge tank(s) standpipe low level |
| (5) RCCWS surge tank(s) low and high level |
| (6) DCS, FAPC, CWS, and RWCU/SDC heat exchanger(s) outlet temperature high |
| (7) RCCWS pump motor overload or trip |
| (8) RCCWS/PSW logic power failure |
| (9) RCCWS standby pump(s) auto-start |

Table 4.10-2
System Dependencies Matrix

| COMPONENT | TRANSMISSION SIGNAL | 6.9KV (R11) | 480V (R12) | UPS (R13) (NO 1E) | IA (P52) | PSW (P41) |
|--|------------------------|----------------|---------------|-------------------------|------------------|--------------|
| PUMPS C001A/2A/3A | C62 (DIV A DID) | BUS A2 | | | | |
| P21-F022A | C62 (DIV A DID) | | DIV A | DIV A | X ⁽²⁾ | |
| HX- 1A/2A/3A | | | | | | X |
| P21-F010/32 | C62 (DIV A DID) | | DIV A | DIV A | X ⁽³⁾ | |
| P21-F016A/18A/20A | | | DIV A | | | |
| PUMPS C001B/2B/3B | | BUS B2 | | | | |
| P21-F022B | C62 (DIV B DID) | | DIV B | DIV B | X ⁽²⁾ | |
| HX- 1B/2B/3B | | | | | | X |
| P21-F013/27 | C62 (DIV B DID) | | DIV B | DIV B | X ⁽³⁾ | |
| P21-F016B/18B/20B | | | DIV B | | | |
| P21-F025A | C62 (DIV A DID) | | DIV A | DIV A | X ⁽¹⁾ | |
| P21-F025B | C62 (DIV B DID) | | DIV B | DIV B | X ⁽¹⁾ | |
| Notes (1) Valves fail OPEN (2) Valves fail CLOSED (3) Valves fail AS IS | | | | | | |

Table 4.10-3
Component Tests

A periodic change in train operation is assumed so that the relevant train components are checked quarterly.

Table 4.10-4
Component Maintenance

| Basic Event | Probability | Description |
|--------------------|--------------------|---|
| P21-MP_-TM-C002A | 2.00E-03 | PUMP C002A IN MAINTENANCE |
| P21-MP_-TM-C002B | 2.00E-03 | PUMP C002B IN MAINTENANCE |
| P21-HX_-TM-B002A | 2.20E-03 | HEAT EXCHANGER B002A UNAVAILABLE DUE TO MAINTENANCE |
| P21-HX_-TM-B002B | 2.20E-03 | HEAT EXCHANGER B002B UNAVAILABLE DUE TO MAINTENANCE |

Table 4.10-5
Common Cause Failure

| Basic Event | Probability | Description |
|---------------------|--------------------|--|
| C62-DTM-CF-N1EALL | 5.50E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| P21-ACV-CF-F010/13 | 9.00E-08 | CCF AIR OPERATED VALVES F010 AND F013 TO REMAIN OPEN |
| P21-ACV-CF-F022/25A | 5.00E-05 | CCF VALV ACV F022A/ F025A TRAIN A |
| P21-ACV-CF-F022/25B | 5.00E-05 | CCF VALV ACV F022B/F025B TRAIN B |
| P21-ACV-CF-F027/32 | 4.00E-08 | CCF AIR OPERATED VALVES F027 AND F032 TO REMAIN OPEN |
| P21-MP_-CR-3A | 2.40E-06 | CCF 3 PUMPS TO RUN (INTRADIVISION A) |
| P21-MP_-CR-TRAINAB | 1.80E-05 | CCF TO RUN PUMPS TRAINS A AND B |
| P21-MP_-CR-3B | 1.20E-06 | CCF 3 PUMPS TO RUN (INTRADIVISION B) |
| P21-MP_-CR-5ALL | 6.00E-07 | CCF TO RUN PUMPS TRAINS A AND B |
| P21-MP_-CS-2A | 2.60E-05 | CCF 2 PUMPS (INTRADIVISION A) TO START |
| P21-MP_-CS-2B | 2.60E-05 | CCF 2 PUMPS (INTRADIVISION B) TO START |
| P21-MP_-CS-TRAINAB | 1.95E-04 | CCF TO START PUMPS TRAINS A AND B |
| P21-MP_-CS-5ALL | 6.50E-06 | CCF TO START PUMPS DIVISIONS A AND B |
| P21-TT_-CF-N038A/B | 4.00E-08 | CCF OF TEMPERATURE XMTR |

Table 4.10-6
Human Errors

| Basic Event | Probability | Description |
|----------------------|-------------|--|
| P21-XHE-FO-STDBYPUMP | 2.69E-01 | OP. FAILS MAN. ACT. OF STDBY PUMP WHEN AUT. ACTUATION FAIL |

Table 4.10-7
System Basic Events

| Basic Event | Probability | Description |
|----------------------|-------------|---|
| C62-DTM-FC-N1EA | 9.00E-04 | DIGITAL TRIP MODULE TRAIN A NO 1E FAILS |
| C62-DTM-FC-N1EB | 9.00E-04 | DIGITAL TRIP MODULE TRAIN B NO 1E FAILS |
| H23-RMU-FC-N038B | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C002A1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C002A2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C002B1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C002B2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C003A1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C003A2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C003B1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21C003B2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P21N038A | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| P21-ACV-FT-F022A | 2.00E-03 | AIR OPERATED VALVE F022A FAILS TO TRANSFER |
| P21-ACV-FT-F022B | 2.00E-03 | AIR OPERATED VALVE F022B FAILS TO TRANSFER |
| P21-ACV-FT-F025A | 2.00E-03 | AIR OPERATED VALVE F025A FAILS TO TRANSFER |
| P21-ACV-FT-F025B | 2.00E-03 | AIR OPERATED VALVE F025B FAILS TO TRANSFER |
| P21-ACV-OC-F010 | 3.60E-06 | AIR OPERATED VALVE F010 FAILS TO REMAIN OPEN |
| P21-ACV-OC-F013 | 3.60E-06 | AIR OPERATED VALVE F013 FAILS TO REMAIN OPEN |
| P21-ACV-OC-F027 | 3.60E-06 | AIR OPERATED VALVE F027 FAILS TO REMAIN OPEN |
| P21-ACV-OC-F032 | 3.60E-06 | AIR OPERATED VALVE F032 FAILS TO REMAIN OPEN |
| P21-HX_-LK-B001A | 2.40E-05 | HEAT EXCHANGER B001A FAILS WHILE OPERATING |
| P21-HX_-LK-B001B | 2.40E-05 | HEAT EXCHANGER B001B FAILS WHILE OPERATING |
| P21-HX_-LK-B002A | 7.20E-04 | HEAT EXCHANGER FAILURE (LEAK OR RUPTURE) |
| P21-HX_-LK-B002B | 7.20E-04 | HEAT EXCHANGER B002B FAILS WHILE OPERATING |
| P21-HX_-LK-B003A | 7.20E-04 | HEAT EXCHANGER (LEAK OR RUPTURE) |
| P21-HX_-LK-B003B | 7.20E-04 | HEAT EXCHANGER B003B FAILS WHILE OPERATING |
| P21-HX_-TM-B002A | 2.20E-03 | HEAT EXCHANGER B002A UNAVAILABLE DUE TO MAINTENANCE |
| P21-HX_-TM-B002B | 2.20E-03 | HEAT EXCHANGER B002B UNAVAILABLE DUE TO MAINTENANCE |
| P21-MCB-CC-C001A | 1.00E-03 | MOTOR DRIVEN PUMP CIRCUIT BEAKER FAILS TO OPEN |
| P21-MCB-CC-C001B | 1.00E-03 | MOTOR DRIVEN PUMP CIRCUIT BEAKER FAILS TO OPEN |

Table 4.10-7
System Basic Events

| Basic Event | Probability | Description |
|---------------------|--------------------|--|
| P21-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER OF PUMP C001A FAILS TO CLOSE |
| P21-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER OF PM C001B FAILS TO CLOSE |
| P21-MCB-OO-C002A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MCB-OO-C002B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MCB-OO-C003A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MCB-OO-C003B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MOV-OC-F016A | 3.36E-06 | MOTOR OPERATED VALVE F016A FAILS TO REMAIN OPEN |
| P21-MOV-OC-F016B | 3.36E-06 | MOTOR OPERATED VALVE F016B FAILS TO REMAIN OPEN |
| P21-MOV-OC-F018A | 1.01E-04 | MOTOR OPERATED VALVE F018A FAILS TO REMAIN OPEN |
| P21-MOV-OC-F018B | 1.01E-04 | MOTOR OPERATED VALVE F018B FAILS TO REMAIN OPEN |
| P21-MOV-OC-F020A | 1.01E-04 | MOTOR OPERATED VALVE F020A FAILS TO REMAIN OPEN |
| P21-MOV-OC-F020B | 1.01E-04 | MOTOR OPERATED VALVE MV-F020B FAIL TO REMAIN OPEN |
| P21-MP_-FR-C001A | 1.20E-04 | MOTOR DRIVEN PUMP C001A FAILS TO RUN |
| P21-MP_-FR-C001B | 1.20E-04 | MOTOR DRIVEN PUMP C001B FAILS TO RUN |
| P21-MP_-FR-C002A | 1.20E-04 | MOTOR-DRIVEN PUMP C002A FAILS TO RUN |
| P21-MP_-FR-C002B | 1.20E-04 | MOTOR-DRIVEN PUMP C002B FAILS TO RUN |
| P21-MP_-FR-C003A | 1.20E-04 | MOTOR-DRIVEN PUMP C0003A FAILS TO RUN |
| P21-MP_-FR-C003B | 1.20E-04 | MOTOR-DRIVEN PUMP C003B FAILS TO RUN |
| P21-MP_-FS-C001A | 1.30E-03 | MOTOR DRIVEN PUMP C001A FAILS TO START |
| P21-MP_-FS-C001B | 1.30E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO START |
| P21-MP_-FS-C002A | 1.30E-03 | MOTOR-DRIVEN PUMP (ALL TYPES) FAILS TO START |
| P21-MP_-FS-C002B | 1.30E-03 | MOTOR-DRIVEN PUMP C002B FAILS TO START |
| P21-MP_-FS-C003A | 1.30E-03 | MOTOR-DRIVEN PUMP C0003A FAILS TO START |
| P21-MP_-FS-C003B | 1.30E-03 | MOTOR-DRIVEN PUMP C003B FAILS TO START |
| P21-MP_-TM-C002A | 2.00E-03 | PUMP C002A IN MAINTENANCE |
| P21-MP_-TM-C002B | 2.00E-03 | PUMP C002B IN MAINTENANCE |
| P21-TT_-NO-N038B | 8.40E-06 | TEMPERATURE XMTR FAILS TO RESPOND TO CHANGE IN TEMPERATURE |
| P21-TT_-NO-P21N038A | 8.40E-06 | TEMPERATURE XMTR FAILS TO RESPOND TO CHANGE IN TEMPERATURE |
| P21-UV_-CC-F001A | 2.00E-04 | CHECK VALVE F001A FAILS TO OPEN |
| P21-UV_-CC-F001B | 2.00E-04 | CHECK VALVE F001B FAILS TO OPEN |

Table 4.10-7
System Basic Events

| Basic Event | Probability | Description |
|--------------------|--------------------|---|
| P21-UV_-CC-F005A | 2.00E-04 | CHECK VALVE F005A FAILS TO OPEN |
| P21-UV_-CC-F005B | 2.00E-04 | CHECK VALVE F005B FAILS TO OPEN |
| P21-UV_-CC-F007A | 2.00E-04 | CHECK VALVE F007A FAILS TO OPEN |
| P21-UV_-CC-F007B | 2.00E-04 | CHECK VALVE F007B FAILS TO OPEN |
| P41-ACV-OC-F002A | 3.60E-06 | AIR OPERATED VALVE MV-F002A FAIL TO REMAIN OPEN |
| P41-ACV-OC-F002B | 3.60E-06 | AIR OPERATED VALVE MV-F002B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F003A | 3.60E-06 | MOTOR OPERATED VALVE FAILS TO REMAIN OPEN |
| P41-ACV-OC-F003B | 3.60E-06 | AIR OPERATED VALVE MV-F003B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F004A | 3.60E-06 | AIR OPERATED VALVE MV-F004A FAIL TO REMAIN OPEN |
| P41-ACV-OC-F004B | 3.60E-06 | AIR OPERATED VALVE MV-F004B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F005A | 3.60E-06 | AIR OPERATED VALVE MV-F005A FAIL TO REMAIN OPEN |
| P41-ACV-OC-F005B | 3.60E-06 | AIR OPERATED VALVE MV-F005B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F006A | 3.60E-06 | AIR OPERATED VALVE MV-F006A FAIL TO REMAIN OPEN |
| P41-ACV-OC-F006B | 3.60E-06 | AIR OPERATED VALVE MV-F006B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F007A | 3.60E-06 | AIR OPERATED VALVE MV-F007A FAIL TO REMAIN OPEN |
| P41-ACV-OC-F007B | 3.60E-06 | AIR OPERATED VALVE MV-F007B FAIL TO REMAIN OPEN |
| P41-ACV-OC-F010A | 3.60E-06 | AIR OPERATED VALVE P41-F010A FAILS TO REMAIN OPEN |
| P41-ACV-OC-F010B | 3.60E-06 | AIR OPERATED VALVE MV-F010B FAIL TO REMAIN OPEN |

Table 4.10-8
System Top Events

| TOP EVENT | Description |
|------------------|-------------------------------------|
| GP21-0001-_1 | RCCWS TRAINS FAIL TO COOL THE LOADS |



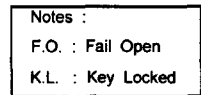


Figure 4.10-1 (2/2). Reactor Component Cooling Water System Flow Diagram

4.11 PLANT SERVICE WATER SYSTEM - (P41)

4.11.1 Purpose

PSW (P41) removes heat, during normal and shutdown operations, from the Reactor Cooling Component Water (RCCW) and Turbine Component Cooling Water (TCCW) systems and rejects this heat to the environment via two mechanical cooling towers.

Plant Service Water System has no safety-related functions.

4.11.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) The normal supply source for the pumps is the Cooling Tower Makeup System (Y41). Upon a malfunction of this system, the cooling water supply to the exchangers is supplied from the PSW pumps. The malfunction of the Y41 system is currently represented by special event P41-SYS-FF-3CTMP in the event that the flow from two pumps is required (malfunction of three pumps) and by special event P41-SYS-FF-2CTMP if flow from three pumps is required (malfunction of two pumps).
- (2) The volume of water in the Plant Service Water Basin is sufficient to supply water to the pumps during 72 hours, including losses through the cooling towers.
- (3) All of the components of train A receive power supply from Division I, while those from train B are powered from Division II.
- (4) It has been assumed that the relevant measures will be taken so that the air operated valves do not depend on the instrument air system in an emergency situation.
- (5) In the event of malfunction of a pump, another pump from the same train receives an initiation signal. This signal is considered to be sent from internal feedback, therefore, it is not modeled (except for the transmission of the signal within the division, which is included).
- (6) In the event of LOPP with malfunction of the startup of one pump that was in operation, no attention is paid to the manual startup from the control room. The support actuation for automatic startup is considered in the event of malfunction of the automatic startup on a pump on standby. In this case, a signal from the operating pump circuit breaker actuates the pump in standby, and if the automatic signal fails, the operator initiates the standby pump.
- (7) The valves at the discharge of the pumps to the towers, F005A/B, F011A/B, F012 and F013, are locked open (key-locked).
- (8) The mechanical draft cooling tower has multiple cells. Each tower cell set has a two-speed reversible 100% motor fan units or sets of 100% fans, with 50% of each tower cells powered from each electrical division.

4.11.3 System Description

The PSWS analysis described here refers to the system configuration shown in the diagram of Figure 4.11-1.

4.11.3.1 Hardware Configuration

PSW comprises two interconnected divisions of cooling water, each division consisting of 2 vertical pumps that take suction from a plant service water basin. Each pump is equipped with a self cleaning discharge strainer, discharge check valve and discharge isolation valve. The pumps discharge to a common header from which two parallel piping trains are fed. This configuration allows for increased flexibility during all operational modes and minimizes actions required in case of loss of PSWS. After passing through the RCCW and TCCW heat exchangers, the heated water from each train returns through two mechanical draft cooling towers (MCT).

The primary source of cooling water for all modes of operation are four Cooling Tower Makeup Water (CTMW) pumps. Two CTMW pumps are normally in operation sending water through P21 and P22 heat exchangers and discharging to the Main Cooling Tower basin to provide makeup water to account for evaporation losses associated with the operation of cooling towers. The backup source of cooling water when the CTMW are not in operation are PSWS pumps. Each pump is sized to provide a maximum flow rate of 4540 m³/h (20,000 gpm).

PSWS receives electrical power from the Standby On Site AC Power Supply System via two independent buses. During LOPP, the pumps can be powered from their respective divisional non-safety related onsite standby diesel generator.

4.11.3.2 System Operation

Initially, there are two pumps in operation. It is assumed that one pump in each train provides cooling flow to two heat exchangers from the RCCW system and one heat exchanger of the TCCW system.

Given a plant trip, the system configuration changes depending on the initiating event that caused the trip:

- Non-LOPP events: operation of two pumps is required for the cooling of the loads and a cooling tower. Pumps 1A and 1B are assumed to be in operation, and the remaining two pumps are in standby. Water flows through the RCCW heat exchangers and the TCCW heat exchangers. In the event of malfunction of one pump, a different pump from the same train initiates.
- All the train interconnection valves remain open.
- LOPP events: one pump of each train is in operation when the LOPP occurs. Both pumps trip, and they restart after power is recovered. In the event of a LOPP with startup of the diesel generators, flow from three PSW pumps and one cooling tower is required.
- Motor-operated and air valves remain open.

The same success criteria applies to the pumps of the Cooling Tower Makeup System (Y41).

4.11.3.3 Components Location

The TCCW and RCCW heat exchangers are located in the Turbine Building. Therefore, the PSWS piping that is connected to these heat exchangers is routed through sections of the Turbine Building.

The PSWS pumps, discharge strainers, a section of PSWS piping, and the mechanical draft cooling towers are located in the Service and Fire Building.

4.11.4 Automatic and Manual Control

In case of failure of one working pump, one standby pump is automatically actuated. PSWS is designed for remote operation from the Main Control Room.

Manual initiation of the standby pump is also possible. Status lights for valves and pumps are expected to be in the control room.

4.11.5 System Interfaces

The PSWS functional dependencies are indicated by the dependency matrix given in Table 4.11-2.

4.11.6 System Testing

The PSWS is a non-safety related system so that no Technical Specifications exist for it.

However, a periodic change in train operation is assumed such that the relevant train components are checked quarterly, as indicated in Table 4.11-3.

No plant outage time is required for testing.

4.11.7 System Maintenance

Considering the redundancy of the PSWS, maintenance can be performed on pumps and/or heat exchangers during power operation.

As regards the models, it has been considered that maintenance can be performed on both trains, granted it is not performed simultaneously and it does not affect more than one pump or heat exchanger in each train (see Table 4.11-4).

4.11.8 Common Cause Failures

The common cause failures are set out in Table 4.11-5.

4.11.9 Fault Tree Analysis

- A fault tree has been developed. Its TOP (gate GP41-0001-_1) represents the system malfunctions with insufficient cooling flow from the PSWS.
- The TOP includes two situations that are mutually exclusive.
- Gate GP41-0001-_2 represents the system malfunctions during normal and shutdown cooling operation.
- Gate GP41-0001-_3 represents the system malfunctions after a Loss of Preferred Power (LOPP).

The LOPP event is represented with event XHOSLOSP, which would have value TRUE, so gate GP41-0001-_2 would be cancelled. Otherwise, the gate that would be cancelled would be GP41-0001-_3.

4.11.9.1 Top Event Definitions

- A fault tree has been developed with gate GP41-0001-_1 as the TOP. It represents the system failures with insufficient cooling flow from the PSWS.

4.11.9.2 Model Assumptions

- System limits: For the purpose of the PRA, PSW system includes PSW pumps, suction lines, discharge lines up to heat exchangers connections, interconnection line and associated valves, cooling towers and fan coolers. RCCW and TCCW heat exchangers, piping and valves are included in P21 and P22 models.
- Gate GP41-0001-_2 shows the system malfunctions during non-LOPP situations. The success criterion in this situation is the makeup with two pumps and cooling through a cooling tower. The system is considered to be initially aligned with a pump of each train running. The interconnection between trains must be open in order to guarantee the cooling to the consumers, regardless of the train where the loads to be cooled are included and of the configuration of the pumps and exchangers.
- Gate GP21-0001-_3 shows the system malfunctions upon a loss of preferred power (LOPP). In this case, the success criterion is the makeup with three pumps and one cooling tower. It is considered that the pumps are tripped as a consequence of the LOPP. The position of the valves is not modified in this case. The model shows the need to start up three pumps and to cool their flow by means of one tower. The connection of trains guarantees the cooling of the consumers regardless of the train they belong to and of the initial configuration of the pumps.
- In non-LOPP events the failure of one running pump is considered a mechanical failure and it is not considered recoverable and manual actions are not taken into account. The support actuation for automatic startup is considered in the event of malfunction of the automatic startup on a pump on standby. In this case, a signal directly from the operating pump breaker actuates to the pump on standby, and if the automatic signal fails, the operator should start up the pump on standby. This human action is represented by means of basic event P41-XHE-FO-STDBYPUMP.
- After a LOPP, 3 PSW pumps must initiate. In this case, it is considered that the automatic start signals are the diesel generator start signals and are represented by gates GR21-0025-_2 and GR21-0025-_1. If these signals fail, the operator initiates the pump in standby.
- No malfunctions because of incorrect positioning of manual or air operated valves are considered because the system is normally in operation with periodic rotation of the trains and the motor-operated valves locked open (key-locked). Alarms, administrative controls, etc. allow immediate detection of the improper closing of any valve.
- Loss of supply from the Plant Service Water Basin is not considered to be a cause of system malfunction because the design of the system includes supply from system Y41. This system has been represented with a basic event in each part of the tree that shows the corresponding success criterion.

- The model includes the need to have one group of fan units in one tower available for cooling.
- During power operation, one pump or one heat exchanger of each train can be under maintenance. In the model, the maintenance events are assigned to exchangers 2A or 2B and to pumps 2A or 2B, always supposing their maintenance does not coincide.
- The two interconnection lines provide flow to the consumers of any train for any combination of pumps and heat exchangers that meets the success criteria of the system.
- Air operated valves open if instrument air supply fails, so their dependency from the instrument air supply system is not modeled.
- The valves at the discharge of the pumps to the towers, F005A/B, F011A/B, F012 and F013, are locked open (key-locked). Therefore, it is considered that they cannot be in a wrong position because there are alarms, administrative controls, etc., that would allow immediate detection of the improper closing of a valve.
- In case one of the pumps fails, another pump from the same train receives an initiation signal. It is assumed that this signal is generated directly from the running pump circuit breaker. It is also assumed that the pumps can be started from the control room.
- In case of LOPP, standby pumps receive starting an initiation directly from the DGs starting signal. These signals are represented by gates GR21-0025-_1 and GR21-0025-_2.
- CTMS power is assumed to be the same supply as PSW and failure of PSW power also fails CTMS. In addition, the CTMS water source is considered not to be limiting. No other supports are required for CTMS.

4.11.9.3 Human Interactions

One human action is considered in the fault tree:

P21-XHE-FO-STDBYPUMP represents the operator fails to manually start the standby pump when automatic actuation fails.

This human action is connected to the tree by means of event XHOSMNA, which represents the availability of the operator to perform manual actuations.

4.11.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.11-7.

The fault tree results provide quantitative values of system unavailability and of the importance of specific components to that total.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This qualification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.11.11 Model Differences for 72-Hour Operation

The long term PSWS operation is similar to that during the first 24 hours following an initiating event. There are no fault tree model changes in logic necessary to extend the operation to 72 hours.

4.11.12 References

4.11-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.11-1
Control Room Instrumentation and Alarms

| |
|---|
| Main Control Room Panel Controls |
| PSW Pump 1A start / stop and control mode status |
| PSW Pump 2A start / stop and control mode status |
| PSW Pump 1B start / stop and control mode status |
| PSW Pump 2B start / stop and control mode status |
| Main Control Console Displays |
| PSW pump 1A motor current, winding temperature and bearing temperature |
| PSW pump 2A motor current, winding temperature and bearing temperature |
| PSW pump 1B motor current, winding temperature and bearing temperature |
| PSW pump 2B motor current, winding temperature and bearing temperature |
| PSWS discharge pressure and discharge flow |
| PSW pump 1A strainer differential pressure |
| PSW pump 2A strainer differential pressure |
| PSW pump 1B strainer differential pressure |
| PSW pump 2B strainer differential pressure |
| PSWS pump discharge header pressure and temperature |
| Motor operated valves position |
| Main Control Room Alarms |
| High differential pressure across pump discharge strainers |
| PSWS supply pressure below a predetermined set point |
| Low PSWS pump discharge flow |
| PSWS supply temperature above/below a predetermined setpoint |
| PSWS pump motor overload trip |
| PSWS standby pump auto-start |
| A significant difference between the return header flowmeter reading and the pump discharge |
| PSWS pump motor winding high temperature |
| PSWS pump motor bearing high temperature |
| PSWS high radioactivity level |

Table 4.11-2
System Dependencies Matrix

| Component | Transmission Signal C62 DID | | AC Power | | Uninterruptible AC | | IA | Room Cooling ⁽³⁾ |
|----------------------|--------------------------------|-------|-----------|-----------|--------------------|-------------------|-----|--------------------------------|
| | Div A | Div B | RH Bus A2 | RH Bus B2 | R13 Div A | R13 Div B B1-1 | | |
| PSW PMP C001A | X | | X | | X | | | |
| PSW PMP C002A | X | | X | | X | | | |
| PSW PMP C001B | | X | | X | | X | | |
| PSW PMP C002B | | X | | X | | X | | |
| F002A MOV | X | | X | | X | | | |
| F004A MOV | X | | X | | X | | | |
| D001A ⁽¹⁾ | X | | X | | X | | | |
| D002A ⁽¹⁾ | X | | X | | X | | | |
| F002A AOV | X | | | | X | | (2) | |
| F006A AOV | X | | | | X | | (2) | |
| F004A AOV | X | | | | X | | (2) | |
| F009A AOV | X | | | | X | | (2) | |
| F001A AOV | X | | | | X | | (2) | |
| F007A AOV | X | | | | X | | (2) | |
| F005A AOV | X | | | | X | | (2) | |
| F003A AOV | X | | | | X | | (2) | |
| F008A AOV | X | | | | X | | (2) | |
| F005A MOV | X | | X | | X | | | |
| FAN 1A | X | | X | | X | | | |
| FAN 2A | X | | X | | X | | | |
| CTM PMP C001A | X | | X | | X | | | |
| CTM PMP C002A | X | | X | | X | | | |
| CTM PMP C001B | | X | | X | | X | | |
| CTM PMP C002B | | X | | X | | X | | |
| F002B MOV | | X | | | | X | | |
| F004B MOV | | X | | | | X | | |
| D001B ⁽¹⁾ | | X | | | | X | | |
| D002B ⁽¹⁾ | | X | | | | X | | |
| F002B AOV | | X | | | | X | (2) | |

Table 4.11-2
System Dependencies Matrix

| Component | Transmission Signal C62 DID | | AC Power | | Uninterruptible AC | | IA | Room Cooling ⁽³⁾ |
|-----------|--------------------------------|-------|-----------|-----------|--------------------|-------------------|-----|--------------------------------|
| | Div A | Div B | RH Bus A2 | RH Bus B2 | R13 Div A | R13 Div B B1-1 | | |
| F006B AOV | | X | | | | X | (2) | |
| F004B AOV | | X | | | | X | (2) | |
| F009B AOV | | X | | | | X | (2) | |
| F001B AOV | | X | | | | X | (2) | |
| F007B AOV | | X | | | | X | (2) | |
| F005B AOV | | X | | | | X | (2) | |
| F003B AOV | | X | | | | X | (2) | |
| F008B AOV | | X | | | | X | (2) | |
| F005B MOV | | X | | X | | X | | |
| FAN 1B | | X | | X | | X | | |
| FAN 2B | | X | | X | | X | | |

Notes to Table 4.11-2

- (1) Double basket strainer.
- (2) The AOVs are provided with IA. The AOVs are fail open valves. If necessary for system operation, AOVs will be made operable by alternative means.
- (3) There are no active room cooling system requirements for 72 hours.

Table 4.11-3
Component Tests

A periodic change in train operation is assumed such that the relevant train components are checked quarterly.

Table 4.11-4
Component Maintenance

| Basic Event | Probability | Description |
|------------------|-------------|---------------------------|
| P41-MP_-TM-C002A | 2.00E-03 | PUMP C002A IN MAINTENANCE |
| P41-MP_-TM-C002B | 2.00E-03 | PUMP C002B IN MAINTENANCE |

Table 4.11-5
Common Cause Failure

| Basic Event | Probability | Description |
|-----------------|-------------|--|
| H23-RMU-CF-P41A | 3.60E-06 | CCF TO OPERATERMU 001 AND RMU 002 P41 TRAIN A FOR MANUAL ACTUAT. |
| H23-RMU-CF-P41B | 3.60E-06 | CCF TO OPERATERMU 001 AND RMU 002 P41 TRAIN B FOR MANUAL ACTUAT. |
| P41-FAN-CR-3ALL | 7.20E-07 | CCF TO RUN 3 FAN UNITS |
| P41-FAN-CS-2ALL | 7.20E-06 | CCF TO START 2 FAN UNITS |
| P41-FAN-CS-3ALL | 1.80E-06 | CCF TO START 3 FAN UNITS |
| P41-MP_-CR-2ALL | 3.39E-06 | CCF TO RUN 2 PUMPS TRAINS A AND B |
| P41-MP_-CR-3ALL | 1.17E-05 | CCF TO RUN 3 PUMPS TRAINS A AND B |
| P41-MP_-CS-2ALL | 4.29E-05 | CCF TO START 2 PUMPS TRAINS A AND B |
| P41-MP_-CS-3ALL | 6.26E-06 | CCF TO START PUMPS TRAINS A AND B |
| P41-STR-CF-2ALL | 1.07E-05 | CCF 2 STRAINERS PLUGGED |
| P41-STR-CF-3ALL | 1.07E-05 | CCF 3 STRAINERS PLUGGED |

Table 4.11-6
Human Errors

| Basic Event | Probability | Description |
|----------------------|-------------|---|
| P41-XHE-FO-STDBYPUMP | 1.77E-02 | OPER. FAILS TO MAN. ACTUATE STDBY PUMP WHEN AUT. ACT. FAIL |

Table 4.11-7
List of System Basic Events

| Basic Event | Probability | Description |
|----------------------|-------------|--|
| H23-RMU-FC-P41C002A1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P41C002A2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P41C002B1 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-P41C002B2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| P41-FAN-FR-0001A | 2.40E-04 | FAN FAILS TO RUN |
| P41-FAN-FR-0001B | 2.40E-04 | FAN FAILS TO RUN |
| P41-FAN-FR-0002A | 2.40E-04 | FAN FAILS TO RUN |
| P41-FAN-FR-0002B | 2.40E-04 | FAN FAILS TO RUN |
| P41-FAN-FS-0001A | 6.00E-04 | FAN FAILS TO START |
| P41-FAN-FS-0001B | 6.00E-04 | FAN FAILS TO START |
| P41-FAN-FS-0002A | 6.00E-04 | FAN FAILS TO START |
| P41-FAN-FS-0002B | 6.00E-04 | FAN FAILS TO START |
| P41-MCB-CC-C001A | 1.00E-03 | MEDIUM CIRCUIT BREAKER (VOLTAGE > 480 V) (FAILURE TO OPEN) |
| P41-MCB-CC-C001B | 1.00E-03 | MEDIUM CIRCUIT BREAKER (VOLTAGE > 480 V) (FAILURE TO OPEN) |
| P41-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER OF PUMP C001A FAILS TO CLOSE |
| P41-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER PUMP C001B FAILS TO CLOSE |
| P41-MCB-OO-C002A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P41-MCB-OO-C002B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P41-MOV-OC-F002A | 3.36E-06 | MOTOR OPERATED VALVE MV-F002A FAIL TO REMAIN OPEN |
| P41-MOV-OC-F002B | 3.36E-06 | MOTOR OPERATED VALVE MV-F002B FAIL TO REMAIN OPEN |
| P41-MOV-OC-F004A | 3.36E-06 | MOTOR OPERATED VALVE MV-F004A FAIL TO REMAIN OPEN |
| P41-MOV-OC-F004B | 3.36E-06 | MOTOR OPERATED VALVE MV-F004B FAIL TO REMAIN OPEN |
| P41-MP_-FR-C001A | 7.68E-04 | MOTOR DRIVEN PUMP C001A FAILS TO RUN |
| P41-MP_-FR-C001B | 7.68E-04 | MOTOR DRIVEN PUMP C001B FAILS TO RUN, |
| P41-MP_-FR-C002A | 7.68E-04 | MOTOR DRIVEN PUMP C002A FAILS TO RUN |
| P41-MP_-FR-C002B | 7.68E-04 | MOTOR-DRIVEN PUMP C002B FAILS TO RUN |
| P41-MP_-FS-C001A | 2.40E-03 | MOTOR-DRIVEN PUMP C001A FAILS TO RESTART |
| P41-MP_-FS-C001B | 2.40E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO START |
| P41-MP_-FS-C002A | 2.40E-03 | MOTOR-DRIVEN PUMP C002A FAILS TO START |
| P41-MP_-FS-C002B | 2.40E-03 | MOTOR-DRIVEN PUMP C002B FAILS TO START |

Table 4.11-7
List of System Basic Events

| Basic Event | Probability | Description |
|--------------------|--------------------|---|
| P41-STR-PG-D01A | 2.40E-04 | STRAINER P41-D001 A PLUGGED |
| P41-STR-PG-D01B | 2.40E-04 | STRAINER P41-D001 B PLUGGED |
| P41-STR-PG-D02A | 3.59E-03 | STRAINER P41-D002A PLUGGED |
| P41-STR-PG-D02B | 3.59E-03 | STRAINER P41-D002B PLUGGED |
| P41-SYS-FF-2CTMP | 5.04E-05 | INSUFFICIENT FLOW FROM CTM PUMPS (2 PUMPS FAIL) |
| P41-SYS-FF-3CTMP | 9.20E-06 | INSUFFICIENT FLOW FROM CTM PUMPS (3 PUMPS FAIL) |
| P41-UV_-CC-F001A | 2.00E-04 | CHECK VALVE F001A FAILS TO OPEN |
| P41-UV_-CC-F001B | 2.00E-04 | CHECK VALVE F001B FAILS TO OPEN |
| P41-UV_-CC-F003A | 2.00E-04 | CHECK VALVE F003A FAILS TO OPEN |
| P41-UV_-CC-F003B | 2.00E-04 | CHECK VALVE F003B FAILS TO OPEN |

Table 4.11-8
List of System Top Events

| TOP EVENT | Description |
|------------------|------------------------------|
| GP41-0001-_1 | PSWS FAILS TO COOL THE LOADS |

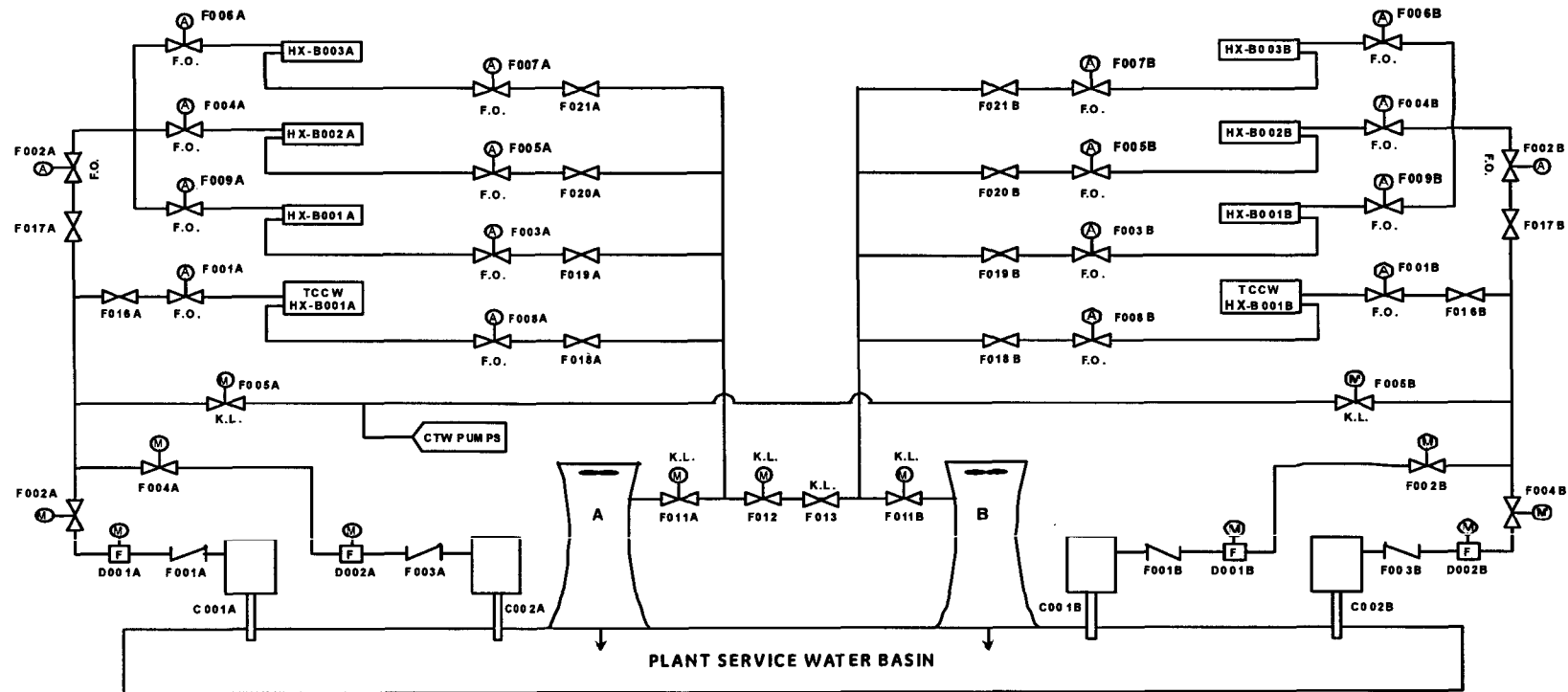


Figure 4.11-1. Plant Service Water System Simplified Flow Diagram for Plant Service Water System

4.12 INSTRUMENT AIR SYSTEM AND SERVICE AIR SYSTEM - (P51 and P52)

4.12.1 Purpose

The function of the Instrument Air System (IAS) is to provide a continuous supply of dry, oil free, filtered compressed air for pneumatic equipment, valves, controls, and instrumentation located in the following areas: areas outside of the primary containment in the Reactor Building, Turbine Building, Radwaste Building, Control Building, Fuel Storage Building, Electrical Building, Hot and Cold Machine Shop. It also supplies air to nitrogen consumers during refueling. The IAS supplies compressed air to the primary containment supply piping for non-essential instrumentation and control equipment during refueling. IAS piping transports nitrogen from the HPNSS to users inside the primary containment during normal operation. It is also used as a backup supply for the High Pressure Nitrogen Supply System (HPNSS) in case nitrogen pressure drops below the system low-pressure setpoint.

The function of the Service Air System (SAS) is to provide a continuous supply of compressed air for tank sparging, pneumatic tools, filter/demineralizer backwashing, and other services that do not require air that has been processed to the same levels as what is provided by the IAS. SAS provides a backup supply to the IAS. SAS also provides a supply of low pressure, purified compressed air for breathing air.

4.12.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) The electrical supply for the dryers is assumed to be the same as for the compressors.
- (2) The load/unload operation for compressor and dryer trains is assumed to be changed monthly.

4.12.3 System Description

4.12.3.1 Hardware Configuration

The IAS is a non-safety related system consisting of two 100 percent air compressor units arranged in parallel which supply air, properly filtered, cleaned, and dried, to two 50 percent capacity receivers. These, in turn, supply air to either of two 100 percent dryers whose outputs join at a common header and into a receiver/surge tank. From this point, several branches are routed to the different plant buildings and on to the individual users.

A system interconnection with the SAS is provided downstream of the air receiver and upstream of the dryers as a backup supply of compressed air. The SAS provides air automatically to the IAS via a tie line if the IAS pressure falls below a desired limit. Another interconnection via a normally closed air-operated valve is provided to supply air to the HPNSS users if the HPNSS is unavailable.

The SAS has a similar structure, including two 50 percent air compressor units, two breathing air purifiers, and related valves.

4.12.3.2 System Operation

The IAS and the SAS are continuously operating. One air compressor normally operates to feed two receivers and one filter and dryer train. The compressors operate on a load/unload sequence with one compressor selected as the lead compressor and the other in standby.

The standby compressor automatically starts when the air pressure at the receiver drops below a pre-determined set point. As the receiver pressure is returned to within normal range, the standby compressor stops and the lead unit is kept in operation. The lead and standby air compressors are switched periodically; assignment is performed at the local master control panel.

The skid of the standby dryer automatically starts when the differential pressure across the dryer, discharge pressure, or the dew point reaches the predetermined set points. The standby unit is warmed prior to start and prior to switching off an operating dryer skid. The assignment of lead or standby dryer is switched periodically and performed at the local master control panel.

4.12.3.3 Component Location

All the main components are located in the turbine building.

4.12.4 Automatic and Manual Control

4.12.4.1 Automatic Actuation

The standby compressor automatically starts when the air pressure at the receiver drops below a set point. As the receiver pressure is returned to the normal range, the standby compressor stops and the lead unit is kept in operation.

The valve in the connection line with the SAS opens automatically in the event that the receiver discharge header pressure continues to fall below a predetermined low-low pressure value.

When the high moisture switches at the operating air dryer outlets indicate high moisture or low pressure detected downstream of the afterfilter, the isolation valve upstream of the standby air dryer opens and the standby air dryer starts.

The IAS automatically switches to the standby AC power source during a loss of preferred power (LOPP).

4.12.4.2 Manual Actuation

The assignment of lead and stand-by air compressors is performed at the local master control panel.

The stand-by compressors, for both the IAS and the SAS, can be started locally and manually following a failure of the automatic actuation. The connection between the IAS and the SAS can be opened locally and manually after automatic actuation has failed.

4.12.4.3 Instrumentation and Alarms

Control room instrumentation and alarms are listed in Table 4.12-1.

4.12.5 System Interfaces

Table 4.12-2 shows a dependency matrix for the systems.

The supporting systems are the Medium Voltage Distribution System for the compressors and the dryers, the Low Voltage Distribution System for the motor operated containment isolation valves, the Uninterruptible AC Power Supply System for controls and instrumentation, the Reactor Component Cooling Water System (RCCW), which cools the IAS compressors, and the Turbine Component Cooling Water System (TCCW), which cools the SAS compressors.

4.12.6 System Testing

The SAS and IAS do not have in-service inspection requirements.

The system operational test for components normally closed to airflow is performed periodically to ensure system capability and integrity.

The only components that are subject to testing are the valves on the connection line between the IAS and the SAS and the pressure transmitters.

It is assumed that the valves are tested quarterly, while the transmitters are tested with every refueling.

The lead and stand-by compressor units follow an alternating operational cycle, so they are tested when called into operation.

4.12.7 System Maintenance

Maintenance of one compressor or one dryer at a time is possible during system operation, and the related unavailability is considered in the fault trees.

Filters are to be periodically inspected for the cleanliness, and the desiccant in the breathing air purifiers is sampled every six months to verify its useful life.

4.12.8 Common Cause Failures

Common cause failures (CCF) within the system are summarized in Table 4.12-5.

Calibration errors of the transmitters are judged to be included in the data for Common Cause Failure of the transmitters.

4.12.9 Fault Tree Analysis

4.12.9.1 Top Event Definitions

The top events considered in the study are the loss of air supply from the IAS and the loss of supply from the SAS.

4.12.9.2 Fault Tree Description

IAS and SAS fault trees have been developed to represent the loss of the instrument air supply and the loss of service air supply, Appendix B.4.12.

IAS and SAS fault trees are developed Appendix B.4.12.

4.12.9.3 Human Interaction

The following human interactions have been considered:

- Operator failure to recognize the need to align the SAS because of an IAS failure or failure when aligning
- Failure when actuating the compressors and dryers which are in standby mode

Table 4.12-6 reports the events considered with the related probabilities.

4.12.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.12-7.

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.12.11 Model Differences for 72-Hour Operation

There are no fault tree changes necessary to extend the operation to 72 hours.

4.12.12 References

4.12-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.12-1
Control Room Instrumentation and Alarms

| Alarms (IAS) |
|---|
| Compressor: |
| -High receiver air pressure |
| -High vibration |
| -High water level in the after cooler drip leg |
| -High compressor outlet air temperature |
| -Compressor low oil pressure and high oil temperature |
| -High cooling water discharge temperature |
| -High motor winding temperature |
| -High motor bearing temperature |
| -Motor overload |
| High temperature in the moisture separator outlet |
| Low receiver air pressure |
| High moisture content in the air dryer outlet |
| Low instrument air supply pressure |
| Standby power for IAS initiated |
| IAS standby dryer/filter train activated |
| High RCCWS temperatures |
| High pre-filter differential pressure |
| High after-filter differential pressure |
| |
| Alarms (SAS) |
| Compressor: |
| -High receiver air pressure |
| -High water level in the after cooler drip leg |
| -High compressor outlet air temperature |
| -Compressor low oil pressure and high oil temperature |
| -High cooling water discharge temperature |
| Increasing temperature in the moisture separator outlet above a predetermined setpoint |
| Decreasing pressure in the air receiver falling below a predetermined setpoint |
| Increasing moisture in the air dryer outlet rising above a predetermined setpoint |
| Increasing moisture in the air purifier outlet rising above a predetermined setpoint |
| Decreasing pressure in the air purifier outlet falling below a predetermined setpoint |
| Increasing pressure in the breathing air receiver rising above a predetermined setpoint |
| |
| Displays (IAS) |
| Instrument air receiver temperature |
| Instrument air receiver pressure |
| Instrument air dryer outlet pressure |
| Instrument air dryer prefilter differential pressure |
| Instrument air dryer afterfilter differential pressure |
| Instrument air header pressure |
| Instrument air dryer outlet moisture |

Table 4.14-2
System Dependency Matrix

| Component | Type | Support System | | | | | |
|-------------|--------|----------------|---------------|--------------------------|----------|----------------|-------|
| | | Power Supply | | | | Cooling System | |
| | | 480 V AC: | | Uninterruptible 208/120V | | | |
| | | Train A in TB | Train B in TB | R13-A2-1 | R13-B2-1 | RCCWS | TCCWS |
| IA Dryer 1 | Dryer | X | | X | | | |
| IA Dryer 2 | Dryer | | X | | X | | |
| IA Compr 1. | Compr. | X | | X | | X | |
| IA Compr. 2 | Compr. | | X | | X | X | |
| F006 | AOV | | | X | | | |
| SA Dryer 1 | Dryer | X | | X | | | |
| SA Dryer 2 | Dryer | | X | | X | | |
| SA Compr 1. | Compr. | X | | X | | | X |
| SA Compr. 2 | Compr. | | X | | X | | X |
| F015A | AOV | | | X | | | |
| F015B | AOV | | | | X | | |

Table 4.12-3
Component Test

| Component | Type | Expected Test Interval |
|------------------|--------------|-------------------------------|
| | | |
| F006 | AOV | 3 months |
| Transmitters | Transmitters | Refueling (24 months) |

Table 4.12-4
Component Maintenance

| Component | Unavailability |
|------------------|-----------------------|
| | |
| Compressors | 2E-5 |
| Dryers | 2E-3 |

Table 4.12-5
System Common Cause Failures

| Basic Event | Probability | Description |
|--------------------|-------------|--|
| | | |
| P52-ACV-CF-DEENERG | 2.20E-05 | CCF OF AOVs TO OPERATE TO OTHER THEN DEENERG. POS. |
| P52-ACV-CF-OPEN | 1.35E-05 | CCF OF AOV TO OPEN |
| P52-CMP-CR-C001AB | 1.49E-04 | CCF TO RUN COMPRESSORS LINES 1 & 2 |
| P52-CMP-CS-C001AB | 1.71E-03 | CCF TO START COMPRESSORS LINES 1 & 2 |
| P52-CMP-CS-P52/P51 | 1.00E-04 | CCF OF P52/P51 COMPRESSORS TO START |
| P52-PT_-CF-P51/P52 | 2.59E-06 | CCF OF PRESSURE TRANSMITTERS |
| C62-DTM-CF-N1EALL | 5.50E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |
| P51-ACV-CF-OPEN | 1.35E-05 | CCF OF AOV TO OPEN |
| P52-CMP-CR-RUN | 1.49E-04 | CCF OF COMPRESSORS TO RUN |
| P52-CMP-CR-START | 1.71E-03 | CCF OF COMPRESSORS TO START |

Table 4.12-6
System Human Errors

| Basic Event | Prob | Description |
|------------------------|----------|---|
| | | |
| P52-XHE-FO-ALIGNDRYER2 | 1.61E-03 | OPERATOR FAILS TO ACTUATE VALVE F015B TO ALIGN DRYER LINE 2 |
| P52-XHE-FO-ALIGNSAS | 1.61E-03 | OPERATOR FAILS TO ACTUATE TO ALIGN SAS |
| P52-XHE-FO-IAS/SAS | 1.61E-02 | OPER. FAILS TO REC. NEED FOR MANUAL INTERV. ON IAS/SAS |
| P52-XHE-FO-STDBYCOMP | 1.61E-03 | OPERATOR FAILS TO ACTUATE STAND-BY COMPRESSOR |
| P51-XHE-FO-STDBYCOMP | 1.77E-02 | OPERATOR FAILS TO ACTUATE STAND-BY COMPRESSOR |
| P52-XHE-FO-STDBYCOMP | 1.61E-03 | OPERATOR FAILS TO ACTUATE STAND-BY COMPRESSOR |

| Table 4.12-7 | | |
|-----------------------------|----------|---|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| P52-XHE-FO-STDBYCOMP | 1.61E-03 | OPERATOR FAILS TO ACTUATE STAND-BY COMPRESSOR |
| P52-XHE-FO-IAS/SAS | 1.61E-02 | OPER. FAILS TO REC. NEED FOR MANUAL INTERV. ON IAS/SAS |
| P52-XHE-FO-ALIGN SAS | 1.61E-03 | OPERATOR FAILS TO ACTUATE TO ALIGN SAS |
| P52-XHE-FO-ALIGN DRYER2 | 1.61E-03 | OPERATOR FAILS TO ACTUATE VALVE F015B TO ALIGN DRYER LINE 2 |
| P52-UV_-CC-F002B | 2.00E-04 | HEATER 2B CHECK VALVE FAILS TO OPEN |
| P52-UV_-CC-F002A | 2.00E-04 | HEATER 2A CHECK VALVE FAILS TO OPEN |
| P52-UV_-CC-F001B | 2.00E-04 | HEATER 1B CHECK VALVE FAILS TO OPEN |
| P52-UV_-CC-F001A | 2.00E-04 | HEATER 1A CHECK VALVE FAILS TO OPEN |
| P52-TNK-RP-A001B | 2.40E-06 | TANK RECEIVER TKA001B LEAKS CATASTROPHICALLY |
| P52-TNK-RP-A001A | 2.40E-06 | TANK RECEIVER TKA001A LEAKS CATASTROPHICALLY |
| P52-STR-PG-D009C | 3.59E-03 | STRAINER/FILTER D009C PLUGS DURING OPERATION |
| P52-STR-PG-D009A | 2.40E-04 | STRAINER/FILTER D009A PLUGS DURING OPERATION |
| P52-STR-PG-D005C | 3.59E-03 | STRAINER/FILTER D005C PLUGS DURING OPERATION |
| P52-STR-PG-D005A | 2.40E-04 | STRAINER/FILTER D009A PLUGS DURING OPERATION |
| P52-STR-PG-D001B | 3.59E-03 | STRAINER/FILTER D001B PLUGS DURING OPERATION |
| P52-STR-PG-D001A | 2.40E-04 | STRAINER/FILTER D001A PLUGS DURING OPERATION |
| P52-PT_-NO-A002 | 5.18E-04 | PRESSURE TRANSMITT. IA002 FAILS TO RESPOND TO CHANGE |
| P52-PT_-NO-A001 | 5.18E-04 | PRESSURE TRANSMITT. IA001 FAILS TO RESPOND TO CHANGE |
| P52-PT_-CF-P51/P52 | 2.59E-06 | CCF OF PRESSURE TRANSMITTERS |
| P52-HX_-PG-D007D | 7.92E-04 | HEATER (PRESSURIZER D007D ALL FAILURES [#2]) |
| P52-HX_-PG-D007C | 7.92E-04 | HEATER (PRESSURIZER D007C ALL FAILURES [#2]) |
| P52-HX_-PG-D007B | 5.28E-05 | HEATER (PRESSURIZER D007B ALL FAILURES [#2]) |
| P52-HX_-PG-D007A | 5.28E-05 | HEATER (PRESSURIZER D007A ALL FAILURES [#2]) |
| P52-HX_-PG-C001B | 3.60E-04 | COOLER COMPRESSOR C001B FAILS WHILE OPERATING |
| P52-HX_-PG-C001A | 2.40E-05 | COOLER COMPRESSOR C001A FAILS WHILE OPERATING |
| P52-DRY-TM-B002B | 2.00E-05 | DRYER K002B IN MAINTENANCE |
| P52-CMP-TM-C001B | 2.00E-03 | COMPRESSOR C001B IN MAINTENANCE |
| P52-CMP-FS-C001B | 2.00E-02 | MOTOR-DRIVEN AIR COMPRESS. C001B FAILS TO START |
| P52-CMP-FS-C001A | 2.00E-02 | MOTOR-DRIVEN AIR COMPRESS. C001A FAILS TO START |
| P52-CMP-FR-C001B | 2.40E-03 | MOTOR-DRIVEN AIR COMPRESS. C001B FAILS TO CONT.OPER. |
| P52-CMP-FR-C001A | 2.40E-03 | MOTOR-DRIVEN AIR COMPRESS. C001A FAILS TO CONT.OPER. |
| P52-CMP-CS-P52/P51 | 1.00E-04 | CCF OF P52/P51 COMPRESSORS TO START |
| P52-CMP-CS-C001AB | 1.71E-03 | CCF TO START COMPRESSORS LINES 1 & 2 |
| P52-CMP-CR-START | 1.71E-03 | CCF OF COMPRESSORS TO START |

| Table 4.12-7 | | |
|-----------------------------|----------|--|
| List of System Basic Events | | |
| Basic Event | Prob | Description |
| P52-CMP-CR-RUN | 1.49E-04 | CCF OF COMPRESSORS TO RUN |
| P52-CMP-CR-C001AB | 1.49E-04 | CCF TO RUN COMPRESSORS LINES 1 & 2 |
| P52-ACV-OC-F010F | 5.40E-05 | AOV FAILS F010F TO REMAIN OPEN (FOR FC VALVE)[#2] |
| P52-ACV-OC-F010D | 5.40E-05 | AOV F010D FAILS TO REMAIN OPEN (FOR FC VALVE)[#2] |
| P52-ACV-OC-F010C | 3.60E-06 | AOV FAILS F010C TO REMAIN OPEN (FOR FC VALVE) |
| P52-ACV-OC-F010A | 3.60E-06 | AOV F010A FAILS TO REMAIN OPEN (FOR FC VALVE)[#2] |
| P52-ACV-OC-F005B | 5.40E-05 | AOV F005A SPURIOUSLY TRANSFER TO ENERGIZED POSIT. |
| P52-ACV-OC-F005A | 3.60E-06 | AOV F005A SPURIOUSLY TRANSFER TO ENERGIZED POSIT. |
| P52-ACV-CF-OPEN | 1.35E-05 | CCF OF AOV TO OPEN |
| P52-ACV-CF-DEENERG | 2.20E-05 | CCF OF AOVs TO OPERATE TO OTHER THEN DEENERG. POS. |
| P52-ACV-CC-RTBUILD | 2.00E-03 | THE AOV TO RADWASTE REACTOR, AND TURB BUILD FAILS (NOFO) |
| P52-ACV-CC-F015B | 2.00E-03 | AIR OPERATED VALVE FAILS TO OPEN |
| P52-ACV-CC-F010F | 2.00E-03 | AOV F010F FAILS TO OPEN REPETITIOUSLY [#3] |
| P52-ACV-CC-F010D | 2.00E-03 | AOV F010D FAILS TO OPEN REPETITIOUSLY [#3] |
| P52-ACV-CC-F010C | 2.00E-03 | AOV F010C FAILS TO OPEN REPETITIOUSLY [#3] |
| P52-ACV-CC-F010A | 2.00E-03 | AOV F010A FAILS TO OPEN REPETITIOUSLY [#3] |
| P52-ACV-CC-F006 | 2.00E-03 | AOV F006 FAILS TO OPERATE TO NOT DEENERG.POS. |
| P51-XHE-FO-STDBYCOMP | 1.77E-02 | OPERATOR FAILS TO ACTUATE STAND-BY COMPRESSOR |
| P51-UV_-CC-F001D | 2.00E-04 | DESICCANT 1D CHECK VALVE FAILS TO OPEN |
| P51-UV_-CC-F001C | 2.00E-04 | DESICCANT 1C CHECK VALVE FAILS TO OPEN |
| P51-UV_-CC-F001B | 2.00E-04 | DESICCANT 1B CHECK VALVE FAILS TO OPEN |
| P51-UV_-CC-F001A | 2.00E-04 | DESICCANT 1A CHECK VALVE FAILS TO OPEN |
| P51-TNK-RP-A002 | 2.40E-06 | AIR RECEIVER TANK LEAKS CATASTROPHICALLY |
| P51-TNK-RP-A001B | 2.40E-06 | AIR RECEIVER TANK A001B LEAKS CATASTROPHICALLY |
| P51-TNK-RP-A001A | 2.40E-06 | AIR RECEIVER TANK A001A LEAKS CATASTROPHICALLY |
| P51-STR-PG-TK07D | 3.59E-03 | STRAINER/FILTER TK07D PLUGS DURING OPERATION |
| P51-STR-PG-TK07C | 3.59E-03 | STRAINER/FILTER TK07C PLUGS DURING OPERATION |
| P51-STR-PG-TK07B | 2.40E-04 | STRAINER/FILTER TK07B PLUGS DURING OPERATION |
| P51-STR-PG-TK07A | 2.40E-04 | STRAINER/FILTER TK07A PLUGS DURING OPERATION |
| P51-STR-PG-K002B | 3.59E-03 | STRAINER/FILTER K002B PLUGS DURING OPERATION |
| P51-STR-PG-K002A | 2.40E-04 | STRAINER/FILTER K002A PLUGS DURING OPERATION |
| P51-STR-PG-D005C | 3.59E-03 | STRAINER/FILTER D005C PLUGS DURING OPERATION |
| P51-STR-PG-D005A | 2.40E-04 | STRAINER/FILTER D005A PLUGS DURING OPERATION |
| P51-STR-PG-D001B | 3.59E-03 | STRAINER/FILTER D001B PLUGS DURING OPERATION |
| P51-STR-PG-D001A | 2.40E-04 | STRAINER/FILTER D001A PLUGS DURING OPERATION |
| P51-PT_-NO-A001 | 1.15E-05 | PRESSURE TRANSMITT. SA001 FAILS TO RESPOND TO CHANGE |
| P51-HX_-PG-C001B | 3.60E-04 | COOLER COMPRESSOR C001B FAILS WHILE OPERATING |
| P51-HX_-PG-C001A | 2.40E-05 | COOLER COMPRESSOR C001A FAILS WHILE OPERATING |
| P51-DRY-TM-B002B | 2.00E-05 | DRYER B002B IN MAINTENANCE |

| Table 4.12-7 List of System Basic Events | | |
|---|----------|--|
| Basic Event | Prob | Description |
| P51-CMP-TM-C001B | 2.00E-03 | COMPRESSOR C001B IN MAINTENANCE |
| P51-CMP-FS-C001B | 2.00E-02 | MOTOR-DRIVEN AIR COMPRESS. C001B FAILS TO START |
| P51-CMP-FS-C001A | 2.00E-02 | MOTOR-DRIVEN AIR COMPRESS. C001A FAILS TO START |
| P51-CMP-FR-C001B | 2.40E-03 | MOTOR-DRIVEN AIR COMPRESS. C001B FAILS TO CONT.OPER. |
| P51-CMP-FR-C001A | 2.40E-03 | MOTOR-DRIVEN AIR COMPRESS. C001A FAILS TO CONT.OPER. |
| P51-ACV-OC-F010C | 5.40E-05 | AOV F010C FAILS TO REMAIN OPEN (FOR FC VALVE) |
| P51-ACV-OC-F010A | 3.60E-06 | AOV F010A FAILS TO REMAIN OPEN (FOR FC VALVE) |
| P51-ACV-CF-OPEN | 1.35E-05 | CCF OF AOV TO OPEN |
| P51-ACV-CC-F010F | 2.00E-03 | AOV F010F FAILS TO OPERATE TO NOT DEENERG. POSITION. |
| P51-ACV-CC-F010C | 2.00E-03 | AOV F010C FAILS TO OPERATE TO NOT DEENERG. POSITION |
| P51-ACV-CC-F008B | 2.00E-03 | AOV F008B FAILS TO OPERATE TO DEENERG. POSITION. |
| P51-ACV-CC-F008A | 2.00E-03 | AIR OPERATED VALVE F008A FAILS TO OP. TO DEENERG. POSIT. |
| H23-RMU-FC-SA001 | 3.00E-04 | RMU SA001 FAILS TO OPERATE |
| H23-RMU-FC-IAS2 | 3.00E-04 | RMU 002 FOR IAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-IAS1 | 3.00E-04 | RMU 001 FOR IAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-IA001 | 3.00E-04 | RMU IA001 FAILS TO OPERATE |
| H23-RMU-FC-2PU2 | 3.00E-04 | RMU 2 PURIFIER 2 FAILS |
| H23-RMU-FC-2P51B | 3.00E-04 | RMU 002 FOR SAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-2P51A | 3.00E-04 | RMU 002 FOR SAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-2IA2 | 3.00E-04 | RMU 2 IA002 FOR INPUT PARAMETER FAILS TO OPERATE |
| H23-RMU-FC-2DRYER2 | 3.00E-04 | RMU 001 DRYER 2 FAIL |
| H23-RMU-FC-1PU2 | 3.00E-04 | RMU 1 PURIFIER 2 FAILS |
| H23-RMU-FC-1P51B | 3.00E-04 | RMU 001 FOR SAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-1P51A | 3.00E-04 | RMU 001 FOR SAS C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-1DRYER2 | 3.00E-04 | RMU 001 DRYER 2 FAIL |
| C74-VLU-FC-SASALIG | 6.24E-04 | VOTER CARD FOR SAS ALIGNMENT FAILS TO OPERATE (3M TEST) |
| C62-DTM-FC-N1EB | 9.00E-04 | DIGITAL TRIP MODULE TRAIN B NO 1E FAILS |
| C62-DTM-FC-N1EA | 9.00E-04 | DIGITAL TRIP MODULE TRAIN A NO 1E FAILS |
| C62-DTM-CF-N1EALL | 5.50E-05 | CCF OF DIGITAL TRIP MODULES NO 1E |

Table 4.12-8
System Top Events

| Top Event | Description |
|------------------|-------------------------------|
| | |
| GP52-0001-_1 | LOSS OF INSTRUMENT AIR SYSTEM |
| GP51-0001-_1 | LOSS OF SERVICE AIR SYSTEM |

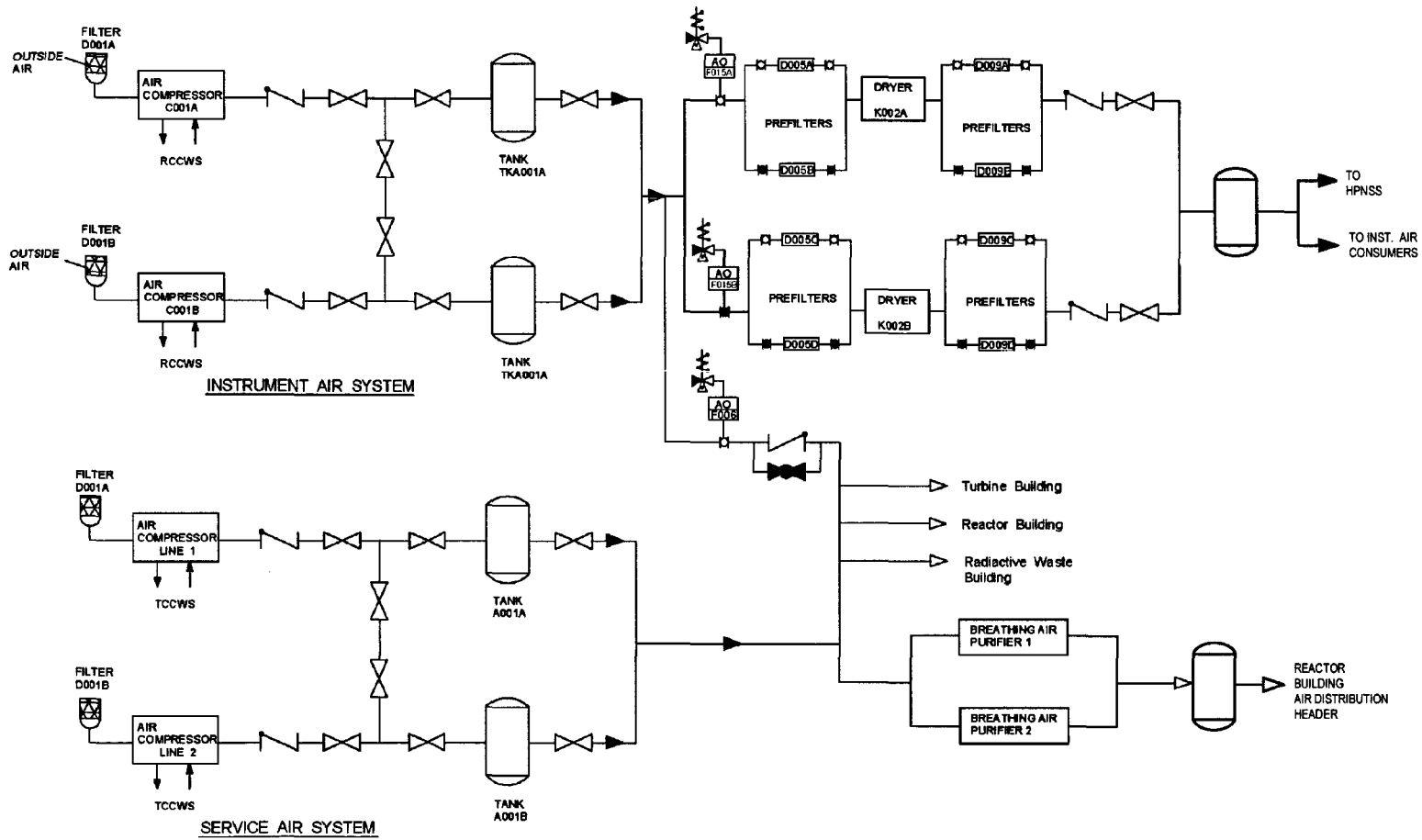


Figure 4.12-1. Simplified Process Flow Diagram for IAS and SAS

4.13 HIGH PRESSURE NITROGEN SUPPLY SYSTEM– (P54)

4.13.1 Purpose

The High Pressure Nitrogen Supply System (HPNSS) consists of distribution piping between the Containment Inerting System and the containment nitrogen users. Its function is to supply nitrogen gas for instrumentation and control systems and for pneumatic valve operators located within the primary containment. The HPNSS is a backup to the Containment Inerting System.

The containment high pressure nitrogen consumers include the ADS SRVs and their accumulators and the isolation condenser steam and condensate line isolation valve (ICIV) accumulator. The containment low pressure nitrogen consumers include the inboard MSIVs and their accumulators, the main steam drain header isolation valves, instruments, inboard RWCU containment isolation valves and the pneumatically operated valves.

4.13.2 Design Assumptions

The following assumptions with respect to the design were made:

- (1) Instrumentation logic for the actuation of valves located in the high pressure portion of the HPNSS is the same type as is used for SRV operation in both designs (discrete logic components for ESBWR).
- (2) The isolation valves on the lines that penetrate the primary containment are self-actuated, air-operated valves. These valves close on a loss of nitrogen pressure.
- (3) Valves F003, F005 and F017 are nitrogen operated.
- (4) A quarterly test of the supply from the bottles is assumed.
- (5) It is assumed that valves F016A and F016B possess their own accumulator for their actuation.

4.13.3 System Description

4.13.3.1 Hardware Configuration

A simplified sketch of the HPNSS used in the analysis is shown in Figure 4.13-1. The sketch shows the interface systems needed for the HPNSS functions.

The HPNSS fulfils two functions:

- (1) Supplies nitrogen gas to essential safety-related systems inside the primary containment;
- (2) Supplies nitrogen gas to non-essential systems inside the primary containment.

For the first function, the nitrogen is normally obtained from a non-safety-related gas supply subsystem comprised of a liquid nitrogen storage tank that is part of the ESBWR Containment Inerting System providing nitrogen gas for containment inerting.

The two low pressure branch lines incorporate a pressure reducing station in each branch line (F014 A and B) to lower the Containment Inerting System nitrogen supply. A check valve (F015 A and B) is located downstream of each pressure reducing station to prevent contamination from traveling upstream. During refueling, the IAS provides instrument air supply to consumers in the

containment via connections to the two low pressure branch lines. As a backup, a connection to the IAS is provided in order to supply air to the non-essential nitrogen users located inside the primary containment when the nitrogen gas supply pressure drops below a preset point.

The one high pressure branch line provides clean, dry, oil-free high pressure nitrogen to the containment nitrogen consumers from either the Containment Inerting System or from two racks of nitrogen storage bottles. Each rack consists of ten or more bottles, each bottle contains a pigtail and station valve that are piped into a manifold. A pressure control valve, F012, in series with an air-operated on-off valve, is provided in the line that connects the nitrogen bottles to the valve operators inside the containment. A connection to the Containment Inerting System Supply is provided downstream of these valves.

4.13.3.2 System Operation

During normal operating conditions, only the line from the tank of the Containment Inerting System to the essential safety systems and non-essential system inside containment is open. The normal valve status is as follows:

- F001 Open
- NOV F003, F017 Open
- NOV F005 Closed

When low pressure is detected in the normal nitrogen supply line, valve F003 automatically closes and nitrogen supply is provided to essential safety systems by the nitrogen storage bottle backup system. On a low-low pressure alarm, the line supplying nitrogen to non-essential systems inside the containment is manually isolated by the operator and, as a backup, the IAS supply line is opened.

The IAS supplies instrument and quality air for the instruments and controls inside the Primary Containment Vessel during refueling outages and/or when the primary containment is open for personnel entry.

4.13.3.3 Component Location

With the exception of the Containment Inerting System Tank and the lines connected to the nitrogen users located inside the primary containment, the system components (valves, piping, etc.) are mainly located in the Reactor Building.

4.13.4 Automatic and Manual Control

4.13.4.1 Automatic Actuation

When low nitrogen gas pressure is detected by pressure transmitter PT006, pressure switch PS-001 closes NOV F003. A low nitrogen gas pressure is simultaneously detected by the pressure transmitter (PT-002) causing NOV F005 to open and thus connecting to the nitrogen gas bottle supply.

No automatic actuation is provided for the portion of the system feeding the low pressure nitrogen consumers inside the containment; only a low pressure alarm in the control room is

available for the operator. Operator action is therefore required in order to close the normal and bottled nitrogen supplying lines and to align the backup from Instrumental Air System.

4.13.4.2 Manual Actuation

The following assumptions with respect to the design were made:

- (1) Operation of valves and associated equipment used for the system functions can be performed by manual actuation of switches located in the main control room (MCR).

4.13.4.3 Instrumentation and Alarms

Table 4.13-1 shows the alarms and indications located in the MCR.

4.13.5 System Interfaces

The only supporting-system interface is with the electrical system.

In case of loss of the HPNSS, the IAS supplies filtered dry air to non-essential containment pneumatic users.

HPNSS supplies nitrogen to operate valves F003, F005 and F017.

Table 4.13-2 reports the system dependency matrix assumed for the analysis.

4.13.6 System Testing

The high pressure nitrogen isolation valves are tested by manual actuation of a switch located in the control room and by observation of associated position indication lights. Operation of valves and associated equipment used to switch from the non-safety to safety nitrogen supply are also tested by manual actuation of a switch located in the control room and by observation of the associated position indication lights. Periodic tests of the check valves and accumulators are conducted to assure valve operability. The HPNSS nitrogen supply pressure for the SRVs is verified at monthly intervals to assure that the valves are capable of performing their intended functions. A system operational test is performed by isolating the Containment Inerting System supply line and allowing the nitrogen pressure to decrease. The HPNSS responds to a predetermined low pressure by opening the appropriate valves and allowing HPNSS backup nitrogen supply. In addition, Containment Inerting System isolation also occurs at this predetermined low nitrogen pressure. Table 4.13-3 reports the expected test frequency of the components considered in the analysis.

4.13.7 System Maintenance

The HPNSS is designed for on-line maintenance to the maximum extent practical. All major components have manual isolation valves to permit maintenance activities without having to shut down the HPNSS. Table 4.13-4 lists the on-line maintenance activities.

4.13.8 Common Cause Failures

Common cause failures (CCFs) within the system are summarized in Table 4.13-5.

4.13.9 Fault Tree Analysis

The HPNSS fault tree is evaluated to determine the system unavailability and the components contributing significantly to system unavailability.

4.13.9.1 Top Event Definitions

Because of the two different functions of the HPNSS, the following two top events are defined:

- Loss of the HPNSS to high pressure nitrogen consumers inside primary containment (safety relief valves of Nuclear Boiler System (NBS) (B21) and nitrogen motor-operated valves of IC (B32)), Fault Tree Top event GP54-0001-_1.
- Loss of HPNSS/air to the low pressure nitrogen consumers inside primary containment, Fault Tree Top event GP54-0001-_2.

4.13.9.2 Fault Tree Description

The fault tree for GP54-0001-_1 shows the unavailability of both the normal and the essential HPNSS.

The fault tree for GP54-0001-_2 shows the unavailability of both the normal HPNSS and the IAS used as a backup supply.

Both fault trees are shown in the Appendix B.4.13.

4.13.9.3 Human Interactions

The following human actions are accounted for in the fault trees:

- Operator failure to manually open valves F005 when automatic opening fails.
- Misposition of valve F003 causes loss of nitrogen for Containment Inerting System supply line.

Table 4.13-6 reports a summary of system mispositions and operator actions considered in the analysis.

4.13.10 Results of Fault Tree Analysis

The definition of each basic event for the HPNSS fault tree is reported in Table 4.13-7.

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow the identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.13.11 Model Differences for 72-Hour Operation

There are no changes to the model for operation for 72 hours other than the missing time.

4.13.12 References

4.13-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.13-1
Control Room Instrumentation and Alarms

| Displays |
|---|
| Nitrogen storage bottle rack pressure |
| Containment Inerting System line header pressure |
| Low line header pressure in the Division 1 low pressure supply branch |
| Low line header pressure in the Division 2 low pressure supply branch |
| High pressure branch line header pressure |
| |
| Alarms |
| Low nitrogen storage bottle rack pressure |
| Low line header pressure in Division 1 low pressure supply branch |
| Low line header pressure in Division 1 low pressure supply branch |
| High line header pressure in Division 1 low pressure supply branch |
| Low line header pressure in Division 2 low pressure supply branch |
| High line header pressure in Division 2 low pressure supply branch |
| Low line header pressure in the high pressure supply branch |
| High line header pressure in the high pressure supply branch |

Table 4.13-2
System Dependency Matrix

| | | Support System | | | | |
|-----------|------|---|----------------------------------|----------------------------|---------------------------|------------|
| | | Power Supply | | | | Air Supply |
| | | Uninterruptible AC Power Supply System (class 1E) | | | | |
| Component | Type | 208-120 V DIV 1 (R13-11-2) | 208-120 V DIV 2 (R13-21-2) | 480 V DIV 1 (R13-11) | 480 V DIV2 (R13-21) | |
| F016A | AOV | | | X | | |
| F016B | AOV | | | | X | |
| F003 | AOV | X | | | | X |
| F005 | AOV | X | | | | X |
| F017 | AOV | X | | | | X |

Table 4.13-3
Component Test

| Component | Type | Expected Test Interval |
|----------------------|-------------|-------------------------------|
| F005 | AOV | Quarterly |
| F012 | CPV | Quarterly |
| Pressure switch | PS | Quarterly |
| Pressure transmitter | PT | Quarterly |

Table 4.13-4
Component Maintenance

Component maintenance has not been included in the model.

Table 4.13-5
System Common Cause Failures

| Basic Event | Prob | Description |
|--------------------|----------|---|
| P54-CPV-CF-CONTROL | 1.57E-04 | CCF OF PRESSURE CONTROL VALVES FAILURES |

Table 4.13-6
System Human Errors

| Basic Event | Prob | Description |
|---------------------|-------------|--|
| P54-XHE-FO-OPENF005 | 1.77E-02 | OPERATOR FAILS TO MANUALLY OPEN VALVE F005 |
| P54-XHE-MH-F003 | 4.00E-03 | INADVERTENT MISPOSITION OF VALVE F003 |

Table 4.13-7
System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|---|
| H23-RMU-FC-P54F005 | 3.00E-04 | ATM VALVE F005 FAILS TO TRIP |
| H23-RMU-FC-P54PT2 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| P54-ACV-CC-F005 | 2.00E-03 | AIR OPERATED VALVE F005 FAILS TO OPEN |
| P54-ACV-OC-F016A | 3.60E-06 | AIR OPER. VALVE F016A SPUR. CLOSE |
| P54-ACV-OC-F016B | 3.60E-06 | AIR OPER. VALVE F016B SPUR. CLOSE |
| P54-ACV-OO-F003 | 2.00E-03 | AIR OPERATED VALVE F003 FAILS TO CLOSE |
| P54-CPV-CF-CONTROL | 1.57E-04 | CCF OF PRESSURE CONTROL VALVES FAILURES |
| P54-CPV-FC-F012 | 3.13E-03 | REGULATING PRESSURE VALVE F012 ALL MODES FAILURE |
| P54-CPV-FC-F014A | 6.96E-05 | REGULATING PRESSURE VALVE F014A ALL MODES FAILURE |
| P54-CPV-FC-F014B | 6.96E-05 | REGULATING PRESSURE VALVE F014B ALL MODES FAILURE |
| P54-CPV-FC-F015 | 6.96E-05 | REGULATING PRESSURE VALVE F015 ALL MODES FAILURE |
| P54-PSF-RP-ESSENT | 4.12E-06 | PIPE SECTION TO ESSENT. SYSTEMS LEAKS OR PLUGS |
| P54-PSF-RP-NONESSENT | 4.12E-06 | PIPE SECTION TO NON ESSENT. USERS LEAKS OR PLUGS (#2) |
| P54-PS_-CO-PS001 | 1.19E-04 | PRESSURE SWITCH PS001 (PT001) OPERATES SPURIOUSLY |
| P54-PT_-NO-002 | 5.18E-04 | PRESSURE TRANSMITT. PT002 FAILS TO RESPOND TO CHANGE |
| P54-PT_-NO-006 | 5.18E-04 | PRESSURE TRANSMITT. PT006 FAILS TO RESPOND TO CHANGE |
| P54-TNK-RP-A001 | 2.40E-06 | TANK A001 LEAKS CATASTROPHICALLY |
| P54-UV_-OO-F004 | 1.00E-03 | CHECK VALVE F004 FAILS TO CLOSE |
| P54-XHE-FO-OPENF005 | 1.77E-02 | OPERATOR FAILS TO MANUALLY OPEN VALVE F005 |
| P54-XHE-MH-F003 | 4.00E-03 | INADVERTENT MISPOSITION OF VALVE F003 |

Table 4.13-8
System Top Events

| Top Event | Description |
|------------------|--|
| GP54-0001-_1 | LOSS OF HIGH PRESS. NIT. GAS SUPPLY SYST. TO HIGH PRESS. CONSUMERS |
| GP54-0001-_2 | LOSS OF NITROGEN TO LOW PRESSURE USERS INSIDE P.C. |

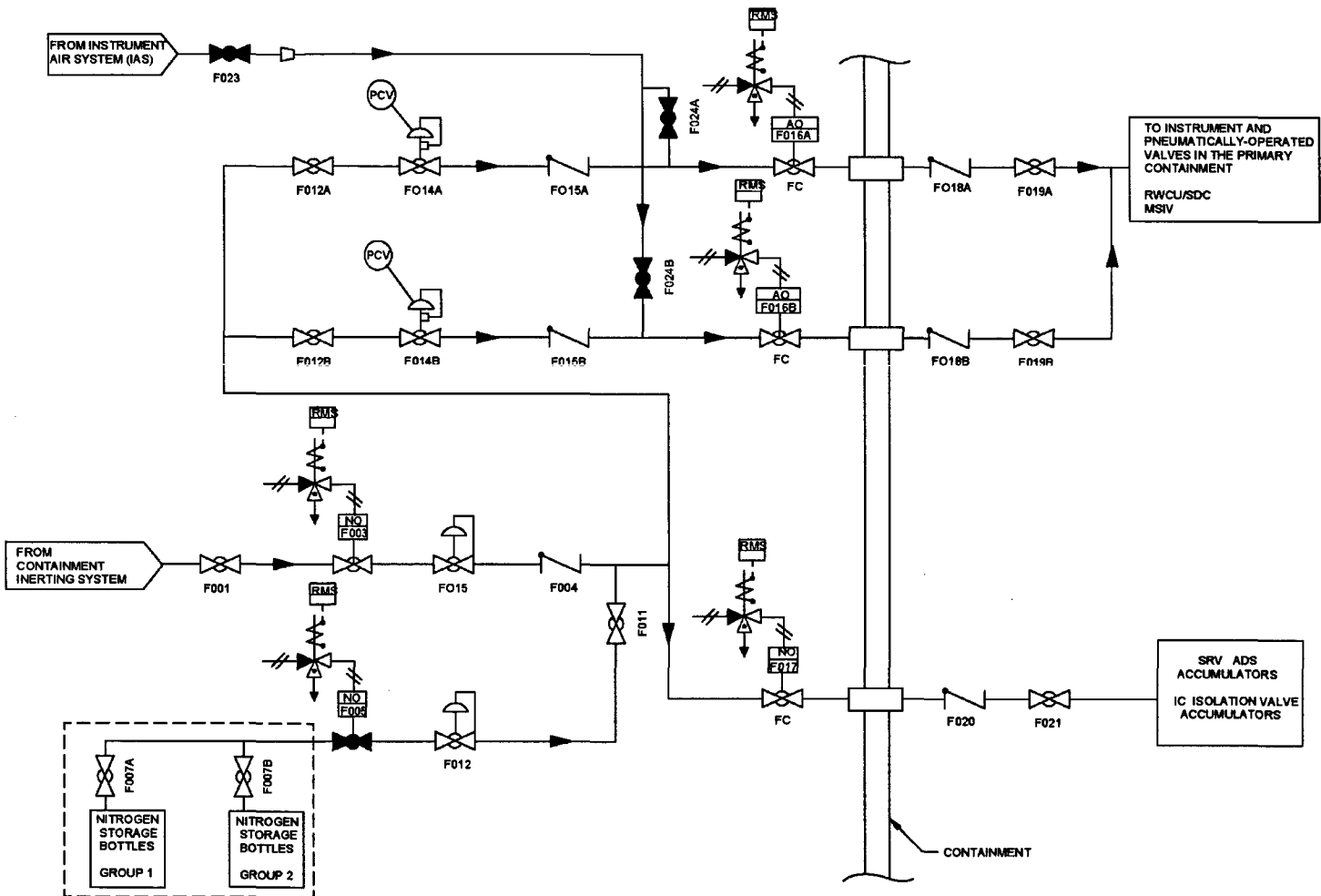


Figure 4.13-1. Simplified Process Flow Diagram for HPNSS

4.14 A.C. ELECTRIC POWER SYSTEM – (R10, R11 and R12)

4.14.1 System Purpose

To ensure satisfactory operation of the plant equipment, the main Diesel Generator and Off-site Power System supply electrical power to the Medium Voltage AC System, which consists of 13.8 kV PG (plant generation) buses and 6.9 kV PIP (Plant Investment Protection) buses. PG loads are divided into four non-safety buses (A1, B1, C1, and D1) and are needed during startup, normal operation, and normal shutdown. The 6.9 kV PIP loads are divided into four non-safety buses (A2, A3, B2, B3) and are the auxiliary and service loads that must remain operational. These buses are backed up with on-site non-safety-related diesel generators.

The Low Voltage AC Power System supplies electric power at 480V and consists of non-safety buses that are normally powered from the PG buses and non-safety buses that are normally powered from the 6.9 kV PIP buses. Four of the buses (A31, B31, C31, D31) are Class 1E divisions and are fed from the two 6.9 kV PIP buses (A2 and B2) that are backed up by on-site non-safety related diesel generators.

The Instrumentation and Control (I&C) Power Supply System provides non-vital low voltage AC power for operation of instrumentation and control system loads and other important plant loads. Receiving 480 V and supplying 208-120 V.

4.14.2 Design Assumptions

The following design assumptions with respect to the design are made:

- (1) It is assumed that the diesel generators are tested every three months.
- (2) It is assumed that the buses that can be powered either by bus A2 or by bus B2 (A31, B31, C31, D31 and C23) change their current supply every three months.
- (3) It is assumed that no preventive maintenance actions performed during the normal operation of the plant. Nevertheless, a corrective maintenance could be done if required.

4.14.3 System Description

4.14.3.1 Hardware Configuration

Figure 4.14-1, Simplified Electrical One-line Diagram, shows the principal components of the system.

The Off-site Power System supply the plant from two electrically independent and physically separated off-site power sources:

- "Normal" (or preferred) source via the unit auxiliary transformer
- "Alternate" (or reserve) source via the reserve auxiliary transformers

During normal plant operation, the main plant generator supplies the plant loads through two auxiliary transformers. These transformers have two output voltage levels:

- 13.8kv which feeds two buses for PG loads (either buses A1 and C1 or buses B1 and D1)
- 6.9kv which feeds PIP buses (bus A2 or B2)

The utility power grid is the normal preferred power supply when the turbine generator is tripped. Off-site power via two reserve transformers, similar to the auxiliaries, is the alternate or reserve power supply.

The four PG buses (A1, B1, C1, D1) supply power to the largest pumps (feedwater, circulating water and condensate) and to the power centers of the different buildings that supply low voltage power to users via 13.8kV and 480V transformers.

Generally, the 6.9 kV buses (A2, A3, B2, B3) directly feed motor loads larger than approximately 250 kW and supply power to the power centers of the different buildings via 6.9kV/480V transformers.

The safety-related portion of the low voltage system consists of four Distribution Panels (DP) (A31, B31, C31, D31), which are fed from PIP buses (A2 and B2), and provide power to the safety-related battery chargers and regulating transformers. These DP are provided with plug-in connections for transportable AC generators, which provide a convenient and secure means of restoring long-term power to the plant safety and selected non-safety loads following a loss of all other sources.

The I&C Power Supply system is comprised of five subsystems. Each Subsystem is powered through a regulatory transformer at a ratio of 480V/208-120V. Four of the subsystems are supplied by power center buses A31, B31, C31 and D31, with a power rating of 50kVA. The fifth is supplied by power center DCIS swing bus C23 with a power rating of 150 kVA.

The Standby Onsite AC Power Supply System (SPSS) consists of two independent and redundant non-safety related diesel generator units. This system is designed to supply safety-related AC power to the plant permanent non-safety loads and to the DP when the normal preferred and alternate preferred AC sources are not available. Diesel generators are connected to the PIP buses A2 and B2.

4.14.3.2 System Operation

The 13.8kV and 6.9kV power distribution system receives power from the unit auxiliary transformers. During normal power operation, the unit auxiliary switchgear buses receive power from the main generator through the generator breaker and the unit auxiliary transformers.

If the main generator trips, the low voltage generator breaker opens and power to the unit auxiliary transformers is backfed from the normal preferred power (utility power grid). When a failure occurs in an auxiliary transformer or in a medium voltage busbar, plant shutdown does not result. The electric loads are transferred from the preferred power supply to the reserve power supply. In this situation, the plant is capable of restoring the production of energy in a short time.

During loss of preferred power (LOPP), the medium voltage AC power system supplies power from the on-site standby AC power system to selected medium voltage loads and selected portions of the medium and low voltage AC power systems.

During station blackout conditions, low voltage AC power system supplies power from transportable AC generators to selected PIP loads and safety-related buses of the low voltage AC power system and selected low voltage loads.

The I&C Power Supply System supplies constant regulated voltage power from the low voltage AC power system to its connected I&C loads. During LOPP conditions, the system is powered from the on-site power system.

4.14.3.3 System Location

The switchgear of the 13.8 kV and 6.9 kV are located in the electrical building.

The load centers consist of transformers and associated low voltage switchgear, and are located in the electrical building, control building, turbine building, fuel building, cooling tower building, pump house building, service water building, water treatment building and radwaste building.

Whenever practical, MCCs, distribution transformers, and distribution panels are located in the proximity of the loads.

The four class 1E isolation buses (A31, B31, C31, D31) are located in the Control Building.

Each diesel generator unit with its dedicated auxiliaries is located within its own room in the electric building

4.14.4 Automatic and Manual Control

Both breakers, in alternate and reserve transformers connection lines to the medium voltage power generation buses independently and automatically transfer to the alternate feed that has been verified for proper voltage and frequency. The transfer scheme is automatic in either direction and occurs any time the bus loses power. If both feeds have power, the transfer is manual, synchronized and fast, or manual and slow. If the fast transfer fails, then a slow (dead bus) transfer is automatically initiated. However originated, the slow bus automatic transfer verifies bus voltage less than 10-30% before closing the source breaker.

With both source breakers in "auto", each 6.9 kV bus individually and automatically transfers to its standby generator. The transfer scheme is automatic in the direction of the standby diesel generator and occurs any time a bus loses power and only after proper voltage and frequency have been verified. The loads are automatically sequenced on the diesels to allow the diesel generator's voltage regulator and speed control system to maintain voltage and frequency within the design range of all automatically connected loads. The return of source power to any non-diesel generator feeder is manual and synchronized.

It is possible to periodically test each standby diesel generator by manually or automatically synchronizing it to its associated medium voltage bus and loading the diesel to its rated capacity while the bus is powered from either of its incoming power feeds.

4.14.5 System Interfaces

The Medium Voltage AC Power System begins at the low voltage terminals of the unit auxiliary and reserve transformers, and at the output terminals of the sources breakers of the plant On-site Standby AC Power System. The system ends at the high voltage terminals of the power center (unit substation) transformers and the input terminals of the medium voltage loads.

The Low Voltage AC Power System begins at the high voltage terminals of the power center transformers, and at the output terminals of the sources breakers of the plant low voltage on-site

standby AC power sources. The system ends at the input terminals of low voltage loads and the boundaries of other systems.

The I&C Power Supply System begins at the input terminals of the regulating transformers and it ends at the input terminals of the system loads.

To start the diesel-generator and connect it to the bus, Direct and Uninterruptible Power Supply Systems are required.

Table 4.14-2 shows the assumed system dependencies.

4.14.6 System Testing

The electrical system design permits operability testing, during normal station operation, of components that are not normally exercised.

Provisions are made for system isolation so as to preclude unacceptable interference of system testing with unit operation. Continuous indication is provided to monitor the conditions of the systems undergoing testing.

The diesel-generators are started and manually loaded every three months; the automatic startup and loading sequence is tested during every refueling. Automatic transfer to the alternate-preferred power source is tested every refueling. Voltage availability on the alternate preferred source is checked each week.

Table 4.14-3 shows the component tests for the AC system.

4.14.7 System Maintenance

Electrical System components are designed and installed to facilitate maintenance during normal plant operation or plant scheduled shutdown.

The equipment and components of the Standby Onsite AC Power Supply System are designed for easy inspection and maintenance during operation.

Only corrective maintenance of buses can be performed.

4.14.8 Potential for Common Cause Failure

Generic CCF of the circuit breakers has been taken into account as follows:

- Spurious opening CCFs are judged to be negligible for 6.9 kV and 13.8 kV.
- CCFs to open or close are conservatively included in the fault tree model, even if the involved circuit breakers operate in different conditions.

Several CCFs have been taking into account about Diesel Generators function:

- Start, load, and run.
- Logic and feed.
- Motor driven fuel transfer loads.

CCFs are summarized in Table 4.14-5.

4.14.9 Fault Tree Analysis

4.14.9.1 Top Event Definition

A fault tree was developed for each of the 13.8kV and 6.9kV buses, for several 480V load centers and for several 480V downstream motor-control center buses. All the fault trees model the failure to supply power for 72 hours.

4.14.9.2 Fault Tree Description

Fault trees GR10-0001-_1 to GR10-0001-_4 model the loss of power of the four 13.8 kV PG load buses.

Fault trees GR11-0001-_1 to GR11-0001-_4 model the loss of power of the four 6.9 kV PIP load buses.

Fault trees GR12-0001-_1 to GR12-0001-_19 model the loss of power of nineteen 480V power center buses.

Fault trees are shown in the Appendix B.4.14.

4.14.9.3 Human Interactions

No human actions have been included in the AC Power System fault tree model, see Table 4.14-6.

4.14.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.14-7.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.14.11 Model Differences for 72-Hour Operation

There are no changes to the model for operation to 72 hours other than the mission time.

4.14.12 References

4.14-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.14-1
Control Room Instrumentation and Alarms

| Controls |
|--|
| |
| Displays (Standby On-site AC Power Supply System) |
| Diesel engine running |
| D/G trouble |
| D/G controls not in Auto |
| G main breaker trip |
| D/G maintenance mode |
| D/G parallel mode |
| D/G overspeed |
| Engine failed to start |
| Generator Differential relay |
| Reverse power relay |
| Field relay |
| Overcurrent relay |
| Lock-out relay operated |
| Overvoltage relay |
| Ground relay |
| Overtemperature relay |
| Undervoltage relay |
| Frequency relay |
| Engine speed |
| Engine hour meter |
| Generator output voltage |
| Current |
| Active power output |
| Reactive power output |
| |
| Displays (Electric Power Distribution System) |
| Switchgear breaker load currents |
| Source bus currents |
| Switchgear bus voltages |
| Switchgear bus frequency |
| |
| Alarms |

Table 4.14-2
System Dependency Matrix

| Component | Type | Support System | | | | | |
|-----------|------|-------------------|-------------------|-------------------|-------------------|--------------------------|-----------------------|
| | | Power Supply | | | | | |
| | | 500kV (XFRM-A) | 500kV (XFRM-B) | 230kV (XFRM-C) | 230kV (XFRM-D) | Diesel Generator A | Diesel Generator B |
| BUS A1 | BUS | X | | X | | | |
| BUS B1 | BUS | | X | | X | | |
| BUS C1 | BUS | X | | X | | | |
| BUS D1 | BUS | | X | | X | | |
| BUS A2 | BUS | X | | X | | X | |
| BUS B2 | BUS | | X | | X | | X |

| Component | Type | Support System | | | | | |
|--------------------------|------|----------------|--------|--------|--------|--------|--------|
| | | Power Supply | | | | | |
| | | BUS A1 | BUS B1 | BUS A2 | BUS B2 | BUS A3 | BUS B3 |
| BUS A3 | BUS | | | X | | | |
| BUS B3 | BUS | | | | X | | |
| TRAIN A IN TB (NO 1E) | BUS | X | | | | | |
| TRAIN B IN TB (NO 1E) | BUS | | X | | | | |
| BUS A2-01A | BUS | | | X | | | |
| BUS A2-02A | BUS | | | X | | | |
| BUS A2-04A | BUS | | | X | | | |
| BUS A3-01A | BUS | | | | | X | |
| BUS A3-02A | BUS | | | | | X | |
| TRAIN A IN TB (DID) | BUS | | | | | X | |
| BUS B2-01B | BUS | | | | X | | |
| BUS B2-02B | BUS | | | | X | | |
| BUS B2-04B | BUS | | | | X | | |
| BUS B3-01B | BUS | | | | | | X |
| BUS B3-02B | BUS | | | | | | X |
| TRAIN B IN TB (DID) | BUS | | | | | | X |
| BUS A31 | BUS | | | X | X | | |
| BUS B31 | BUS | | | X | X | | |
| BUS C31 | BUS | | | X | X | | |
| BUS D31 | BUS | | | X | X | | |
| BUS C23 | BUS | | | X | X | | |

| | | Support System | |
|--------------------------|------|------------------------------|------------------------------|
| | | Power Supply | |
| Component | Type | 125 VDC Control Power BUS A3 | 125 VDC Control Power BUS B3 |
| BUS A1 | BUS | X | |
| BUS B1 | BUS | | X |
| BUS C1 | BUS | X | |
| BUS D1 | BUS | | X |
| BUS A2 | BUS | X | |
| BUS B2 | BUS | | X |
| BUS A3 | BUS | X | |
| BUS B3 | BUS | | X |
| TRAIN A IN TB (NO 1E) | BUS | X | |
| TRAIN B IN TB (NO 1E) | BUS | | X |
| BUS A2-01A | BUS | X | |
| BUS A2-02A | BUS | X | |
| BUS A2-04A | BUS | X | |
| BUS A3-01A | BUS | X | |
| BUS A3-02A | BUS | X | |
| TRAIN A IN TB (DID) | BUS | X | |
| BUS B2-01B | BUS | | X |
| BUS B2-02B | BUS | | X |
| BUS B2-04B | BUS | | X |
| BUS B3-01B | BUS | | X |
| BUS B3-02B | BUS | | X |
| TRAIN B IN TB (DID) | BUS | | X |
| BUS A31 | BUS | X | |
| BUS B31 | BUS | | X |
| BUS C31 | BUS | X | |
| BUS D31 | BUS | | X |
| BUS C23 | BUS | X | |

Table 4.14-3
Component Tests

| Component | Expected Test Interval |
|--|-------------------------------|
| Diesel generator manually started | 3 months |
| Automatic startup and loading sequence | Refueling (2 years) |
| Automatic transfer to the alternate preferred power source | Refueling (2 years) |
| Alternate preferred power source voltage level | 1 week |

Table 4.14-4
Component Maintenance

No preventive maintenance actions are expected to be performed during normal plant operation.
Only corrective maintenance can be performed.

Table 4.14-5
System Common Cause Failures

| Basic Event | Prob | Description |
|----------------------|----------|---|
| R10-MCB-CF-138CLOSE | 7.43E-04 | CCF CIRCUIT BREAKERS TO CLOSE |
| C62-VLU-CF-DIDALL | 3.12E-05 | CCF OF VOTER LOGIC UNITS |
| C74-BYP-CF-DIDALL | 9.00E-07 | CCF OF BYPASS UNITS (DID) |
| H23-EMS-CF-DIDALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| P21-MP_-CR-TRAINA | 1.20E-06 | CCF TO RUN RCCW PUMPS TRAIN A |
| P21-MP_-CR-TRAINB | 1.20E-06 | CCF TO RUN RCCW PUMPS TRAIN B |
| P21-MP_-CS-TRAINA | 1.20E-06 | CCF TO START RCCW PUMPS TRAIN A |
| P21-MP_-CS-TRAINB | 1.30E-05 | CCF TO START RCCW PUMPS TRAIN B |
| R11-MCB-CF-69CLOSE | 9.29E-05 | CCF CIRCUIT BREAKERS 6.9 KV TO CLOSE |
| R21-DG_-CR-ALLDG | 4.44E-03 | CCF OF DIESEL GENERATORS TO RUN |
| R21-DG_-CS-ALLDG | 8.52E-04 | CCF OF DIESEL GENERATORS TO START AND LOAD |
| R21-MP_-CR-FUELTRANS | 3.72E-05 | CCF TO RUN MOTOR-DRIVEN FUEL TRANSFER PUMPS |
| R21-MP_-CS-FUELTRANS | 1.71E-04 | CCF TO START MOTOR-DRIVEN FUEL TRANSFER PUMPS |
| R12-LCB-CF-480CLOSE | 2.79E-05 | CCF CIRCUIT BREAKERS 480 V AC TO CLOSE |

Table 4.14-6
System Human Errors

No human actions have been included in the fault tree model.

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|---------------------|----------|--|
| R10-BAC-LP-R10A1 | 4.80E-06 | BUS R10-A1 FAILS DURING OPERATION |
| R10-BAC-LP-R10B1 | 4.80E-06 | BUS R10-B1 FAILS DURING OPERATION |
| R10-BAC-LP-R10C1 | 4.80E-06 | BUS R10-C1 FAILS DURING OPERATION |
| R10-BAC-LP-R10D1 | 4.80E-06 | BUS R10-D1 FAILS DURING OPERATION |
| R10-BAC-TM-R10A1 | 4.80E-06 | 13,8 KV AC BUS R10-A1-IN MAINTENANCE |
| R10-BAC-TM-R10B1 | 4.80E-06 | 13,8 KV AC BUS R10-B1-IN MAINTENANCE |
| R10-BAC-TM-R10C1 | 4.80E-06 | 13,8 KV AC BUS R10-C1-IN MAINTENANCE |
| R10-BAC-TM-R10D1 | 4.80E-06 | 13,8 KV AC BUS R10-D1-IN MAINTENANCE |
| R10-MCB-CC-XFRMAA1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-A FAILS TO OPEN |
| R10-MCB-CC-XFRMAC1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-A FAILS TO OPEN |
| R10-MCB-CC-XFRMBB1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-B FAILS TO OPEN |
| R10-MCB-CC-XFRMBD1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-B FAILS TO OPEN |
| R10-MCB-CF-138CLOSE | 7.43E-04 | CCF CIRCUIT BREAKERS TO CLOSE |
| R10-MCB-CO-XFRMAA1 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-A OPENS SPURIOUSLY |
| R10-MCB-CO-XFRMAC1 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-A OPENS SPURIOUSLY |
| R10-MCB-CO-XFRMBB1 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-B OPENS SPURIOUSLY |
| R10-MCB-CO-XFRMBD1 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-B OPENS SPURIOUSLY |
| R10-MCB-OO-XFRMCA1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-C FAILS TO CLOSE |
| R10-MCB-OO-XFRMCC1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-C FAILS TO CLOSE |
| R10-MCB-OO-XFRMDB1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-D FAILS TO CLOSE |
| R10-MCB-OO-XFRMDD1 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-D FAILS TO CLOSE |
| R10-XFH-LP-XFRMA | 2.88E-05 | TRANSFORMER XFRM-A (HIGH VOLTAGE) FAILS TO OPERATE |
| R10-XFH-LP-XFRMB | 2.88E-05 | TRANSFORMER XFRM-B (HIGH VOLTAGE) FAILS TO OPERATE |
| R10-XFH-LP-XFRMC | 2.88E-05 | TRANSFORMER XFRM-C FAILS TO OPERATE |
| R10-XFH-LP-XFRMD | 2.88E-05 | TRANSFORMER XFRM-D FAILS TO OPERATE |
| R10-XFH-TM-XFRMA | 1.00E-04 | TRANSFORMER XFRM-A IN MAINTENANCE |
| R10-XFH-TM-XFRMB | 1.00E-04 | TRANSFORMER XFRM-B IN MAINTENANCE |
| R10-XFH-TM-XFRMC | 1.00E-04 | TRANSFORMER XFRM-C IN MAINTENANCE |
| R10-XFH-TM-XFRMD | 1.00E-04 | TRANSFORMER XFRM-D IN MAINTENANCE |
| R11-SYS-FF-NOREC | 6.13E-01 | FAILURE IN OFFSITE POWER RECOVERY |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|---------------------|----------|--|
| C62-VLU-CF-DIDALL | 3.12E-05 | CCF OF VOTER LOGIC UNITS |
| C62-VLU-FC-ESF1ADID | 6.24E-04 | 1ST DIV A ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF1BDID | 6.24E-04 | 1ST DIV B ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2ADID | 6.24E-04 | 2ND DIV A (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C62-VLU-FC-ESF2BDID | 6.24E-04 | 2ND DIV B (DID) ESF VOTER LOGIC UNIT FAILS TO FUNCTION |
| C74-BYP-CF-DIDALL | 9.00E-07 | CCF OF BYPASS UNITS (DID) |
| C74-BYP-FC-DIVADID | 3.00E-04 | BYPASS UNIT DIV A (DID) FAILS TO FUNCTION |
| C74-BYP-FC-DIVBDID | 3.00E-04 | BYPASS UNIT DIV B (DID) FAILS TO FUNCTION |
| H23-EMS-CF-DIDALL | 1.80E-06 | CCF OF ALL DIVISION OF THE EMS |
| H23-EMS-FC-DIVADID | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV A (DID) FAILS TO FUNCTION |
| H23-EMS-FC-DIVBDID | 6.00E-04 | ESSENTIAL MULTIPLEXING SYSTEM DIV B (DID) FAILS TO FUNCTION |
| H23-RMU-CF-DIDALL | 9.00E-07 | CCF OF REMOTE MULTIPLEXING UNITS (DID) |
| H23-RMU-FC-10A11 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-10A21 | 3.00E-04 | REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1ADID | 3.00E-04 | 1ST DIV A (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF1BDID | 3.00E-04 | 1ST DIV B (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2ADID | 3.00E-04 | 2ND DIV A (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-ESF2BDID | 3.00E-04 | 2ND DIV B (DID) ESF REMOTE MULTIPLEXING UNIT FAILS TO FUNCTION |
| H23-RMU-FC-R101 | 3.00E-04 | DTM/TLU 001 OR MUX INTERFACE FAIL TO TRIP (2Y TEST) |
| H23-RMU-FC-R102 | 3.00E-04 | DTM/TLU 002 OR MUX INTERFACE FAIL TO TRIP (2Y TEST) |
| P21-MCB-OO-C001A | 1.00E-03 | CIRCUIT BREAKER OF PUMP C001A FAILS TO CLOSE |
| P21-MCB-OO-C001B | 1.00E-03 | CIRCUIT BREAKER OF PM C001B FAILS TO CLOSE |
| P21-MCB-OO-C002A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MCB-OO-C002B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MCB-OO-C003A | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|---|
| P21-MCB-OO-C003B | 1.00E-03 | MEDIUM VOLTAGE CIRCUIT BREAKER FAILS TO CLOSE |
| P21-MP_-CR-TRAINA | 1.20E-06 | CCF TO RUN RCCW PUMPS TRAIN A |
| P21-MP_-CR-TRAINB | 1.20E-06 | CCF TO RUN RCCW PUMPS TRAIN B |
| P21-MP_-CS-TRAINA | 1.20E-06 | CCF TO START RCCW PUMPS TRAIN A |
| P21-MP_-CS-TRAINB | 1.30E-05 | CCF TO START RCCW PUMPS TRAIN B |
| P21-MP_-FR-C001A | 1.20E-04 | MOTOR DRIVEN PUMP C001A FAILS TO RUN |
| P21-MP_-FR-C001B | 1.20E-04 | MOTOR DRIVEN PUMP C001B FAILS TO RUN |
| P21-MP_-FR-C002A | 1.20E-04 | MOTOR-DRIVEN PUMP C002A FAILS TO RUN |
| P21-MP_-FR-C002B | 1.20E-04 | MOTOR-DRIVEN PUMP C002B FAILS TO RUN |
| P21-MP_-FR-C003A | 1.20E-04 | MOTOR-DRIVEN PUMP C0003A FAILS TO RUN |
| P21-MP_-FR-C003B | 1.20E-04 | MOTOR-DRIVEN PUMP C003B FAILS TO RUN |
| P21-MP_-FS-C001A | 1.30E-03 | MOTOR DRIVEN PUMP C001A FAILS TO START |
| P21-MP_-FS-C001B | 1.30E-03 | MOTOR-DRIVEN PUMP C001B FAILS TO START |
| P21-MP_-FS-C002A | 1.30E-03 | MOTOR-DRIVEN PUMP (ALL TYPES) FAILS TO START |
| P21-MP_-FS-C002B | 1.30E-03 | MOTOR-DRIVEN PUMP C002B FAILS TO START |
| P21-MP_-FS-C003A | 1.30E-03 | MOTOR-DRIVEN PUMP C0003A FAILS TO START |
| P21-MP_-FS-C003B | 1.30E-03 | MOTOR-DRIVEN PUMP C003B FAILS TO START |
| P21-MP_-TM-C002A | 2.00E-03 | PUMP C002A IN MAINTENANCE |
| P21-MP_-TM-C002B | 2.00E-03 | PUMP C002B IN MAINTENANCE |
| P21-UV_-CC-F001A | 2.00E-04 | CHECK VALVE F001A FAILS TO OPEN |
| P21-UV_-CC-F001B | 2.00E-04 | CHECK VALVE F001B FAILS TO OPEN |
| P21-UV_-CC-F005A | 2.00E-04 | CHECK VALVE F005A FAILS TO OPEN |
| P21-UV_-CC-F005B | 2.00E-04 | CHECK VALVE F005B FAILS TO OPEN |
| P21-UV_-CC-F007A | 2.00E-04 | CHECK VALVE F007A FAILS TO OPEN |
| P21-UV_-CC-F007B | 2.00E-04 | CHECK VALVE F007B FAILS TO OPEN |
| R10-BAC-LP-230KVMAIN | 4.80E-06 | 230 KV MAIN DISTRIBUTION FAILS DURING OPERATION |
| R10-BAC-LP-500KVMAIN | 4.80E-06 | 500 KV MAIN DISTRIBUTION BUS FAILS DURING OPERATION |
| R10-CBU-FC-PRE230KV | 7.20E-05 | 230 KV POWER TRANSMISSION LINE FAILS |
| R10-CBU-FC-PRE500KV | 7.20E-05 | 500KV TRANSMISSION LINE FAILS |
| R10-SYS-FF-230KV | 5.00E-04 | 230 KV SWITCHYARD FAILS DURING OPEATION |
| R10-SYS-FF-500KV | 1.20E-06 | 500KV SWITCHYARD FAILS DURING OPERATION |
| R10-SYS-TM-230KV | 1.00E-02 | 230 KV SWITCHYARD IN MAINTENANCE |
| R10-XFH-LP-XFRMA | 2.88E-05 | TRANSFORMER XFRM-A (HIGH VOLTAGE) FAILS TO OPERATE |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|-----------------------|----------|--|
| R10-XFH-LP-XFRMB | 2.88E-05 | TRANSFORMER XFRM-B (HIGH VOLTAGE) FAILS TO OPERATE |
| R10-XFH-LP-XFRMC | 2.88E-05 | TRANSFORMER XFRM-C FAILS TO OPERATE |
| R10-XFH-LP-XFRMD | 2.88E-05 | TRANSFORMER XFRM-D FAILS TO OPERATE |
| R10-XFH-TM-XFRMA | 1.00E-04 | TRANSFORMER XFRM-A IN MAINTENANCE |
| R10-XFH-TM-XFRMB | 1.00E-04 | TRANSFORMER XFRM-B IN MAINTENANCE |
| R10-XFH-TM-XFRMC | 1.00E-04 | TRANSFORMER XFRM-C IN MAINTENANCE |
| R10-XFH-TM-XFRMD | 1.00E-04 | TRANSFORMER XFRM-D IN MAINTENANCE |
| R11-BAC-LP-R11A2 | 4.80E-06 | BUS R11-A2 FAILS DURING OPERATION |
| R11-BAC-LP-R11B2 | 4.80E-06 | BUS R11-B2 FAILS DURING OPERATION |
| R11-BAC-TM-R11A2 | 4.80E-06 | 6.9 KV AC BUS R11-A2 IN MANTENANCE |
| R11-BAC-TM-R11B2 | 4.80E-06 | 6.9 KV AC BUS R11-B2 IN MANTENANCE |
| R11-MCB-CC-XFRMAA2 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-A FAILS TO OPEN |
| R11-MCB-CC-XFRMBB2 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-B FAILS TO OPEN |
| R11-MCB-CF-69CLOSE | 9.29E-05 | CCF CIRCUIT BREAKERS 6.9 KV TO CLOSE |
| R11-MCB-CO-XFRMAA2 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-A OPENS SPURIOUSLY |
| R11-MCB-CO-XFRMBB2 | 1.20E-05 | CIRCUIT BREAKER FROM XFRM-B OPENS SPURIOUSLY |
| R11-MCB-OO-XFRMCA2 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-C FAILS TO CLOSE |
| R11-MCB-OO-XFRMDB2 | 7.96E-03 | CIRCUIT BREAKER FROM XFRM-D FAILS TO CLOSE |
| R11-SYS-FF-NOREC | 6.13E-01 | FAILURE IN OFFSITE POWER RECOVERY |
| R13-BAC-LP-R13A1 | 4.80E-06 | BUS R13-A1 FAILS DURING OPERATION |
| R13-BAC-LP-R13A11 | 4.80E-06 | BUS R13-A1-1 FAILS DURING OPERATION |
| R13-BAC-LP-R13A2 | 4.80E-06 | BUS R13-A2 FAILS DURING OPERATION |
| R13-BAC-LP-R13A21 | 4.80E-06 | BUS R13-A2-1 FAILS DURING OPERATION |
| R13-BAC-LP-R13B1 | 4.80E-06 | BUS R13-B1 FAILS DURING OPERATION |
| R13-BAC-LP-R13B11 | 4.80E-06 | BUS R13-B1-1 FAILS DURING OPERATION |
| R13-BAC-LP-R13B2 | 4.80E-06 | BUS R13-B2 FAILS DURING OPERATION |
| R13-BAC-LP-R13B21 | 4.80E-06 | BUS R13-B2-1 FAILS DURING OPERATION |
| R13-DIO-FC-R16A1R13A1 | 1.43E-06 | DIODE FROM R16-A1 FAILS TO OPERATE |
| R13-DIO-FC-R16A1R13A2 | 1.43E-06 | DIODE FROM R16-A2 FAILS TO OPERATE |
| R13-DIO-FC-R16A1R13B1 | 1.43E-06 | DIODE FROM R16-B1 FAILS TO OPERATE |
| R13-DIO-FC-R16A1R13B2 | 1.43E-06 | DIODE FROM R16-B2 FAILS TO OPERATE |
| R13-INV-FC-R13A1 | 4.80E-04 | INVERTER TO R13-A1 FAILS |
| R13-INV-FC-R13A2 | 4.80E-04 | INVERTER TO R13-A2 FAILS |
| R13-INV-FC-R13B1 | 4.80E-04 | INVERTER TO R13-B1 FAILS |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|-----------------------|----------|--|
| R13-INV-FC-R13B2 | 4.80E-04 | INVERTER TO R13-B2 FAILS |
| R13-LCB-CO-FR13A11 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-FR13A21 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-FR13B11 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-FR13B21 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-R13A11 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-R13A21 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-R13B11 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-R13B21 | 1.44E-05 | CIRCUIT BREAKER OPENS SPURIOUSLY |
| R13-LCB-CO-R16A1R13A1 | 1.44E-05 | CIRCUIT BREAKER FROM R16-A1 OPENS SPURIOUSLY |
| R13-LCB-CO-R16A1R13A2 | 1.44E-05 | CIRCUIT BREAKER FROM R16-A2 OPENS SPURIOUSLY |
| R13-LCB-CO-R16A1R13B1 | 1.44E-05 | CIRCUIT BREAKER FROM R16-B1 OPENS SPURIOUSLY |
| R13-LCB-CO-R16A1R13B2 | 1.44E-05 | CIRCUIT BREAKER FROM R16-B2 OPENS SPURIOUSLY |
| R13-LCB-CO-TOR13A1 | 1.44E-05 | CIRCUIT BREAKER TO R13-A1 OPENS SPURIOUSLY |
| R13-LCB-CO-TOR13A2 | 1.44E-05 | CIRCUIT BREAKER TO R13-A2 OPENS SPURIOUSLY |
| R13-LCB-CO-TOR13B1 | 1.44E-05 | CIRCUIT BREAKER TO R13-B1 OPENS SPURIOUSLY |
| R13-LCB-CO-TOR13B2 | 1.44E-05 | CIRCUIT BREAKER TO R13-B2 OPENS SPURIOUSLY |
| R13-XFL-LP-R13A11 | 1.92E-05 | TRANSFORMER FAILS DURING OPERATION |
| R13-XFL-LP-R13A21 | 1.92E-05 | TRANSFORMER FAILS DURING OPERATION |
| R13-XFL-LP-R13B11 | 1.92E-05 | TRANSFORMER FAILS DURING OPERATION |
| R13-XFL-LP-R13B21 | 1.92E-05 | TRANSFORMER FAILS DURING OPERATION |
| R16-BDC-LP-R16A | 4.80E-06 | DC BUS R16-A FAILS DURING OPERATION |
| R16-BDC-LP-R16A1 | 4.80E-06 | DC BUS R16-A1 FAILS DURING OPERATION |
| R16-BDC-LP-R16A2 | 4.80E-06 | DC BUS R16-A2 FAILS DURING OPERATION |
| R16-BDC-LP-R16B | 4.80E-06 | DC BUS R16-B FAILS DURING OPERATION |
| R16-BDC-LP-R16B1 | 4.80E-06 | DC BUS R16-B1 FAILS DURING OPERATION |
| R16-BDC-LP-R16B2 | 4.80E-06 | DC BUS R16-B2 FAILS DURING OPERATION |
| R16-BDC-TM-R16A | 4.80E-06 | DC BUS R16-A IN MAINTENANCE |
| R16-BDC-TM-R16A1 | 4.80E-06 | DC BUS R16-A1 IN MAINTENANCE |
| R16-BDC-TM-R16A2 | 4.80E-06 | DC BUS R16-A2 IN MAINTENANCE |
| R16-BDC-TM-R16B | 4.80E-06 | DC BUS R16-B IN MAINTENANCE |
| R16-BDC-TM-R16B1 | 4.80E-06 | DC BUS R16-B1 IN MAINTENANCE |
| R16-BDC-TM-R16B2 | 4.80E-06 | DC BUS R16-B2 IN MAINTENANCE |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|------------------------|----------|--|
| R16-BT_-LP-R16BTA2 | 5.00E-04 | BATTERY R16-BTA2 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTB1 | 5.00E-04 | BATTERY R16-BTB1 FAILS TO PROVIDE OUTPUT |
| R16-BT_-TM-R16BTA2 | 1.00E-03 | BATTERY R16-BTA2 IN TEST |
| R16-BT_-TM-R16BTB1 | 1.00E-03 | BATTERY R16-BTB1 IN TEST |
| R16-LCB-CO-FROMR16BTA2 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTA2 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTB1 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTB1 OPENS SPURIOUSLY |
| R21-DG_-CR-ALLDG | 4.44E-03 | CCF OF DIESEL GENERATORS TO RUN |
| R21-DG_-CS-ALLDG | 8.52E-04 | CCF OF DIESEL GENERATORS TO START AND LOAD |
| R21-DG_-FR-DGA | 5.60E-02 | DIESEL GENERATOR "A" FAILS TO RUN GIVEN START |
| R21-DG_-FR-DGB | 5.60E-02 | DIESEL GENERATOR "B" FAILS TO RUN GIVEN START |
| R21-DG_-FS-DGA | 1.40E-02 | DIESEL GENERATOR "A" FAILS TO START |
| R21-DG_-FS-DGB | 1.40E-02 | D/G "B" FAILS TO START AND LOAD |
| R21-DG_-TM-DGA | 6.00E-03 | STANDBY DIESEL GENERATOR "A" IN MAINTENANCE |
| R21-DG_-TM-DGB | 6.00E-03 | STANDBY DIESEL GENERATOR "B" IN MAINTENANCE |
| R21-MCB-OO-DGAR11A2 | 1.00E-03 | CIRCUIT BREAKER FROM DG-A TO R11-A2 FAILS TO CLOSE |
| R21-MCB-OO-DGBR11B2 | 1.00E-03 | CIRCUIT BREAKER FROM DG-B TO R11-B2 FAILS TO CLOSE |
| R21-MP_-CR-FUELTRANS | 3.72E-05 | CCF TO RUN MOTOR-DRIVEN FUEL TRANSFER PUMPS |
| R21-MP_-CS-FUELTRANS | 1.71E-04 | CCF TO START MOTOR-DRIVEN FUEL TRANSFER PUMPS |
| R21-SYS-FC-AIRDG4 | 1.00E-03 | AIR STARTING SYSTEM FAILURE [#13] |
| R21-SYS-FC-AIRDG5 | 1.00E-03 | AIR STARTING SYSTEM FAILURE |
| R21-SYS-FC-FUELDG4 | 4.00E-03 | FUEL OIL STORAGE & TRANSFER SYSTEM FAILURE [#14] |
| R21-SYS-FC-FUELDG5 | 4.00E-03 | FUEL OIL STORAGE & TRANSFER SYSTEM FAILURE [#14] |
| R22-MCB-CC-1LOAD1 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 1 FAILS TO OPEN |
| R22-MCB-CC-1LOAD2 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 2 FAILS TO OPEN |
| R22-MCB-CC-1LOAD3 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 3 FAILS TO OPEN |
| R22-MCB-CC-1LOAD4 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 4 FAILS TO OPEN |
| R22-MCB-CC-1LOAD5 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 5 FAILS TO OPEN |
| R22-MCB-CC-2LOAD1 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 1 FAILS TO OPEN |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|------------------------|----------|---|
| R22-MCB-CC-2LOAD2 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 2 FAILS TO OPEN |
| R22-MCB-CC-2LOAD3 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 3 FAILS TO OPEN |
| R22-MCB-CC-2LOAD4 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 4 FAILS TO OPEN |
| R22-MCB-CC-2LOAD5 | 7.96E-03 | CIRCUIT BREAKER TO LOAD 5 FAILS TO OPEN |
| R22-RE_-FD-10A11 | 8.00E-04 | UNDERVOLTAGE RELAY FOR R22-10A11 FAILS TO OPERATE |
| R22-RE_-FD-10A21 | 8.00E-04 | UNDERVOLTAGE RELAY SENSING VOLTAGE ON BUS 10A21 FAILS |
| R11-MCB-CO-A2A3 | 1.20E-05 | CIRCUIT BREAKER FROM R11-A2 OPENS SPURIOUSLY |
| R11-MCB-CO-B2B3 | 1.20E-05 | CIRCUIT BREAKER FROM R11-B2 OPENS SPURIOUSLY |
| R12-BAC-LP-R12A201A | 4.80E-06 | BUS R12-A2-01A FAILS DURING OPERATION |
| R12-BAC-LP-R12A202A | 4.80E-06 | BUS R12-A2-02A FAILS DURING OPERATION |
| R12-BAC-LP-R12A204A | 4.80E-06 | BUS R12-A2-04A FAILS DURING OPERATION |
| R12-BAC-LP-R12A301A | 4.80E-06 | BUS R12-A3-01A FAILS DURING OPERATION |
| R12-BAC-LP-R12A302A | 4.80E-06 | BUS R12-A3-02A FAILS DURING OPERATION |
| R12-BAC-LP-R12A31 | 4.80E-06 | BUS R12-A31 FAILS DURING OPERATION |
| R12-BAC-LP-R12B201B | 4.80E-06 | BUS R12-B2-01B FAILS DURING OPERATION |
| R12-BAC-LP-R12B202B | 4.80E-06 | BUS R12-B2-02B FAILS DURING OPERATION |
| R12-BAC-LP-R12B204B | 4.80E-06 | BUS R12-B2-04B FAILS DURING OPERATION |
| R12-BAC-LP-R12B301B | 4.80E-06 | BUS R12-B3-01B FAILS DURING OPERATION |
| R12-BAC-LP-R12B302B | 4.80E-06 | BUS R12-B3-02B FAILS DURING OPERATION |
| R12-BAC-LP-R12B31 | 4.80E-06 | BUS R12-B31 FAILS DURING OPERATION |
| R12-BAC-LP-R12C23 | 4.80E-06 | BUS R12-C23 FAILS DURING OPERATION |
| R12-BAC-LP-R12C31 | 4.80E-06 | BUS R12-C31 FAILS DURING OPERATION |
| R12-BAC-LP-R12D31 | 4.80E-06 | BUS R12-D31 FAILS DURING OPERATION |
| R12-BAC-LP-R12TBAN1E | 4.80E-06 | BUS TRAIN A IN TB (NO 1E) FAILS DURING OPERATION |
| R12-BAC-LP-R12TBARTNSS | 4.80E-06 | BUS TRAIN A IN TB (RTNSS) FAILS DURING OPERATION |
| R12-BAC-LP-R12TBBN1E | 4.80E-06 | BUS TRAIN B IN TB (NO 1E) FAILS DURING OPERATION |
| R12-BAC-LP-R12TBBRTNSS | 4.80E-06 | BUS TRAIN B IN TB (RTNSS) FAILS DURING OPERATION |
| R12-BAC-TM-R12A201A | 4.80E-06 | BUS R12-A2-01A IN MAINTENANCE |
| R12-BAC-TM-R12A202A | 4.80E-06 | BUS R12-A2-02A IN MAINTENANCE |
| R12-BAC-TM-R12A204A | 4.80E-06 | BUS R12-A2-04A IN MAINTENANCE |
| R12-BAC-TM-R12A301A | 4.80E-06 | BUS R12-A3-01A IN MAINTENANCE |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|------------------------|----------|---|
| R12-BAC-TM-R12A302A | 4.80E-06 | BUS R12-A3-02A IN MAINTENANCE |
| R12-BAC-TM-R12A31 | 4.80E-06 | BUS R12-A31 IN MAINTENANCE |
| R12-BAC-TM-R12B201B | 4.80E-06 | BUS R12-B2-01B IN MAINTENANCE |
| R12-BAC-TM-R12B202B | 4.80E-06 | BUS R12-B2-02B IN MAINTENANCE |
| R12-BAC-TM-R12B204B | 4.80E-06 | BUS R12-B2-04B IN MAINTENANCE |
| R12-BAC-TM-R12B301B | 4.80E-06 | BUS R12-B3-01B IN MAINTENANCE |
| R12-BAC-TM-R12B302B | 4.80E-06 | BUS R12-B3-02B IN MAINTENANCE |
| R12-BAC-TM-R12B31 | 4.80E-06 | BUS R12-B31 IN MAINTENANCE |
| R12-BAC-TM-R12C23 | 4.80E-06 | BUS R12-C23 IN MAINTENANCE |
| R12-BAC-TM-R12C31 | 4.80E-06 | BUS R12-C31 IN MAINTENANCE |
| R12-BAC-TM-R12D31 | 4.80E-06 | BUS R12-D31 IN MAINTENANCE |
| R12-BAC-TM-R12TBAN1E | 4.80E-06 | BUS TRAIN A IN TB (NO 1E) MAINTENANCE |
| R12-BAC-TM-R12TBARTNSS | 4.80E-06 | BUS TRAIN A IN TB (RTNSS) MAINTENANCE |
| R12-BAC-TM-R12TBBN1E | 4.80E-06 | BUS TRAIN B IN TB (NO 1E) MAINTENANCE |
| R12-BAC-TM-R12TBBRTNSS | 4.80E-06 | BUS TRAIN B IN TB (RTNSS) MAINTENANCE |
| R12-LCB-CC-A2R12A31 | 5.00E-04 | CIRCUIT BREAKER IN BUS R12-A31 FROM BUS R11-A2 FAILS TO OPEN |
| R12-LCB-CC-A2R12B31 | 5.00E-04 | CIRCUIT BREAKER IN BUS R12-B31 FROM BUS R11-A2 FAILS TO OPEN |
| R12-LCB-CC-A2R12C23 | 5.00E-04 | CIRCUIT BREAKER IN BUS R12-C23 FROM BUS R11-A2 FAILS TO OPEN |
| R12-LCB-CC-A2R12C31 | 5.00E-04 | CIRCUIT BREAKER IN BUS R12-C31 FROM BUS R11-A2 FAILS TO OPEN |
| R12-LCB-CC-A2R12D31 | 5.00E-04 | CIRCUIT BREAKER IN BUS R12-D31 FROM BUS R11-A2 FAILS TO OPEN |
| R12-LCB-CF-480CLOSE | 2.79E-05 | CCF CIRCUIT BREAKERS 480 V AC TO CLOSE |
| R12-LCB-CO-A2R12A201A | 1.44E-05 | CIRCUIT BREAKER IN R12-A2-01A OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12A202A | 1.44E-05 | CIRCUIT BREAKER IN R12-A2-02A OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12A204A | 1.44E-05 | CIRCUIT BREAKER IN R12-A2-04A OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12A31 | 1.44E-05 | CIRCUIT BREAKER IN BUS R12-A31 FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12B31 | 1.44E-05 | CIRCUIT BREAKER IN BUS R12-B31 FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12C23 | 1.44E-05 | CIRCUIT BREAKER IN BUS R12-C23 FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12C31 | 1.44E-05 | CIRCUIT BREAKER IN BUS R12-C31 FROM BUS R11-A2 OPENS SPURIOUSLY |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|--------------------------|----------|---|
| R12-LCB-CO-A2R12D31 | 1.44E-05 | CIRCUIT BREAKER IN BUS R12-D31 FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-A2R12TBAN1E | 1.44E-05 | CIRCUIT BREAKER IN BUS TRAIN A IN TB (NO 1E) OPENS SPURIOUSLY |
| R12-LCB-CO-A3R12A301A | 1.44E-05 | CIRCUIT BREAKER IN R12-A3-01A OPENS SPURIOUSLY |
| R12-LCB-CO-A3R12A302A | 1.44E-05 | CIRCUIT BREAKER IN R12-A3-02A OPENS SPURIOUSLY |
| R12-LCB-CO-A3R12TBARTNSS | 1.44E-05 | CIRCUIT BREAKER IN BUS TRAIN A IN TB (RTNSS) OPENS SPURIOUSLY |
| R12-LCB-CO-B2R12B202B | 1.44E-05 | CIRCUIT BREAKER IN R12-B2-02B OPENS SPURIOUSLY |
| R12-LCB-CO-B2R12B204B | 1.44E-05 | CIRCUIT BREAKER IN R12-B2-04B OPENS SPURIOUSLY |
| R12-LCB-CO-B3R12B201B | 1.44E-05 | CIRCUIT BREAKER IN R12-B2-01B OPENS SPURIOUSLY |
| R12-LCB-CO-B3R12B301B | 1.44E-05 | CIRCUIT BREAKER IN R12-B3-01B OPENS SPURIOUSLY |
| R12-LCB-CO-B3R12B302B | 1.44E-05 | CIRCUIT BREAKER IN R12-B3-02B OPENS SPURIOUSLY |
| R12-LCB-CO-B3R12TBBN1E | 1.44E-05 | CIRCUIT BREAKER IN BUS TRAIN B IN TB (NO 1E) OPENS SPURIOUSLY |
| R12-LCB-CO-B3R12TBBRTNSS | 1.44E-05 | CIRCUIT BREAKER IN BUS TRAIN B IN TB (RTNSS) OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12A201A | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A1 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12A202A | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12A204A | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12A31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12B31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12C23 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12C31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12D31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A2 OPENS SPURIOUSLY |
| R12-LCB-CO-FA2R12TBAN1E | 1.44E-05 | CIRCUIT BREAKER FROM BUS R10-A1 OPENS SPURIOUSLY |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|---------------------------|----------|---|
| R12-LCB-CO-FA3R12A301A | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A3 OPENS SPURIOUSLY |
| R12-LCB-CO-FA3R12A302A | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A3 OPENS SPURIOUSLY |
| R12-LCB-CO-FA3R12TBARTNSS | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-A3 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12A31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12B202B | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12B204B | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12B31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12C23 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12C31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB2R12D31 | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB3R12B201B | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B2 OPENS SPURIOUSLY |
| R12-LCB-CO-FB3R12B301B | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B3 OPENS SPURIOUSLY |
| R12-LCB-CO-FB3R12B302B | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B3 OPENS SPURIOUSLY |
| R12-LCB-CO-FB3R12TBBN1E | 1.44E-05 | CIRCUIT BREAKER FROM BUS R10-B1 OPENS SPURIOUSLY |
| R12-LCB-CO-FB3R12TBBRTNSS | 1.44E-05 | CIRCUIT BREAKER FROM BUS R11-B3 OPENS SPURIOUSLY |
| R12-LCB-OO-B2R12A31 | 3.00E-04 | CIRCUIT BREAKER IN BUS R11-A31 FAILS TO CLOSE |
| R12-LCB-OO-B2R12B31 | 3.00E-04 | CIRCUIT BREAKER IN BUS R11-B31 FAILS TO CLOSE |
| R12-LCB-OO-B2R12C23 | 3.00E-04 | CIRCUIT BREAKER IN BUS R11-C23 FAILS TO CLOSE |
| R12-LCB-OO-B2R12C31 | 3.00E-04 | CIRCUIT BREAKER IN BUS R11-C31 FAILS TO CLOSE |
| R12-LCB-OO-B2R12D31 | 3.00E-04 | CIRCUIT BREAKER IN BUS R11-D31 FAILS TO CLOSE |
| R12-XFL-LP-A2R12A31 | 1.92E-05 | TRANSFORMER FROM R11-A2 TO R12-A31 FAILS TO OPERATE |
| R12-XFL-LP-A2R12B31 | 1.92E-05 | TRANSFORMER FROM R11-A2 TO R12-B31 FAILS TO OPERATE |
| R12-XFL-LP-A2R12C23 | 1.92E-05 | TRANSFORMER FROM R11-A2 TO R12-C23 FAILS TO OPERATE |

Table 4.14-7
List of System Basic Events

| Basic Event | Prob | Description |
|------------------------|----------|---|
| R12-XFL-LP-A2R12C31 | 1.92E-05 | TRANSFORMER FROM R11-A2 TO R12-C31 FAILS TO OPERATE |
| R12-XFL-LP-A2R12D31 | 1.92E-05 | TRANSFORMER FROM R11-A2 TO R12-D31 FAILS TO OPERATE |
| R12-XFL-LP-B2R12A31 | 1.92E-05 | TRANSFORMER FROM R11-B2 TO R12-A31 FAILS TO OPERATE |
| R12-XFL-LP-B2R12B31 | 1.92E-05 | TRANSFORMER FROM R11-B2 TO R12-B31 FAILS TO OPERATE |
| R12-XFL-LP-B2R12C23 | 1.92E-05 | TRANSFORMER FROM R11-B2 TO R12-C23 FAILS TO OPERATE |
| R12-XFL-LP-B2R12C31 | 1.92E-05 | TRANSFORMER FROM R11-B2 TO R12-C31 FAILS TO OPERATE |
| R12-XFL-LP-B2R12D31 | 1.92E-05 | TRANSFORMER FROM R11-B2 TO R12-D31 FAILS TO OPERATE |
| R12-XFL-LP-R12A201A | 1.92E-05 | TRANSFORMER TO R12-A2-01A FAILS TO OPERATE |
| R12-XFL-LP-R12A202A | 1.92E-05 | TRANSFORMER TO R12-A2-02A FAILS TO OPERATE |
| R12-XFL-LP-R12A204A | 1.92E-05 | TRANSFORMER TO R12-A2-04A FAILS TO OPERATE |
| R12-XFL-LP-R12A301A | 1.92E-05 | TRANSFORMER TO R12-A3-01A FAILS TO OPERATE |
| R12-XFL-LP-R12A302A | 1.92E-05 | TRANSFORMER TO R12-A3-02A FAILS TO OPERATE |
| R12-XFL-LP-R12B201B | 1.92E-05 | TRANSFORMER TO R12-B2-01B FAILS TO OPERATE |
| R12-XFL-LP-R12B202B | 1.92E-05 | TRANSFORMER TO R12-B2-02B FAILS TO OPERATE |
| R12-XFL-LP-R12B204B | 1.92E-05 | TRANSFORMER TO R12-B2-04B FAILS TO OPERATE |
| R12-XFL-LP-R12B301B | 1.92E-05 | TRANSFORMER TO R12-B3-01B FAILS TO OPERATE |
| R12-XFL-LP-R12B302B | 1.92E-05 | TRANSFORMER TO R12-B3-02B FAILS TO OPERATE |
| R12-XFL-LP-R12TBAN1E | 1.92E-05 | TRANSFORMER TO BUS TRAIN A IN TB (NO 1E) FAILS TO OPERATE |
| R12-XFL-LP-R12TBARTNSS | 1.92E-05 | TRANSFORMER TO BUS TRAIN A IN TB (RTNSS) FAILS TO OPERATE |
| R12-XFL-LP-R12TBBN1E | 1.92E-05 | TRANSFORMER TO BUS TRAIN B IN TB (NO 1E) FAILS TO OPERATE |
| R12-XFL-LP-R12TBBRTNSS | 1.92E-05 | TRANSFORMER TO BUS TRAIN B IN TB (RTNSS) FAILS TO OPERATE |

Table 4.14-8
System Top Events

| Top Event | Description |
|------------------|--|
| GR10-0001-_1 | LOSS OF 13.8 KV AC FROM BUS R10-A1 |
| GR10-0001-_2 | LOSS OF 13.8 KV AC FROM BUS R10-B1 |
| GR10-0001-_3 | LOSS OF 13.8 KV AC FROM BUS R10-C1 |
| GR10-0001-_4 | LOSS OF 13.8 KV AC FROM BUS R10-D1 |
| GR11-0001-_1 | LOSS OF 6.9 KV AC FROM BUS R11-A2 |
| GR11-0001-_2 | LOSS OF 6.9 KV AC FROM BUS R11-B2 |
| GR11-0001-_3 | LOSS OF 6.9 KV AC FROM BUS R11-A3 |
| GR11-0001-_4 | LOSS OF 6.9 KV AC FROM BUS R11-B3 |
| GR12-0001-_1 | LOSS OF 480 V AC FROM BUS R12-A31 |
| GR12-0001-_2 | LOSS OF 480 V AC FROM BUS R12-B31 |
| GR12-0001-_3 | LOSS OF 480 V AC FROM BUS R12-C31 |
| GR12-0001-_4 | LOSS OF 480 V AC FROM BUS R12-D31 |
| GR12-0001-_5 | LOSS OF 480 V AC FROM BUS R12-C23 |
| GR12-0001-_6 | LOSS OF 480 V AC FROM BUS R12-A3-01A |
| GR12-0001-_7 | LOSS OF 480 V AC FROM BUS R12-A3-02A |
| GR12-0001-_8 | LOSS OF 480 V AC FROM BUS R12-A2-02A |
| GR12-0001-_9 | LOSS OF 480 V AC FROM BUS R12-B3-01B |
| GR12-0001-_10 | LOSS OF 480 V AC FROM BUS R12-B3-02B |
| GR12-0001-_11 | LOSS OF 480 V AC FROM BUS R12-B2-02B |
| GR12-0001-_12 | LOSS OF 480 V AC FROM BUS R12-A2-04A (FUEL BUILDING) |
| GR12-0001-_13 | LOSS OF 480 V AC FROM BUS R12-B2-04B (FUEL BUILDING) |
| GR12-0001-_14 | LOSS OF 480 V AC FROM TRAIN A IN TB (DID) |
| GR12-0001-_15 | LOSS OF 480 V AC FROM TRAIN B IN TB (DID) |
| GR12-0001-_16 | LOSS OF 480 V AC FROM TRAIN A IN TB (NO 1E) |
| GR12-0001-_17 | LOSS OF 480 V AC FROM TRAIN B IN TB (NO 1E) |
| GR12-0001-_18 | LOSS OF 480 V AC FROM BUS R12-A2-01A |
| GR12-0001-_19 | LOSS OF 480 V AC FROM BUS R12-B2-01B |

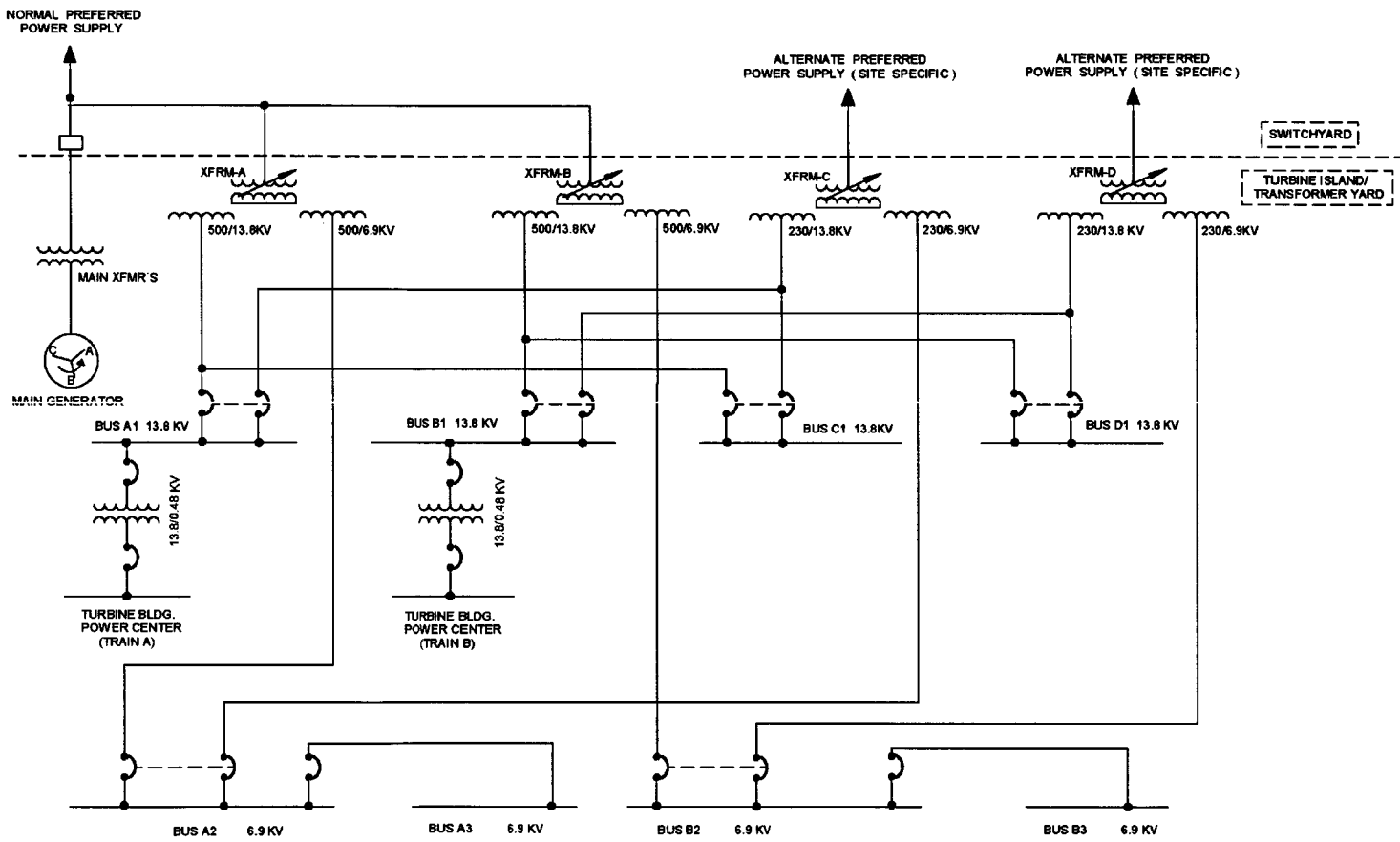


Figure 4.14-1A. A.C. Electric Power System One-line Simplified Diagram

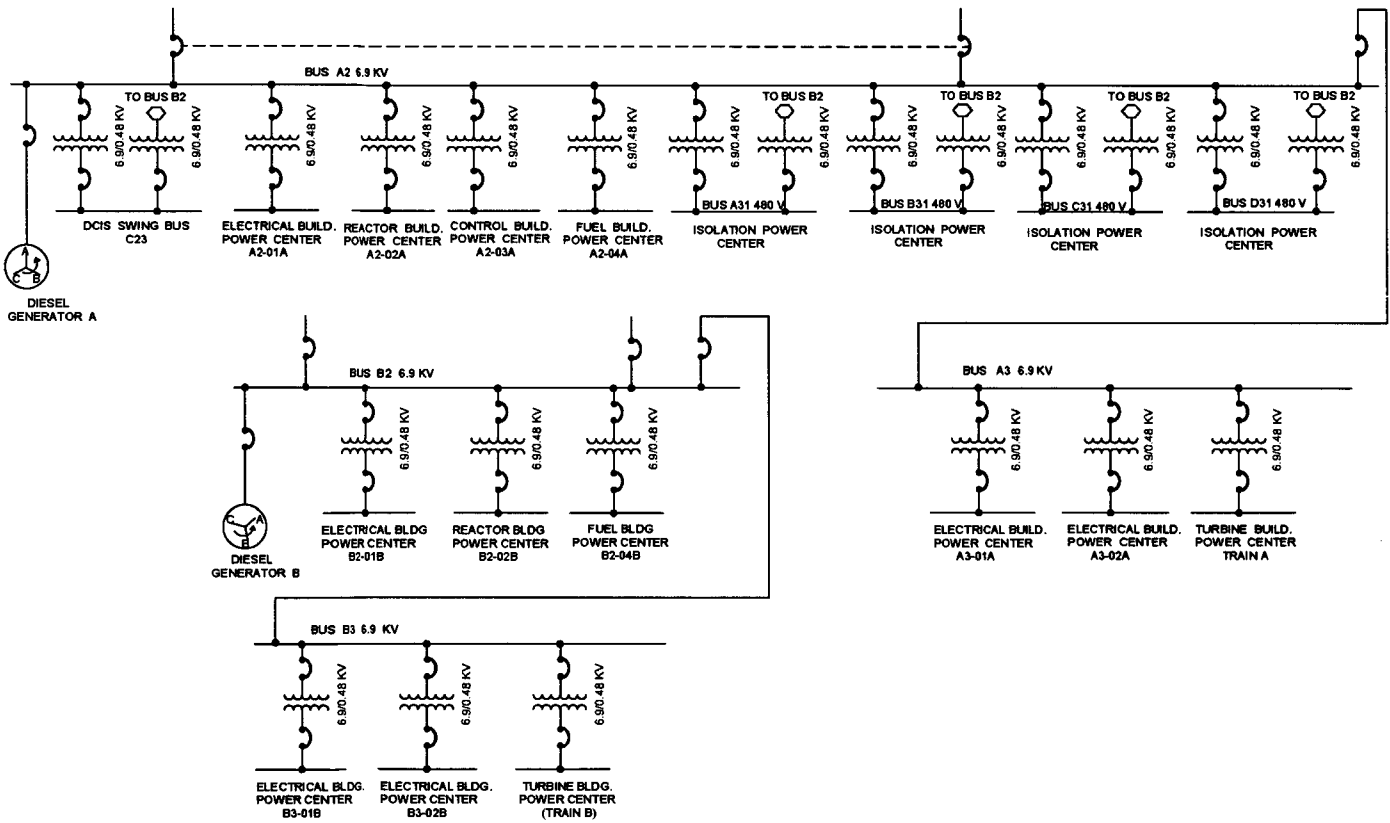


Figure 4.14-1B. A.C. Electric Power System One-line Simplified Diagram

4.15 UNINTERRUPTIBLE AC POWER SUPPLY SYSTEM – (R13)

4.15.1 System Purpose

This system provides continuous and reliable uninterruptible low voltage AC power for instrument loads, computer systems and other important plant loads, including safety-related loads (supplied from safety-related system) and non-safety-related load (supplied from non-safety-related system). This system is required for plant start-up, normal operation, and normal or emergency shutdown.

4.15.2 Design Assumptions

The following assumption with respect to the design is made:

- (1) It is assumed that preventive and corrective maintenance actions that can make a bus unavailable are avoided while the plant is at power.

4.15.3 System Description

4.15.3.1 Hardware Configuration

Figure 4.15-1, shows the principal components of this system.

The safety-related Uninterruptible Power Supply consists of four divisions. Division 1 and 2 include two separate units. One unit supplies 120 V single-phase power and the other unit supplies 480 V AC three-phase power. Divisions 3 and 4 supply 480 V AC three-phase power. Each unit has two power supplies. The main source is from 250 VDC. The auxiliary source is through a voltage regulatory transformer supplied by 480 VAC.

There are five non-safety related uninterruptible power supply systems. Three of them receive 480 V AC and 250 V DC power and supply 480 V AC, while the other two receive 480 V AC and 125 V DC power and supply 120 V AC.

In the systems that are a long distance away from the DC power system batteries, the uninterruptible power can be supplied by a modularized Uninterruptible Power Supply (UPS) that includes batteries and battery chargers.

4.15.3.2 System Operation

During plant start-up, normal operation, normal and emergency shutdown, and during plant outages, the Uninterruptible Power Supply System inverters receive rectified DC power through the battery charger with the batteries floating on line, while supplying constant voltage and constant frequency to its connected loads.

During station blackout conditions the system inverters automatically and without interruption, supply constant voltage and constant frequency AC power from the DC power system or the UPSs batteries, whichever applies, to its connected loads.

When the DC input is lost, the AC input is switched. A static transfer switch, capable of operating within ½ cycle of 60 Hz, automatically switches the inverter output to a regulating transformer when the inverter portion of the supply fails. Prior to the static transfer switch operation, the inverter is kept in synchronism with the bypass source.

4.15.3.3 System Location

The safety-related Uninterruptible Power Supply System (UPSS) is located in the Reactor Building.

The non-safety-related 120V AC power system is located in the Electrical Building.

4.15.4 Automatic and Manual Control

Appropriate manual and automatic controls are provided to maintain the operating parameters and systems within prescribed operating ranges and, specifically, to permit the following operations:

- Automatic and manual transfer to the alternate source
- Automatic and manual re-transfer back to the normal source

Table 4.15-1 lists the signals and alarms that are assumed to be needed.

4.15.5 System Interfaces

The UPSS begins at the input terminals of the inverters and regulating transformers and ends at the input terminals of the system's loads. For AC power provided by a modularized UPSS, complete with battery chargers, the system begins at the input terminals of the UPSS battery charger and alternate source.

The Low Voltage AC power supply system provides power to the battery chargers and regulating transformers, and for operation of their auxiliary equipment, such as cooling fans, heaters, etc.

The DC power supply system provides power to inverters.

The instrumentation and control systems provide remote monitoring and control capabilities.

Table 4.15-2 shows the assumed system dependencies.

4.15.6 System Testing

Periodic tests of the UPSS are conducted in accordance with normal power plant requirements for demonstrating system and component operability and integrity.

Table 4.15-3 shows the assumed component tests.

4.15.7 System Maintenance

A maintenance switch parallels the static transfer switch to allow maintenance of the inverter/static transfer switch while continuing power to the vital loads. It is possible to throw the maintenance switch while the inverter and loads are operating.

The equipment and components of the UPSS are designed for easy inspection and maintenance during plant operation.

4.15.8 Potential for Common Cause Failure (CCF)

Potential CCFs are considered negligible because the system has a continuous operation, continuous surveillance and its preferred alignment is with the DC power supply system.

4.15.9 Fault Tree Analysis

4.15.9.1 Top Event Definition

A fault tree is developed for each one of the safety-related and non-safety-related buses.

Top events are listed in Table 4.15-8

4.15.9.2 Fault Tree Description

Eighteen fault trees are developed to reflect the failure of each class 1E buses:

- Fault trees GR13-0001-_1 to GR13-0001-_8 model the failures of the 208/120V AC buses, for 24 h. supply.
- Fault trees GR13-0001-_9 and GR13-0001-_10 model the failures of the 120V AC buses, for 72 h. supply.
- Fault trees GR13-0001-_11 to GR13-0001-_18 model the failures of the 480V AC buses, for 24 h. supply.

The rest of the fault trees relate to the non-class 1E buses:

- Fault trees GR13-0001-_19 to GR13-0001-_21, GR13-0001-_23 to GR13-0001-_25 and GR13-0001-_27 model the failures of the 208/120V AC buses.
- Fault trees GR13-0001-_22, GR13-0001-_26, GR13-0001-_28 and GR13-0001-_29 model the failures of the 120V AC buses.

4.15.9.3 Human Interactions

There is no human action included in the model.

4.15.10 Results of Fault Tree Analysis

The definition of each basic event is reported in Table 4.15-7.

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.15.11 Model Differences for 72-Hour Operation

There are no changes to the model for operation to 72 hours other than the mission time.

4.15.12 References

4.15-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.15-1
Control Room Instrumentation and Alarms

| |
|--------------------------|
| Controls |
| |
| Displays |
| Inverters input voltages |
| |
| Alarms |

Table 4.15-2
System Dependency Matrix

| Component | Type | Support System | | | | | | | | | |
|-----------|------|----------------------------|----|----|----|----|----|---------------------|-----|-----|-----|
| | | Power Supply | | | | | | | | | |
| | | 250 V DC class 1E (busses) | | | | | | 480 V A.C. (busses) | | | |
| | | 11 | 12 | 21 | 22 | 31 | 41 | A31 | B31 | C31 | D31 |
| Bus 11-1 | Bus | X | | | | | | X | | | |
| Bus 11-2 | Bus | X | | | | | | X | | | |
| Bus 21-1 | Bus | | | X | | | | | X | | |
| Bus 21-2 | Bus | | | X | | | | | X | | |
| Bus 31-1 | Bus | | | | | X | | | | X | |
| Bus 31-2 | Bus | | | | | X | | | | X | |
| Bus 41-1 | Bus | | | | | | X | | | | X |
| Bus 41-2 | Bus | | | | | | X | | | | X |
| Bus 12 | Bus | | X | | | | | X | | | |
| Bus 22 | Bus | | | | X | | | | X | | |
| Bus 11-3 | Bus | X | | | | | | X | | | |
| Bus 11-4 | Bus | X | | | | | | X | | | |
| Bus 21-3 | Bus | | | X | | | | | X | | |
| Bus 21-4 | Bus | | | X | | | | | X | | |
| Bus 31-3 | Bus | | | | | X | | | | X | |
| Bus 31-4 | Bus | | | | | X | | | | X | |
| Bus 41-3 | Bus | | | | | | X | | | | X |
| Bus 41-4 | Bus | | | | | | X | | | | X |

| Component | Type | Support System | | | | | | | | | | |
|-----------|------|-------------------------------|----|----|----|---|---------------------|--------|--------|--------|--------|-----|
| | | Power Supply | | | | | | | | | | |
| | | 250 V DC no class 1E (busses) | | | | | 480 V A.C. (busses) | | | | | |
| | | A1 | A2 | B1 | B2 | C | A2-02A | A3-01A | A3-02A | B3-02B | B3-01B | C23 |
| Bus A1-1 | Bus | X | | | | | | X | X | | | |
| Bus A1-2 | Bus | X | | | | | | X | X | | | |
| Bus A2-1 | Bus | | X | | | | | X | X | | | |
| Bus A2-2 | Bus | | X | | | | | X | X | | | |
| Bus B1-1 | Bus | | | X | | | | | | X | X | |
| Bus B1-2 | Bus | | | X | | | | | | X | X | |
| Bus B2-1 | Bus | | | | X | | | | | X | X | |
| Bus B2-2 | Bus | | | | X | | | | | X | X | |
| Bus C-1 | Bus | | | | | X | X | | | | | X |
| Bus C-2 | Bus | | | | | X | X | | | | | X |
| Bus C-3 | Bus | | | | | X | X | | | | | X |

Table 4.15-3
Component Tests

| Component | Expected Test Interval |
|------------------|-------------------------------|
| | |
| Transfer switch | 3 months |

Table 4.15-4
Component Maintenance

No preventive maintenance actions are expected to be performed during normal plant operation that can cause loss of power in a bus.

Table 4.15-5
System Common Cause Failures

Common cause failures have been considered negligible

Table 4.15-6
System Human Errors

No human actions have been included in the fault tree model.

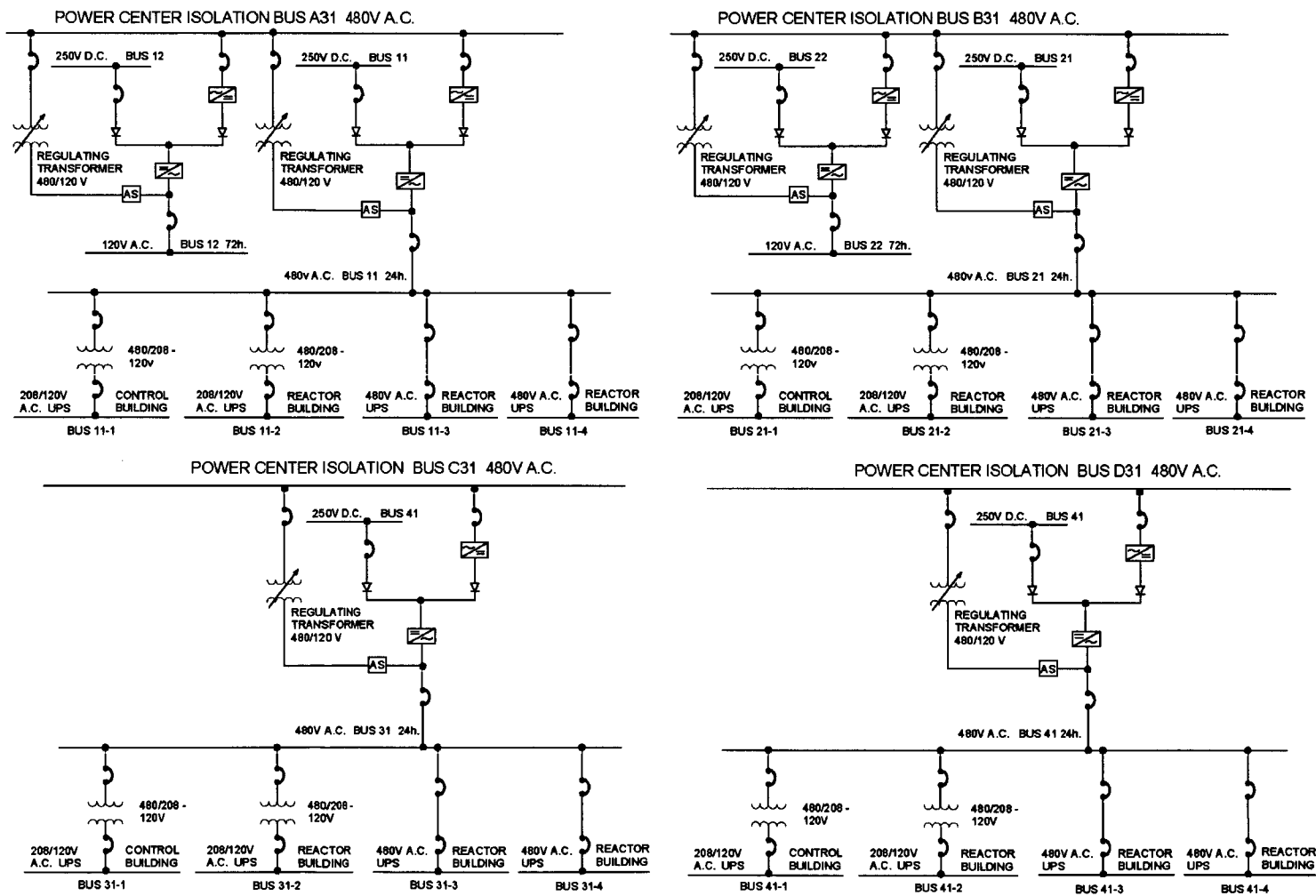
Table 4.15-7
System Basic Events

| Top Event | Description |
|------------------|--|
| GR13-0001-_1 | LOSS OF 208/120 V AC FROM BUS R13-11-1 |
| GR13-0001-_2 | LOSS OF 208/120 V AC FROM BUS R13-11-2 |
| GR13-0001-_3 | LOSS OF 208/120 V AC FROM BUS R13-21-1 |
| GR13-0001-_4 | LOSS OF 208/120 V AC FROM BUS R13-21-2 |
| GR13-0001-_5 | LOSS OF 208/120 V AC FROM BUS R13-31-1 |
| GR13-0001-_6 | LOSS OF 208/120 V AC FROM BUS R13-31-2 |
| GR13-0001-_7 | LOSS OF 208/120 V AC FROM BUS R13-41-1 |
| GR13-0001-_8 | LOSS OF 208/120 V AC FROM BUS R13-41-2 |
| GR13-0001-_9 | LOSS OF 120 V AC FROM R13-12 |
| GR13-0001-_10 | LOSS OF 120 V AC FROM R13-22 |
| GR13-0001-_11 | LOSS OF 480 V AC FROM BUS R13-11-3 |
| GR13-0001-_12 | LOSS OF 480 V AC FROM BUS R13-11-4 |
| GR13-0001-_13 | LOSS OF 480 V AC FROM R13-21-3 |
| GR13-0001-_14 | LOSS OF 480 V AC FROM R13-21-4 |
| GR13-0001-_15 | LOSS OF 480 V AC FROM R13-31-3 |
| GR13-0001-_16 | LOSS OF 480 V AC FROM R13-31-4 |
| GR13-0001-_17 | LOSS OF 480 V AC FROM R13-41-3 |
| GR13-0001-_18 | LOSS OF 480 V AC FROM R13-41-4 |
| GR13-0001-_19 | LOSS OF 208/120 V AC FROM BUS R13-A1-1 |
| GR13-0001-_20 | LOSS OF 208/120 V AC FROM BUS R13-A1-2 |
| GR13-0001-_21 | LOSS OF 208/120 V AC FROM BUS R13-A2-1 |
| GR13-0001-_22 | LOSS OF 120 V AC FROM BUS R13-A2-2 |
| GR13-0001-_23 | LOSS OF 208/120 V AC FROM BUS R13-B1-1 |
| GR13-0001-_24 | LOSS OF 208/120 V AC FROM BUS R13-B1-2 |
| GR13-0001-_25 | LOSS OF 208/120 V AC FROM BUS R13-B2-1 |
| GR13-0001-_26 | LOSS OF 120 V AC FROM BUS R13-B2-2 |
| GR13-0001-_27 | LOSS OF 208/120 V AC FROM BUS R13-C1-1 |
| GR13-0001-_28 | LOSS OF 120 V AC FROM BUS R13-C1-2 |
| GR13-0001-_29 | LOSS OF 120 V AC FROM BUS R13-C1-3 |

Table 4.15-8
System Top Events

| Top Event | Description |
|------------------|--|
| GR13-0001-_1 | LOSS OF 208/120 V AC FROM BUS R13-11-1 |
| GR13-0001-_2 | LOSS OF 208/120 V AC FROM BUS R13-11-2 |
| GR13-0001-_3 | LOSS OF 208/120 V AC FROM BUS R13-21-1 |
| GR13-0001-_4 | LOSS OF 208/120 V AC FROM BUS R13-21-2 |
| GR13-0001-_5 | LOSS OF 208/120 V AC FROM BUS R13-31-1 |
| GR13-0001-_6 | LOSS OF 208/120 V AC FROM BUS R13-31-2 |
| GR13-0001-_7 | LOSS OF 208/120 V AC FROM BUS R13-41-1 |
| GR13-0001-_8 | LOSS OF 208/120 V AC FROM BUS R13-41-2 |
| GR13-0001-_9 | LOSS OF 120 V AC FROM R13-12 |
| GR13-0001-_10 | LOSS OF 120 V AC FROM R13-22 |
| GR13-0001-_11 | LOSS OF 480 V AC FROM BUS R13-11-3 |
| GR13-0001-_12 | LOSS OF 480 V AC FROM BUS R13-11-4 |
| GR13-0001-_13 | LOSS OF 480 V AC FROM R13-21-3 |
| GR13-0001-_14 | LOSS OF 480 V AC FROM R13-21-4 |
| GR13-0001-_15 | LOSS OF 480 V AC FROM R13-31-3 |
| GR13-0001-_16 | LOSS OF 480 V AC FROM R13-31-4 |
| GR13-0001-_17 | LOSS OF 480 V AC FROM R13-41-3 |
| GR13-0001-_18 | LOSS OF 480 V AC FROM R13-41-4 |
| GR13-0001-_19 | LOSS OF 208/120 V AC FROM BUS R13-A1-1 |
| GR13-0001-_20 | LOSS OF 208/120 V AC FROM BUS R13-A1-2 |
| GR13-0001-_21 | LOSS OF 208/120 V AC FROM BUS R13-A2-1 |
| GR13-0001-_22 | LOSS OF 120 V AC FROM BUS R13-A2-2 |
| GR13-0001-_23 | LOSS OF 208/120 V AC FROM BUS R13-B1-1 |
| GR13-0001-_24 | LOSS OF 208/120 V AC FROM BUS R13-B1-2 |
| GR13-0001-_25 | LOSS OF 208/120 V AC FROM BUS R13-B2-1 |
| GR13-0001-_26 | LOSS OF 120 V AC FROM BUS R13-B2-2 |
| GR13-0001-_27 | LOSS OF 208/120 V AC FROM BUS R13-C1-1 |
| GR13-0001-_28 | LOSS OF 120 V AC FROM BUS R13-C1-2 |
| GR13-0001-_29 | LOSS OF 120 V AC FROM BUS R13-C1-3 |

Figure 4.15-1A. Uninterruptible A.C. Power Supply System One Line Diagram



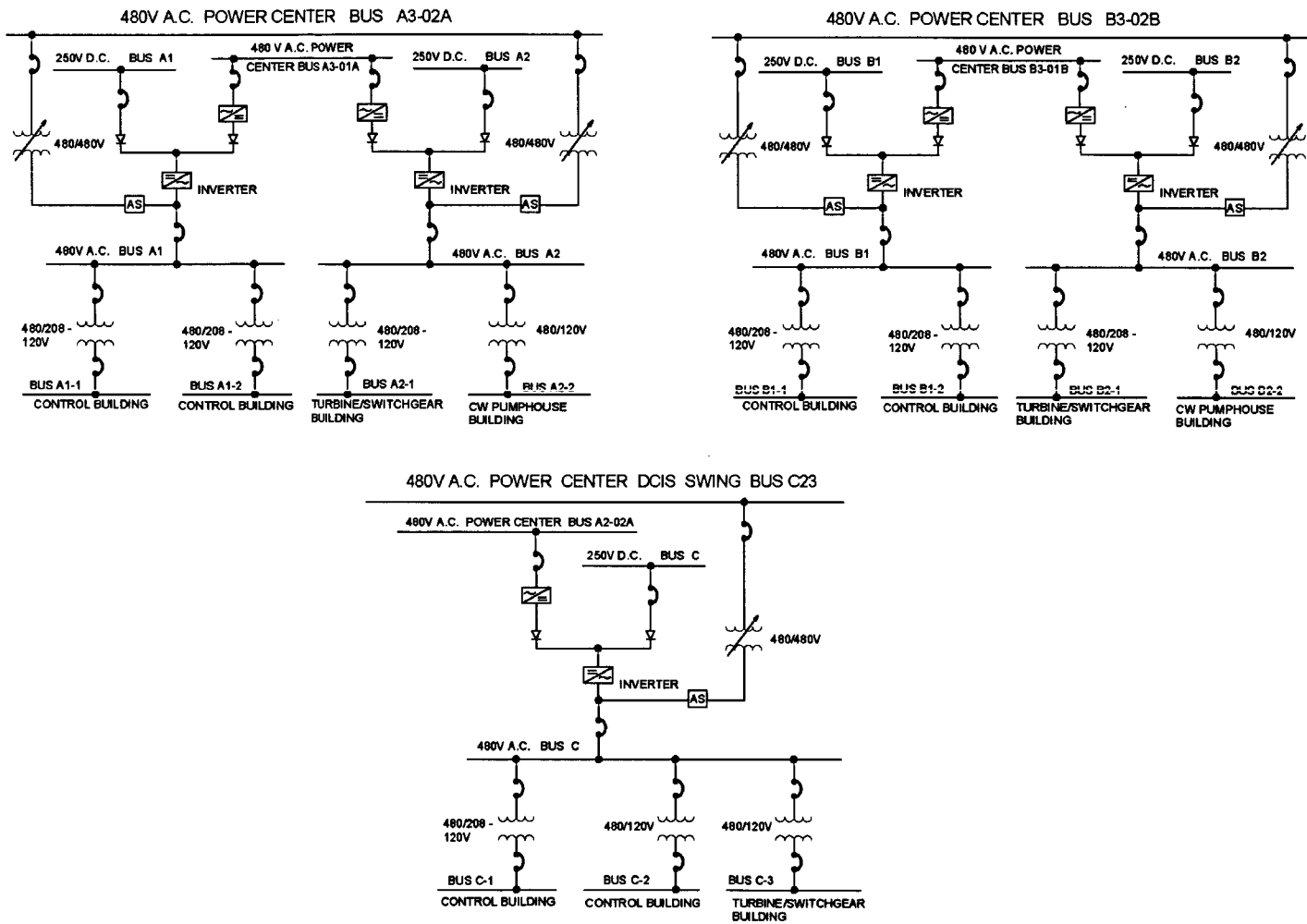


Figure 4.15-1B. Uninterruptible A.C. Power Supply System One Line Diagram

4.16 N/A SECTION 4.16 HAS BEEN DELETED FROM THE PRA

4.17 DIRECT CURRENT POWER SUPPLY SYSTEM - (R16)

4.17.1 System Purpose

The system provides power to the safety-related plant controls and instrumentation required for emergency reactor shutdown and other safety related functions. It also supplies power to non-safety related loads through 250V DC divisions with the necessary isolation between the safety and the non-safety grade part of the system. The non-safety is separated from the safety during normal operation.

4.17.2 Design Assumptions

The following assumptions with respect to the design are made:

- (1) The plug-in power source to supply the 480V AC MCCs is available long term. No human action is modeled as far as system alignment is concerned. The system is assumed to be always aligned and does not need automatic actuation or human action.
- (2) It is assumed that the only maintenance performed to a busbar is corrective maintenance after its failure.
- (3) It is assumed that adequate cooling is needed and available in battery charger room.

4.17.3 System Description

4.17.3.1 Hardware Configuration

The safety related DC distribution system is arranged in four divisional class 1E 250V DC power supplies. Each DC train consists of a battery, battery charger, and DC distribution panels. Divisions 1 and 2 have two separate DC systems. One of the systems has a battery sized to provide power for a 24-hour period. The other system has a battery sized to provide power for a 72-hour period.

Non-class 1E DC power is arranged in three 250V DC trains. The safety and non-safety battery chargers are supplied from the Motor Control Centers (MCCs) fed by diesel generator buses.

In the event of loss of all AC supplies (including diesel generators), divisional batteries supply all the safety related loads. Non-class 1E batteries are sized to provide power to protections, controls, and instrumentation loads for two hours.

Following discharge of the batteries, the battery chargers are able to supply the normal system loads and at the same time recharge the batteries. The battery charger is arranged so that the associated battery can be charged on line at an equalizing charge rate while the normal battery loads are disconnected from the battery and carried by the normal battery charger.

Each distribution panel contains disconnect switches protected by fuses, ground fault detection, and alarm devices.

An alarm located within the control room alerts the operator if the battery connection switch is inadvertently left open after test or maintenance.

Battery rooms are served by a flow-through ventilation system to remove the small amount of hydrogen produced during the charging of the batteries.

Figure 4.17-1 shows the system arrangement.

4.17.3.2 System Operation

The batteries and the battery chargers are always connected to the distribution panel during normal operation. In fault conditions, the automatic opening of the circuit breaker connecting the battery charger isolates each division.

During station blackout conditions and whenever the battery charger output is unavailable, the DC Power Supply System automatically and without interruption supplies DC power from the batteries to its connected loads.

Fuses protect battery chargers and batteries from overcurrent.

4.17.3.3 System Location

Safety related batteries, battery chargers, and related motor control centers and power centers are located in the reactor building.

The DC batteries in each division are installed in separate rooms, apart from the power distribution equipment, such as the chargers and the power center.

Non-safety related 250 V DC systems are located in Electrical Building.

4.17.4 Automatic and Manual Control

There is no logic driven disconnection in case of battery charger failure. In case of a fault to ground, overcurrent protection is assured by fuses and by the overcurrent protection device located inside the circuit breaker connecting the battery charger to the 480 V AC MCC. All switching has to be done locally by the operator through manual switches.

4.17.5 System Interfaces

The system begins at the source terminals of the plant battery chargers. It ends at the input terminals of the plant DC loads (motors, control loads, etc.) and at the DC input terminals of the inverters of the Uninterruptible AC Power Supply System.

The 250V DC division interfaces with the related 480V AC buses at the MCC level.

Battery rooms are served by a flow-through ventilation system to avoid hydrogen concentration during battery charging. In case of a loss of AC power, the battery ceases to charge so the ventilation is not needed.

System dependencies are reported in Table 4.17-2.

4.17.6 System Testing

The non-class 1E and class 1E battery testing is assumed to be the same. Battery voltage is verified weekly. Electrolyte level and density is checked every month.

A complete battery test with a 24-hour and 72-hour discharge at nominal current and a complete recharge is performed at each refueling.

The output of the battery charger and correct alignment of the charger switches is tested as part of the battery discharge test because the charger has to restore the battery charge. Consequently, a misposition of the switch connecting the battery charger is not credible.

4.17.7 System Maintenance

The battery maintenance requires disconnection from the distribution panel, but misposition after maintenance is not credible because an alarm alerts the control room operator.

4.17.8 Potential for Common Cause Failure

Battery chargers, batteries, and the fuses connecting the batteries to the distribution panel are subject to Common Cause Failure (CCF). The batteries are the most critical, since, in some accident sequences involving loss of preferred power, the power from the battery chargers is not expected, they are no longer powered.

CCF data estimated results are summarized in Table 4.17-5.

4.17.9 Fault Tree Analysis

4.17.9.1 Top Event Definitions

A fault tree is developed for each of the 250V DC (class 1E and non class 1E) divisional trains.

The fault trees for non-class 1E trains model the failure to supply power for two hours, which is the design discharge time. They are only used for short-term non-safety related systems alignment.

All top events are listed in Table 4.17-8.

4.17.9.2 Fault Tree Description

Fault trees take into account internal failures and dependency on Low Voltage AC Power System.

Fault trees are shown in the Appendix B.4.17.

4.17.9.3 Human Interaction

No human action is modeled as far as system alignment is concerned. The system is always aligned and does not need automatic actuation or human action. Misposition is possible after a battery test, but a disconnection is alarmed in the control room, so no misposition is taken into account.

4.17.10 Results of Fault Tree Analysis

The quantification of core damage sequences implicitly includes the contribution of basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

The definition of each basic event is reported in Table 4.17-7.

On the other hand, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures

in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.17.11 Model Differences for 72-Hour Operation

For the class 1E 250V DC power supply, the model differences due to a 72-hour or 24-hour mission time are mainly due to running failures and spurious operation.

4.17.12 References

4.17-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.17-1
Control Room Instrumentation and Alarms

| | |
|--|---|
| | Controls |
| | |
| | Displays |
| | Divisions Voltage |
| | Battery chargers output current and voltage |
| | CBs position indication |
| | |
| | Alarms |
| | Loss of Voltage to a Division |
| | Battery charger failure |
| | Battery discharging |
| | Fault to ground |

Table 4.17-2
System Dependency Matrix

| | | Support System | | | |
|-------------------------|----------------------|----------------|-----------|-----------|-----------|
| | | Power Supply | | | |
| | | 480 V AC MCC | | | |
| Component (250 V 1E) | Type | (R12-A31) | (R12-B31) | (R12-C31) | (R12-D31) |
| BYC11 | Battery. Charger. | X | | | |
| BYC12 | Battery. Charger. | X | | | |
| BYC21 | Battery. Charger. | | X | | |
| BYC22 | Battery. Charger. | | X | | |
| BYC31 | Battery. Charger. | | | X | |
| BYC41 | Battery. Charger. | | | | X |

| | | Support System | | |
|----------------------------|----------------------|----------------|-------------|-----------|
| | | Power Supply | | |
| | | 480 V AC MCC | | |
| Component (250 V NO 1E) | Type | (R12-A201A) | (R12-B201A) | (R12-C23) |
| | | | | |
| BYCA1 | Battery. Charger. | X | | |
| BYCA2 | Battery. Charger. | X | | |
| BYCB1 | Battery. Charger. | | X | |
| BYCB2 | Battery. Charger. | | X | |
| BYCC | Battery. Charger. | | | X |

Table 4.17-3
Component Test

| Component | Expected test interval |
|---|-------------------------------|
| Batteries voltage | 1 week |
| Electrolyte level and density | 1 month |
| Complete battery test (discharge and recharge) | Refueling (24 months) |

Table 4.17-4
Component Maintenance

No preventive maintenance actions are expected to be performed during normal plant operation.
Only corrective maintenance can be performed.

Table 4.17-5
System Common Cause Failures

| Basic Event | Prob | Description |
|----------------------|-------------|---------------------|
| R16-BT_-CF-ALLBATT | 9.00E-06 | BATTERY CCF #2 |
| R16-BT_-CF-BATT11&12 | 9.50E-07 | BATTERY 11 & 12 CCF |
| R16-BT_-CF-BATT11&21 | 9.50E-07 | BATTERY 11 & 21 CCF |
| R16-BT_-CF-BATT11&22 | 9.50E-07 | BATTERY 11 & 22 CCF |
| R16-BT_-CF-BATT11&31 | 9.50E-07 | BATTERY 11 & 31 CCF |
| R16-BT_-CF-BATT11&41 | 9.50E-07 | BATTERY 11 & 41 CCF |
| R16-BT_-CF-BATT12&21 | 9.50E-07 | BATTERY 12 & 21 CCF |
| R16-BT_-CF-BATT12&22 | 9.50E-07 | BATTERY 12 & 22 CCF |
| R16-BT_-CF-BATT12&31 | 9.50E-07 | BATTERY 12 & 31 CCF |
| R16-BT_-CF-BATT12&41 | 9.50E-07 | BATTERY 12 & 41 CCF |
| R16-BT_-CF-BATT21&22 | 9.50E-07 | BATTERY 21 & 22 CCF |
| R16-BT_-CF-BATT21&31 | 9.50E-07 | BATTERY 21 & 31 CCF |
| R16-BT_-CF-BATT21&41 | 9.50E-07 | BATTERY 21 & 41 CCF |
| R16-BT_-CF-BATT22&31 | 9.50E-07 | BATTERY 22 & 31 CCF |
| R16-BT_-CF-BATT22&41 | 9.50E-07 | BATTERY 41 & 22 CCF |
| R16-BT_-CF-BATT31&41 | 9.50E-07 | BATTERY 31 & 41 CCF |
| R16-BT_-CF-BATTA1A2 | 1.53E-06 | BATTERY A1 & A2 CCF |
| R16-BT_-CF-BATTA1B1 | 1.53E-06 | BATTERY A1 & B1 CCF |
| R16-BT_-CF-BATTA1B2 | 1.53E-06 | BATTERY A1 & B2 CCF |
| R16-BT_-CF-BATTA1C | 1.53E-06 | BATTERY A1 & C CCF |
| R16-BT_-CF-BATTA2B1 | 1.53E-06 | BATTERY A2 & B1 CCF |
| R16-BT_-CF-BATTA2B2 | 1.53E-06 | BATTERY A2 & B2 CCF |
| R16-BT_-CF-BATTA2C | 1.53E-06 | BATTERY A2 & C CCF |
| R16-BT_-CF-BATTB1B2 | 1.53E-06 | BATTERY B1 & B2 CCF |
| R16-BT_-CF-BATTB1C | 1.53E-06 | BATTERY B1 & C CCF |
| R16-BT_-CF-BATTB2C | 1.53E-06 | BATTERY B2 & C CCF |

Table 4.17-6
System Human Errors

No human actions have been included in the fault tree model.

Table 4.17-7
System Basic Events

| Basic Event | Prob | Description |
|----------------------|----------|---------------------------------------|
| R16-BDC-LP-R1611 | 4.80E-06 | DC BUS R16-11 FAILS DURING OPERATION |
| R16-BDC-LP-R1612 | 4.80E-06 | DC BUS R16-12 FAILS DURING OPERATION |
| R16-BDC-LP-R1621 | 4.80E-06 | DC BUS R16-21 FAILS DURING OPERATION |
| R16-BDC-LP-R1622 | 4.80E-06 | DC BUS R16-22 FAILS DURING OPERATION |
| R16-BDC-LP-R1631 | 4.80E-06 | DC BUS R16-31 FAILS DURING OPERATION |
| R16-BDC-LP-R1641 | 4.80E-06 | DC BUS R16-41 FAILS DURING OPERATION |
| R16-BDC-LP-R16A1 | 4.80E-06 | DC BUS R16-A1 FAILS DURING OPERATION |
| R16-BDC-LP-R16A2 | 4.80E-06 | DC BUS R16-A2 FAILS DURING OPERATION |
| R16-BDC-LP-R16B1 | 4.80E-06 | DC BUS R16-B1 FAILS DURING OPERATION |
| R16-BDC-LP-R16B2 | 4.80E-06 | DC BUS R16-B2 FAILS DURING OPERATION |
| R16-BDC-LP-R16C | 4.80E-06 | DC BUS R16-C FAILS DURING OPERATION |
| R16-BDC-TM-R1611 | 4.80E-06 | DC BUS R16-11 IN MAINTENANCE |
| R16-BDC-TM-R1612 | 4.80E-06 | DC BUS R16-12 IN MAINTENANCE |
| R16-BDC-TM-R1621 | 4.80E-06 | DC BUS R16-21 IN MAINTENANCE |
| R16-BDC-TM-R1622 | 4.80E-06 | DC BUS R16-22 IN MAINTENANCE |
| R16-BDC-TM-R1631 | 4.80E-06 | DC BUS R16-31 IN MAINTENANCE |
| R16-BDC-TM-R1641 | 4.80E-06 | DC BUS R16-41 IN MAINTENANCE |
| R16-BDC-TM-R16A1 | 4.80E-06 | DC BUS R16-A1 IN MAINTENANCE |
| R16-BDC-TM-R16A2 | 4.80E-06 | DC BUS R16-A2 IN MAINTENANCE |
| R16-BDC-TM-R16B1 | 4.80E-06 | DC BUS R16-B1 IN MAINTENANCE |
| R16-BDC-TM-R16B2 | 4.80E-06 | DC BUS R16-B2 IN MAINTENANCE |
| R16-BDC-TM-R16C | 4.80E-06 | DC BUS R16-C IN MAINTENANCE |
| R16-BT_-CF-ALLBATT | 9.00E-06 | BATTERY CCF FOR ALL 250 VDC BATTERIES |
| R16-BT_-CF-BATT11&12 | 9.50E-07 | BATTERY 11 & 12 CCF |
| R16-BT_-CF-BATT11&21 | 9.50E-07 | BATTERY 11 & 21 CCF |
| R16-BT_-CF-BATT11&22 | 9.50E-07 | BATTERY 11 & 22 CCF |
| R16-BT_-CF-BATT11&31 | 9.50E-07 | BATTERY 11 & 31 CCF |
| R16-BT_-CF-BATT11&41 | 9.50E-07 | BATTERY 11 & 41 CCF |
| R16-BT_-CF-BATT12&21 | 9.50E-07 | BATTERY 12 & 21 CCF |
| R16-BT_-CF-BATT12&22 | 9.50E-07 | BATTERY 12 & 22 CCF |
| R16-BT_-CF-BATT12&31 | 9.50E-07 | BATTERY 12 & 31 CCF |
| R16-BT_-CF-BATT12&41 | 9.50E-07 | BATTERY 12 & 41 CCF |
| R16-BT_-CF-BATT21&22 | 9.50E-07 | BATTERY 21 & 22 CCF |
| R16-BT_-CF-BATT21&31 | 9.50E-07 | BATTERY 21 & 31 CCF |
| R16-BT_-CF-BATT21&41 | 9.50E-07 | BATTERY 21 & 41 CCF |
| R16-BT_-CF-BATT22&31 | 9.50E-07 | BATTERY 22 & 31 CCF |
| R16-BT_-CF-BATT22&41 | 9.50E-07 | BATTERY 41 & 22 CCF |
| R16-BT_-CF-BATT31&41 | 9.50E-07 | BATTERY 31 & 41 CCF |
| R16-BT_-CF-BATTA1A2 | 1.53E-06 | BATTERY A1 & A2 CCF |
| R16-BT_-CF-BATTA1B1 | 1.53E-06 | BATTERY A1 & B1 CCF |
| R16-BT_-CF-BATTA1B2 | 1.53E-06 | BATTERY A1 & B2 CCF |
| R16-BT_-CF-BATTA1C | 1.53E-06 | BATTERY A1 & C CCF |
| R16-BT_-CF-BATTA2B1 | 1.53E-06 | BATTERY A2 & B1 CCF |
| R16-BT_-CF-BATTA2B2 | 1.53E-06 | BATTERY A2 & B2 CCF |

Table 4.17-7
System Basic Events

| Basic Event | Prob | Description |
|---------------------|----------|--|
| R16-BT_-CF-BATTA2C | 1.53E-06 | BATTERY A2 & C CCF |
| R16-BT_-CF-BATTB1B2 | 1.53E-06 | BATTERY B1 & B2 CCF |
| R16-BT_-CF-BATTB1C | 1.53E-06 | BATTERY B1 & C CCF |
| R16-BT_-CF-BATTB2C | 1.53E-06 | BATTERY B2 & C CCF |
| R16-BT_-LP-R16BT11 | 5.00E-04 | BATTERY R16-BT11 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BT12 | 5.00E-04 | BATTERY DE2BY001 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BT21 | 5.00E-04 | BATTERY DE3BY001 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BT22 | 5.00E-04 | BATTERY R16-BT22 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BT31 | 5.00E-04 | BATTERY R16-BT31 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BT41 | 5.00E-04 | BATTERY R16-BT41 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTA1 | 5.00E-04 | BATTERY R16-BTA1 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTA2 | 5.00E-04 | BATTERY R16-BTA2 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTB1 | 5.00E-04 | BATTERY R16-BTB1 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTB2 | 5.00E-04 | BATTERY R16-BTB2 FAILS TO PROVIDE OUTPUT |
| R16-BT_-LP-R16BTC | 5.00E-04 | BATTERY R16-BTC FAILS TO PROVIDE OUTPUT |
| R16-BT_-TM-R16BT11 | 1.00E-03 | BATTERY R16-BT11 IN TEST. |
| R16-BT_-TM-R16BT12 | 1.00E-03 | BATTERY R16-BT12 IN TEST |
| R16-BT_-TM-R16BT21 | 1.00E-03 | BATTERY R16-BT21 IN TEST |
| R16-BT_-TM-R16BT22 | 1.00E-03 | BATTERY R16-BT22 IN TEST |
| R16-BT_-TM-R16BT31 | 1.00E-03 | BATTERY R16-BT31 IN TEST |
| R16-BT_-TM-R16BT41 | 1.00E-03 | BATTERY R16-BT41 IN TEST |
| R16-BT_-TM-R16BTA1 | 1.00E-03 | BATTERY R16-BTA1 IN TEST |
| R16-BT_-TM-R16BTA2 | 1.00E-03 | BATTERY R16-BTA2 IN TEST |
| R16-BT_-TM-R16BTB1 | 1.00E-03 | BATTERY R16-BTB1 IN TEST |
| R16-BT_-TM-R16BTB2 | 1.00E-03 | BATTERY R16-BTB2 IN TEST |
| R16-BT_-TM-R16BTC | 1.00E-03 | BATTERY R16-BTC IN TEST |
| R16-BYC-LP-R16BYC11 | 1.68E-04 | BATTERY CHARGER R16-BYC11 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYC12 | 1.68E-04 | BATTERY CHARGER R16-BYC12 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYC21 | 1.68E-04 | BATTERY CHARGER R16-BYC21 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYC22 | 1.68E-04 | BATTERY CHARGER R16-BYC22 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYC31 | 1.68E-04 | BATTERY CHARGER R16-BYC31 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYC41 | 1.68E-04 | BATTERY CHARGER R16-BYC41 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYCA1 | 1.68E-04 | BATTERY CHARGER R16-BYCA1 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYCA2 | 1.68E-04 | BATTERY CHARGER R16-BYCA2 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYCB1 | 1.68E-04 | BATTERY CHARGER R16-BYCB1 FAILS TO MAINTAIN OUTPUT |
| R16-BYC-LP-R16BYCB2 | 1.68E-04 | BATTERY CHARGER R16-BYCB2 FAILS TO MAINTAIN OUTPUT |

Table 4.17-7
System Basic Events

| Basic Event | Prob | Description |
|------------------------|----------|--|
| R16-BYC-LP-R16BYCC | 1.68E-04 | BATTERY CHARGER R16-BYCC FAILS TO MAINTAIN OUTPUT |
| R16-BYC-TM-R16BYC11 | 3.00E-04 | BATTERY CHARGER R16-BYC11 IN TEST |
| R16-BYC-TM-R16BYC12 | 3.00E-04 | BATTERY CHARGER R16-BYC12 IN TEST |
| R16-BYC-TM-R16BYC21 | 3.00E-04 | BATTERY CHARGER R16-BYC21 IN TEST |
| R16-BYC-TM-R16BYC22 | 3.00E-04 | BATTERY CHARGER R16-BYC22 IN TEST |
| R16-BYC-TM-R16BYC31 | 3.00E-04 | BATTERY CHARGER R16-BYC31 IN TEST |
| R16-BYC-TM-R16BYC41 | 3.00E-04 | BATTERY CHARGER R16-BYC41 IN TEST |
| R16-BYC-TM-R16BYCA1 | 3.00E-04 | BATTERY CHARGER R16-BYCA1 IN TEST |
| R16-BYC-TM-R16BYCA2 | 3.00E-04 | BATTERY CHARGER R16-BYCA2 IN TEST |
| R16-BYC-TM-R16BYCB1 | 3.00E-04 | BATTERY CHARGER R16-BYCB1 IN TEST |
| R16-BYC-TM-R16BYCB2 | 3.00E-04 | BATTERY CHARGER R16-BYCB2 IN TEST |
| R16-BYC-TM-R16BYCC | 3.00E-04 | BATTERY CHARGER R16-BYCC IN TEST |
| R16-LCB-CO-FROMR16BT11 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT11 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BT12 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT12 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BT21 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT21 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BT22 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT22 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BT31 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT31 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BT41 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BT41 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTA1 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTA1 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTA2 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTA2 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTB1 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTB1 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTB2 | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTB2 OPENS SPURIOUSLY |
| R16-LCB-CO-FROMR16BTC | 1.44E-05 | CIRCUIT BREAKER FROM R16-BTC OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1611 | 1.44E-05 | BATT. CHARG. R16-BYC11 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1612 | 1.44E-05 | BATT. CHARG. R16-BYC12 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1621 | 1.44E-05 | BATT. CHARG. R16-BYC21 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1622 | 1.44E-05 | BATT. CHARG. R16-BYC22 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1631 | 1.44E-05 | BATT. CHARG. R16-BYC31 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR1641 | 1.44E-05 | BATT. CHARG. R16-BYC41 OUTPUT BREAKER OPENS SPURIOUSLY |

Table 4.17-7
System Basic Events

| Basic Event | Prob | Description |
|-----------------------|----------|--|
| R16-LCB-CO-TOR16A1 | 1.44E-05 | BATT. CHARG. R16-BYCA1 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16A2 | 1.44E-05 | BATT. CHARG. R16-BYCA2 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16B1 | 1.44E-05 | BATT. CHARG. R16-BYCB1 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16B2 | 1.44E-05 | BATT. CHARG. R16-BYCB2 OUTPUT BREAKER OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC11 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC11 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC12 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC12 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC21 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC21 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC22 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC22 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC31 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC31 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYC41 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYC41 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYCA1 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYCA1 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYCA2 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYCA2 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYCB1 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYCB1 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYCB2 | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYCB2 OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16BYCC | 1.44E-05 | 480V FEEDER BREAKER TO R16-BYCC OPENS SPURIOUSLY |
| R16-LCB-CO-TOR16C | 1.44E-05 | BATT. CHARG. R16-BYCC OUTPUT BREAKER OPENS SPURIOUSLY |

Table 4.17-8
System Top Events

| Top Event | Description |
|------------------|---|
| GR42-0001-_1 | LOSS OF POWER FROM 250V DC 1E BUS R16-11 |
| GR42-0001-_2 | LOSS OF POWER FROM 250V DC 1E BUS R16-12 |
| GR42-0001-_3 | LOSS OF POWER FROM 250V DC 1E BUS R16-21 |
| GR42-0001-_4 | LOSS OF POWER FROM 250V DC 1E BUS R16-22 |
| GR42-0001-_5 | LOSS OF POWER FROM 250V DC 1E BUS R16-31 |
| GR42-0001-_6 | LOSS OF POWER FROM 250V DC 1E BUS R16-41 |
| GR42-0001-_7 | LOSS OF POWER FROM 250V DC NO 1E BUS R16-A1 |
| GR42-0001-_8 | LOSS OF POWER FROM 250V DC NO 1E BUS R16-A2 |
| GR42-0001-_9 | LOSS OF POWER FROM 250V DC NO 1E BUS R16-B1 |
| GR42-0001-_10 | LOSS OF POWER FROM 250V DC NO 1E BUS R16-B2 |
| GR42-0001-_11 | LOSS OF POWER FROM 250V DC NO 1E BUS R16-C |

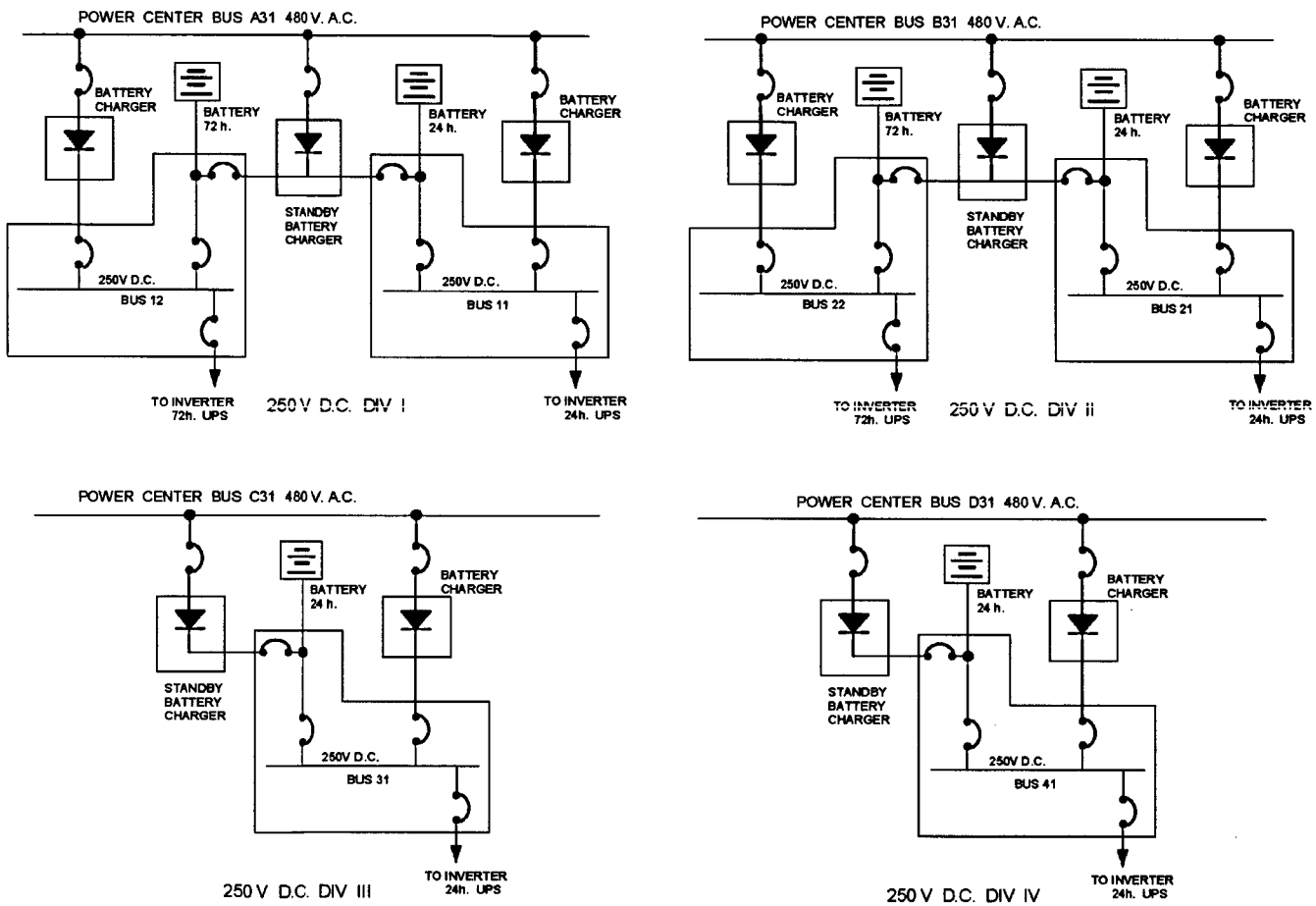


Figure 4.17-1A. Simplified One-line Diagram for 250 V DC Class 1E System

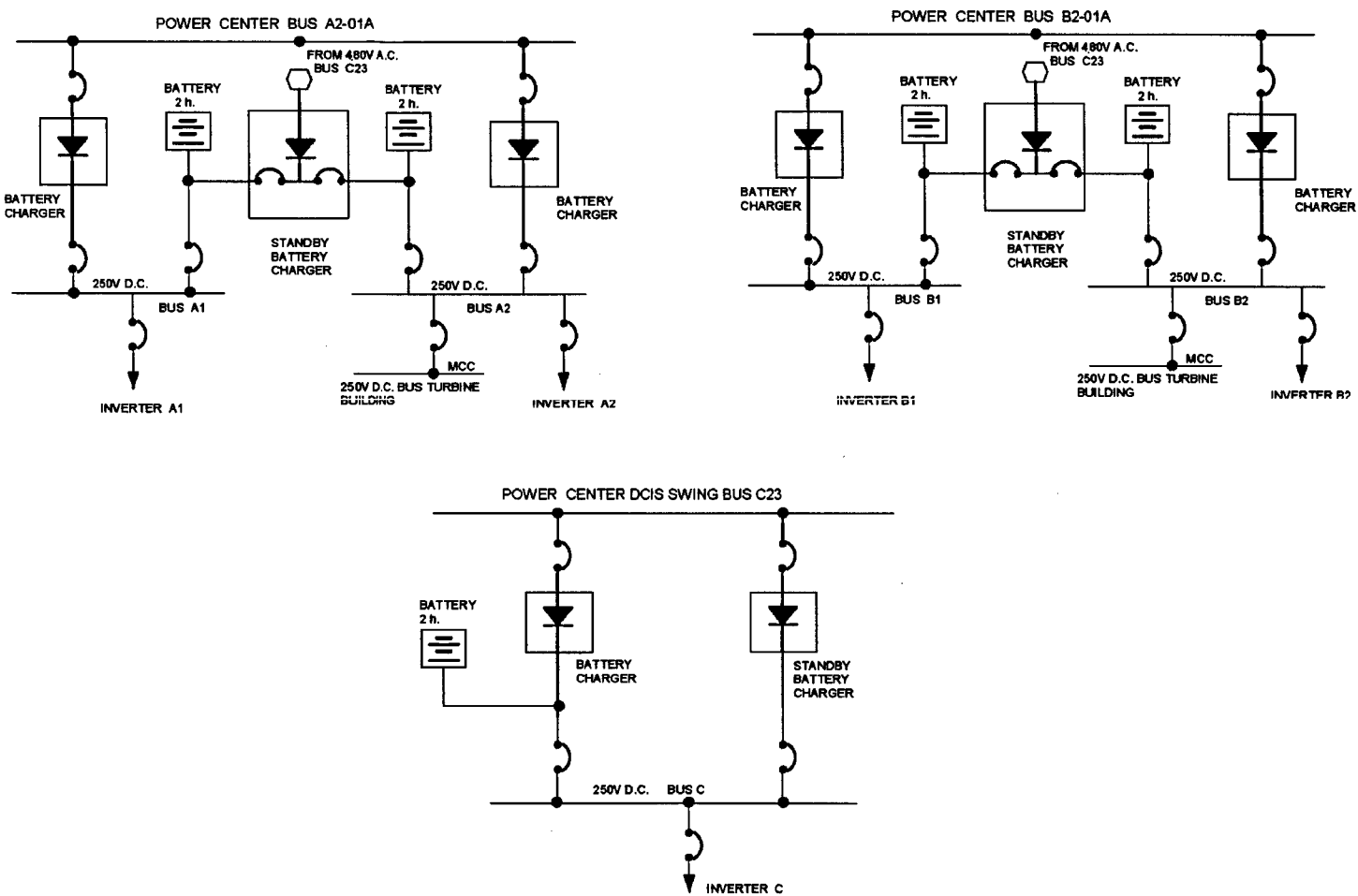


Figure 4.17-1B. Simplified One-line Diagram for 250 V DC Non-Class 1E System

4.18 CONTAINMENT SYSTEM - (TI0)

4.18.1 Purpose

The function of the containment system is to limit the fission product release from the core to the environment and to provide a leak tight vessel enclosing the Reactor Pressure Vessel (RPV) and associated equipment.

4.18.2 Design Assumptions

The following assumptions with respect to the design were made:

- (1) It is assumed that vacuum breakers 1, 2, and 3 are supplied with 250 V DC class 1E from DIV. I, DIV. II and DIV. III, respectively.
- (2) It is assumed that manual action in support of closure of the vacuum breakers is possible from the Control Room and two vacuum breakers closure signal propagates through divisions I and II of the ECCS logic of the SSLC system.
- (3) It is assumed that valve B32-F001A receives automatic isolation signals from DIV. I and DIV. II of the ECCS logic of the SSLC and that the solenoid actuated by DIV. I receives 208/120 V AC uninterruptible power supply DIV. I, class 1E, and the solenoid actuated by DIV. II receives 208/120 V AC uninterruptible power supply DIV. II, class 1E. If automatic signals fail, it is assumed that it is possible for the crew to close the valve remotely from the Control Room.
- (4) It is assumed that valve B32-F002A receives automatic isolation signals from DIV I of the ECCS logic or the SSLC and that its motor is fed by 480 V AC uninterruptible power supply DIV. I, class 1E. If automatic signals fail, it is assumed that it is possible for the crew to close the valve remotely from the Control Room.
- (5) It is assumed that valve G31-F005A receives automatic isolation signals from DIV. I of the ECCS logic of the SSLC and that the electrical supply to the valve motor is 480 V AC uninterruptible power supply DIV. I, class 1E. If automatic signals fail, it is assumed that it is possible for the crew to close the valve remotely from the Control Room.
- (6) It is assumed that valve G31-F006A receives automatic isolation signals from DIV. II of the ECCS logic of the SSLC. If automatic signals fail, it is assumed that it is possible for the crew to close the valve remotely from the Control Room.
- (7) It is assumed that valves B21-F001A / B / C / D receive automatic isolation signals from DIV. I of the RTIF-scrum logic of the SSLC. If automatic signals fail, it is assumed that it is possible for the crew to close the valves remotely from the Control Room.
- (8) It is assumed that valves B21-F002A / B / C / D receive automatic isolation signals from DIV. II of the RTIF-scrum logic of the SSLC. If automatic signals fail, it is assumed that it is possible for the crew to close the valves remotely from the Control Room.
- (9) It is assumed that the logic channels DIV. I, II, III and IV of the RTIF logic of the SSLC are fed from 208/120 V AC uninterruptible power supply DIV. I, II, III and IV class 1E, respectively.

- (10) It is assumed that there is a third valve (G31-F3A or B) in the suction line of each train of the RWCU system. These valves are located, respectively, downstream of the valves G31-F005/7A or B, depending on the train. They are air operated valves that are normally open that fail closed. These valves isolate the line automatically due to the same isolation signals that actuate the valves G31-F005/6/7A or B in the same train. These automatic signals are transmitted by the NE-RTNSS logic sub-system and can be manually overridden.

4.18.3 System Description

The Containment System is comprised of the drywell (DW), suppression pool (SP), wetwell (WW) gas space, and the vent system. The vent system and the Passive Cooling Containment Cooling System (PCCS) vent lines establish flow paths from the drywell to the suppression pool.

In addition, the wetwell-to-drywell vacuum breakers establish a flow path from the wetwell gas space to the drywell for the purpose of limiting the maximum differential pressure that could exist between the wetwell gas space and the drywell. The containment is inerted with nitrogen during normal plant operation.

The containment vessel is a reinforced concrete vessel that has an internal steel liner providing the leak-tight containment boundary.

The Containment System analysis described here refers to the system configuration shown in the diagram of Figure 4.18-1.

4.18.3.1 Hardware Configuration

The containment system mainly consists of the following components:

Drywell (DW)

The DW is made up of two volumes, the upper DW and the lower DW, which are geometrically separated by the Reactor Pressure Vessel (RPV) support skirt and pedestal. The two DW volumes are connected by open flow paths between the upper and lower drywell regions through the RPV support skirt.

Suppression Chamber (SC)

The SC, also referred to as the wetwell, is made up of a water volume (suppression pool) and a gas volume above the pool.

The suppression pool is a large body of water that absorbs energy by condensing steam from Safety Relief Valves (SRV) discharges and postulated pipe breaks. The suppression pool also serves as a source of reactor makeup water. The SP also serves to retain radioactive materials routed to it with the blowdown flow.

DW/SC Connecting Vents

The SC is connected to the DW by a vent system, which includes twelve vent modules. These vent modules are built into the vertical cylindrical wall, which separates the upper DW from the SC.

Spillover Pipes

The ESBWR spillover function provides a DW to SC connection for transferring water from the DW to the suppression pool. Spillover is accomplished by ten pipes, which are built into the vent wall. If water ascending through the DW annulus following a postulated LOCA reaches the entrance of the spillover pipes, it flows to the suppression pool. These lines are fitted with valves that open at the same time as the equalizing lines.

Containment Isolation System

The Containment Isolation System provides protection against release of radioactive materials to the environment as a result of accidents occurring in systems or components within the containment. Protection is provided by isolation of lines and ducts that penetrate the containment boundary.

The Containment Isolation System consists of the valves that are required to close and the portion of the instrumentation, from sensor to the actuation device that generates containment isolation signals (isolation logic) during an accident in which containment integrity is necessary. In the analysis, only the isolation of LOCAs outside containment in IC pipes, RWCU pipes, main steam (MS) pipes, or Feedwater pipes are justified.

Vacuum Relief

The ESBWR design provides a vacuum relief function to limit the magnitude of a negative pressure differential between the DW and the SC. The vacuum relief function is accomplished by three DW/SC vacuum breakers installed in the diaphragm floor. These vacuum breakers operate passively in response to a negative DW-to-SC to pressure gradient and is otherwise held closed by combination of gravity and the normally positive DW-to-SC pressure gradient. Each vacuum breaker is equipped with a DC motor-operated valve which provides isolation capability if the vacuum breaker sticks open or leaks in its closed position. Four position sensors are located around the disk periphery to provide the plant operator with confirmation that the disk is securely seated.

Vent

At the outer wall of the containment structural boundary there is a pipeline forming part of the Containment Inerting System normally isolated capable of venting the suppression pool air volume to the Reactor Building HVAC if required. The bleed line is normally closed by air operated valves that are only opened by operator action under emergency procedures to vent overpressure if required.

4.18.3.2 System Operation

The containment is inerted prior to reactor heatup for startup from an external source before hydrotest. During shutdown and plant outages, pre-entry purge is provided with the establishment of clean breathable atmosphere throughout the containment. The Containment Inerting System (CIS) (T31) provides the nitrogen inerting prior to plant startup and the nitrogen pre-entry purging with breathable atmosphere during shutdowns and plant outages.

During plant operation nitrogen makeup is also provided by CIS. The CIS maintains the drywell and wetwell pressures slightly above the reactor building pressure to ensure that there is no air leakage from reactor building to the containment. The differential pressure setpoint is selected

such that a positive containment differential pressure can be maintained without excessive wasteful inerting with nitrogen. Following a postulated LOCA, high-energy fluids from the RPV pressurize the DW. In the early portion of the post –LOCA transient, the DW pressure is relieved by flow of the non-condensibles and steam through the main vent system to the suppression pool, where the steam is condensed. In the long term, steam generated by decay power flows to the PCCS where its latent heat is transferred via PCCS or IC Heat Exchangers to the IC/PCC pool.

In case of a LOCA outside containment, the Containment Isolation System actuates closing the open lines that penetrate the containment boundary.

4.18.3.3 Components Location

The Containment System components are located within the reactor building, drywell and the steam tunnel.

4.18.4 Automatic and Manual Control

In case of a LOCA outside containment automatic actuation of containment isolation valves is initiated. The actuation signals considered in the isolation of these LOCAs are listed by type of LOCA as follows:

IC pipe breaks

- High radiation in the IC pool
- High flow (differential pressure) in steam supply line
- High flow (differential pressure) in condensate return line

Main Steam line breaks

- Reactor water level 2
- Main steam line high flow rate
- Turbine inlet low pressure

Feedwater line breaks

No automatic actuation is required for feedwater isolation valves (check valves).

RWCU line breaks

- Reactor water level 2
- High flow rate in the RWCU train
- High temperature in the steam tunnel

A remote manual (from the control room) actuation is provided for all isolation valves with automatic closure. Neither automatic nor manual actuation is provided for the self-actuated isolation valves on main feedwater lines.

4.18.5 System Interfaces

The Containment System dependencies are included in the dependency matrix given in Table 4.18-2.

4.18.6 System Testing

Closure of the MSIVs is tested monthly. The isolation function of the check valves located in the feedwater lines is tested during each refueling outage.

RWCU containment isolation valves G31-F005A/B and G31-F006A/B are closed each month when the train is changed.

Closure of the containment isolation valves is done quarterly.

IC steam isolation valves B32-F001A/B/C/D and B32-F002A/B/C/D are tested every three months.

The tests performed during power operation on components included in the fault trees do not affect their availabilities relative to the modeled functions.

4.18.7 System Maintenance

The performance of maintenance on the system components located inside the containment during power operation is not assumed in this model, because the line is isolated during the maintenance.

Maintenance performed on the containment isolation valves during power operation does not make them unavailable to carryout their penetration isolation functions.

4.18.8 Common-Cause Failures

The common cause failures have been grouped in Table 4.18-5.

4.18.9 Fault Tree Analysis

Various fault trees have been developed to allow the determination of the system unavailability and to identify components that contribute significantly to system unavailability of the different containment system functions.

The following systems functions are required:

- Isolation of vacuum breaker leaks
- Steam suppression in the pool after a significant RPV depressurization. This includes the need for at least 2 of 3 vacuum breakers to be closed for the steam suppression function to be successful and at least one vacuum breaker able to open after steam suppression to avoid containment failure due to depressurization.
- Isolation of IC pipe breaks outside containment.
- Isolation of MSL breaks outside containment.
- Isolation of Feedwater pipe breaks outside containment
- Isolation of RWCU pipe breaks outside containment

4.18.9.1 Top Event Definitions

The corresponding fault trees have been developed for the above-mentioned functions. Their TOPS are indicated below:

- Gate GT10-0001-_1 represents the failure to isolate vacuum breaker leaks.
- Gate GT10-0001-_2 represents the failure of the steam suppression in the pool after a significant RPV depressurization.
- Gate GT10-0001-_4 represents the failure to isolate IC pipe breaks outside containment.
- Gate GT10-0001-_5 represents the failure to isolate main Feedwater line A breaks outside containment.
- Gate GT10-0001-_6 represents the failure to isolate main Feedwater line B breaks outside containment.
- Gate GT10-0001-_7 represents the failure to isolate RWCU pipe breaks outside containment in the operating train.
- Gate GT10-0001-_9 represents the failure to isolate main steam pipe breaks outside containment.

4.18.9.2 Model Assumptions

- System boundary: For purposes of the PRA, the Containment System includes the drywell, the suppression chamber, the DW/SC connecting vents, the spillover pipes, the containment isolation system, the vacuum relief valves, the containment overpressure protection system, the containment isolation valves B32-F001A, B32-F002A, B21-F001A/B/C/D, B21-F002A/B/C/D, B21-F102A/B, B21-F103A/B, G31-F005A, G31-F006A and the sensor and logic channels (RTIF) related to the automatic and manual actuations of these components.
- RWCU Train A is the train operating during power operation and that Train B is isolated from the RPV.
- The nitrogen or air stored in their corresponding accumulators is sufficient to close the air-operated containment isolation valves, and therefore, that supply from the High Pressure Nitrogen (P54) or Instrument Air (P52) systems is not necessary.
- In the event of a LOCA outside containment due to ruptures in the Isolation Condenser System, only the failure upon isolation of loop A of the system has been modeled. The behavior of loops B, C and D is similar.

4.18.9.3 Human actions

The human actions considered in the fault trees are collected in Table 4.18-6

T10-XHE-FO-CLOSE represents the operator's failure to manually close the vacuum breakers.

T10-XHE-FO-CLOSEIVS represents the operator's failure to manually close the isolation valves in case of a LOCA outside containment when automatic actuation fails.

Human actions in the fault trees are connected by the house-event XHOSMAN that represents operator unavailability to carry out manual actions.

4.18.10 Results of Fault Tree Analysis

The fault tree results provide quantitative values of system unavailability and of the importance of specific components to that total.

The quantification of core damage sequences implicitly includes the contribution of basic event for this system. This justification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

In addition, the importance measurements obtained from core damage frequency equations, allow identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.18.11 Model Differences for 72-Hour Operation

The long-term Containment System operation is similar to that during the first 24 hours following a transient or LOCA. There are no fault tree model changes necessary to extend the operation to 72 hours.

4.18.12 References

4.18-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.18-1
Control Room Instrumentation and Alarms

| Main Control Room Panel Controls | |
|--|--|
| Vacuum breakers closure | |
| B32-F001A / B / C / D open / close | |
| B32-F002A / B / C / D open / close | |
| G31-F3A / B open / close | |
| G31-F005A / B open / close | |
| G31-F006A / B open / close | |
| B21- F001A / B / C / D open / close | |
| B21- F002A / B / C / D open / close | |
| Main Control Console Displays | |
| DW temperature | |
| DW pressure | |
| Differential pressure DW to SC and DW to outside containment | |
| DW oxygen and hydrogen concentrations | |
| DW radiation level | |
| SC gas space temperature | |
| SC gas space pressure | |
| Differential pressure SC to outside containment | |
| SC oxygen and hydrogen concentrations | |
| SC radiation level | |
| Suppression pool temperature | |
| Suppression pool level | |
| DW water level | |
| Vacuum breakers SC to DW open/close position indication | |
| B32-F001A / B / C / D position open / close | |
| B32-F002A / B / C / D position open / close | |
| G31-F005A / B position open / close | |
| G31-F006A / B position open / close | |
| B21- F001A / B / C / D position open / close | |
| B21- F002A / B / C / D position open / close | |

| Main Control Room Alarms | |
|---|--|
| DW temperature high | |
| DW pressure high | |
| DW oxygen and hydrogen concentrations high | |
| DW radiation level high | |
| SC gas space temperature high | |
| SC gas space pressure high | |
| Differential pressure SC to outside containment | |
| SC oxygen and hydrogen concentrations high | |
| SC radiation level high | |
| Suppression pool temperature high | |
| Suppression pool level low | |

Table 4.18-2
System Dependencies Matrix

| COMPONENT | TRANSMISSION SIGNAL | R16 | R13 (Class 1E) | R13 (No Class 1E) |
|------------------|--------------------------------|------------|---------------------------|------------------------------|
| VB-1 | C74 (DIV I AND II) | DIV 1 | | |
| VB-2 | C74 (DIV I AND II) | DIV 2 | | |
| VB-3 | C74 (DIV I AND II) | DIV 3 | | |
| B32-F001A | C74 (DIV I AND II) | | | |
| B32-F002A | C74 (DIV I) | | DIV 1 | |
| B32-N001A | | | DIV 1 | |
| B32-N002A | | | DIV 2 | |
| B32-N003A | | | DIV 3 | |
| B32-N004A | | | DIV 4 | |
| D11-N001A | | | DIV 1 | |
| D11-N002A | | | DIV 2 | |
| D11-N003A | | | DIV 3 | |
| D11-N004A | | | DIV 4 | |
| G31-F005A | | | DIV 1 | |
| G31-F3A | C62(DIV A) | | | DIV. A |

Table 4.18-3
Component Tests

See Section 4.18.6

Table 4.18-4
Component Maintenance

N/A

Table 4.18-5
Common Cause Failure

| Basic Event | Probability | Description |
|-----------------------|-------------|--|
| B21-ACV-CF-MSIVCLOSE | 1.45E-05 | CCF OF AOV (MSIV) TO CLOSE |
| B21-FT_-CF-MSLA | 3.00E-08 | CCF 3/4 FLOW TRANSMITTERS MSL "A" |
| B21-FT_-CF-MSLB | 3.00E-08 | CCF 3/4 FLOW TRANSMITTERS MSL "B" |
| B21-FT_-CF-MSLC | 3.00E-08 | CCF 3/4 FLOW TRANSMITTERS MSL "C" |
| B21-FT_-CF-MSLD | 3.00E-08 | CCF 3/4 FLOW TRANSMITTERS MSL "D" |
| B21-LT_-CF-N001ABCD | 1.20E-07 | CCF OF DIVERSIFIED LEVEL 1& 2 TRANSM. 1A/B/C/D |
| B21-PT_-CF-MSL | 3.00E-08 | CCF 3/4 PRESSURE TRANSMITTERS MSL |
| B21-UV_-CF-102/3A | 4.00E-04 | CCF OF CHECK VALVES F103A AND F102A TO CLOSE |
| B21-UV_-CF-102/3B | 4.00E-04 | CCF OF CHECK VALVES F103B AND F102B TO CLOSE |
| B32-PDT-CF-ICA | 1.62E-05 | CCF OF PRESSURE DIFF. TRANSMITTERS IN IC-A NO OUTPUT SIGNAL |
| C21-FT_-CF-N001ABCD A | 3.00E-08 | CCF 3/4 FLOW TRANSMITTERS RWCU TRAIN A |
| C21-TT_-CF-N003ABCD | 2.00E-08 | CCF 3/4 STEAM TUNNEL TEMPERATURE TRANSMITTERS |
| C62-DTM-CF-DIDALL | 5.50E-05 | COMMON CAUSE FAILURE 3/4 DTM DID LOGIC |
| C74-DTM-CF-ALL | 1.20E-05 | CCF 3/4 DTM OF SSLC DIV1/2/3/4 |
| C74-DTM-CF-RTIFALL | 1.20E-05 | CCF 3/4 DTM OF SSLC/RTIF DIV1/2/3/4 |
| C74-SLU-CF-OLU | 2.70E-06 | CCF 3/4 OLU IN SSLC/RTIF DIV 1/2/3/4 |
| C74-SLU-CF-TLU | 2.70E-06 | CCF 3/4 TLU IN SSLC/RTIF DIV 1/2/3/4 |
| C74-VLU-CF-LD&IS | 3.12E-06 | CCF VOTING LOGIC CARDS 2/4 LD&IS |
| D11-ACT-CF-ICA | 9.71E-06 | CCF OF RADIATION MONITORS IN IC-A POOL (NO OUTPUT) |
| T10-VB_-CF-CLOSED | 2.00E-06 | CCF 2 OF 3 VB FAIL TO REMAIN CLOSED |
| T10-VB_-CF-OPEN | 1.00E-06 | CCF 3 OF 3 VB FAIL TO OPEN |

Table 4.18-6
Human Errors

| Basic Event | Probability | Description |
|---------------------|-------------|---|
| T10-XHE-FO-CLOSE | 1.77E-02 | Operator Fails To Close Vacuum Breaker |
| T10-XHE-FO-CLOSEIVS | 1.77E-02 | Operator Fails To Manually Close Isolation Valves |

Table 4.18-7**List of System Basic Events**

| Basic Event | Probability | Description |
|---------------------|--------------------|---|
| C74-SLU-FC-TLURTIF2 | 9.00E-04 | TLU OF SSLC/RTIF DIV 2 FAILS |
| C74-SLU-FC-TLURTIF3 | 9.00E-04 | TLU OF SSLC/RTIF DIV 3 FAILS |
| C74-SLU-FC-TLURTIF4 | 9.00E-04 | TLU OF SSLC/RTIF DIV 4 FAILS |
| C74-VLU-FC-LD&IS1 | 6.24E-04 | 2/4 VOTING LOGIC CARD LD&IS DIV.1 FAILURE |
| C74-VLU-FC-LD&IS2 | 6.24E-04 | 2/4 VOTING LOGIC CARD LD&IS DIV.2 FAILURE |
| D11-ACT-NO-1A | 1.94E-03 | RADIATION MONITOR 1A ON IC A FAILS TO RESPOND TO CHANGE |
| D11-ACT-NO-2A | 1.94E-03 | RADIATION MONITOR 2A ON IC A FAILS TO RESPOND TO CHANGE |
| D11-ACT-NO-3A | 1.94E-03 | RADIATION MONITOR 3A ON IC A FAILS TO RESPOND TO CHANGE |
| D11-ACT-NO-4A | 1.94E-03 | RADIATION MONITOR 4A ON IC A FAILS TO RESPOND TO CHANGE |
| H23-RMU-FC-IICA1 | 3.00E-04 | RMU 001 FROM C.R. FAILS |
| H23-RMU-FC-IICA2 | 3.00E-04 | RMU 001 FROM C.R. FAILS |
| H23-RMU-FC-1VB | 3.00E-04 | RMU 001 FOR T10 C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-2ICA1 | 3.00E-04 | RMU 002 FROM C.R. FAILS |
| H23-RMU-FC-2ICA2 | 3.00E-04 | RMU 002 FROM C.R. FAILS |
| H23-RMU-FC-2VB | 3.00E-04 | RMU 002 FOR T10 C.R. MANUAL ACTUAT. FAILS TO OPERATE |
| H23-RMU-FC-ACT1A | 3.00E-04 | LMU 10 A FAILURE |
| H23-RMU-FC-ACT2A | 3.00E-04 | LMU 10B FAILURE |
| H23-RMU-FC-ACT3A | 3.00E-04 | LMU 10C FAILURE |
| H23-RMU-FC-PDT1A | 3.00E-04 | LMU 6A FAILURE |
| H23-RMU-FC-PDT2A | 3.00E-04 | LMU 6B FAILURE |
| H23-RMU-FC-PDT3A | 3.00E-04 | LMU 6C FAILURE |
| H23-RMU-FC-PDT4A | 3.00E-04 | LMU 6D FAILURE |
| T10-VB_-LK-VB1 | 1.00E-04 | PROBABILITY OF LEAK IN VACUUM BREAKER 1 |
| T10-VB_-LK-VB2 | 1.00E-04 | PROBABILITY OF LEAK IN VACUUM BREAKER 2 |
| T10-VB_-LK-VB3 | 1.00E-04 | PROBABILITY OF LEAK IN VACUUM BREAKER 3 |
| T10-VB_-OO-VB1 | 1.00E-04 | VACUUM BREAKER 1 FAILS TO CLOSE |

Table 4.18-7
List of System Basic Events

| Basic Event | Probability | Description |
|------------------|-------------|--|
| T10-VB_-OO-VB2 | 1.00E-04 | VACUUM BREAKER 2 FAILS TO CLOSE |
| T10-VB_-OO-VB3 | 1.00E-04 | VACUUM BREAKER 3 FAILS TO CLOSE |
| B21-ACV-OO-F001A | 2.00E-03 | NOV F001A FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F001B | 2.00E-03 | NOV F001B FAILS TO OPEARTE TO DEENER. POSITION. |
| B21-ACV-OO-F001C | 2.00E-03 | NOV F001C FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F001D | 2.00E-03 | NOV F001D FAILS TO OPERATE TO DEENER. POSITION. |
| B21-ACV-OO-F002A | 2.00E-03 | AOV F002A FAILS TO OPERATE TO DEENER. POSTION. |
| B21-ACV-OO-F002B | 2.00E-03 | AOV F002B FAILS TO OPERATE. TO DENER. POSTION. |
| B21-ACV-OO-F002C | 2.00E-03 | AOV F002C FAILS TO OPERATE TO DEENER. POSTION. |
| B21-ACV-OO-F002D | 2.00E-03 | AOV F002D FAILS TO OPERATE TO DEENER. POSTION. |
| B21-UV_-OO-102B | 7.96E-03 | CHECK VALVE F102B FAILS TO CLOSE |
| B21-UV_-OO-103B | 7.96E-03 | CHECK VALVE F103B FAILS TO CLOSE |
| B21-UV_-OO-F102A | 7.96E-03 | CHECK VALVE F102A FAILS TO CLOSE |
| B21-UV_-OO-F103A | 7.96E-03 | CHECK VALVE F103A FAILS TO CLOSE |
| B32-ACV-OO-F001A | 2.00E-03 | MSIV F001A VALVE FAILS TO CLOSE |
| B32-MOV-OO-F002A | 4.00E-03 | SOLID STATE LOAD DRIVER LD002 FAILS TO CLOSE |
| B32-PDT-NO-N001A | 3.23E-03 | PRESSURE TRANSMITT. 1A IC "A" FAILS TO RESPOND TO CHANGE |
| B32-PDT-NO-N002A | 3.23E-03 | PRESSURE TRANSMITT. 2A IC "A" FAILS TO RESPOND TO CHANGE |
| B32-PDT-NO-N003A | 3.23E-03 | PRESSURE TRANSMITT. 3A IC "A" FAILS TO RESPOND TO CHANGE |
| B32-PDT-NO-N004A | 3.23E-03 | PRESSURE TRANSMITT. 4A IC "A" FAILS TO RESPOND TO CHANGE |
| C74-DTM-FC-DIV1 | 6.00E-04 | DTM OF SSLC DIV 1 FAILS TO TRIP |
| C74-DTM-FC-DIV2 | 6.00E-04 | DTM OF SSLC DIV. 2 FAILS TO TRIP |
| C74-DTM-FC-DIV3 | 6.00E-04 | DTM OF SSLC DIV. 3 FAILS TO TRIP |
| C74-DTM-FC-DIV4 | 6.00E-04 | DTM OF SSLC DIV. 4 FAILS TO TRIP |

Table 4.18-7**List of System Basic Events**

| Basic Event | Probability | Description |
|---------------------|--------------------|------------------------------|
| C74-SLU-FC-LMU10D | 9.00E-04 | LMU 10 D FAILURE |
| C74-SLU-FC-OLURTIF1 | 9.00E-04 | OLU OF SSLC/RTIF DIV 1 FAILS |
| C74-SLU-FC-OLURTIF2 | 9.00E-04 | OLU OF SSLC/RTIF DIV 2 FAILS |
| C74-SLU-FC-OLURTIF3 | 9.00E-04 | OLU OF SSLC/RTIF DIV 3 FAILS |
| C74-SLU-FC-OLURTIF4 | 9.00E-04 | OLU OF SSLC/RTIF DIV 4 FAILS |
| C74-SLU-FC-TLURTIF1 | 9.00E-04 | TLU OF SSLC/RTIF DIV 1 FAILS |

Table 4.18-8
System Top Events

| TOP EVENT | Description |
|------------------|--|
| GT10-0001-_1 | ISOLATION OF VB LEAKS FAILS |
| GT10-0001-_2 | STEAM SUPPRESSION FUNCTION FAILURE |
| GT10-0001-_4 | ICS ISOLATION FAILURE LOCA OUTSIDE CONTAINMENT |
| GT10-0001-_5 | MAIN FEEDWATER LINE A ISOLATION FAILURE |
| GT10-0001-_6 | MAIN FEEDWATER LINE B ISOLATION FAILURE |
| GT10-0001-_7 | RWCU/SDC ISOLATION FAILURE LOCA IN TRAIN A OUTSIDE CONT. |
| GT10-0001-_9 | MSL ISOLATION FAILURE LOCA OUTSIDE CONTAINMENT |

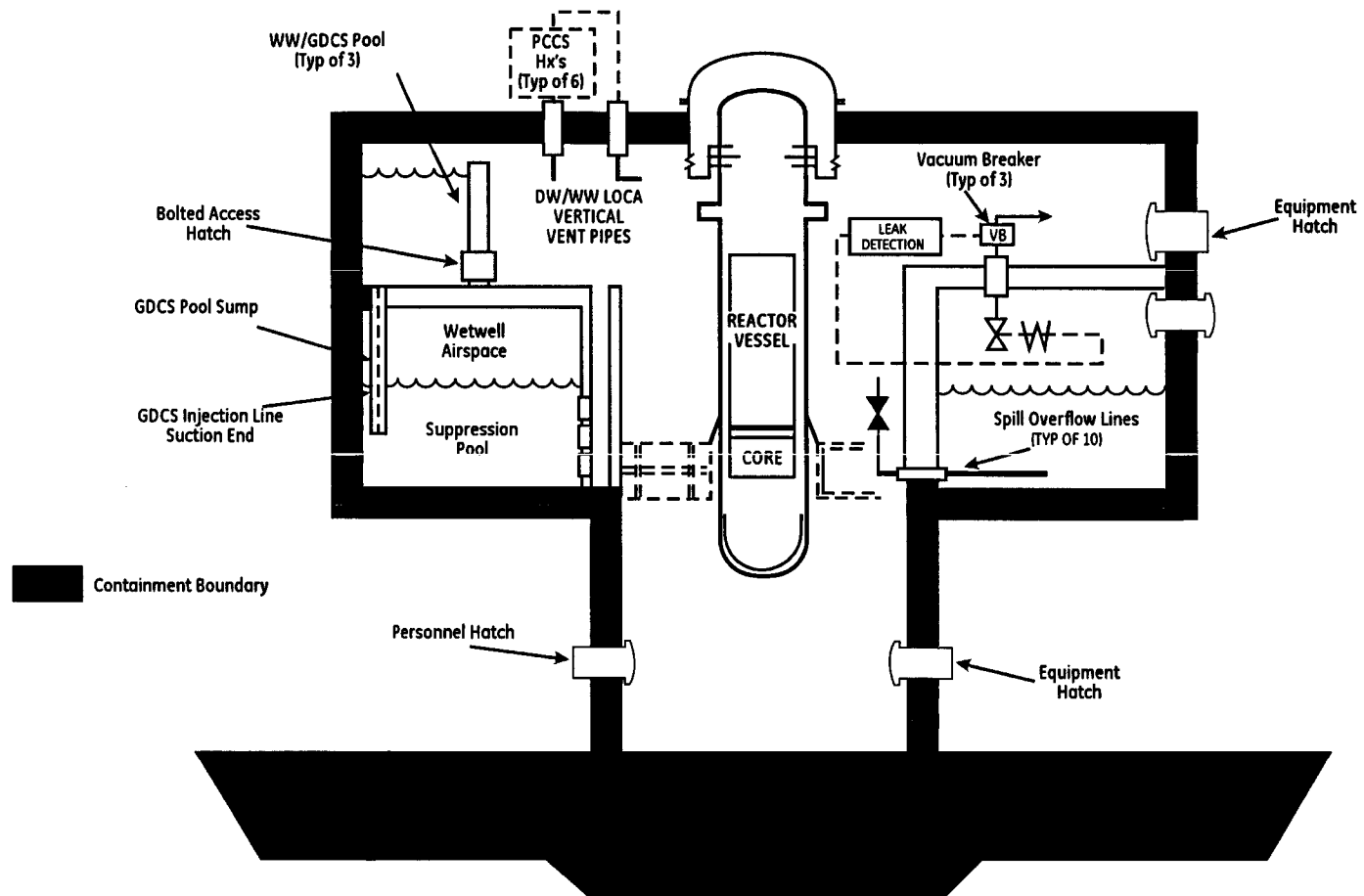
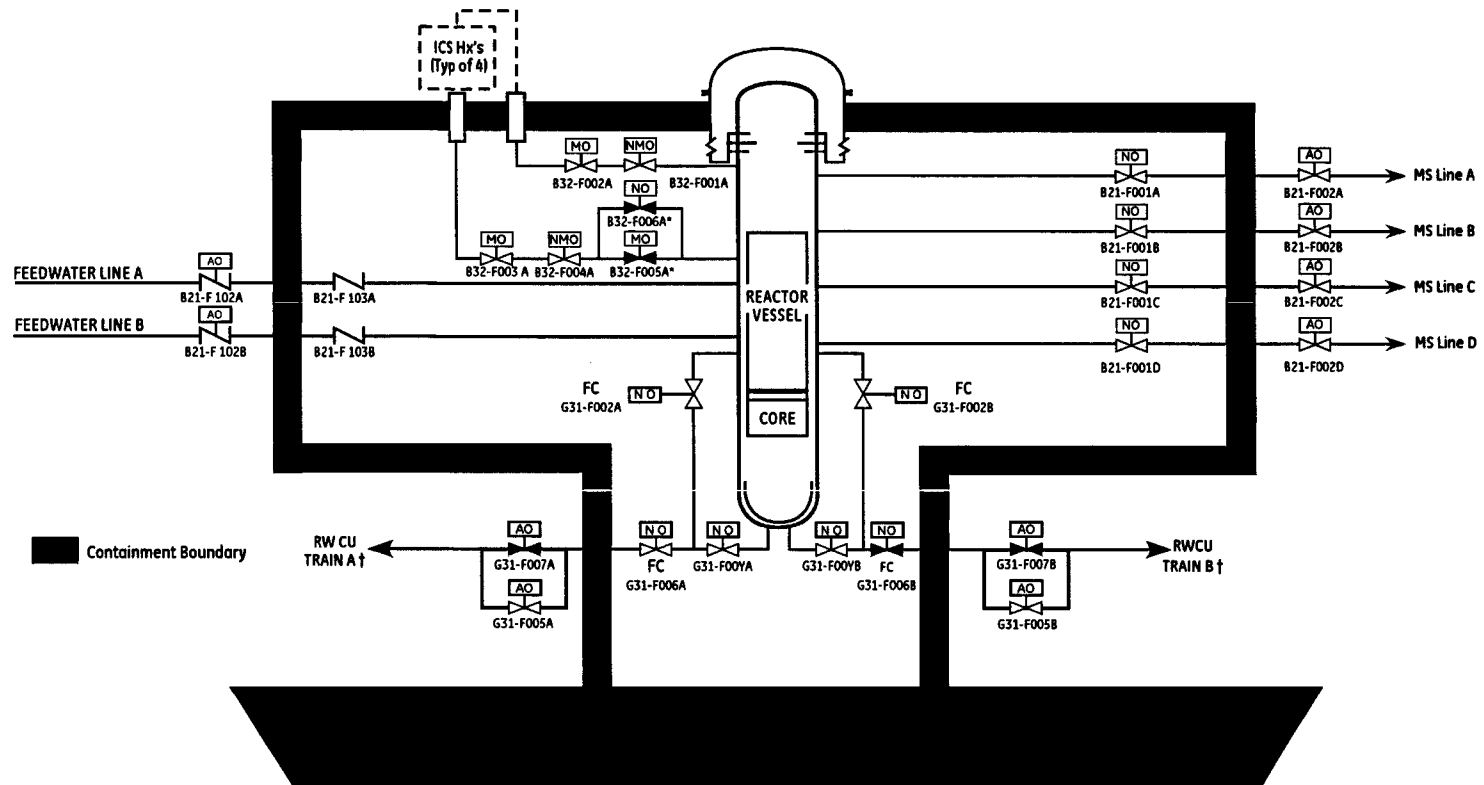


Figure 4.18-1 (1/2). Containment System Simplified Diagram



* Not containment isolation valves. Shown for clarity.

† Upper and lower RWCU paths actually have separate penetrations. For PRA purposes the model is functionally similar.

Figure 4.18-1 (2/2). Containment System Simplified Flow Diagram

4.19 PASSIVE CONTAINMENT COOLING SYSTEM - (T15)

4.19.1 Purpose

The PCCS is a safety-related passive-acting containment heat removal system that maintains the containment within its design pressure and design temperature limits for design basis accidents (DBAs). The PCCS consists of six (6) independent loops, each loop arranged as a closed-loop configuration containing a heat exchanger (PCCS condenser), supply piping to bring containment steam-gas mixture to the loop heat exchanger, discharge piping to return condensed steam back into the containment, and vent piping to recycle non-condensable gas back inside the containment. The condensed steam is drained to the Gravity Driven Cooling System (GDCS) pool and the gas is vented through a line that is submerged in the suppression pool. The heat exchangers are positioned in the lower-most region of a large, elevated, common, ambient pressure pool - Isolation Condenser/Passive Containment Cooling (IC/PCC) Pool - that warms and ultimately boils-down, with exhaust vapor discharged to the plant stack as containment heat removal proceeds following a LOCA. The heat exchangers are open-type drum-and-tube-bundle exchangers, configured into a dual-module arrangement and operated by a combination of buoyancy, pressure and condensation-driven natural circulation processes.

A schematic diagram of PCCS is shown in Figure 4.19-1. The PCCS is classified as an Engineered Safety Feature (ESF). PCCS piping conveys containment steam/non-condensable (nitrogen) mixture to the upper drums of each HX module. Heat is moved to the IC/PCC pool by condensing the steam component inside the vertical tubes. As bulk pool water warms and eventually boils, secondary (non-radioactive) steam is vented through moisture separators to the surroundings.

Condensate drains from the tubes into the HX module lower drums, where it is collected and drained to the GDCS pool. Loop seals in these drain lines insure that PCCS inflow is via the steam supply line to the top of the HXs. Non-condensable gas along with a small proportion of water vapor is piped to a discharge point within the SP and provides continuous purging of any non-condensable gases accumulating within the HX tube bundle regions.

This system design needs no isolation valves, as the HX modules and piping, as extensions of the safety-related containment, are designed for pressures of approximately twice the containment design pressure, and are designed to match the containment design temperature. No actuation valves are needed, since both condensate and non-condensable vent points discharge into containment spaces/volumes, and thus, the system is always available (i.e., in "ready standby" condition). Since it does not have any valves, the system starts into operation immediately following a LOCA event and limits containment pressure to less than its design pressure for at least 72 hours after a LOCA without operator action. The PCCS loops are initially driven by the pressure difference created between the containment drywell and the suppression pool during a LOCA and then by gravity drainage of steam condensed in the tubes, so they require no sensing, control, logic or power-actuated devices to function. In general, PCCS operation is characterized by steam and non-condensable gases entering the HX, condensation of the steam with drain flow to the GDCS pool, and collection of the non-condensable gases in the lower portion of the HX tubes and lower headers with periodic, naturally-occurring venting to the WW. In the long term, when virtually all non-condensable gas has been purged to the WW, the operation of the HX is

characterized by an interface within the tubes with almost pure steam above the interface and a rich mixture of non-condensable gases and steam below the interface. The interface gradually moves upward to compensate for the decreasing decay heat generation rate.

4.19.2 Design Assumptions

The following assumptions with respect to the design were made:

- (1) Water in the IC/PCC pools provides the necessary cooling for greater than 24 hours.
- (2) Connections with a makeup or cooling system are also provided as a back up to ensure cooling for up to 72 hours.

4.19.3 System Description

The system is assumed to operate as described in Reference 4.19-1.

4.19.3.1 Hardware Configuration

Passive Containment Cooling System (PCCS)

The PCCS consists of six (6), low pressure, independent loops, each containing a condenser located in the same large, interconnected IC/PCC pools as the isolation condensers. A simplified diagram is shown in Figure 4.19-1.

The PCCS operates by natural recirculation. Each PCCS loop has no valves; therefore, the system is always in "ready standby."

Each PCCS condenser is open to the primary containment and receives a steam-gas mixture directly from the drywell. The condensate is drained to the Gravity Driven Cooling System (GDSC) pool and the gas is vented through a discharge line, which is submerged in the pressure suppression pool. The water is returned directly to the RPV through the opened GDSC injection lines.

IC/PCC Pool

The isolation condensers and the passive containment cooling condensers are located in the same large interconnected IC/PCC water pools positioned outside the containment and above the drywell. The large IC/PCC pools are partitioned, but both the IC and PCC are able to draw water from the entire pool; the air and steam space are also common. See Figure 4.19-2.

The pool subcompartment interconnections are as follows: The PCCS pool subcompartments are connected to the outer pool below the water level by locked open valves, one for each subcompartment. These valves can be closed to isolate and drain the individual partitioned PCC pool for maintenance of the unit. The remote handwheels on the locked open valves extend above the water level to locations accessible to the operator. All other pool subcompartments are interconnected below the pool water level. Cross connects between the two outer pools are via normally closed valves.

The walls containing the airspace flow path extend above the normal water level; this enhances the flow stability and heat removal capability of the condensers by establishing a flow path for the make-up water through the lower pipes.

4.19.3.2 System Operation

During normal plant operation, each PCCS subloop is in "ready standby". The design basis analysis shows that the PCCS can maintain containment below the design pressure for 39 hours without opening the connection to the reactor well. The design requirement is that this time is greater than 24 hours.

Following a LOCA event, the PCCS receives a steam-gas supply directly from the drywell; it does not have any valves. Noncondensibles, together with steam vapor, enter the PCCS condenser where steam condenses inside vertical tubes that allow the condensate collected in the lower header to be discharged later to the GDCS pools. The noncondensibles are then purged to the wetwell through the large vent line.

PCCS effectiveness in containment heat removal requires that a small pressure differential exist between the drywell and wetwell. To this end, the vacuum breakers and wetwells between the DW and WW must be leak tight to maintain this small DW to WW pressure differential.

4.19.3.3 Component Location

Both the ICS and PCCS condenser are located in multiple, large, interconnected pools positioned above, and outside the ESBWR primary containment (drywell).

The large pool is partitioned between the ICS and PCCS condensers, but each is able to draw water from the entire pool; the air and steam space is also open to both.

The PCCS condensers are extensions of the containment boundary.

4.19.4 Automatic and Manual Control

The PCCS does not have instrumentation; Control logic is not needed for successful PCCS.

The automatic valves interconnecting the upper containment pools (Reactor Well and IC/PCC Pools) have a manual backup. Pool water level indication is provided and is used by the operators to indicate when manual initiation of makeup with demineralized water is needed. Manual makeup is thus used to prevent automatic opening of the valves. Make-up water is also available from the Fire Protection System.

4.19.5 Control Room Instrumentation and Alarms

The PCCS is a passive system and does not require instrumentation to initiate its operation. Nevertheless, for the long-term replenishing of the pools, the water level is monitored and alarmed..

A high and low water level alarm, measuring the common pool area is provided as a part of the FAPCS system.

4.19.6 System Interfaces

Support Systems

The system does not require any support system for the modeled function, in the first 24 hours. Water is required for long-term makeup (72h). This appears in the PCCS fault tree model in Appendix B.4.19 as a transfer to the isolation condenser logic.

The Fire Protection System is able to provide makeup water to the IC/PCC pools in the long term and it has been considered in the PCCS fault tree model (transfer to the isolation condenser fault tree).

The Fuel and Auxiliary Pools Cooling System (FAPCS), which provides for the cooling/clean-up of IC/PCC pool water, is a supporting system that does not enter into the fault tree analysis for post accident mitigation.

4.19.7 System Testing

The PCCS System is tested during refueling outages, with an expected interval between tests of 24 months.

4.19.8 System Maintenance

No preventive maintenance actions are expected to be performed during normal plant operation for the PCCS.

Corrective maintenance for tube plugging following the detection of a tube leak is performed during refueling. If there is considerable damage to some component part of the condenser, each module is easily removable after cutting the connecting feed, drain, and vent lines.

The pool water in each isolation condenser or PCC condenser subcompartment is removable without emptying the entire ICS/PCCS pool. The individual partitioned PCC pool is isolated by closing the locked open valve interconnecting it to the other pools and emptying the compartment using a portable pump. This water is discharged into the common pool. To replenish the pool, the normally locked open valve is reopened and the water refills the pool.

4.19.9 Common Cause Failures

Heat exchanger common cause failures within the PCCS are included in the model.

4.19.10 Fault Tree Analysis

The PCCS fault tree is evaluated to determine the system unavailabilities and to identify the components that contribute significantly.

4.19.10.1 Top Event Definitions

The PCCS is analyzed for the following top events:

- Containment Heat Removal Function (Top Event GT15TOP: 3/6 PCCS fail).
- Loss of Pool Water 72 H (Top Event GT15-0032-_1).

Table 4.19-8 reports the list of PCCS top events.

4.19.10.2 Model Assumptions

The success criteria considered is the operation of four (4) of the six (6) heat exchangers. To maintain the cooling function up to 72 hours long-term make-up would be necessary.

Unavailabilities for more than 24 hours due to testing or at power maintenance are not credible.

For purposes of the model, the PCCS includes pool subcompartment interconnections, and excludes pool makeup and water recirculation system. Subcompartment interconnections are normally open and passive and failure is not credible. Failure of water supply up to 72h appears in the model (in Top event GT15-0032-_1) as a transfer to the IC system fault tree, however.

It is necessary for the vacuum breakers to close in order to establish the required differential pressure through the PCCS, and intermittent operation is acceptable. The failure of the vacuum breaker valves of the SP are considered in a separate model (see Section 4.18 of this report).

The system is capable of passively performing its function for at least 24 hours. It is capable of performing its function for longer periods if the IC/PCC pools are refilled. It is assumed that the action of replenishment of inventory is a procedurized action.

The Type C human actions included in the model are included in the Emergency Operating Procedures as well as in other Operating Procedures

4.19.10.3 Fault Tree Description

The fault tree is shown in Appendix B.4.19.

4.19.11 Results of Fault Tree Analysis

The quantification of core damage sequences implicitly includes the contribution of the basic events for this system. This quantification process enables checking the global consistency of the system fault trees and their relationship with the rest of the systems modeled in the PRA.

A summary of the basic events included in the system fault tree are reported in the Table 4.19-7.

The importance measurements obtained from core damage frequency equations allow the identification of the most relevant basic events and system component failures in an integrated context, as well as the determination of their relative importance with respect to the basic events and component failures of the other systems modeled.

4.19.12 Model Differences for 72-Hour Operation

The IC/PCC pools provide sufficient cooling water for 24 hours. To maintain the cooling function beyond that time, it is necessary to interconnect the two outer IC/PCC pools and Reactor Well or to make up the loss of water from the Fire Protection System.

4.19.13 References

4.19-1. ESBWR Design Control Document, 26A6642 Rev. 01

Table 4.19-1
Control Room Instrumentation and Alarms

Note: See 4.19.5

Table 4.19-2
System Dependency Matrix

The system does not require a support system for the modeled function, except for the long-term system when the Fire Protection System or power to the interconnected valves could be necessary. (See 4.19.6)

Table 4.19-3
Component Test

The system is tested during refueling, with an expected interval of 24 months between tests.

Table 4.19-4
Component Maintenance

No preventive maintenance actions are expected to be performed during normal plant operation for the PCCS. See 4.19.8.

Table 4.19-5
List of System Common Cause Failures

| Basic Event | Prob | Description |
|-----------------|----------|------------------------|
| T15-HX_-CF-PLUG | 4.36E-05 | CCF OF THREE PCCS HX'S |

Table 4.19-6
List of System Human Errors

The human errors for pool makeup are included by transfers to the IC system fault tree. See IC discussion, Section 4.2.

Table 4.19-7**List of System Basic Events**

| Basic Event | Probability | Description |
|--------------------|--------------------|--|
| T10-VB_-LK-VBKRS | 8.00E-04 | LEAKAGE OF THE VACUUM BREAKERS IN DW/WW [# 9] |
| T15-HX_-CF-PLUG | 4.36E-05 | CCF OF THREE PCCS HX'S |
| T15-HX_-PG-B001A | 1.90E-02 | HEAT EXCHANGER B001A (PLUGGED) |
| T15-HX_-PG-B001B | 1.90E-02 | HEAT EXCHANGER B001B (PLUGGED) |
| T15-HX_-PG-B001C | 1.90E-02 | HEAT EXCHANGER B001C (PLUGGED) |
| T15-HX_-PG-B001D | 1.90E-02 | HEAT EXCHANGER B001D (PLUGGED) |
| T15-HX_-PG-B001E | 1.90E-02 | HEAT EXCHANGER B001E (PLUGGED) |
| T15-HX_-PG-B001F | 1.90E-02 | HEAT EXCHANGER B001F (PLUGGED) |
| XXX-POL-RP-IC/PCC | 3.00E-07 | LOSS OF IC/PCC POOL WATER DUE TO BREAK DURING ACCIDENT |

Table 4.19-8
List of System Top Events

| Top Event Name | Description |
|-----------------------|---------------------------|
| GT15TOP | 3/6 PCCS FAIL |
| GT15-0032-_1 | LOSS OF POOL WATER (72 H) |

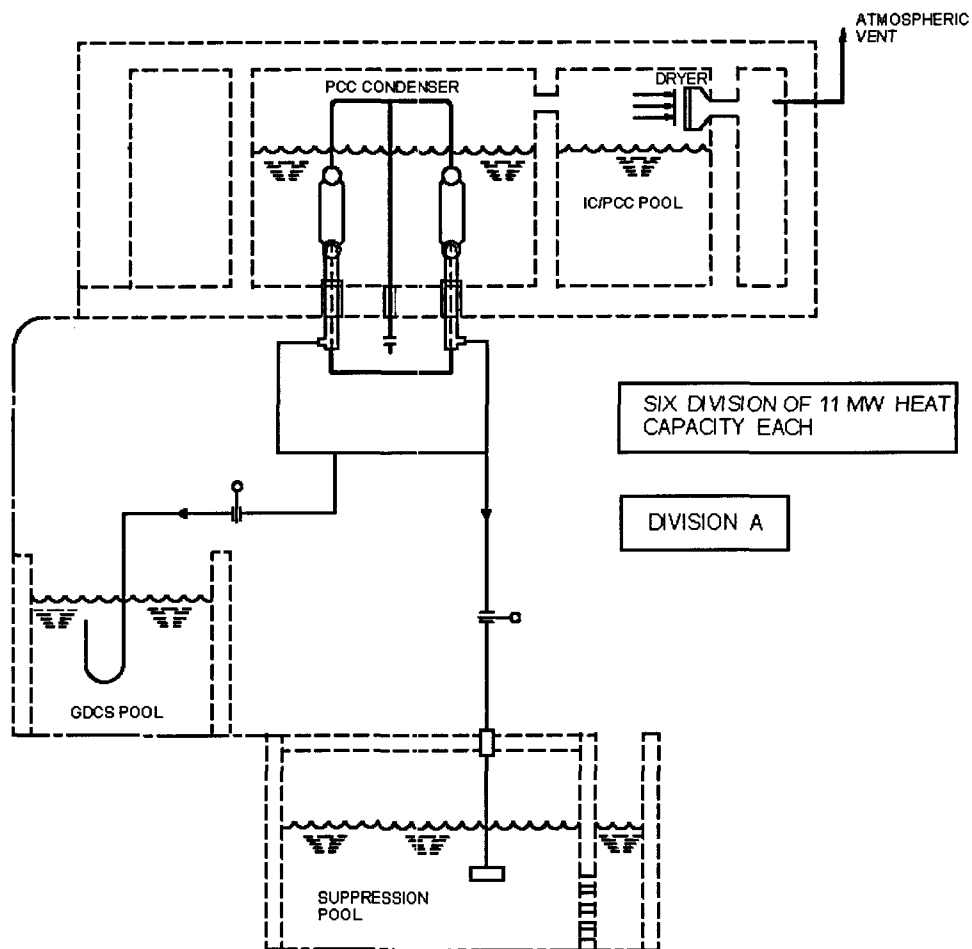


Figure 4.19-1. Simplified Process Flow Diagram for PCCS System

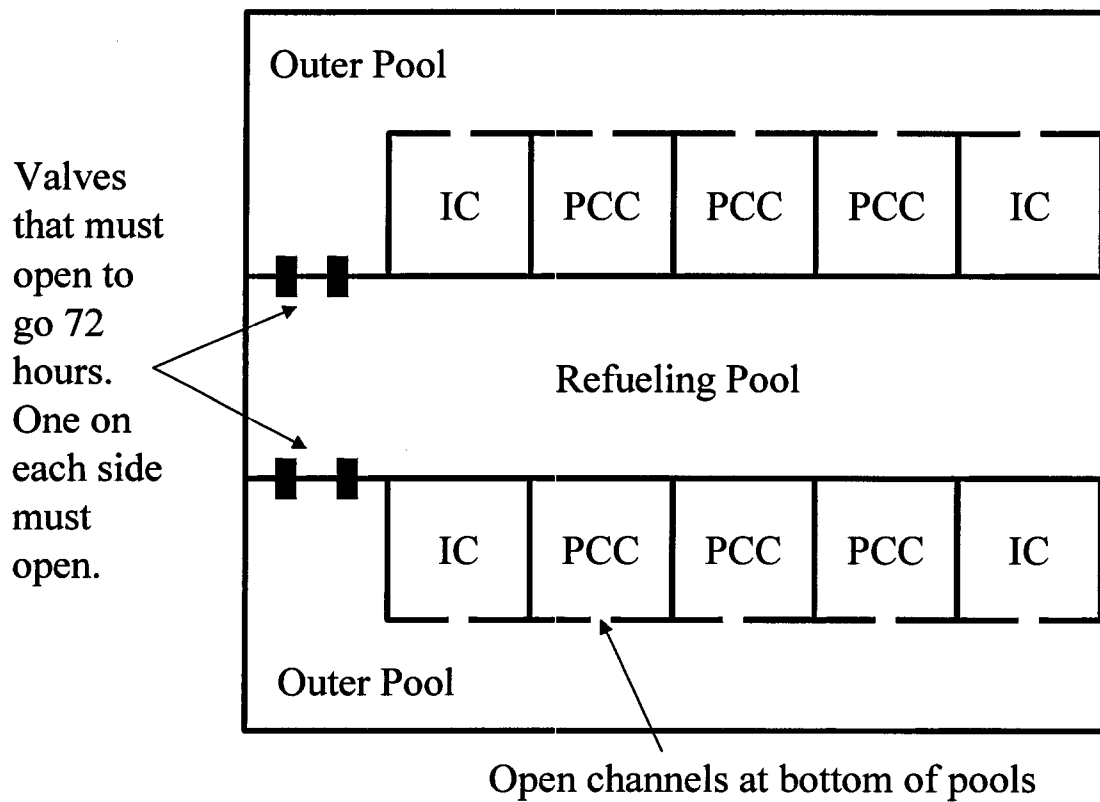


Figure 4.19-2. Schematic of the IC/PCC Pools and Interconnections

Appendix 4.A

Appendix 4.A-1

4.A-1 Initiating Event Naming Convention

Basic events that represent initiating events shall be formatted as follows:

%AAAAAA

Where AAAAA is a descriptive identifier of up to 31 characters.

4.A-2 Basic Event Naming Convention

Basic events that represent individual components shall be named as follows:

XXX-YYY-ZZ-AAAAA

Where the fields are defined as:

| | |
|-------|--|
| XXX | System ID |
| YYY | Component Type Code |
| ZZ | Failure Mode |
| AAAAA | Information to identify the specific component. This should match the identifier on the system sketch, excluding the component code. Use as many digits as necessary up to a total of 21 for this field. The maximum number of characters for the basic event is 32. |

Common cause basic events use a similar convention, except that field AAAAA contains the combination of equipment in the CCF group preceded by the number of components in the group. For example, CCF of 3 of 6 GDSC valves F002A, F002B, and F002D would be:

E50-SQV-CF-6F002ABD

Some systems and recoveries may be modeled as undeveloped events. These are named using the same convention, with the following codes:

| | |
|-------|---|
| XXX | System ID |
| YYY | Use the code "SYS" |
| ZZ | For an undeveloped system, use "FF". For a recovery action use "NR". |
| AAAAA | For an undeveloped system, provide a short description of the system. |

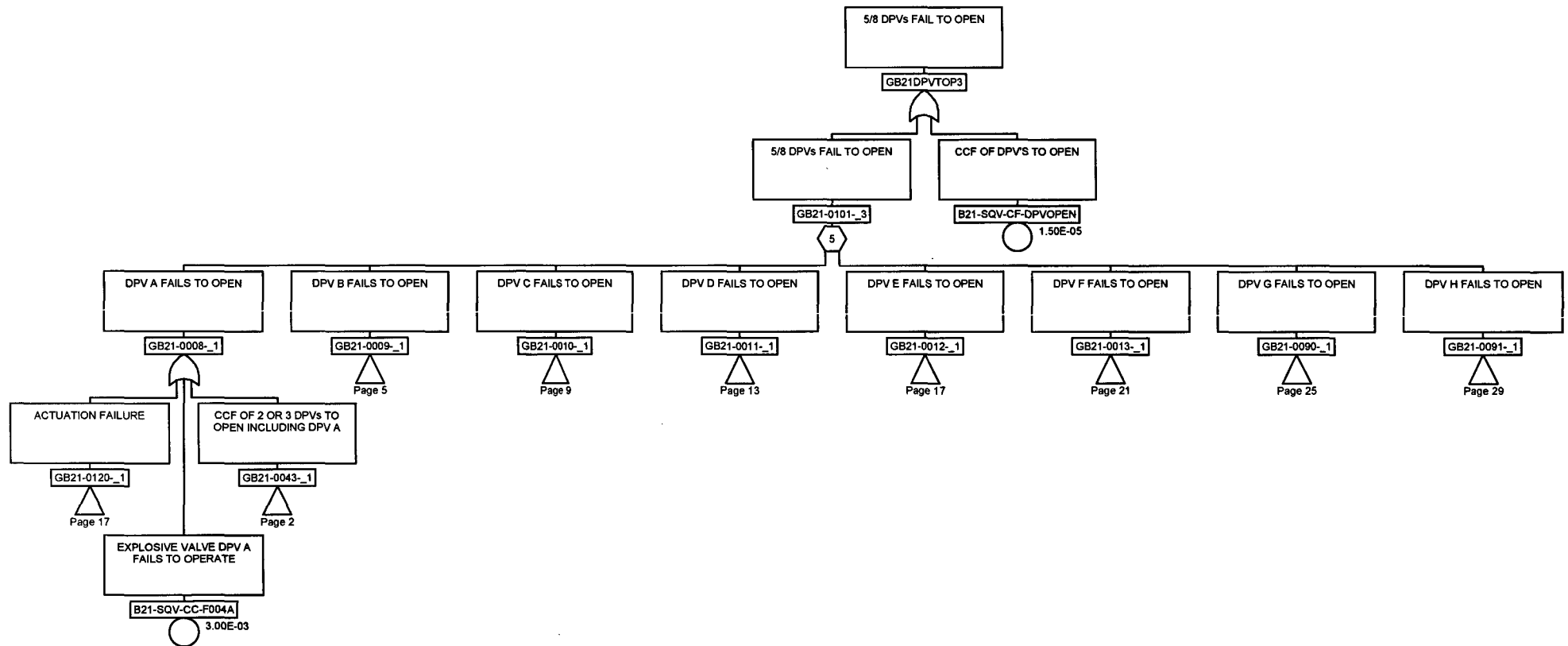
Appendix 4.B
Fault Tree Diagrams

Appendix B Contents

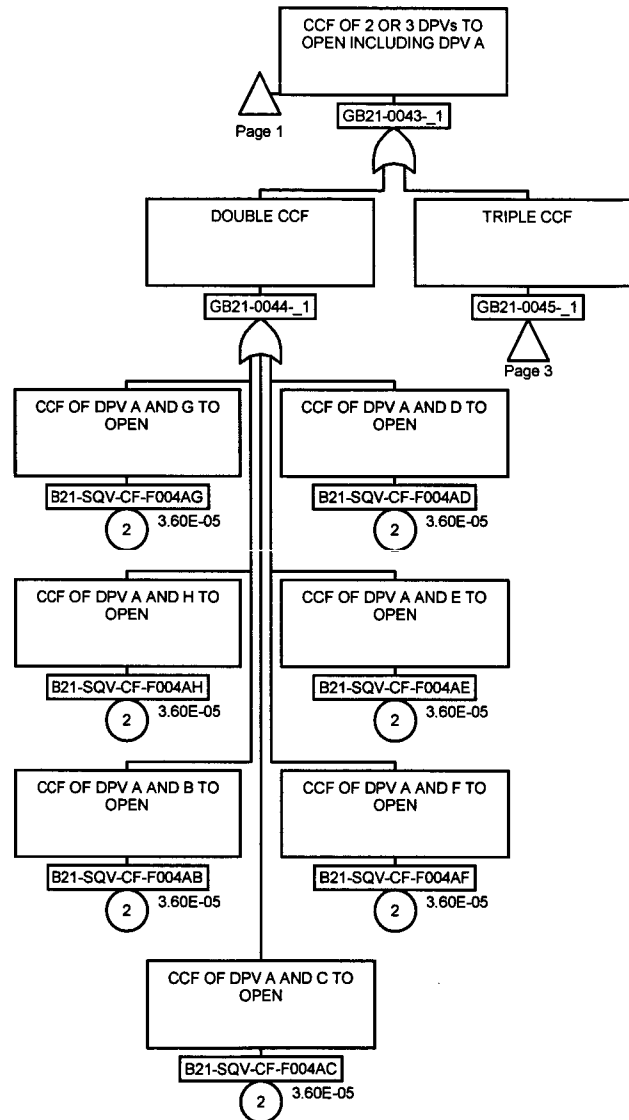
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|-------------------|---|------------|
| Appendix B.4.1-1 | Automatic Depressurization System Fault Tree | B.4.1-1-1 |
| Appendix B.4.1-2 | Automatic Depressurization System Fault Tree | B.4.1-2-1 |
| Appendix B.4.2 | Isolation Condenser System Fault Tree | B.4.2-1 |
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| Appendix B.4.9-2 | Feedwater/Condensate and TCCW Fault Tree | B.4.9-2-1 |

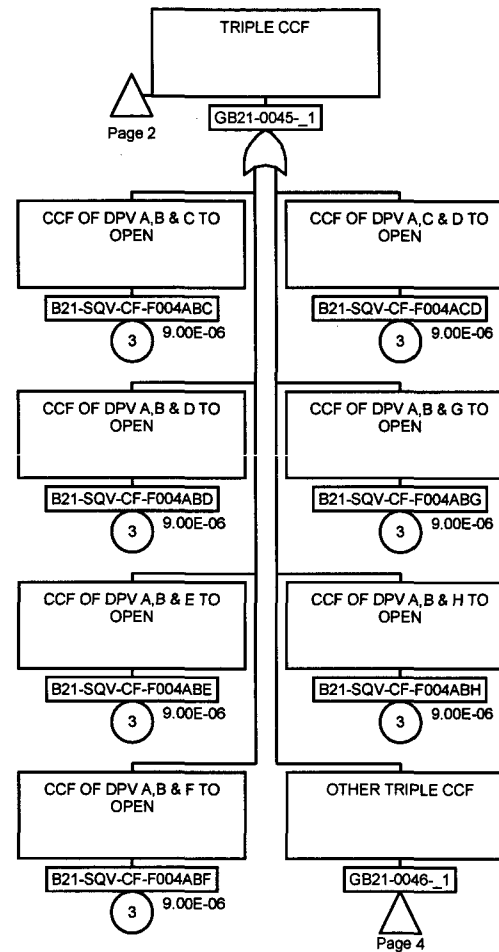
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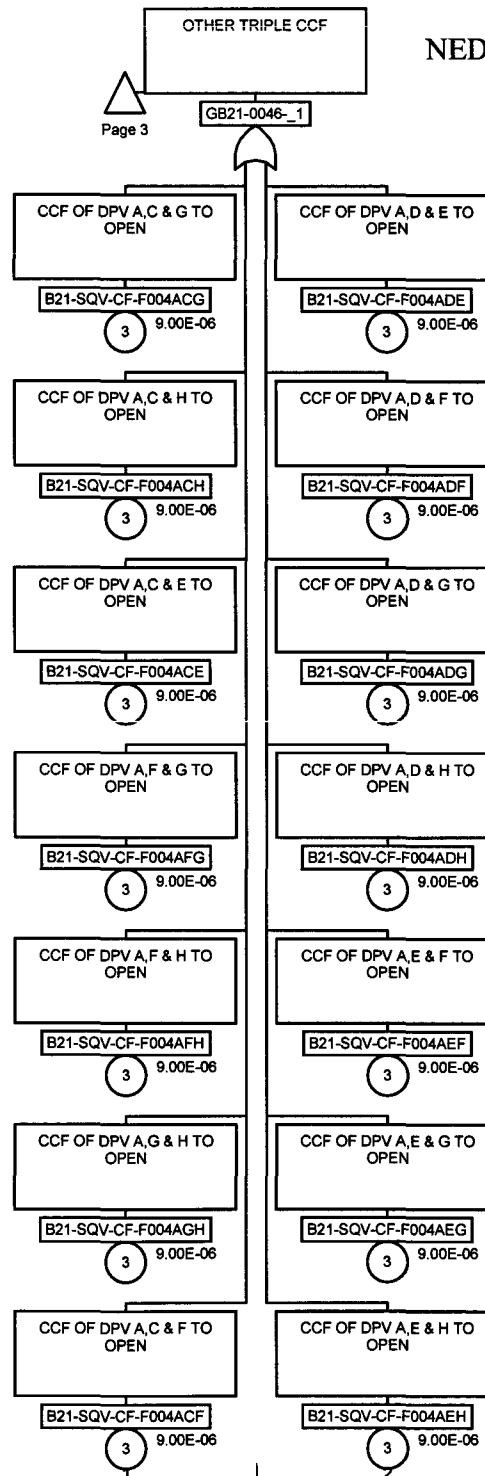
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Automatic Depressurization System
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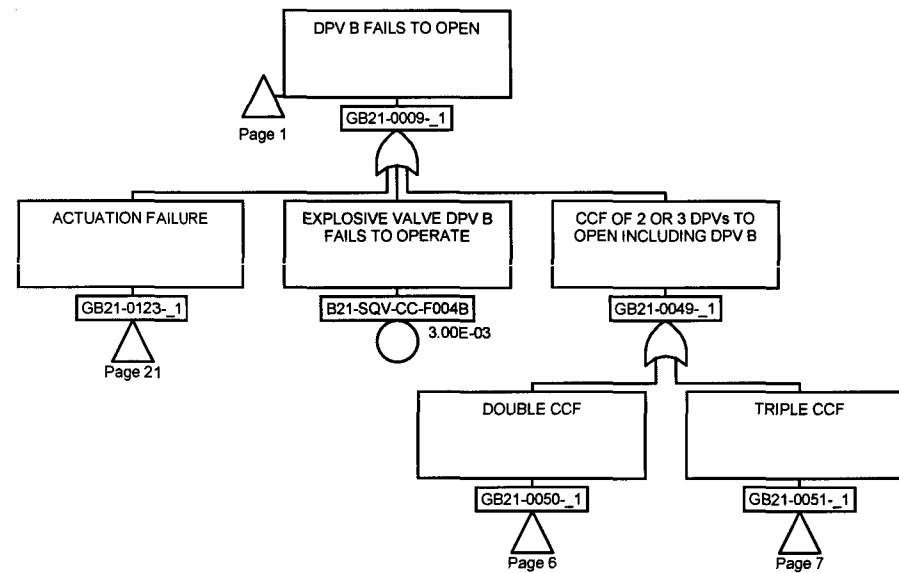


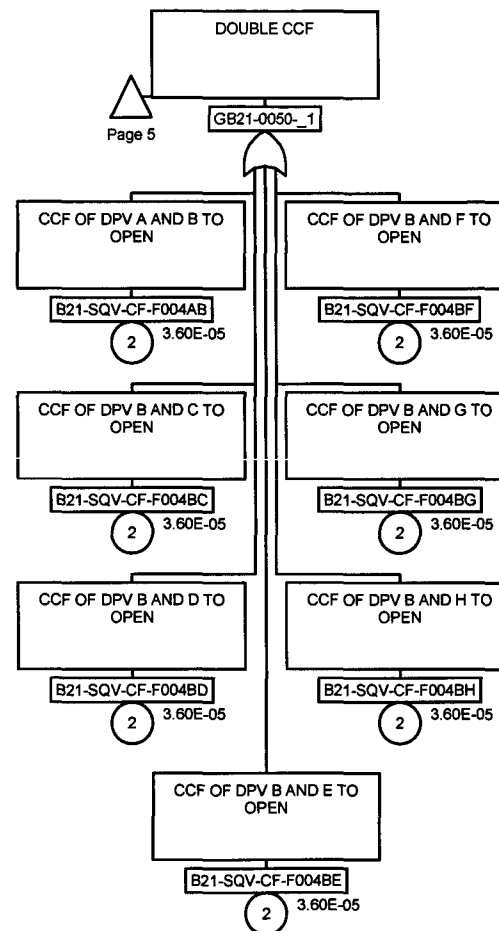
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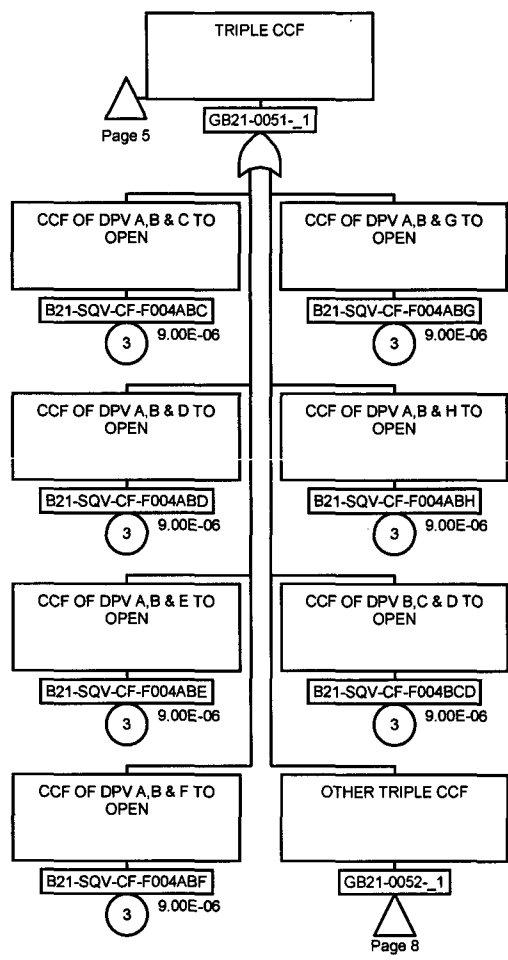




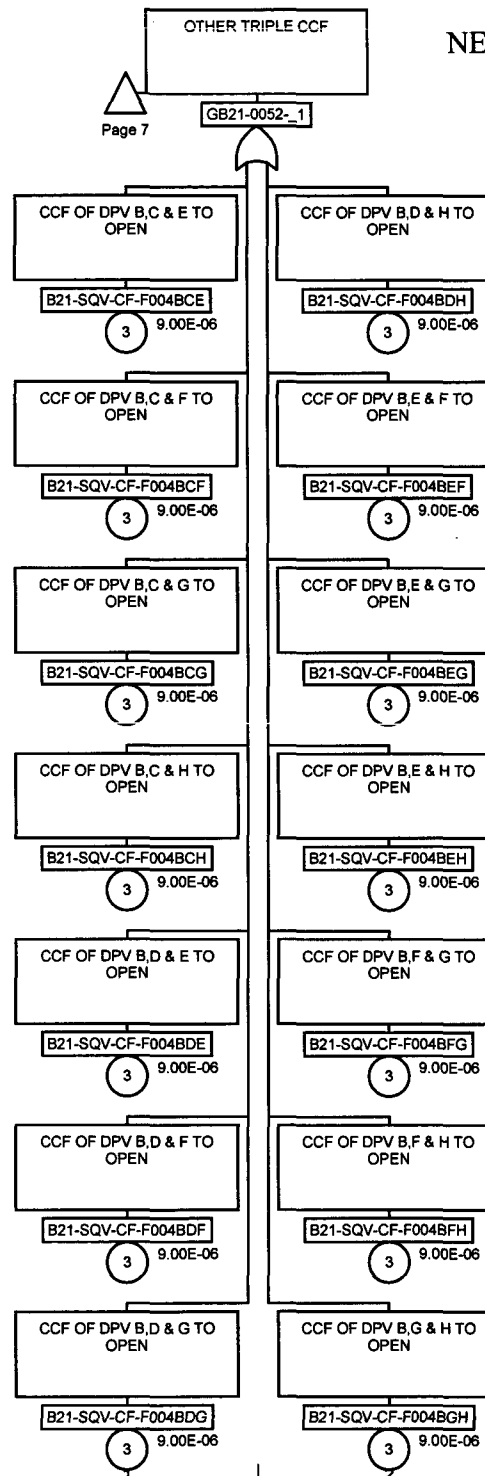


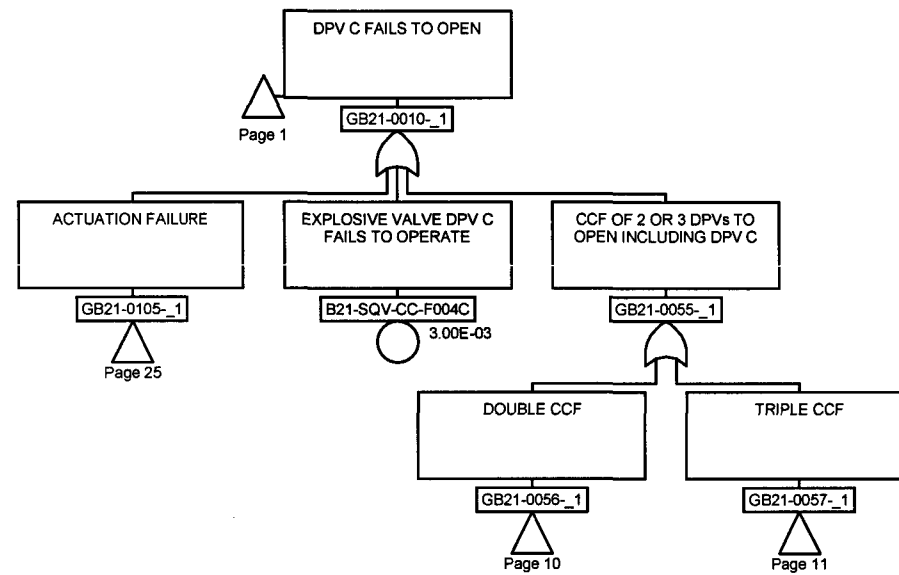


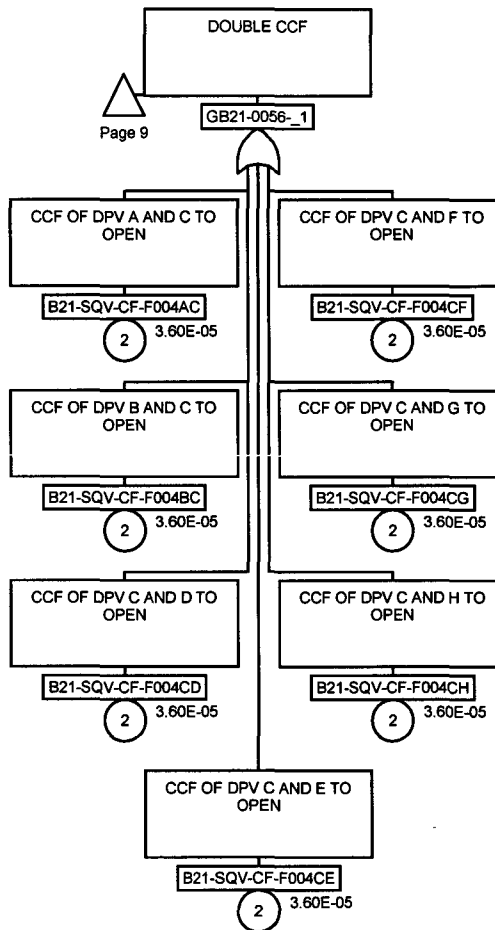


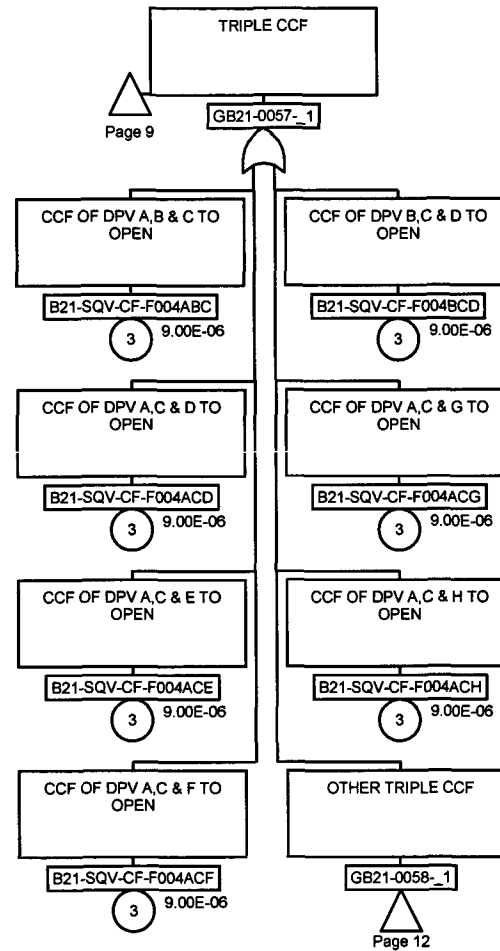


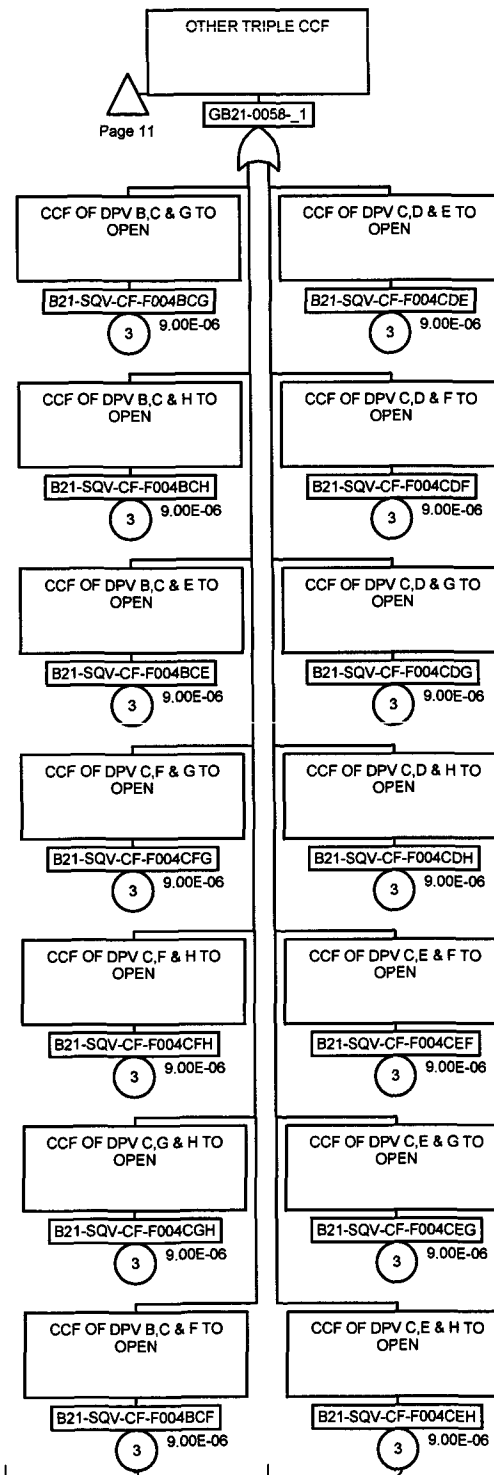
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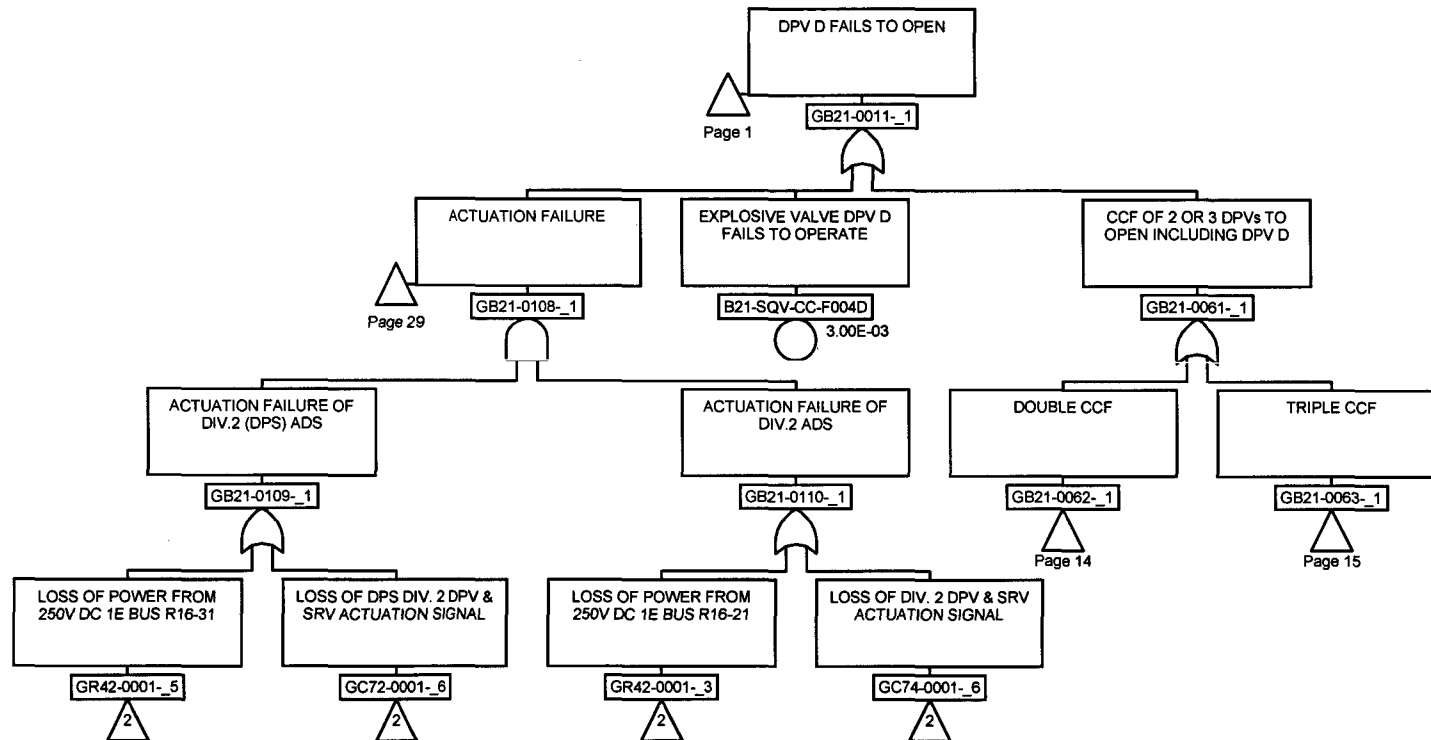


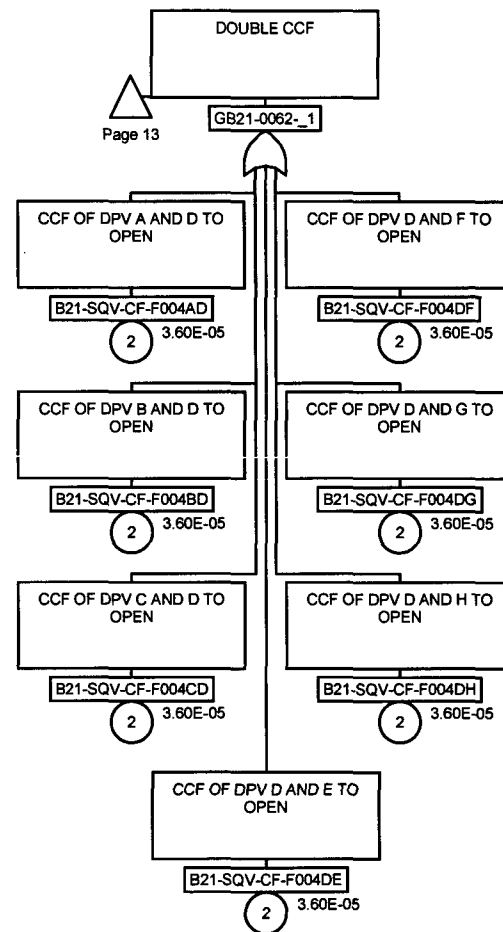


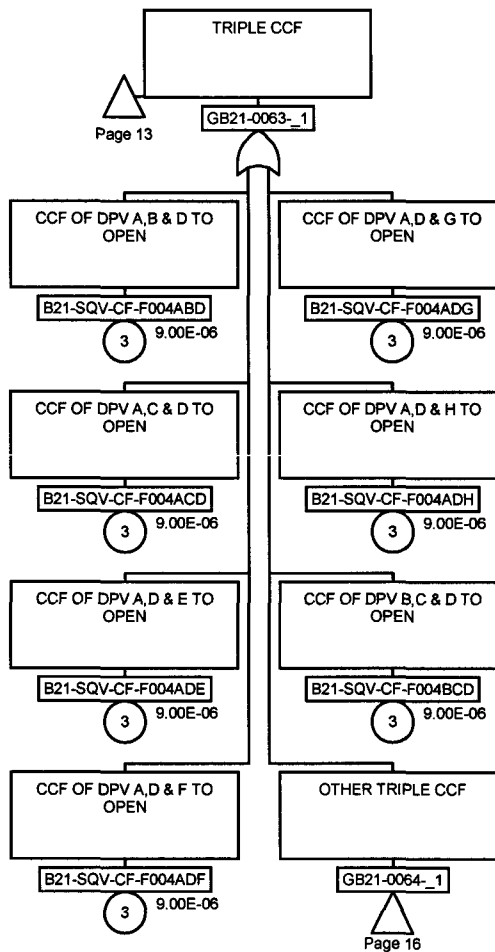


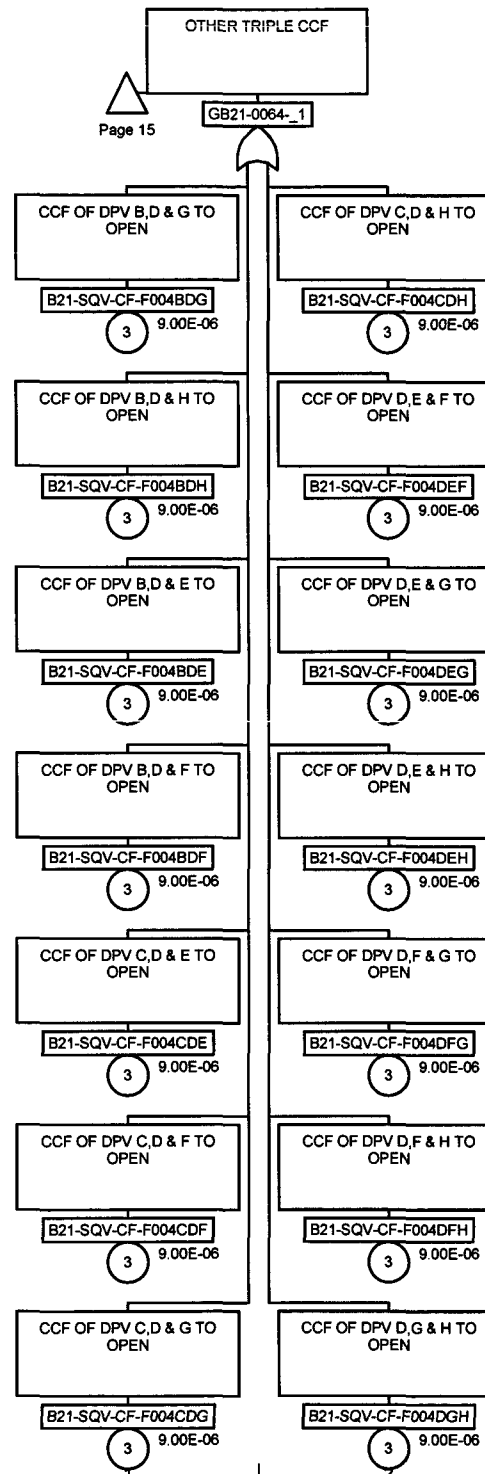


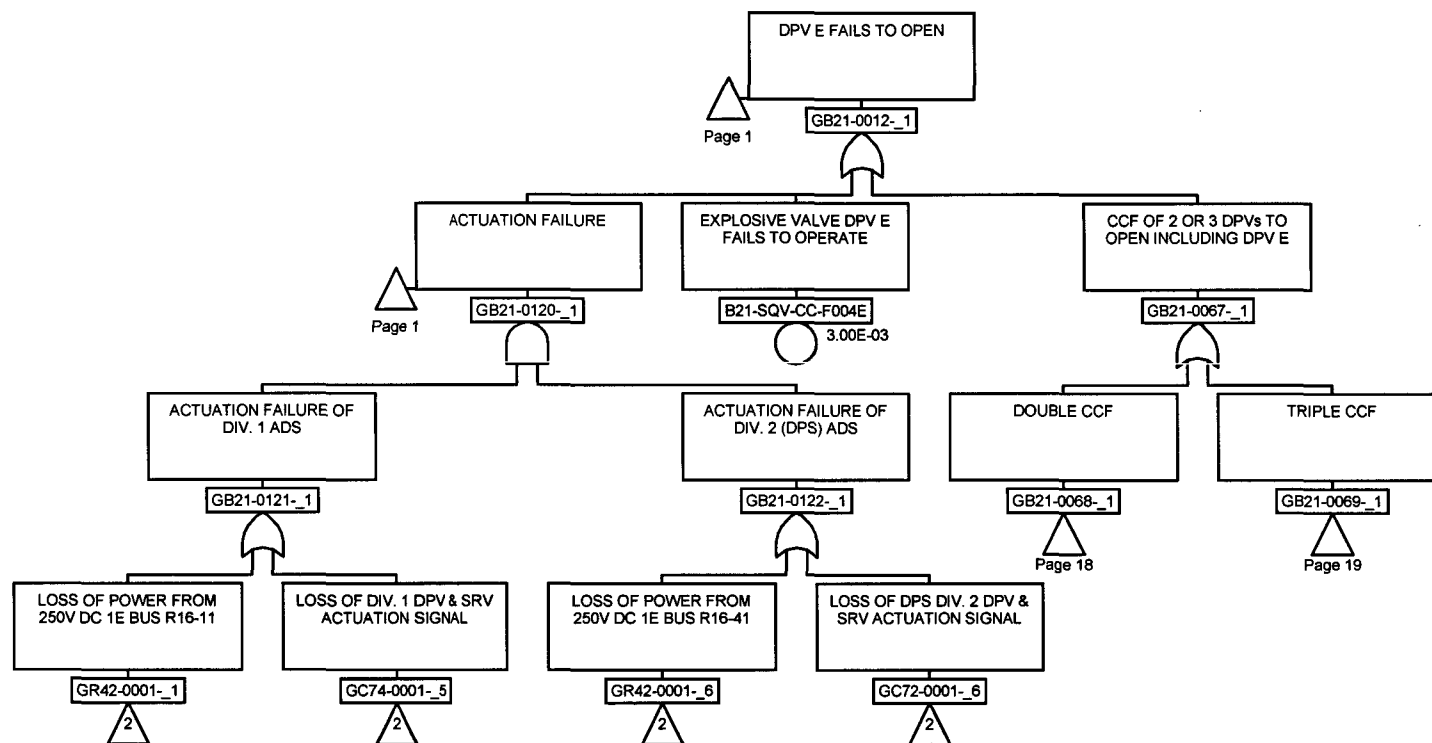


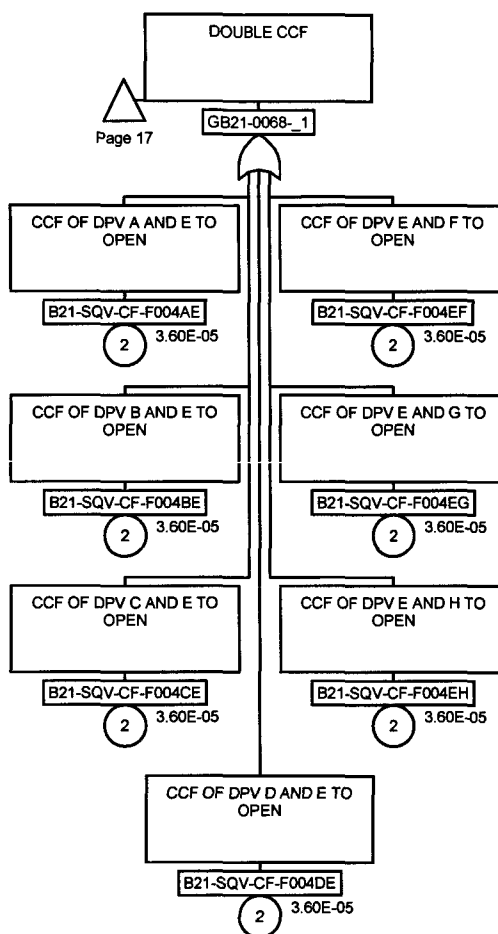


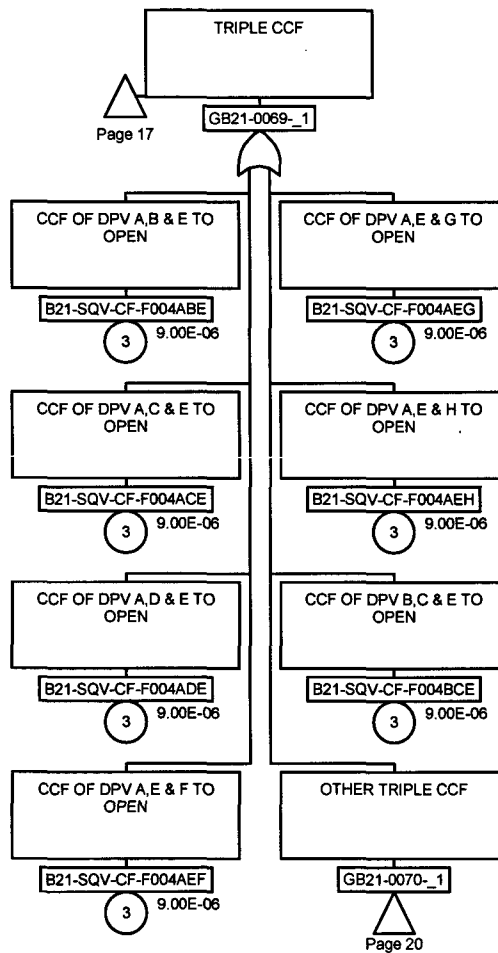


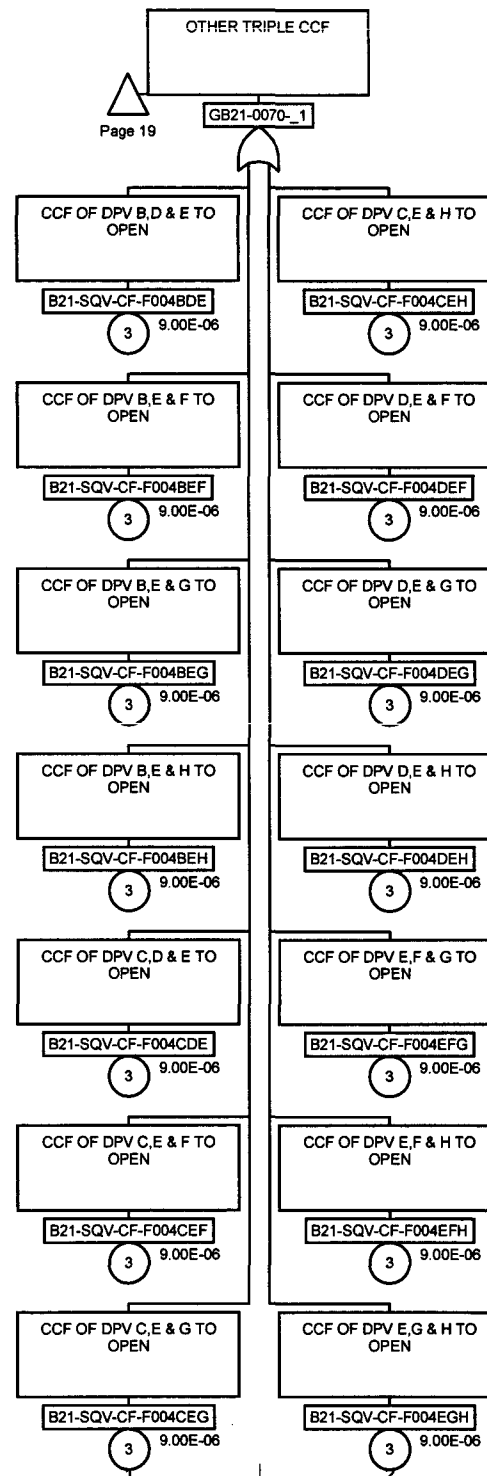


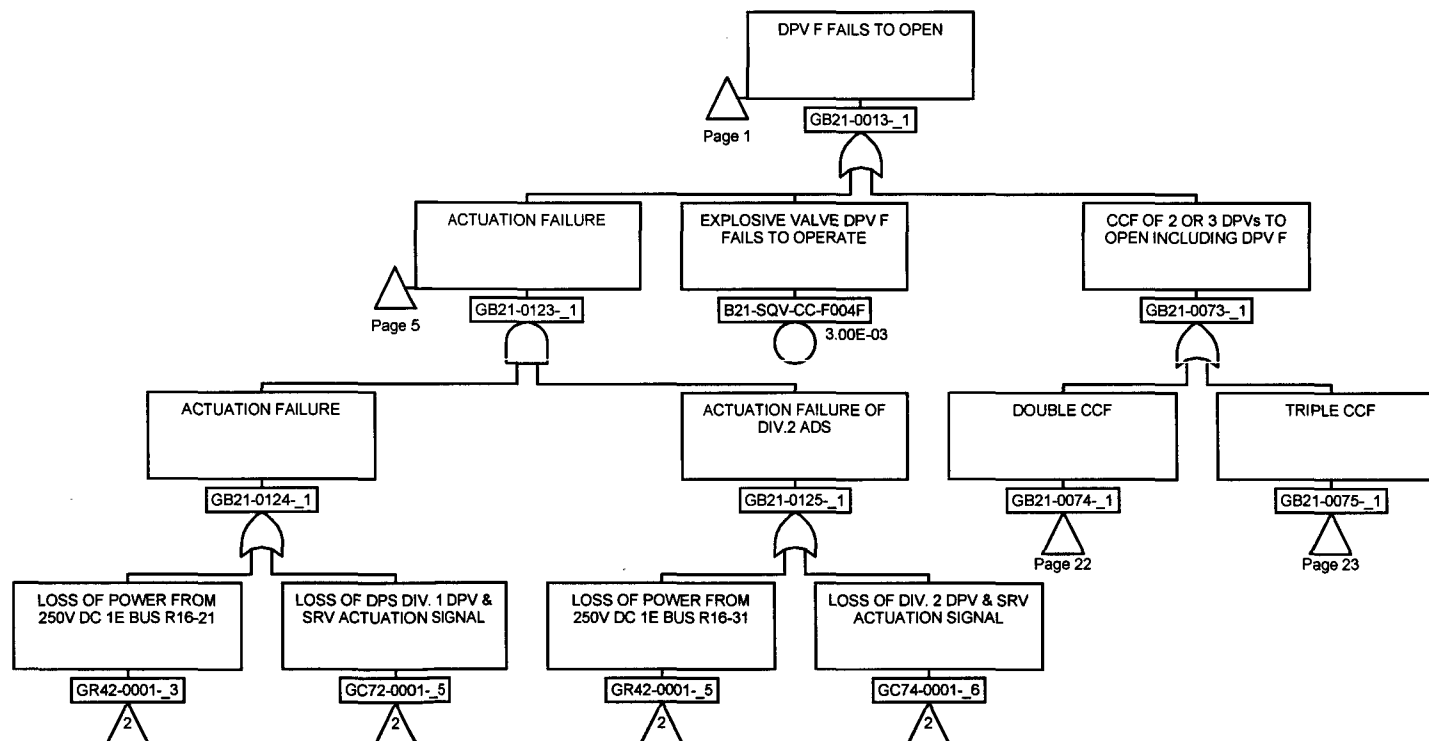


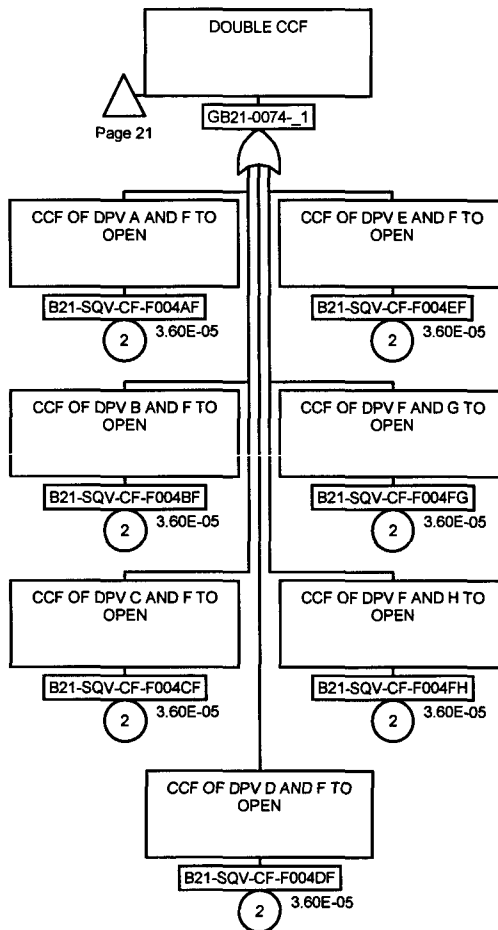


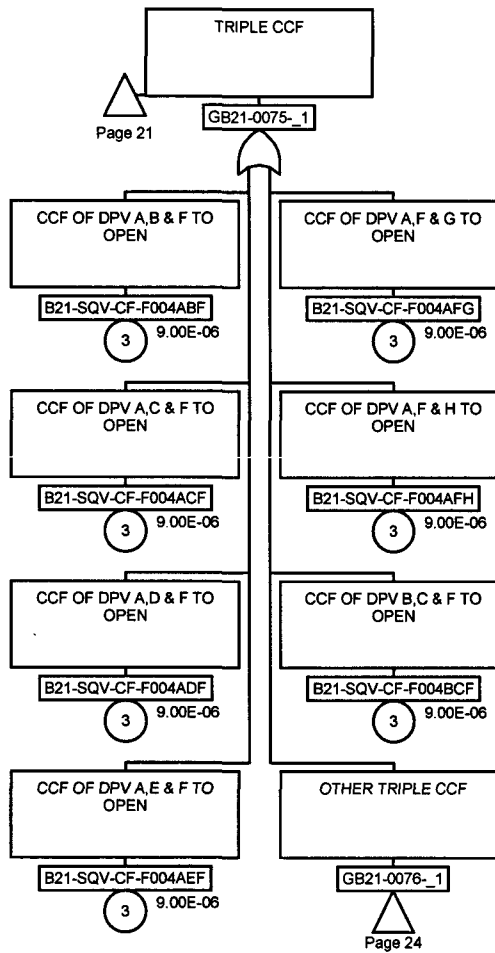


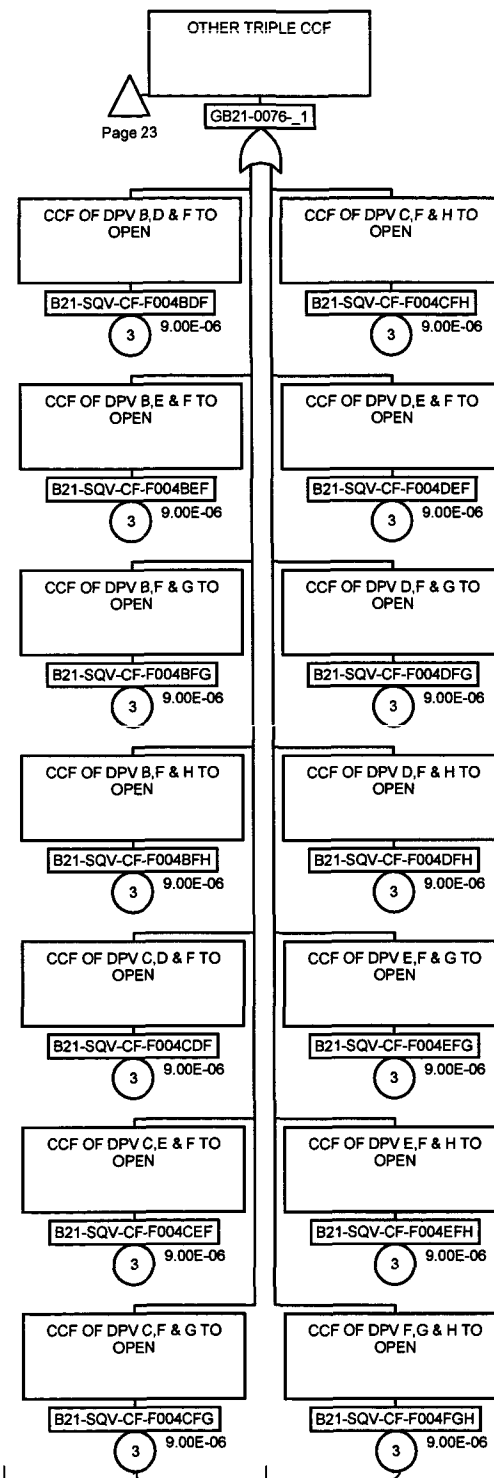


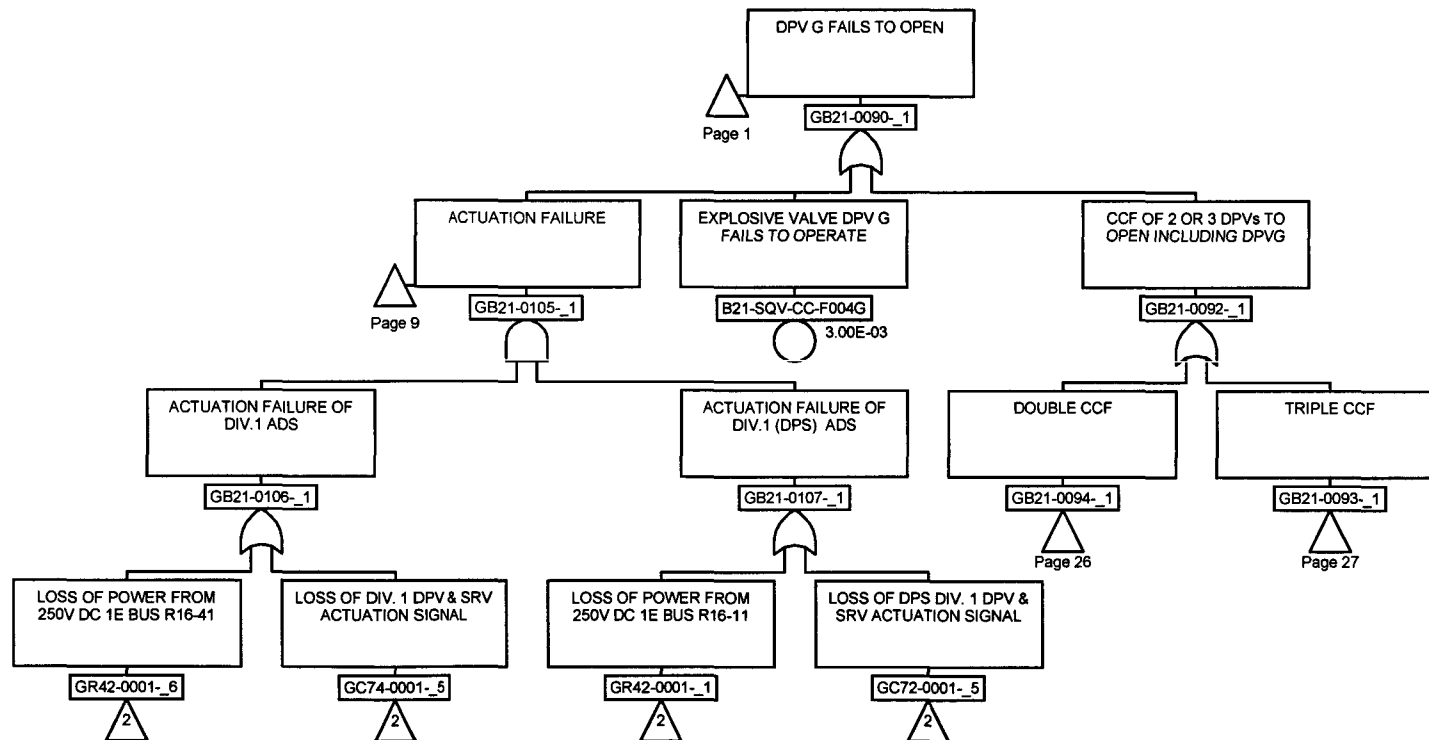


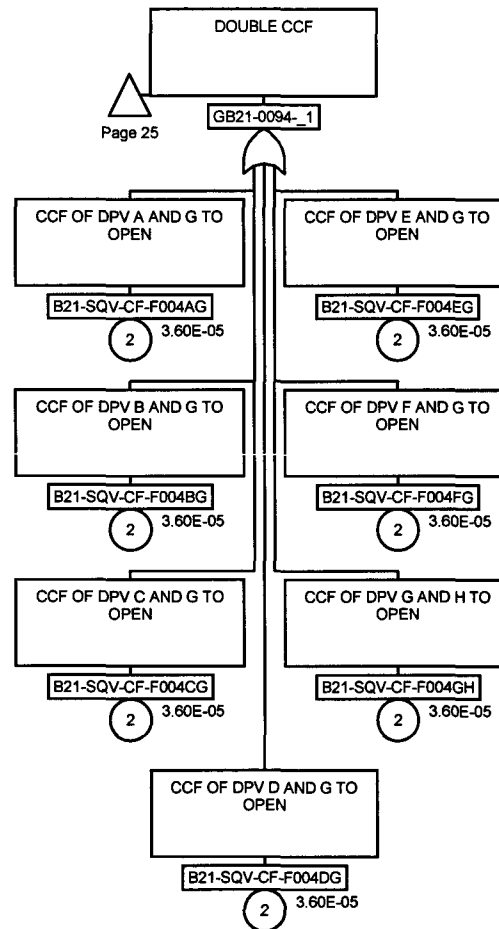




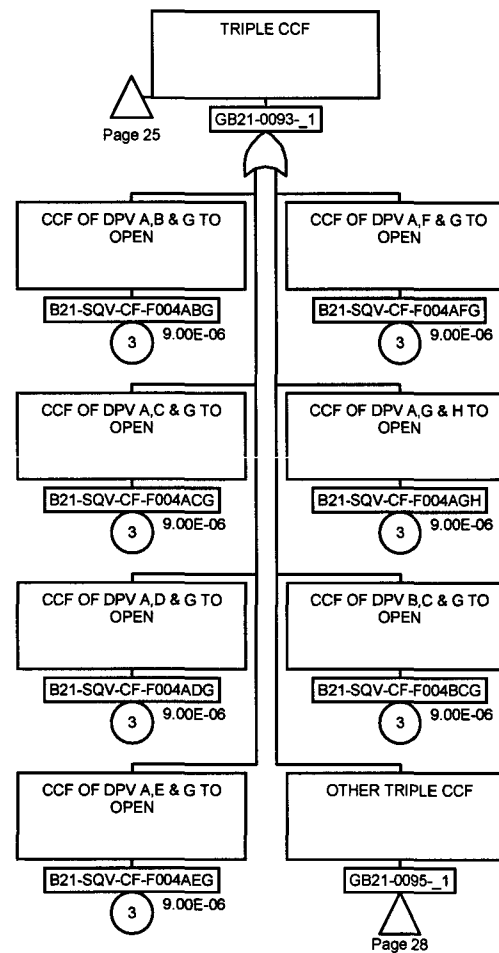




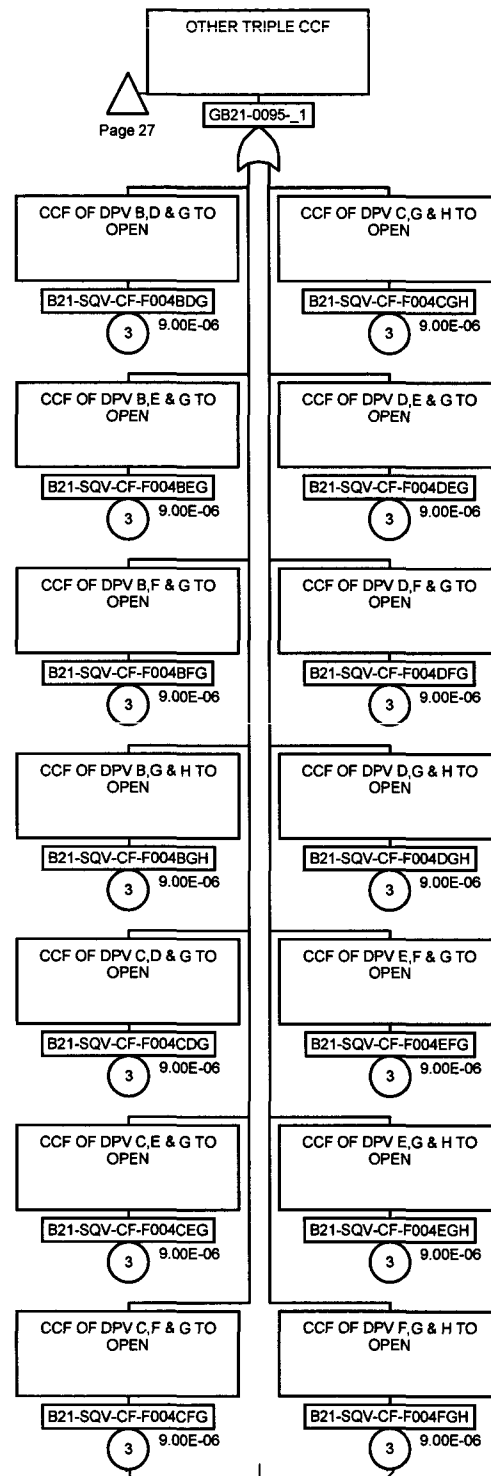


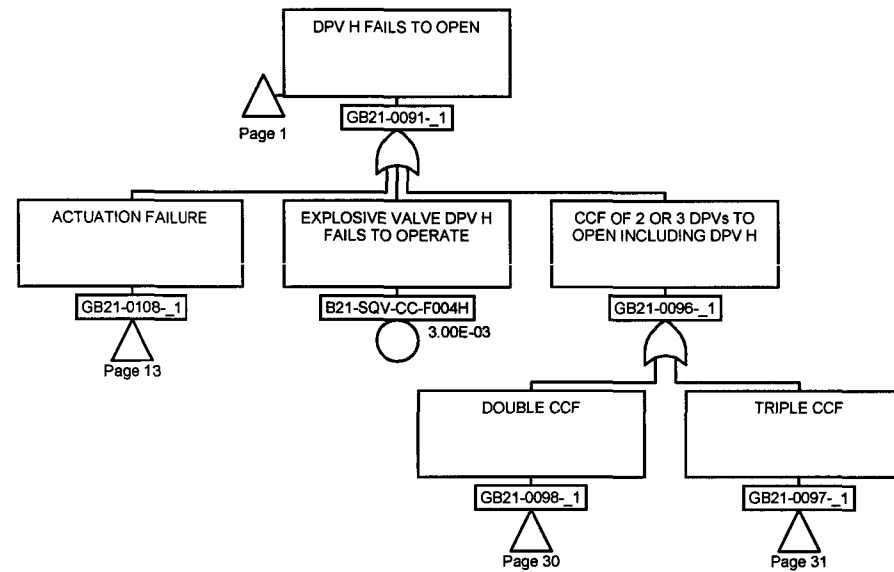


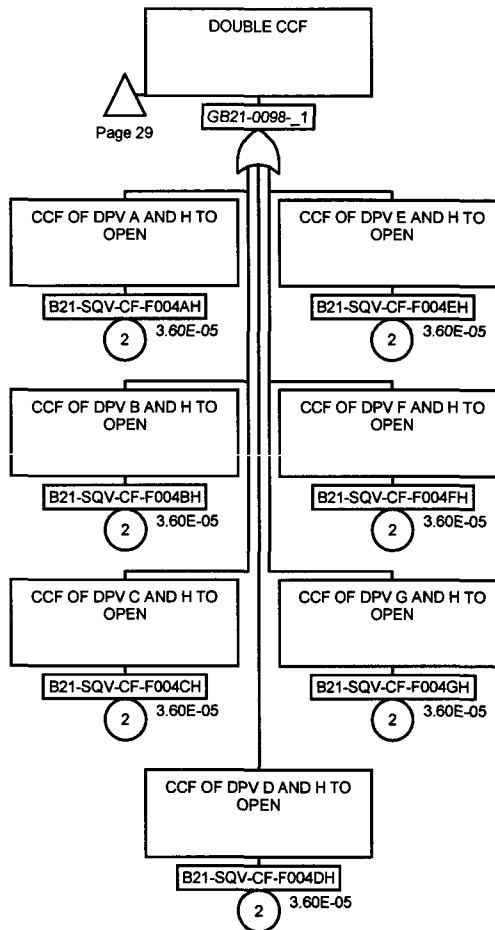
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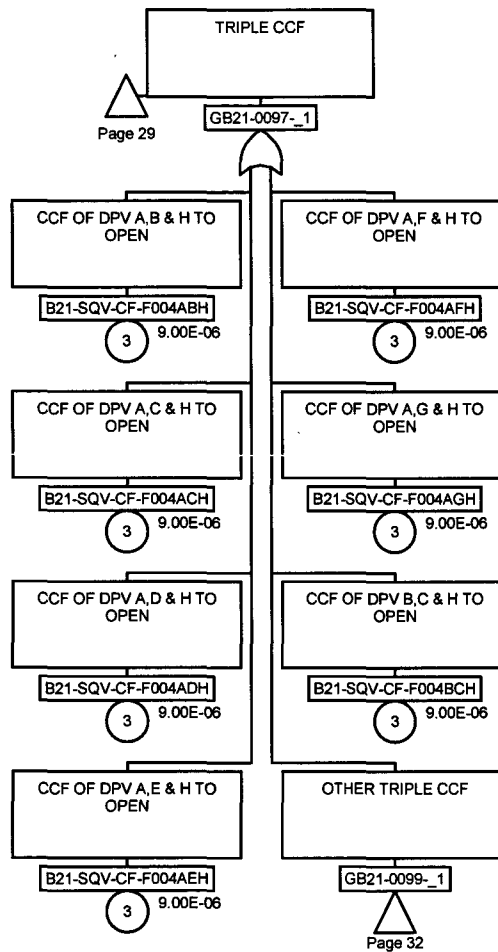
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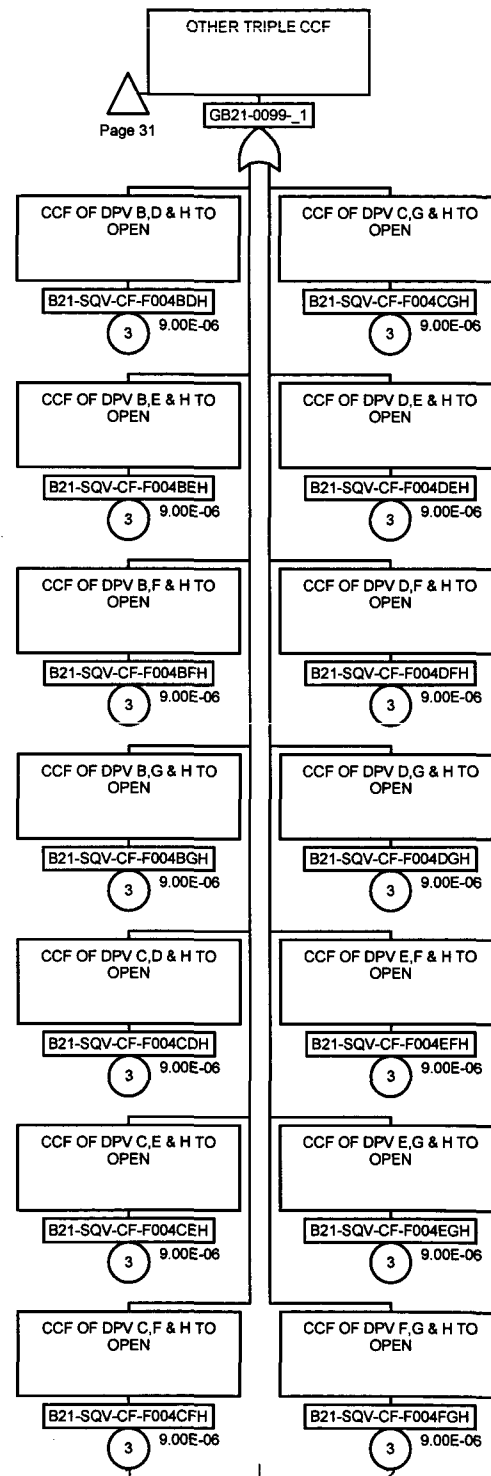




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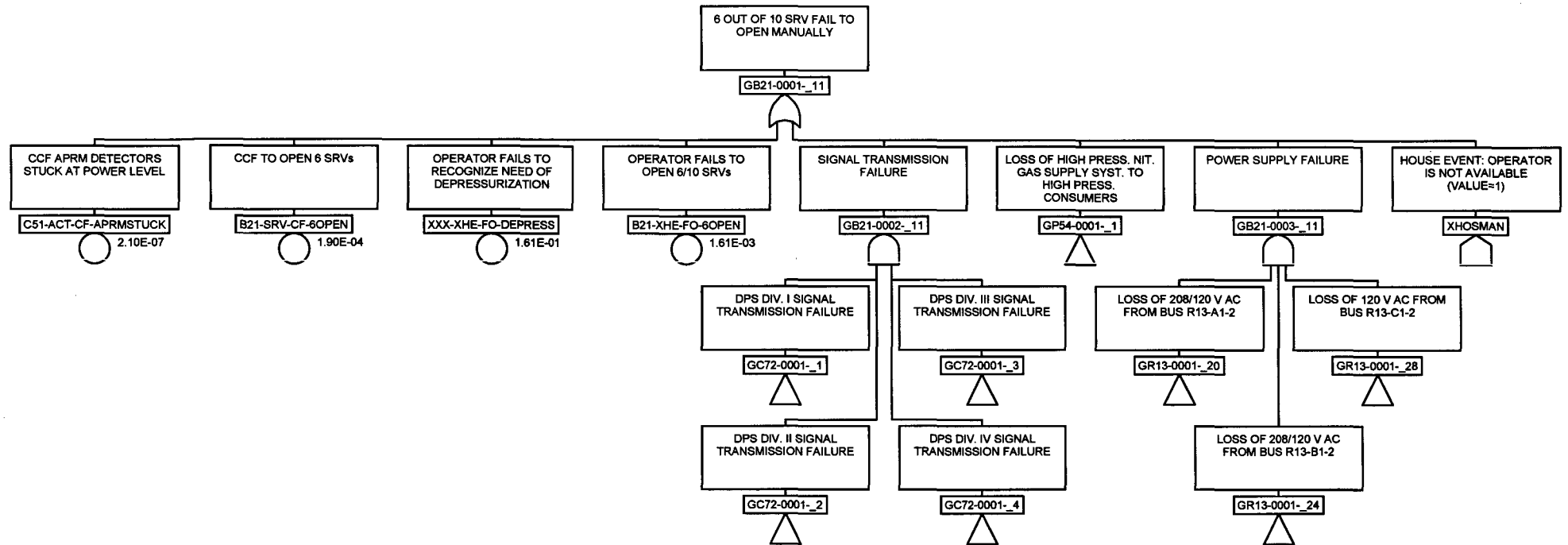
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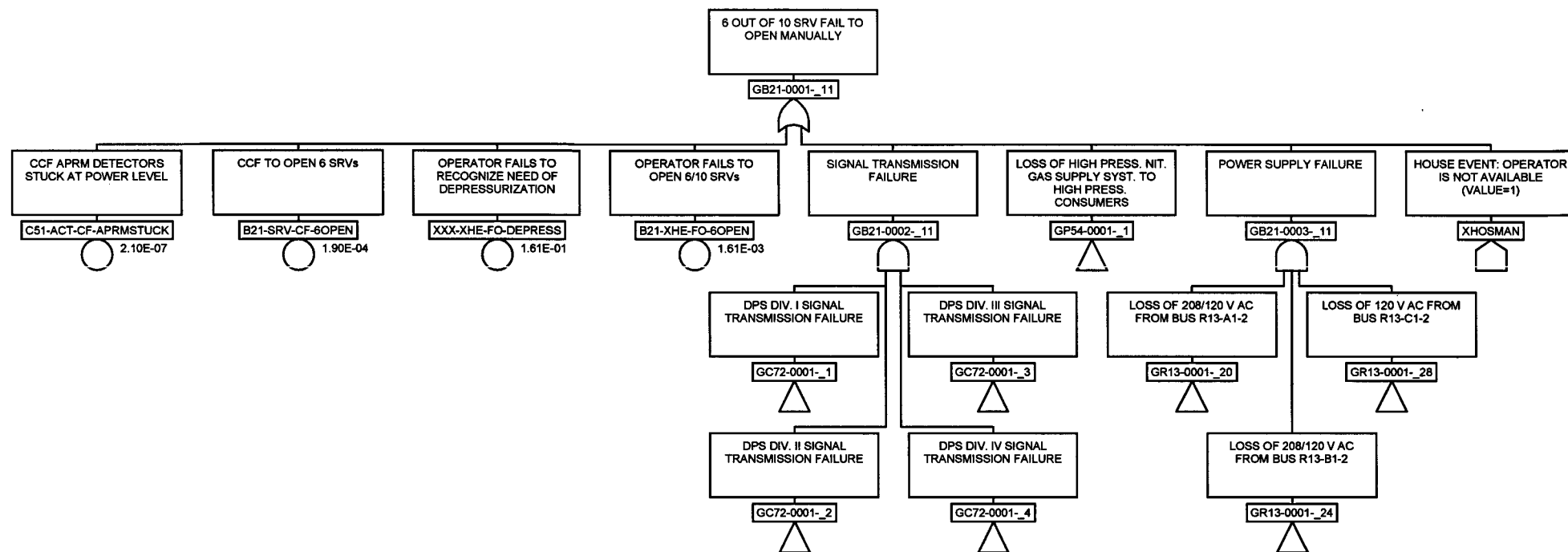
Appendix B.4.1-2
Automatic Depressurization System
Fault Tree



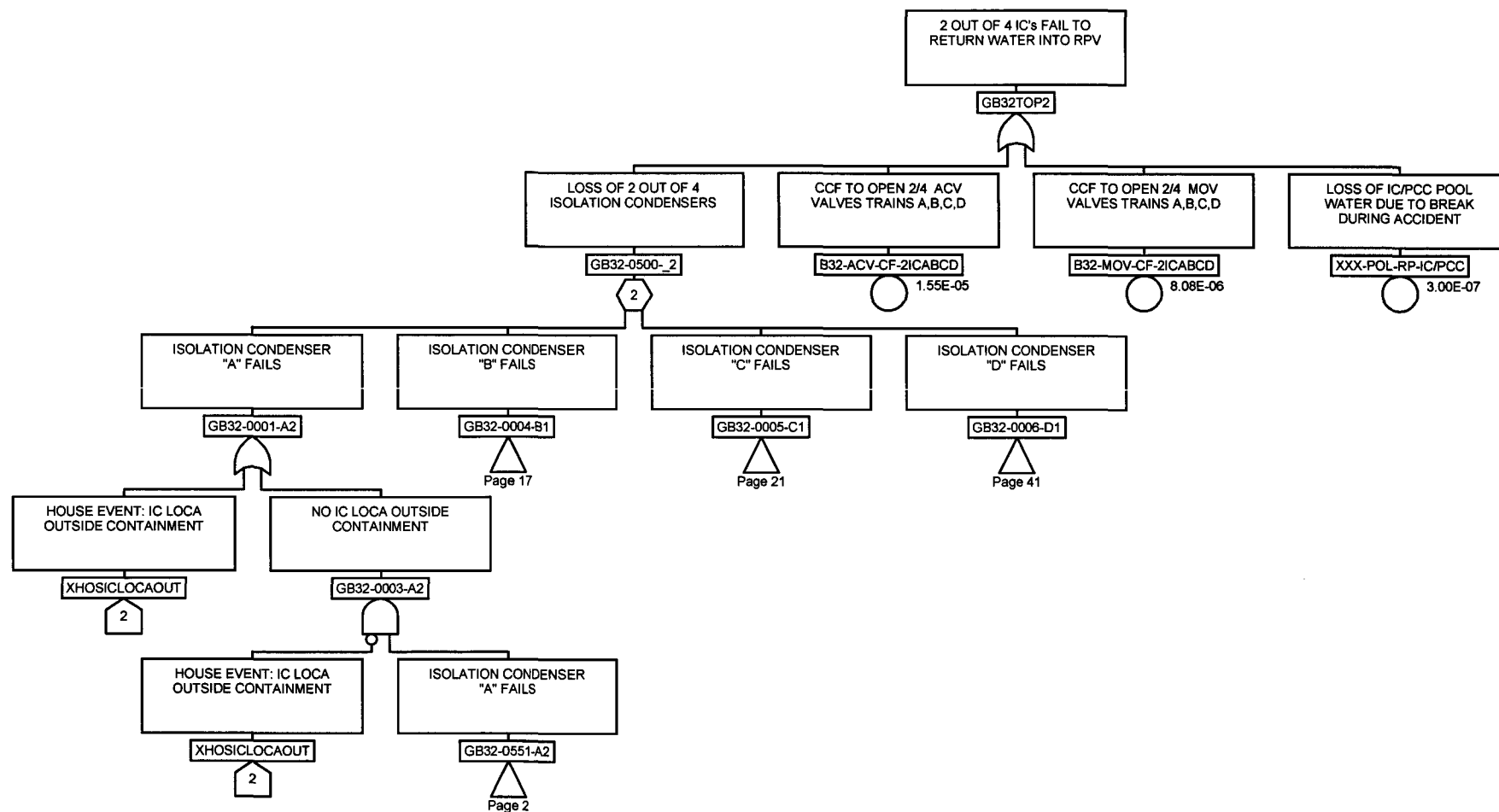
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| B21-SRV-CF-6OPEN | 1 | 2 | | | | | | | |
| B21-XHE-FO-6OPEN | 1 | 4 | | | | | | | |
| C51-ACT-CF-APRMSTUCK | 1 | 1 | | | | | | | |
| GB21-0001- 11 | 1 | 5 | | | | | | | |
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| GP54-0001- 1 | 1 | 6 | | | | | | | |
| GR13-0001- 20 | 1 | 7 | | | | | | | |
| GR13-0001- 24 | 1 | 7 | | | | | | | |
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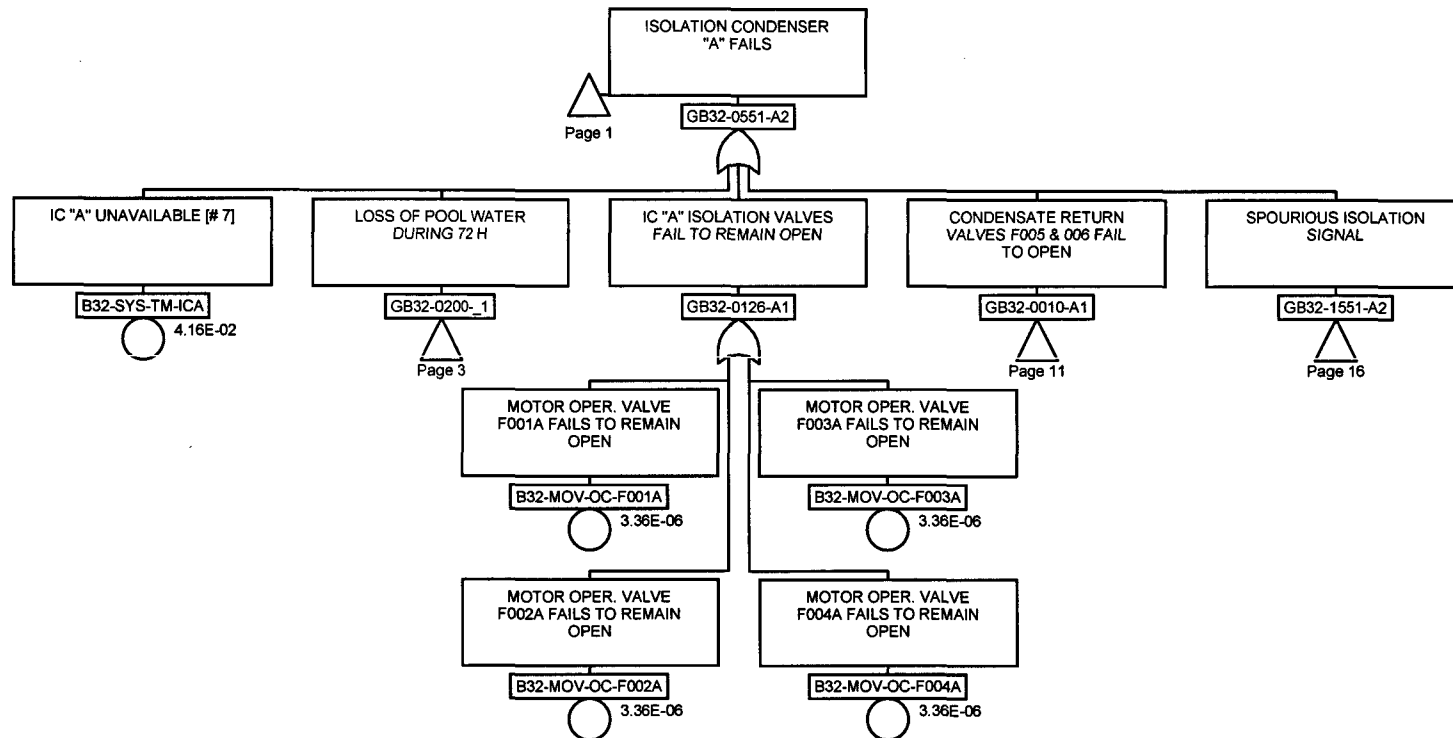
Appendix B.4.1-2
Automatic Depressurization System
Fault Tree

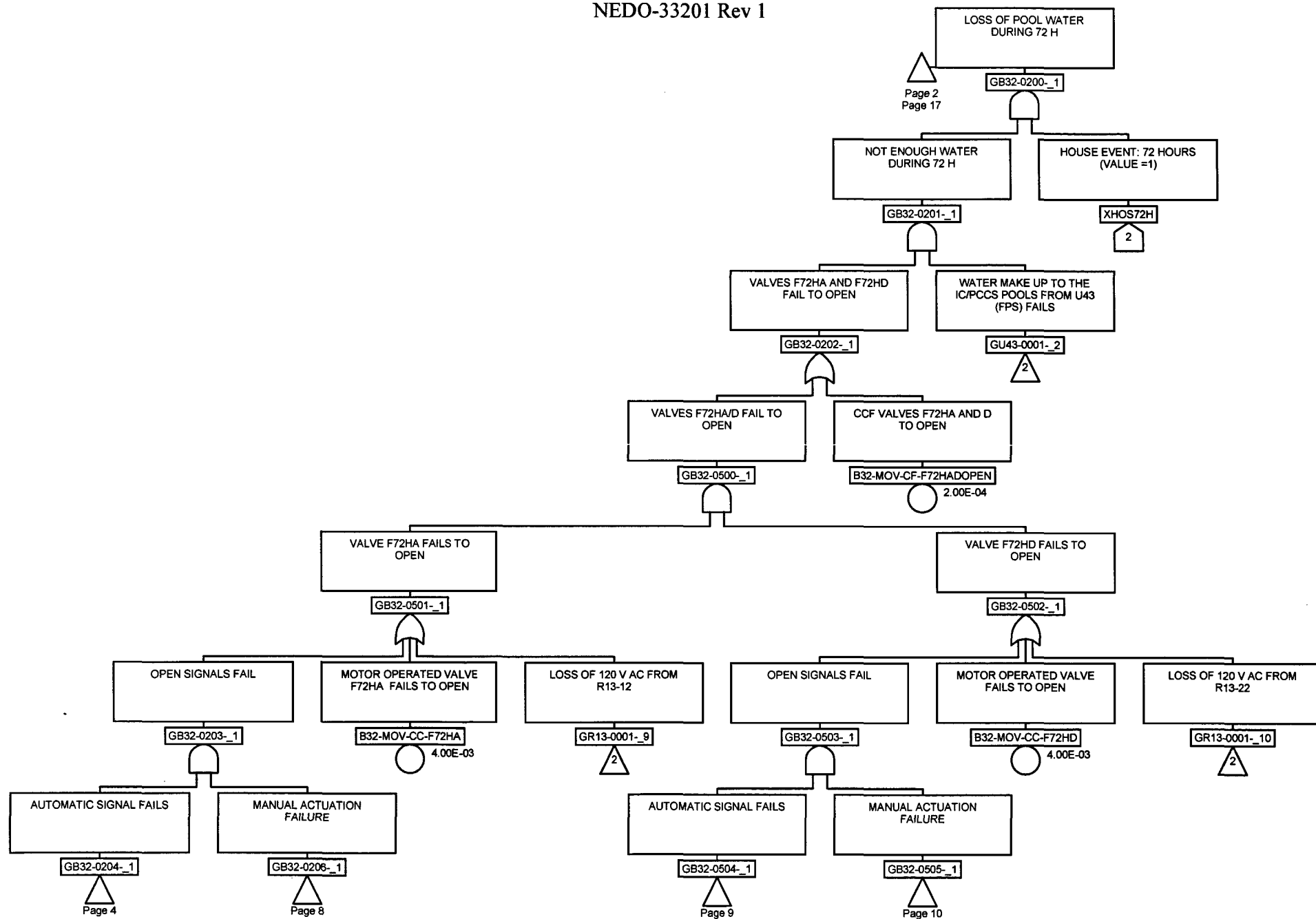
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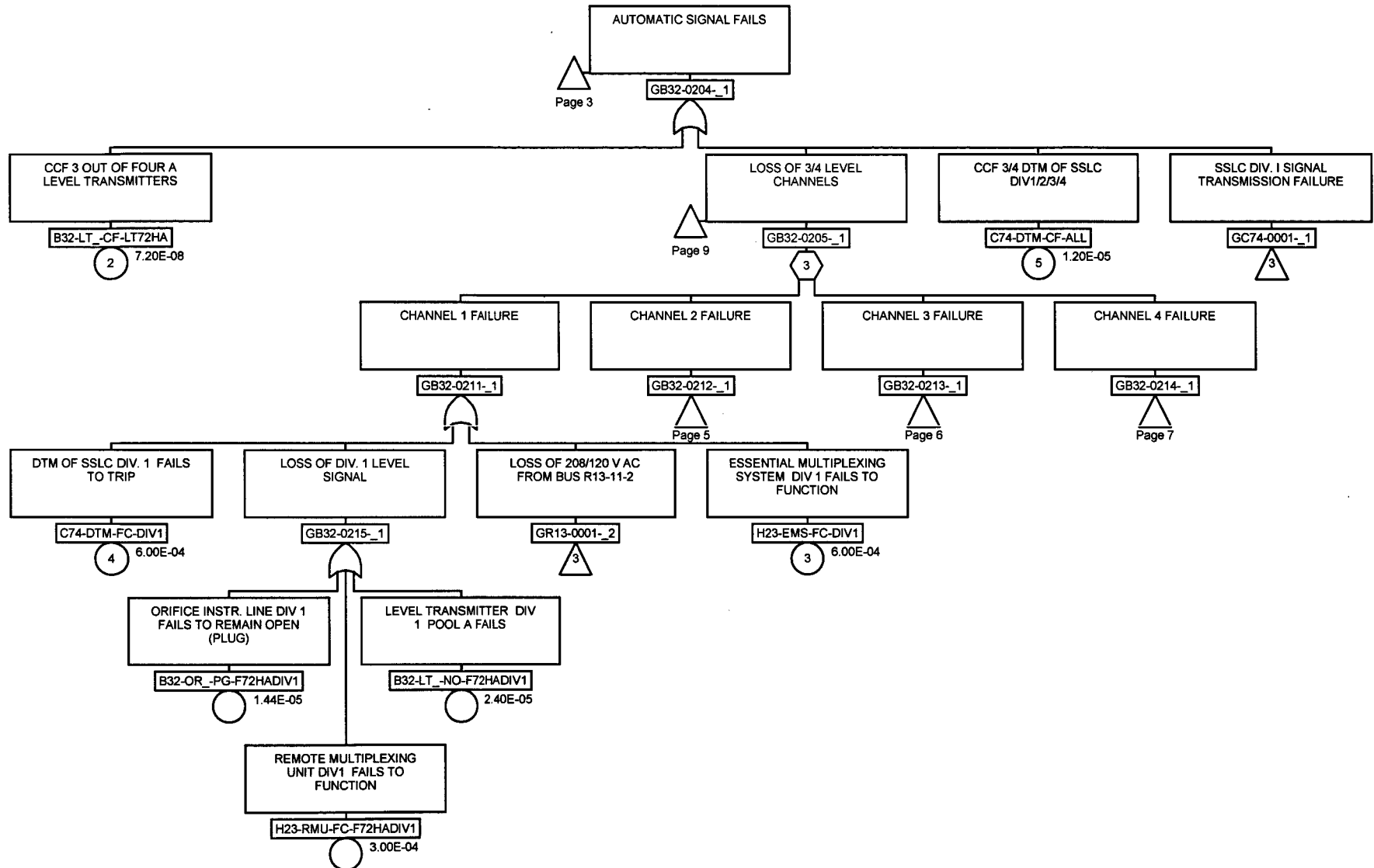
Appendix B.4.2
Isolation Condenser System
Fault Tree

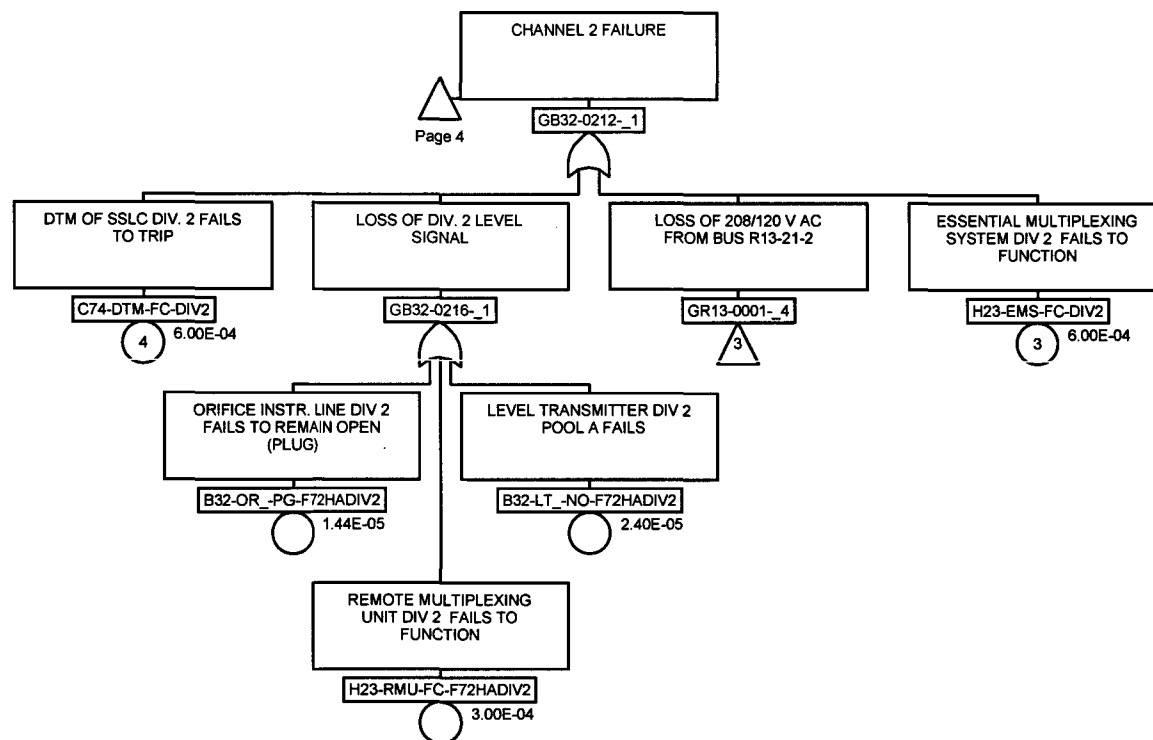


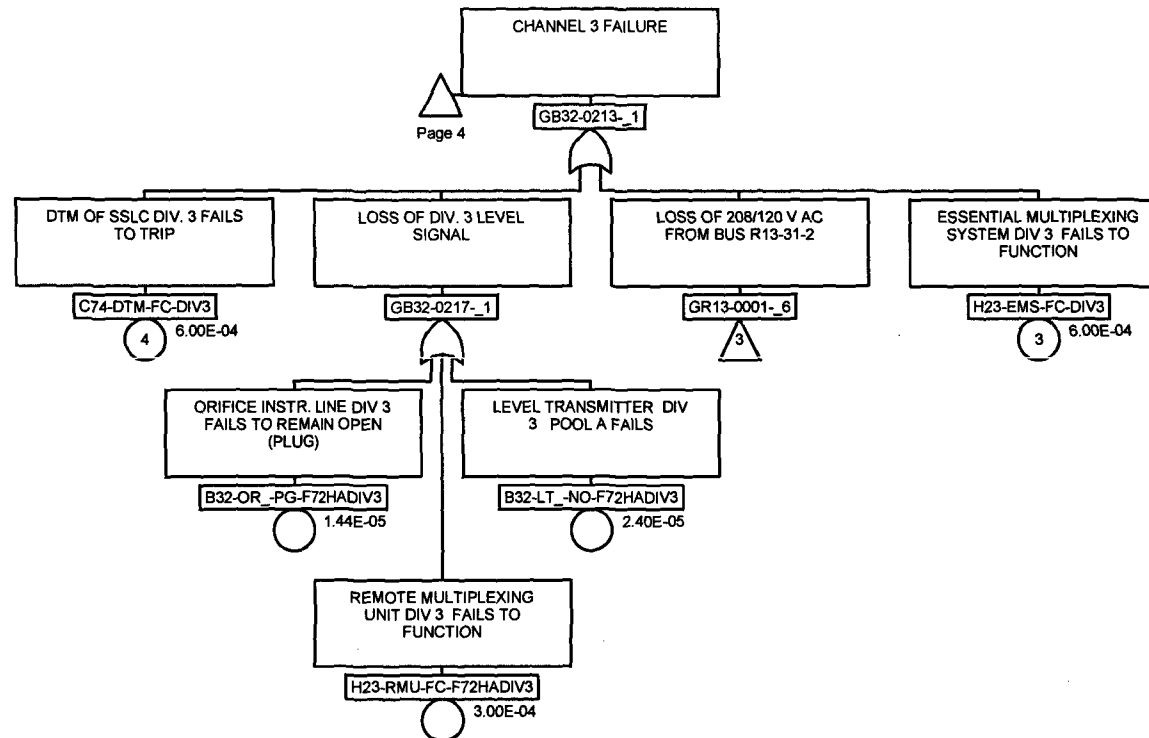


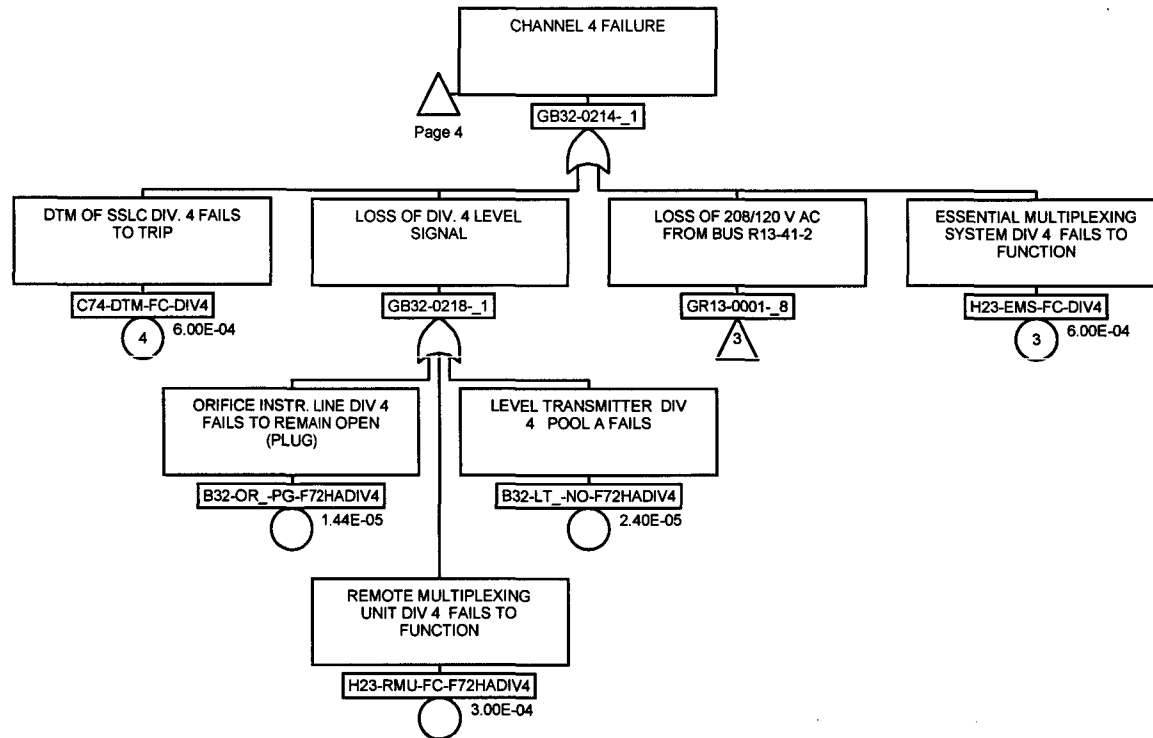


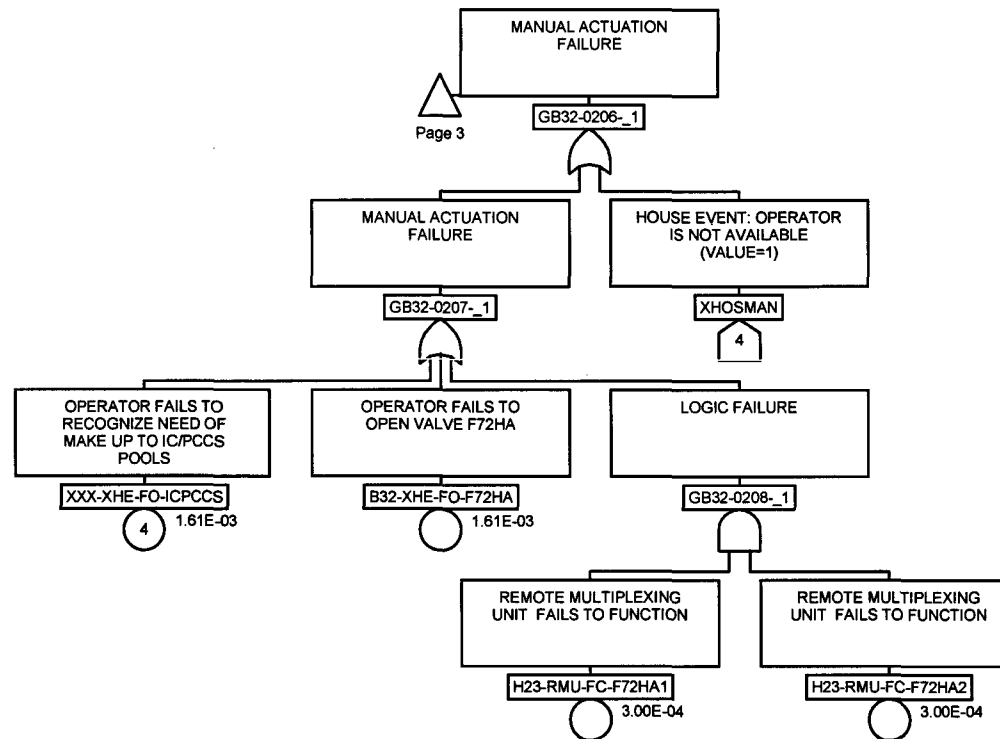
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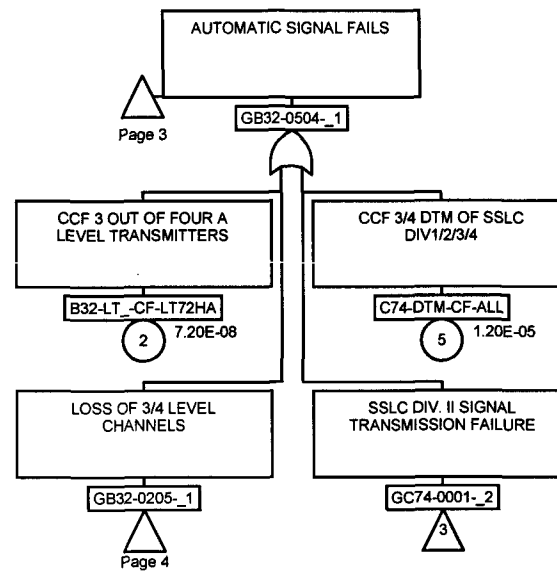


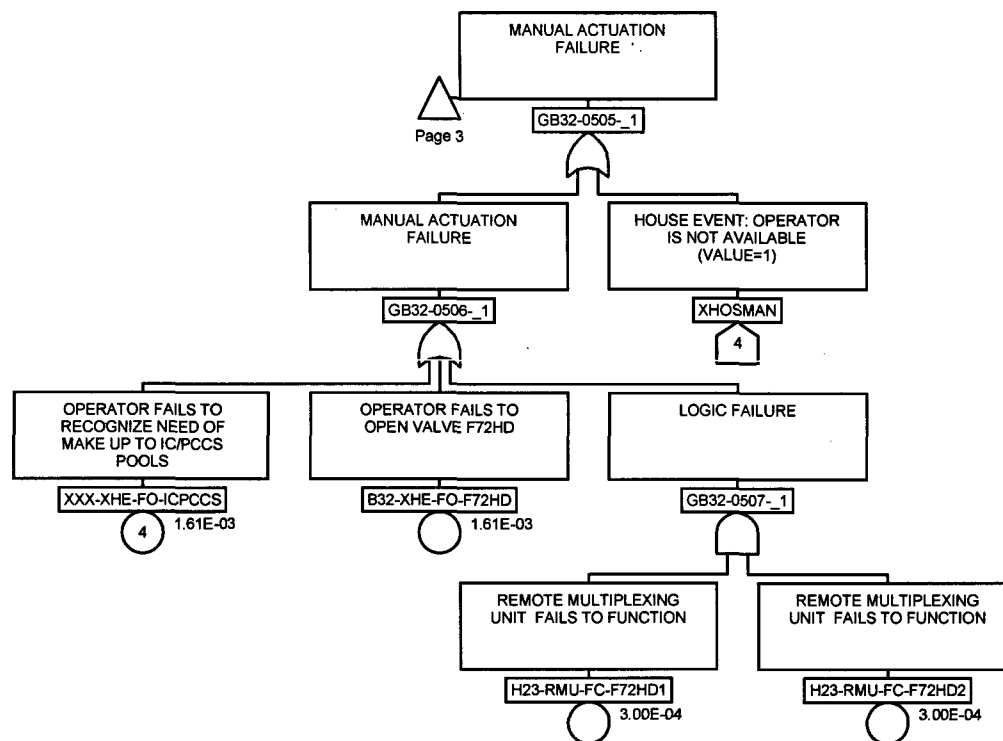


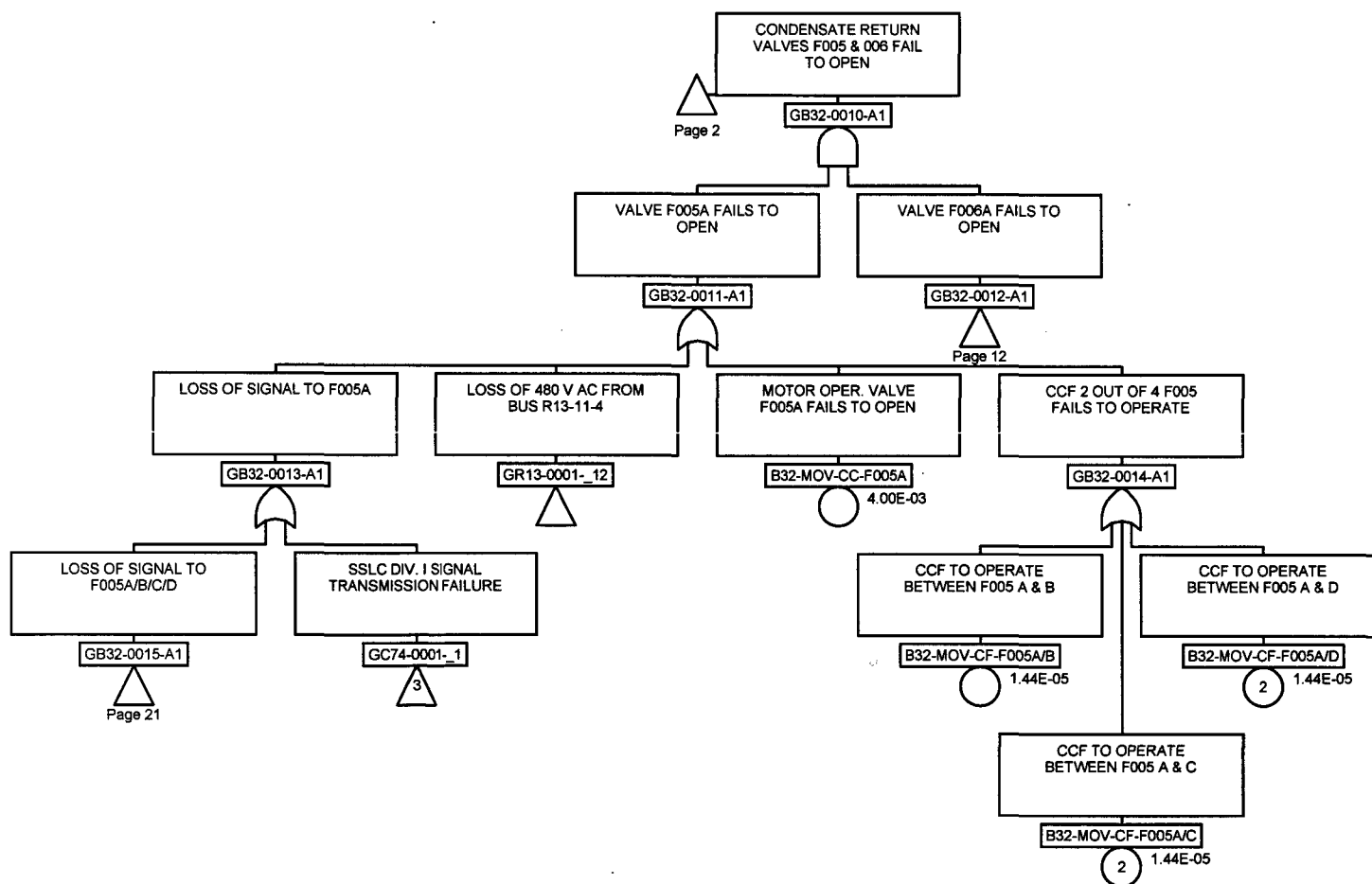


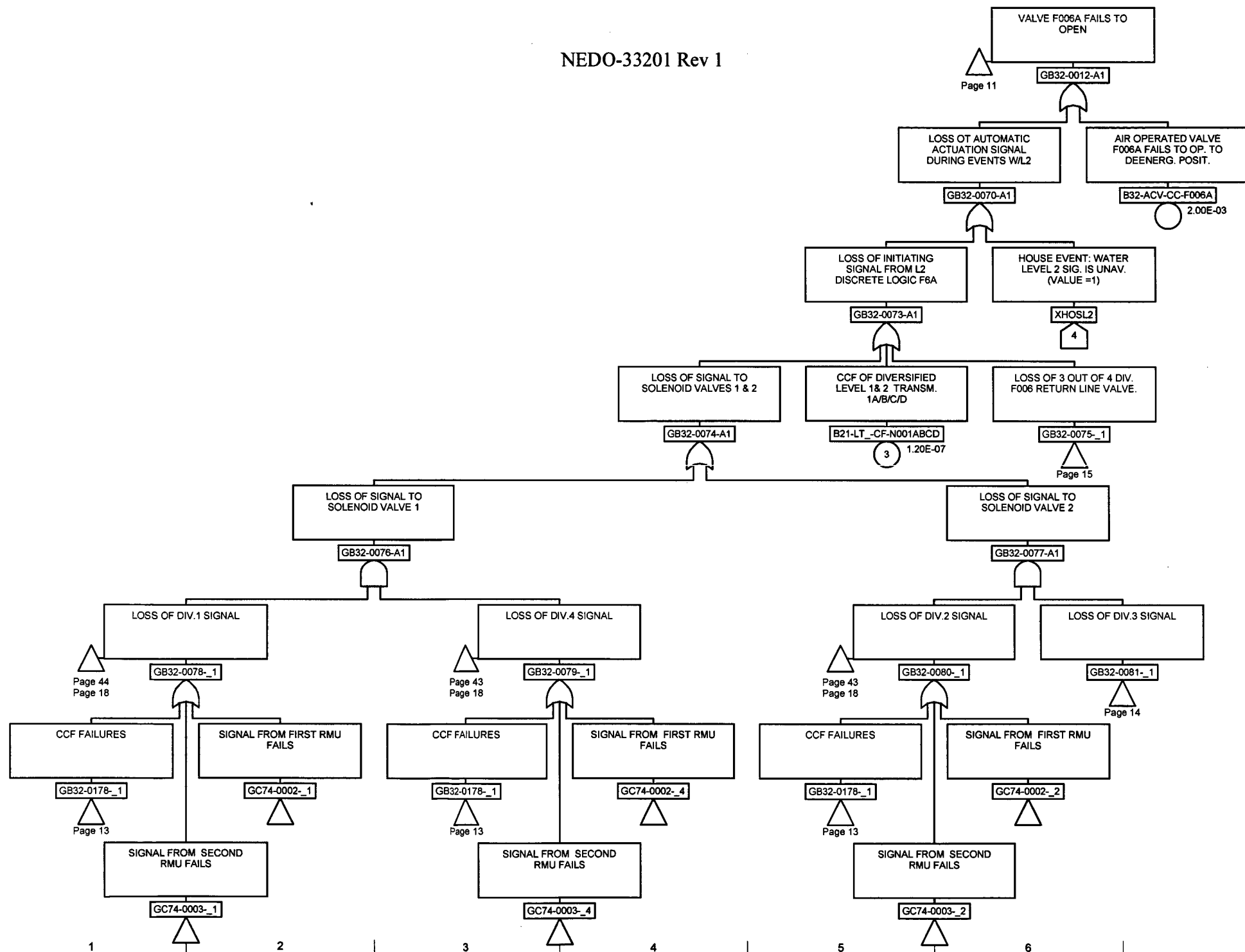


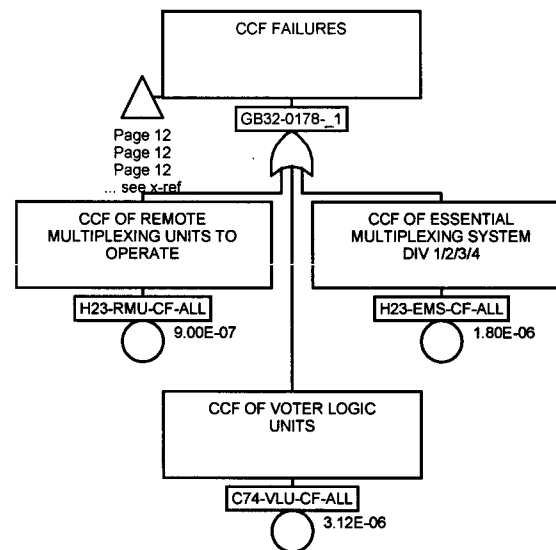


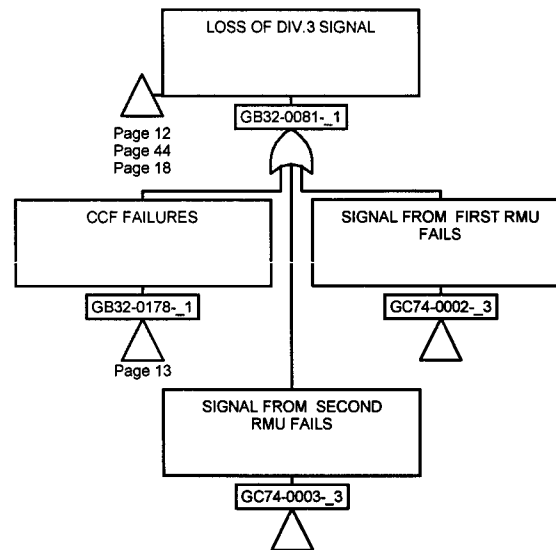


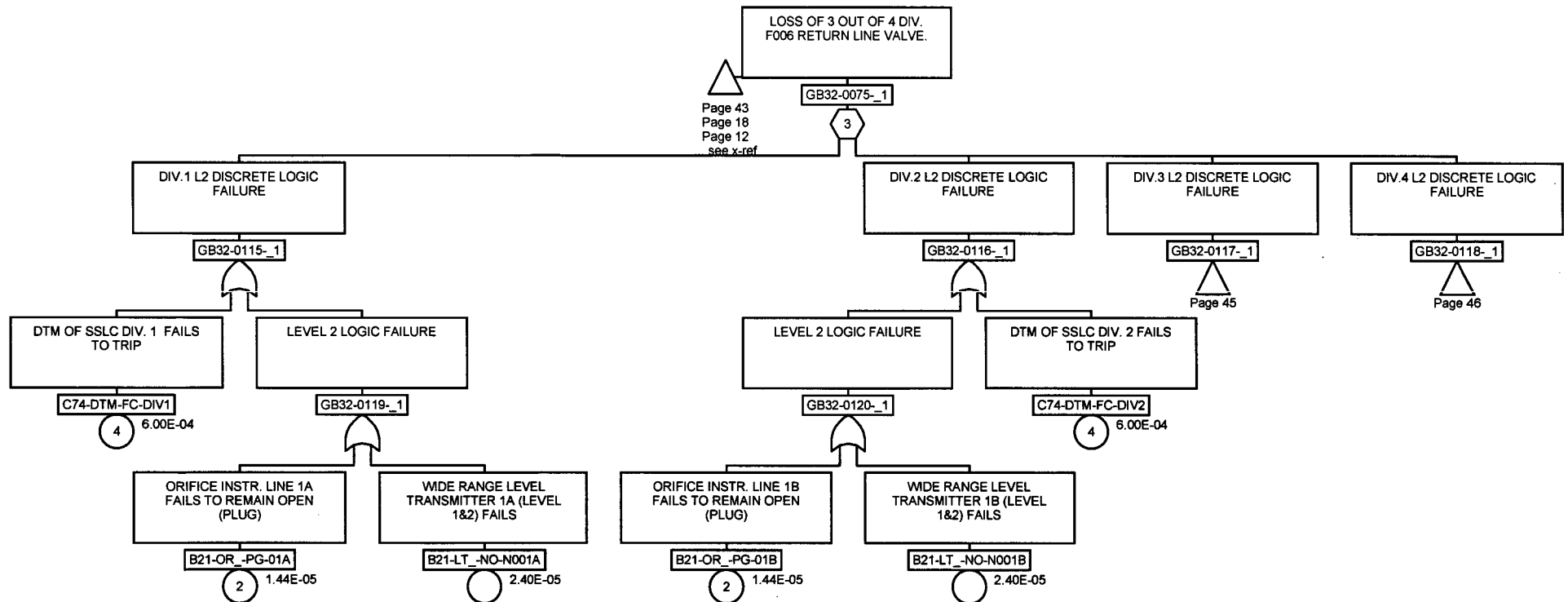


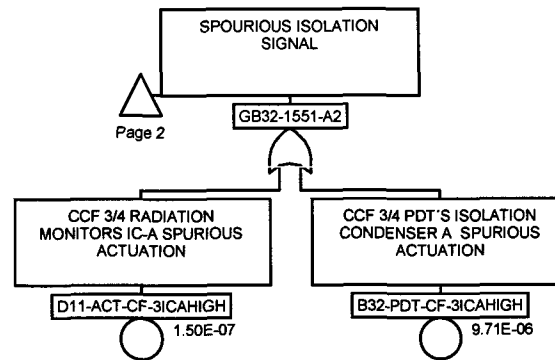




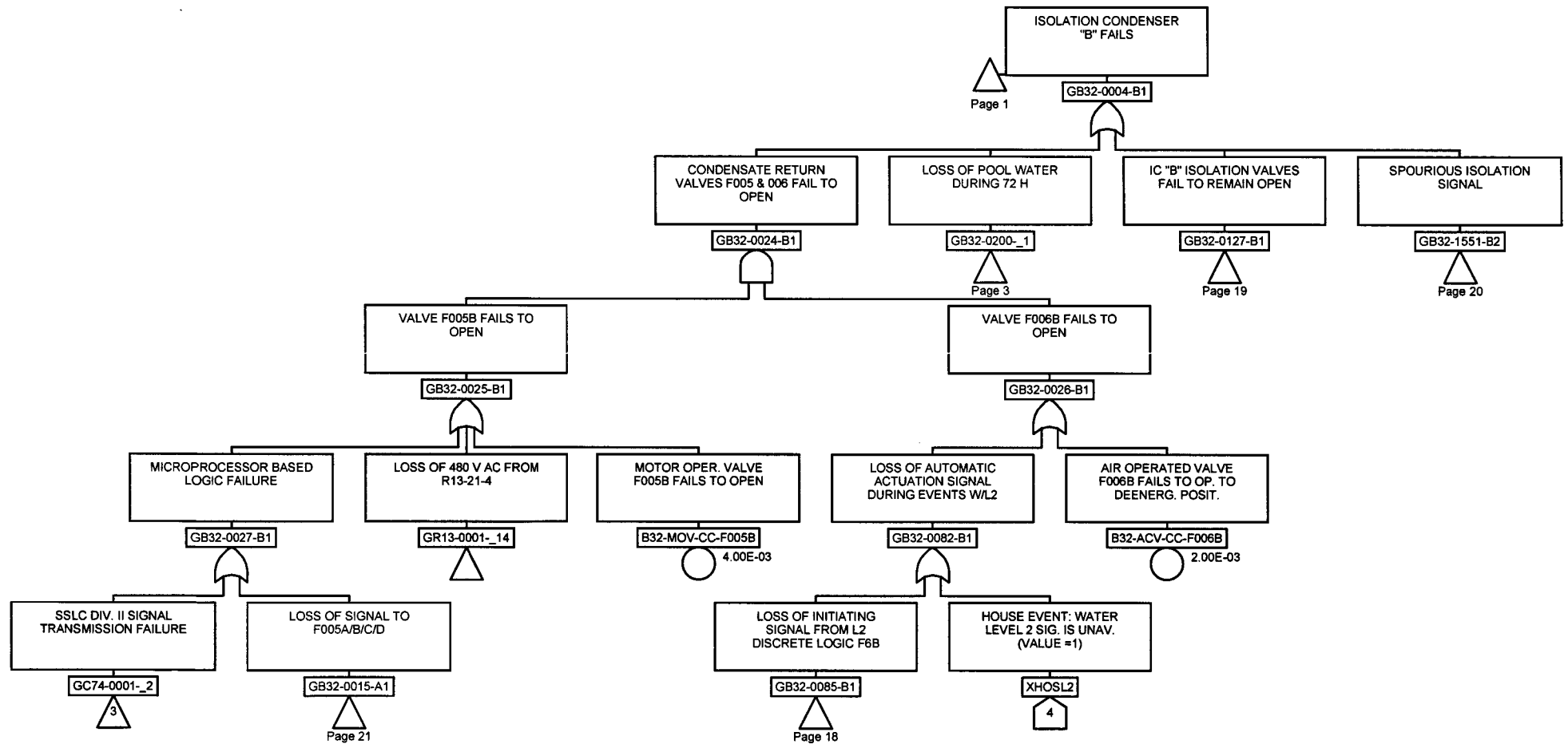


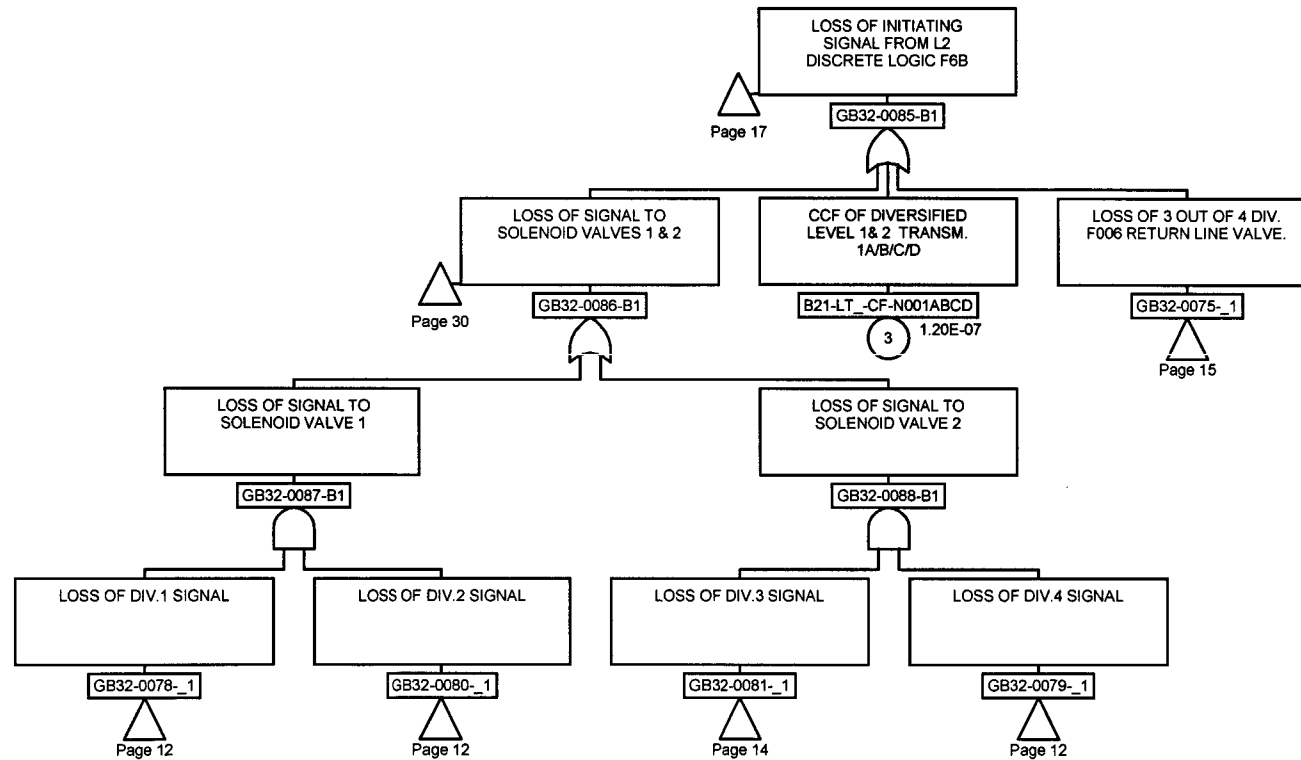


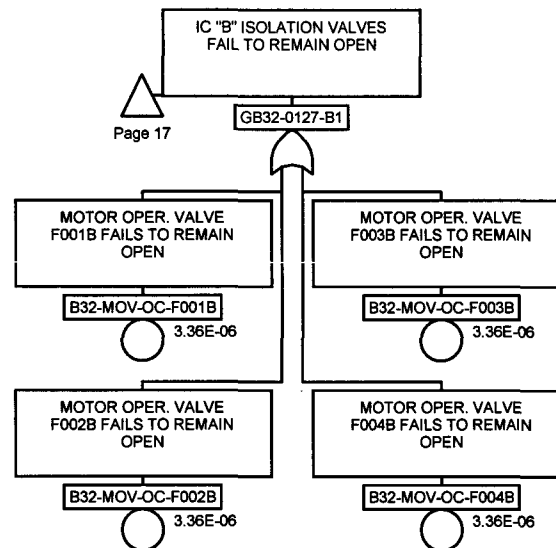


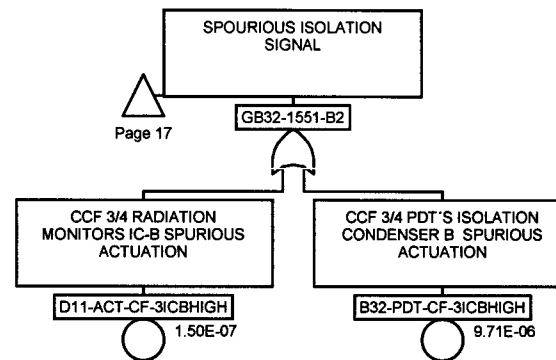


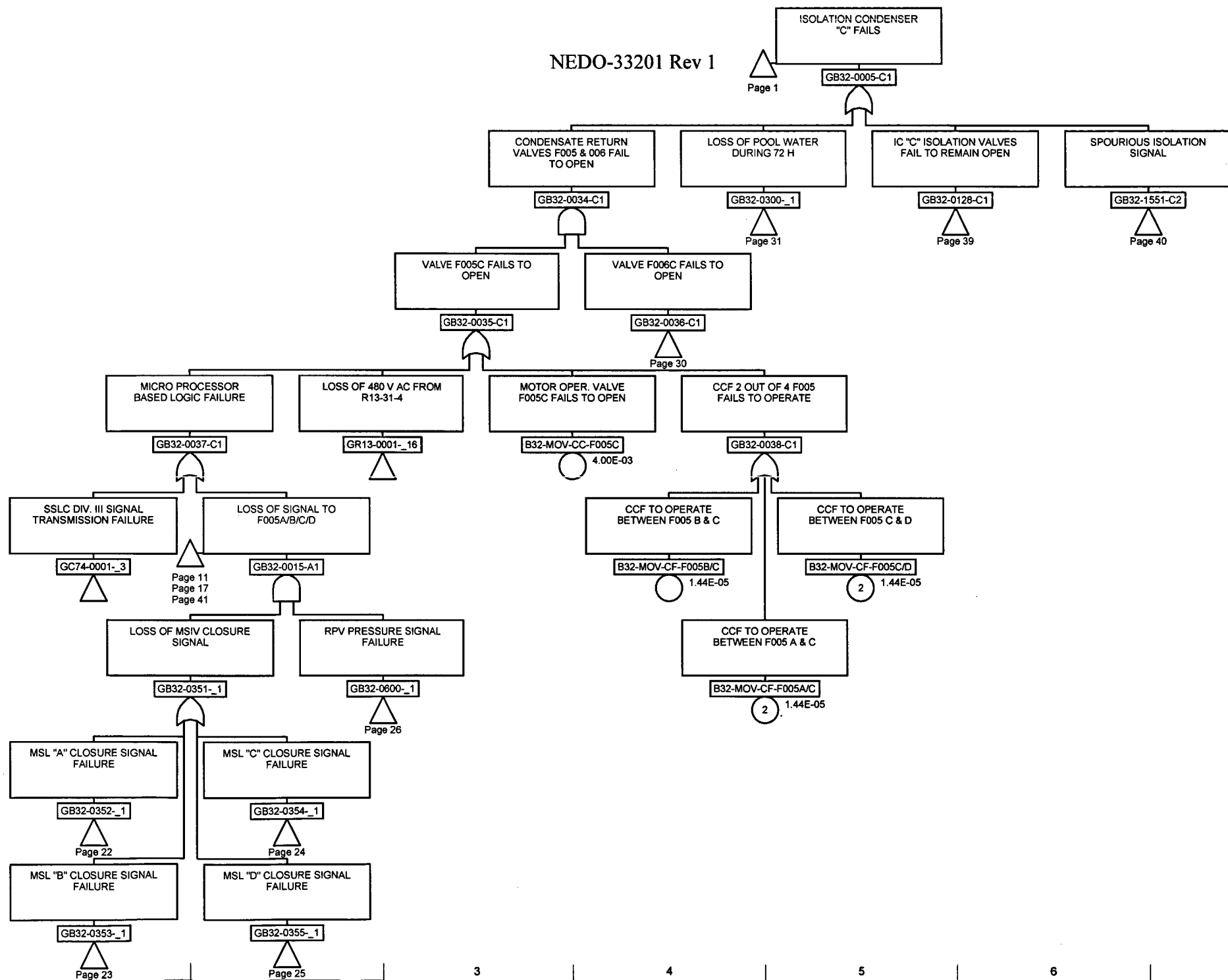
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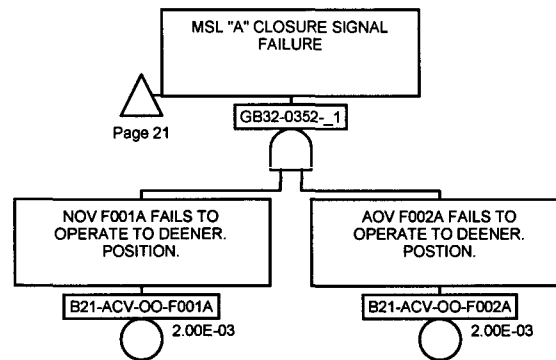


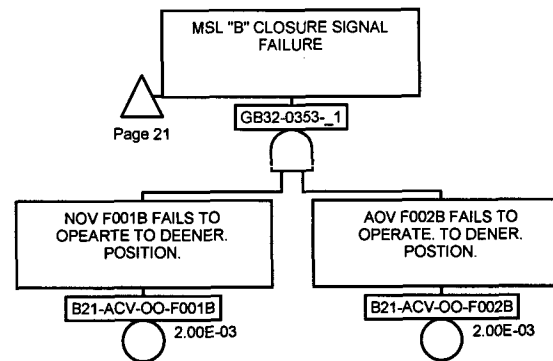


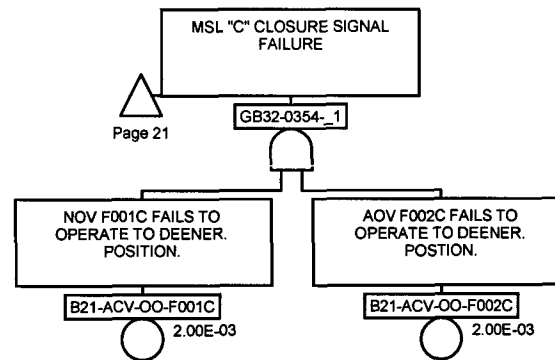




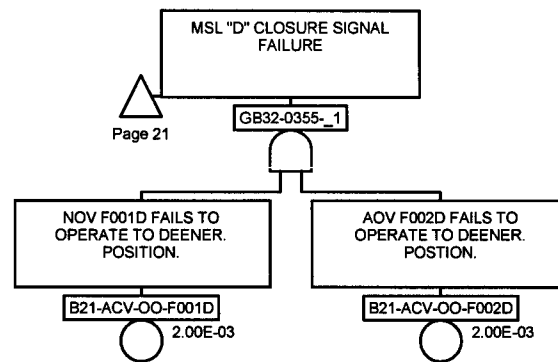


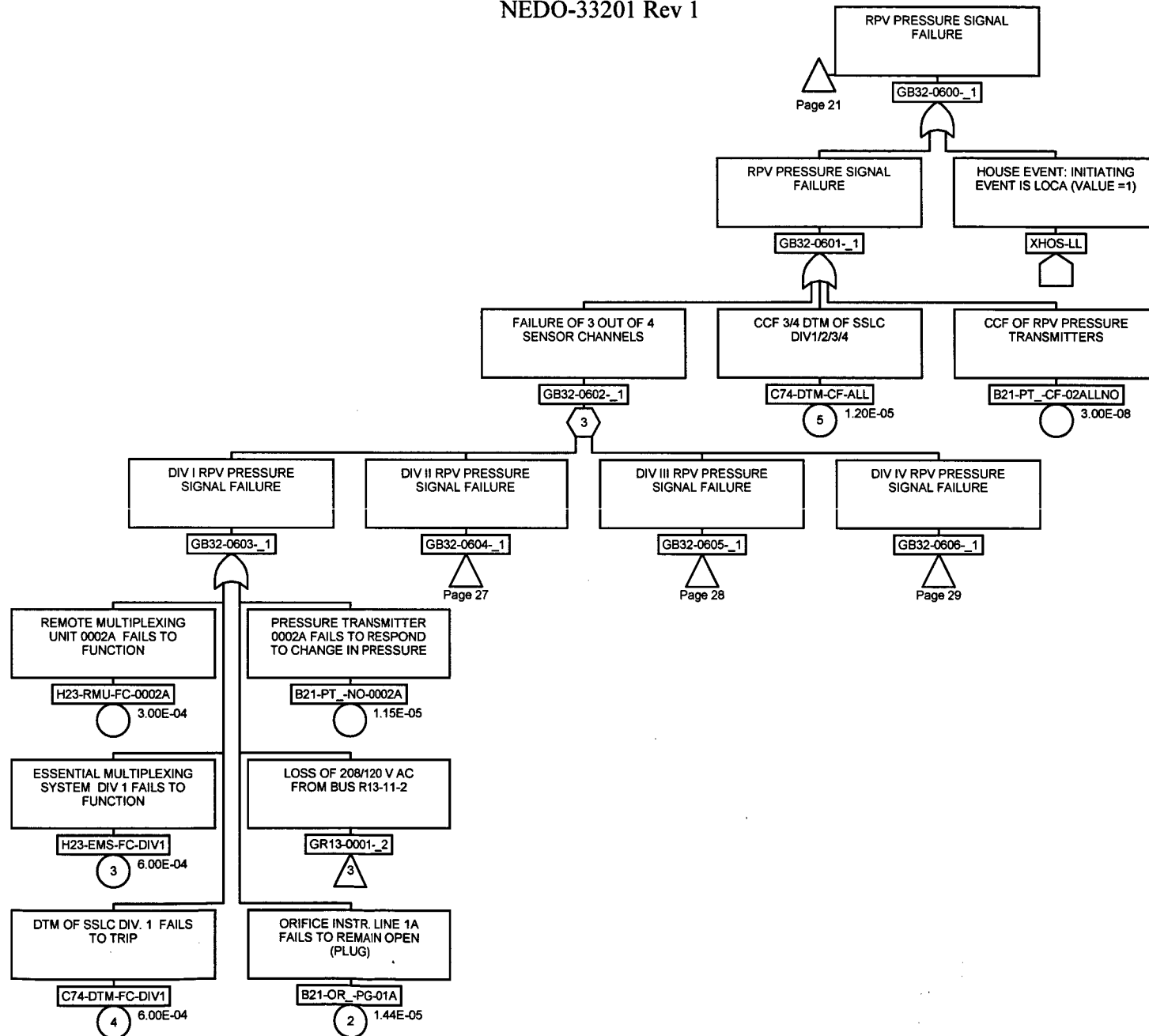


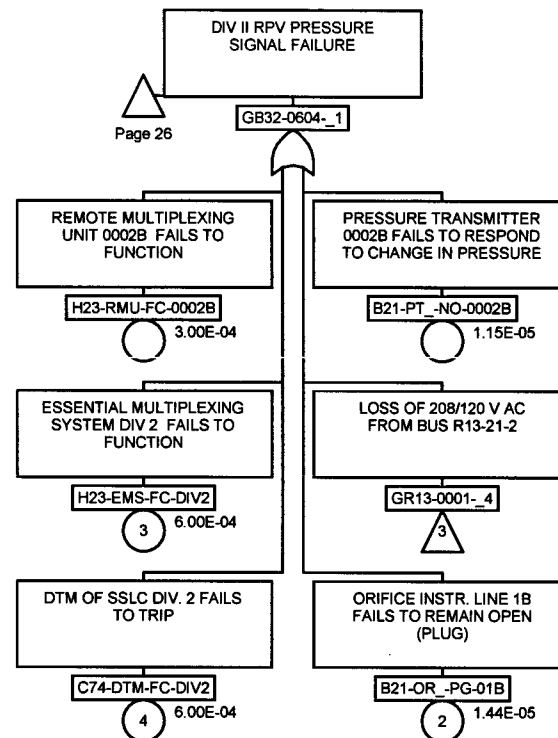


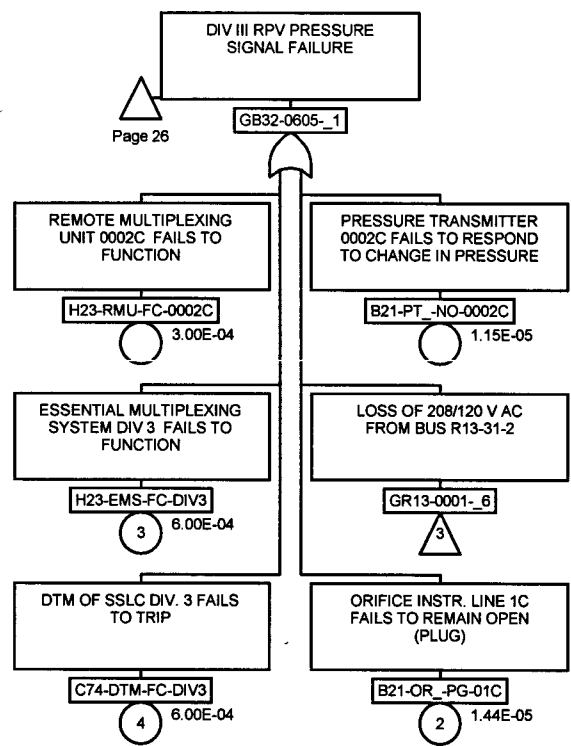


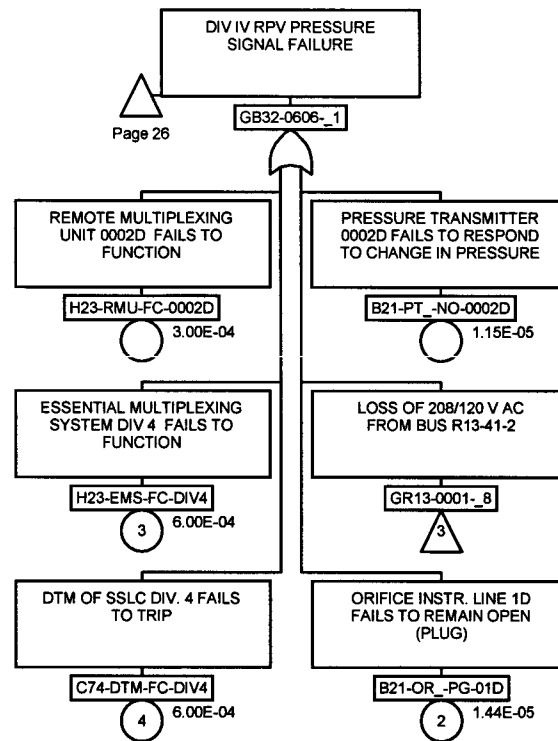
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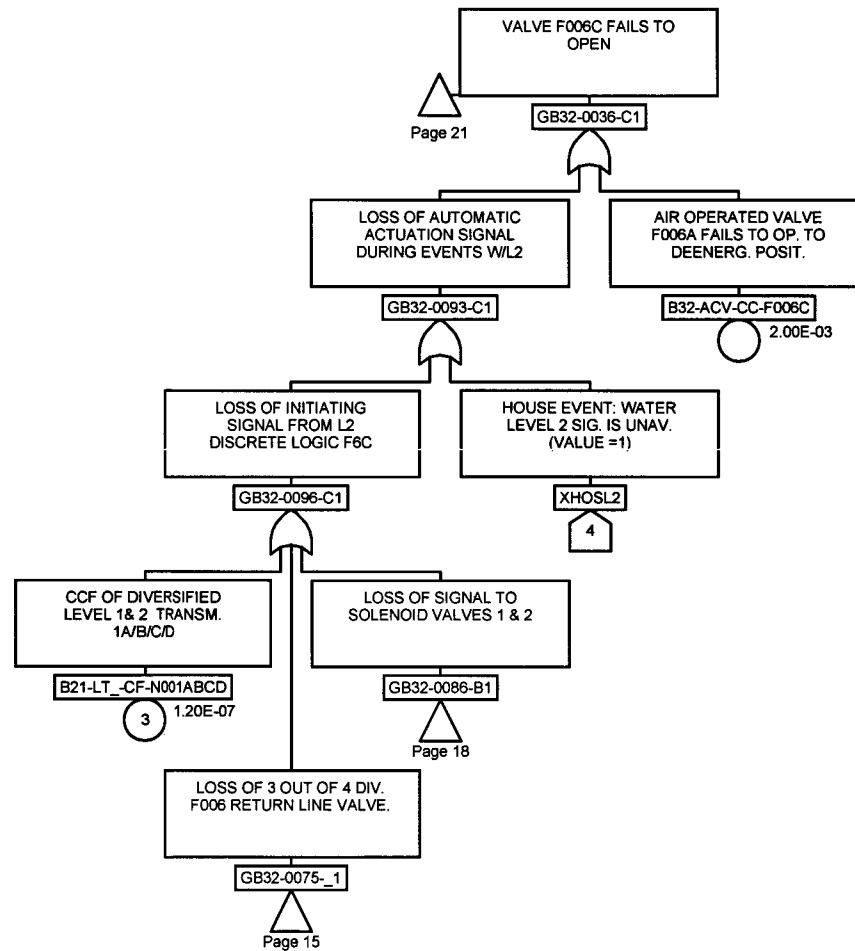


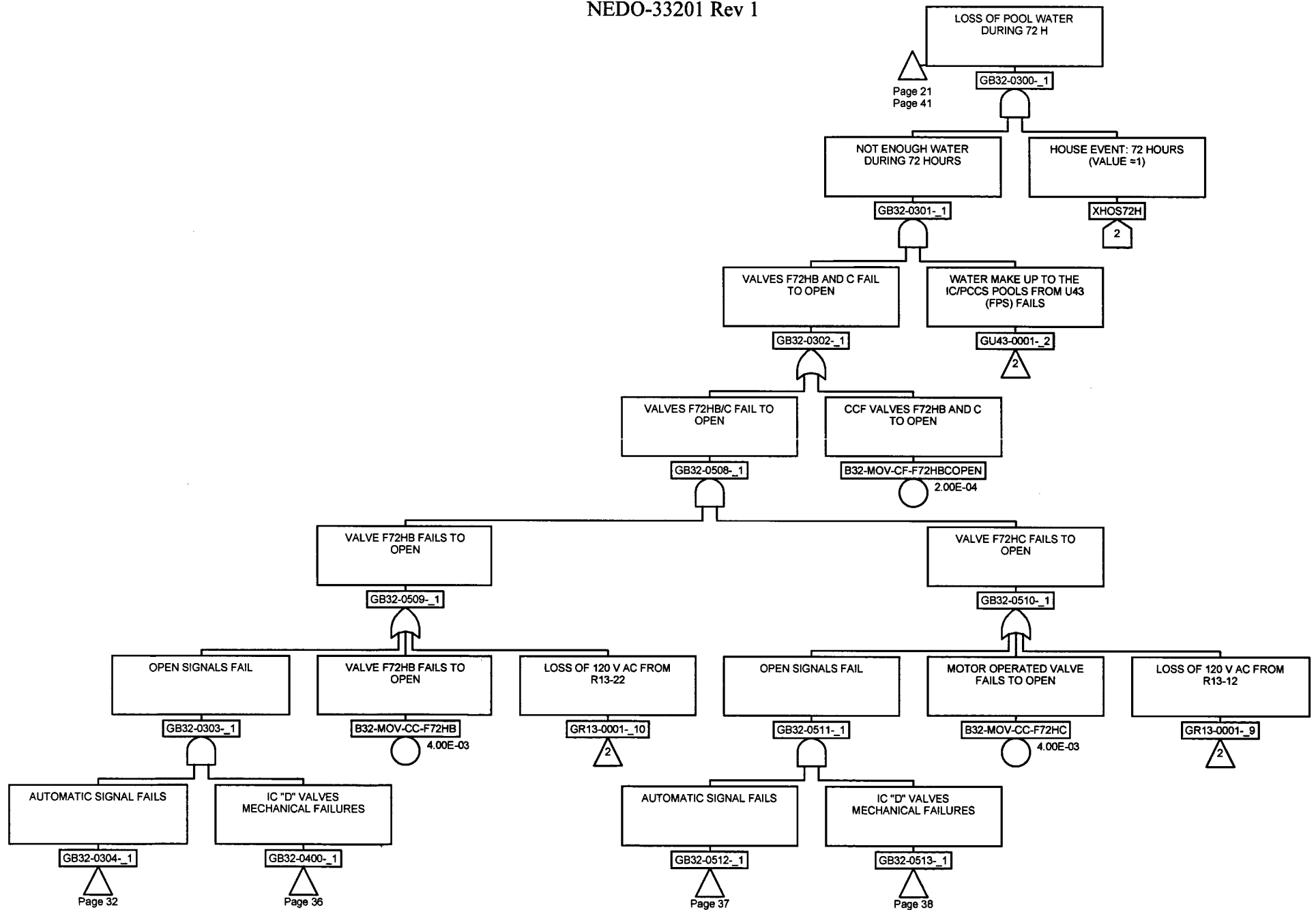


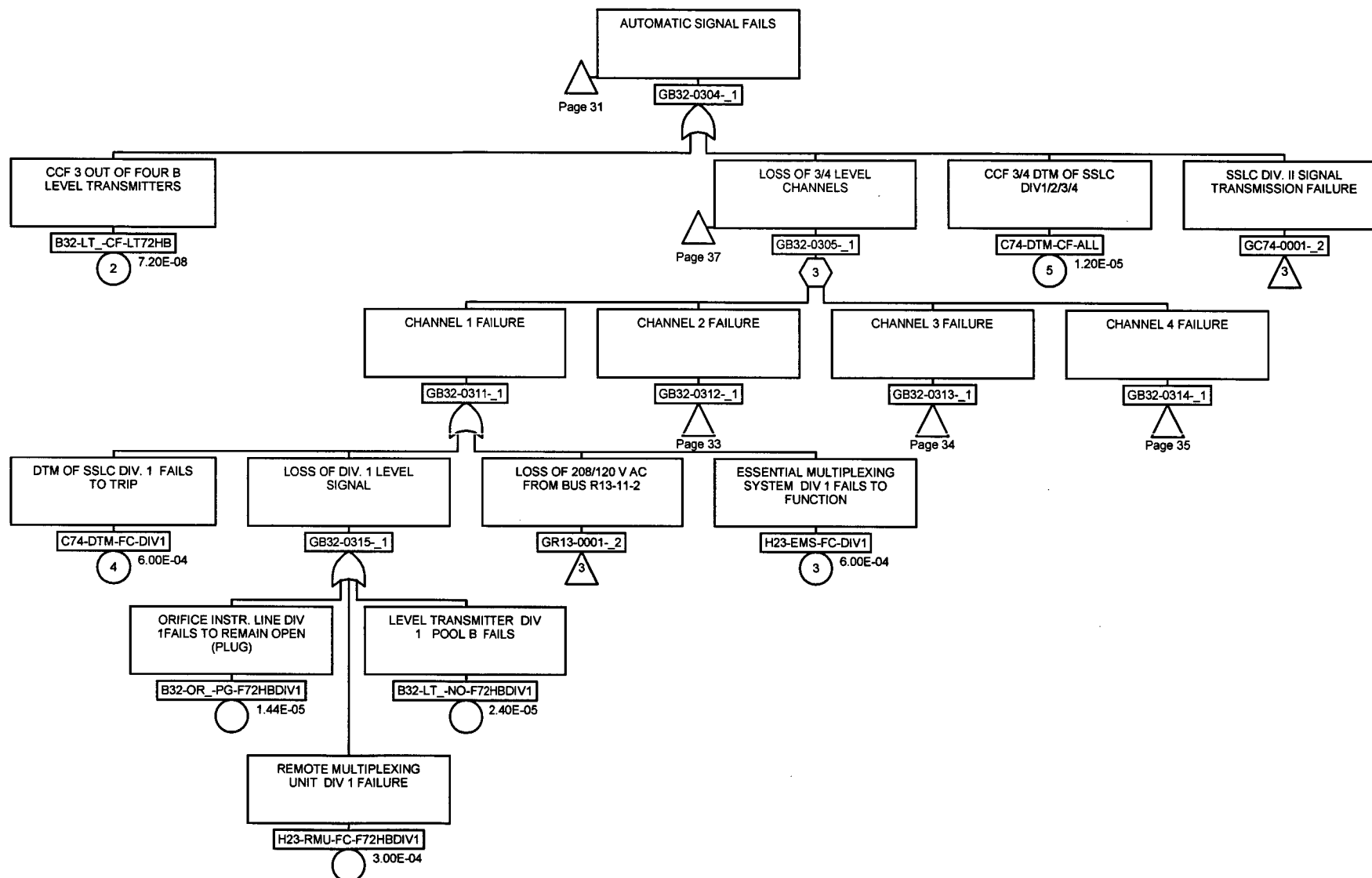


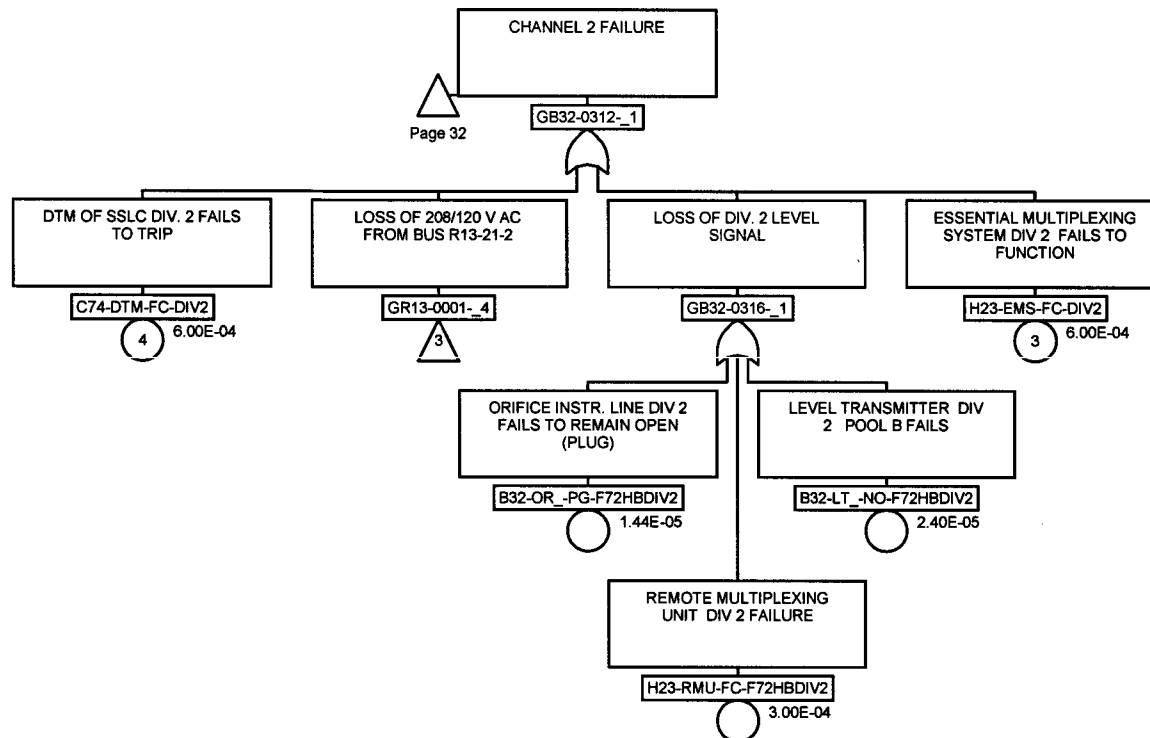


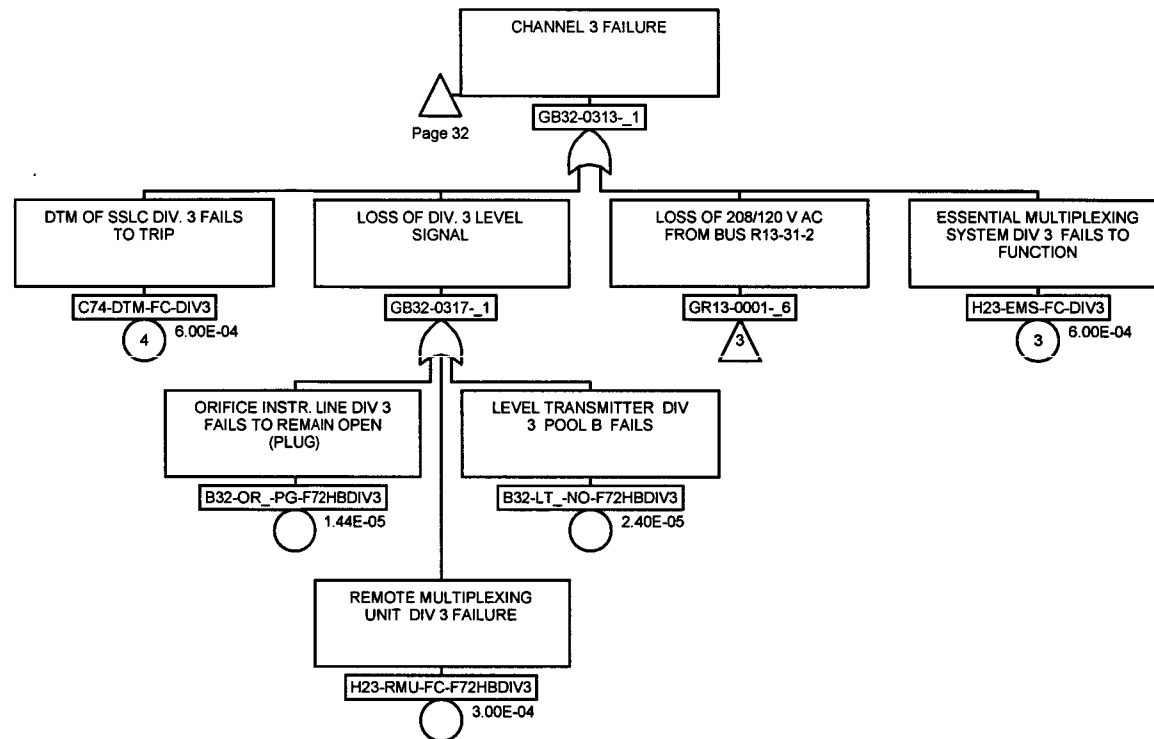


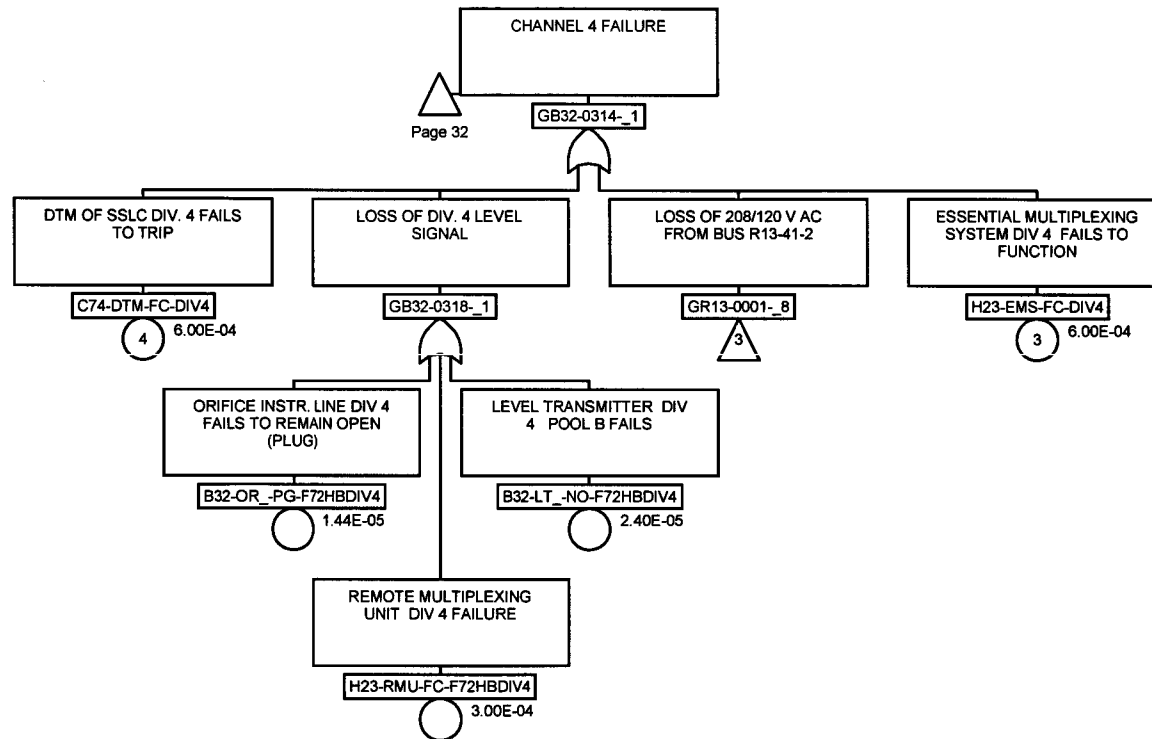


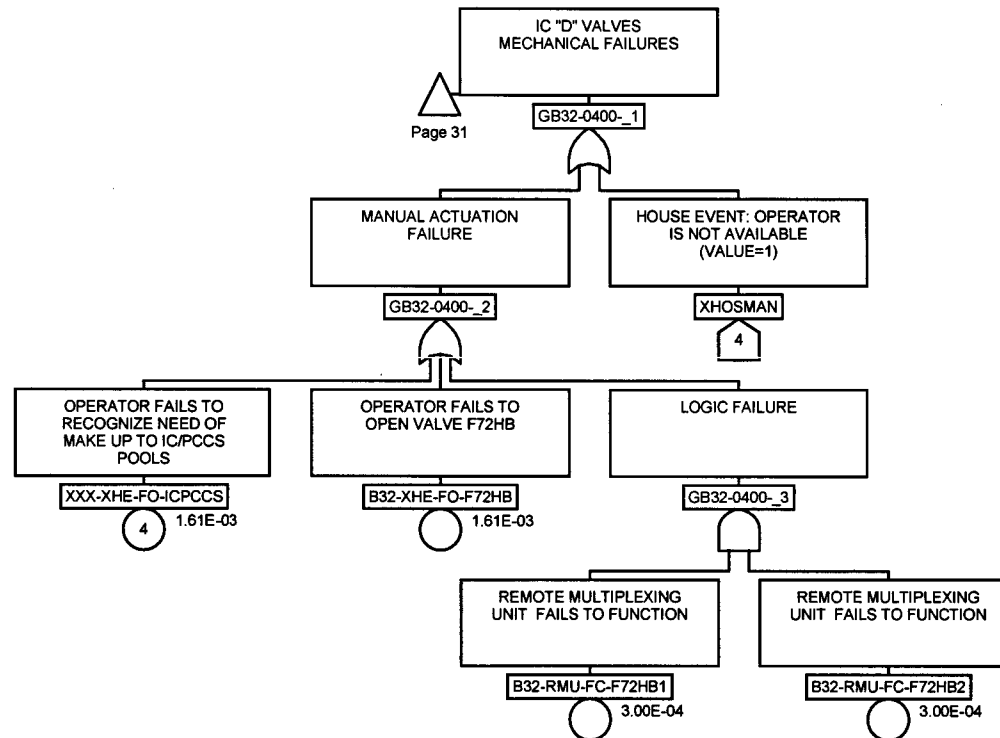


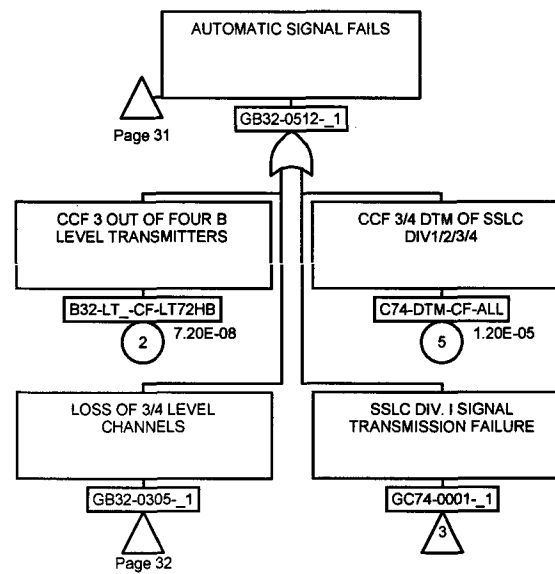


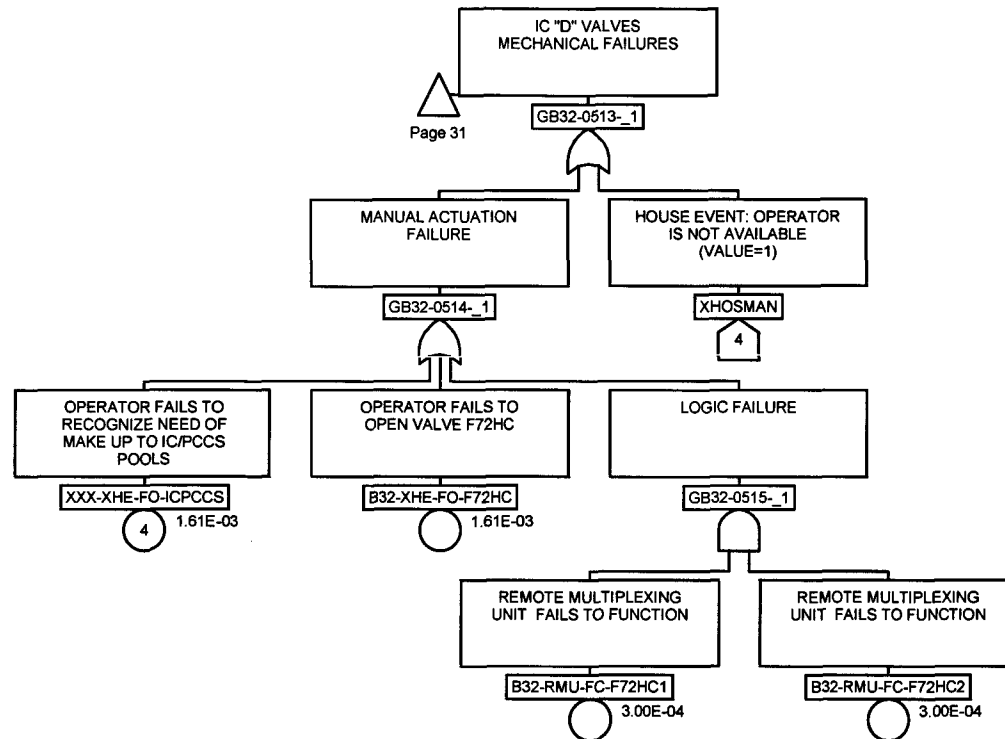


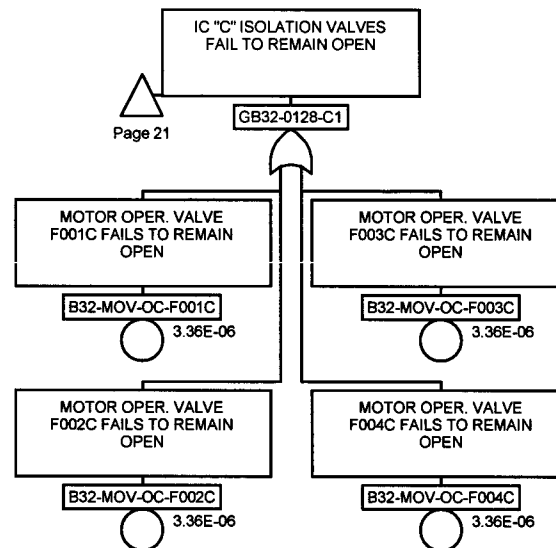


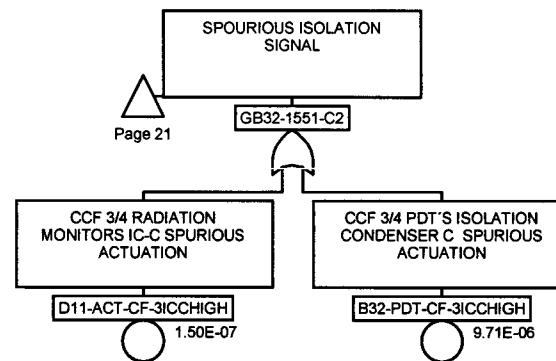


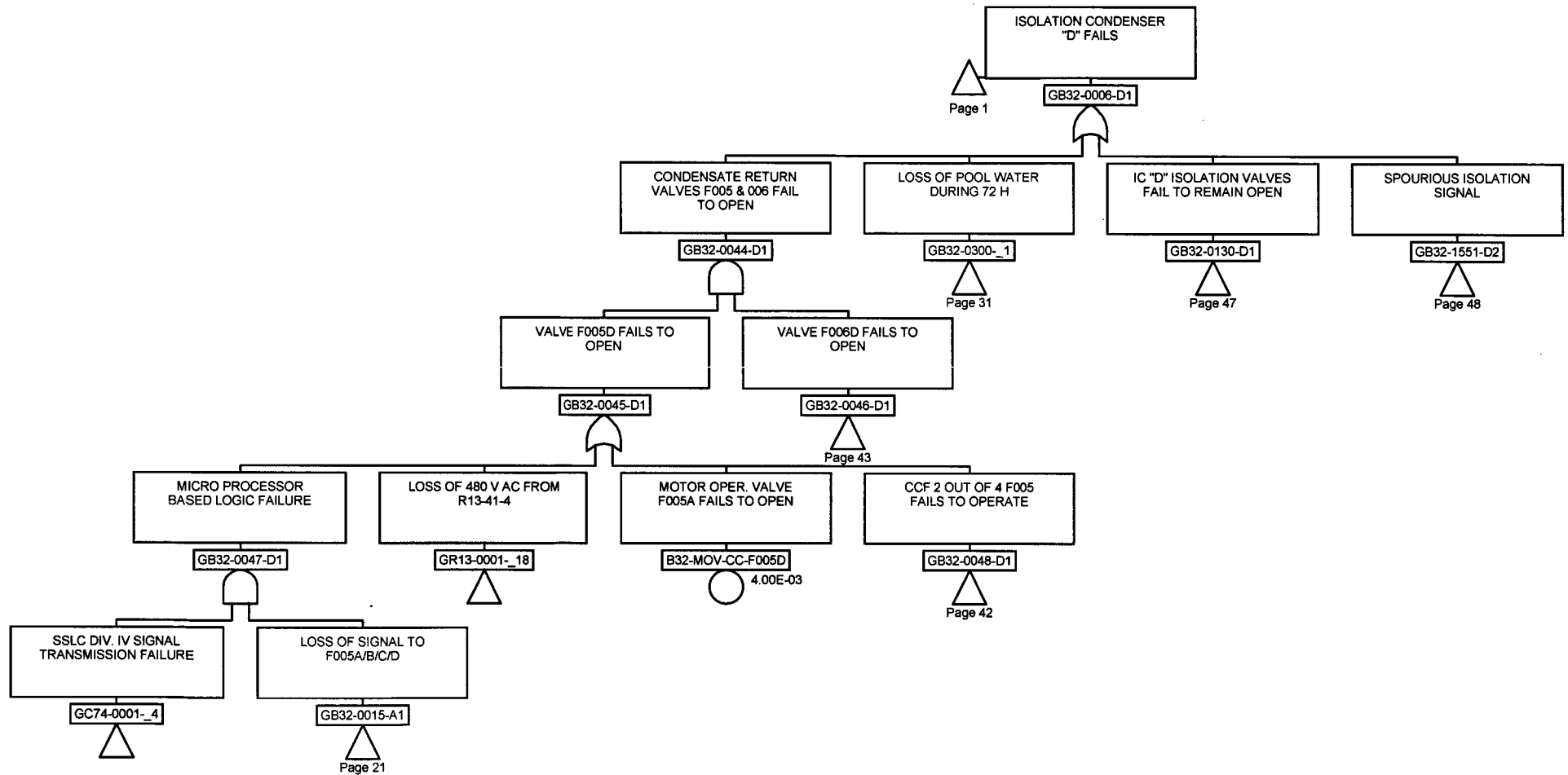


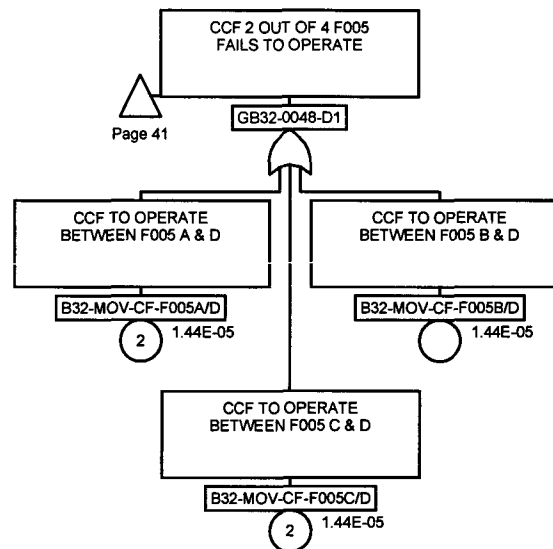


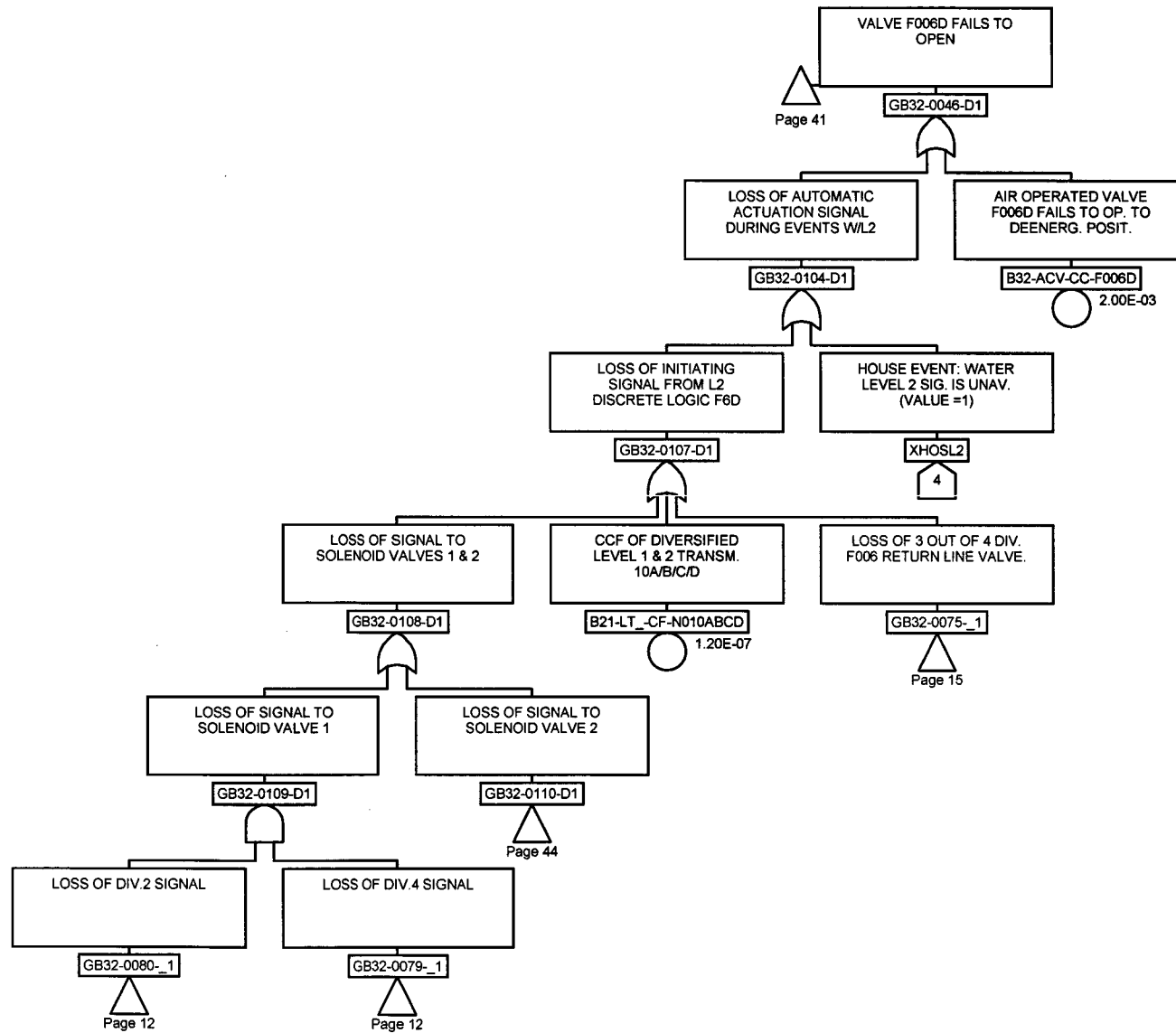


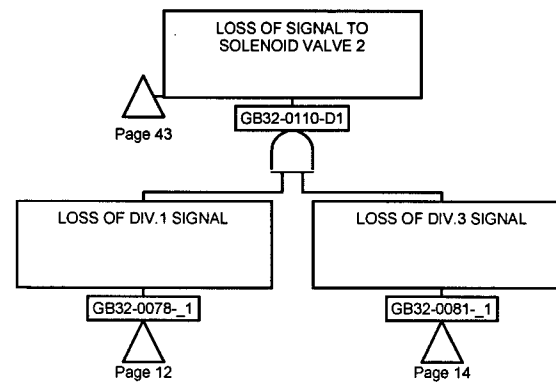


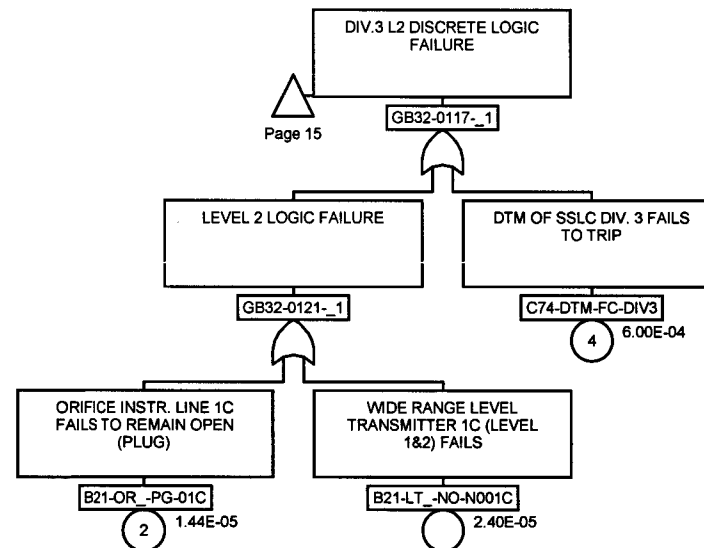


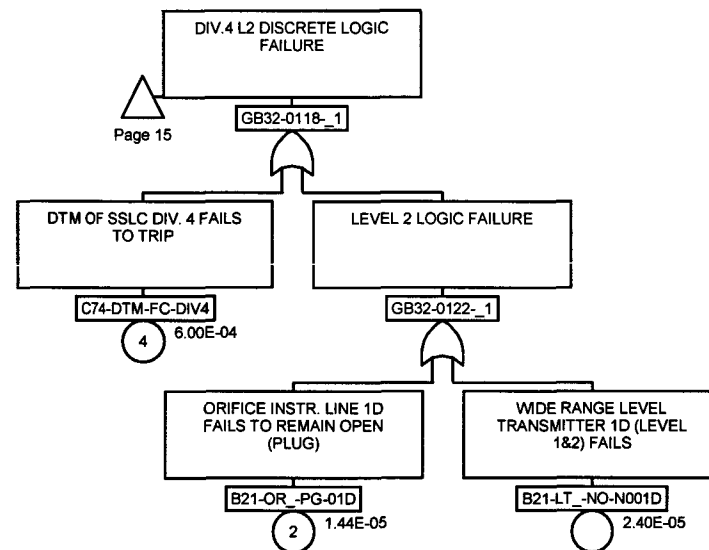


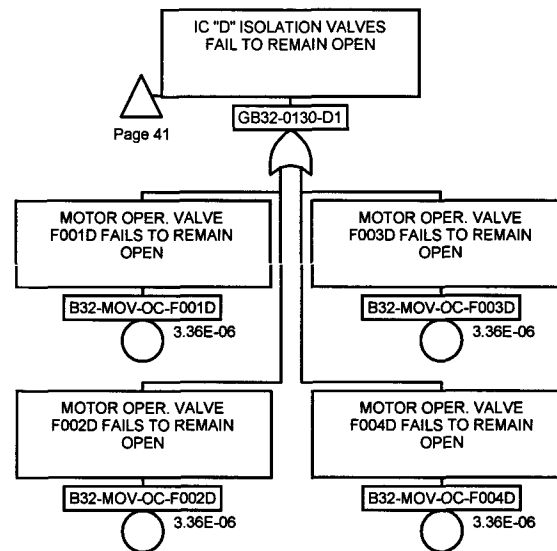


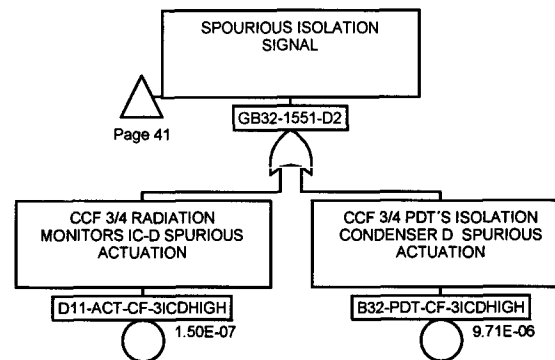












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| NEDO-33201 Rev 1 | | | | | | | | |
| B21-ACV-OO-F001A | 22 | 1 | B32-MOV-CF-2ICABCD | 1 | 5 | C74-DTM-FC-DIV1 | 4 | 1 |
| B21-ACV-OO-F001B | 23 | 1 | B32-MOV-CF-F005A/B | 11 | 4 | C74-DTM-FC-DIV1 | 15 | 1 |
| B21-ACV-OO-F001C | 24 | 1 | B32-MOV-CF-F005A/C | 11 | 5 | C74-DTM-FC-DIV1 | 26 | 1 |
| B21-ACV-OO-F001D | 25 | 1 | B32-MOV-CF-F005A/C | 21 | 5 | C74-DTM-FC-DIV1 | 32 | 1 |
| B21-ACV-OO-F002A | 22 | 2 | B32-MOV-CF-F005A/D | 11 | 5 | C74-DTM-FC-DIV2 | 5 | 1 |
| B21-ACV-OO-F002B | 23 | 2 | B32-MOV-CF-F005A/D | 42 | 1 | C74-DTM-FC-DIV2 | 15 | 5 |
| B21-ACV-OO-F002C | 24 | 2 | B32-MOV-CF-F005B/C | 21 | 4 | C74-DTM-FC-DIV2 | 27 | 1 |
| B21-ACV-OO-F002D | 25 | 2 | B32-MOV-CF-F005B/D | 42 | 2 | C74-DTM-FC-DIV2 | 33 | 1 |
| B21-LT -CF-N001ABCD | 12 | 5 | B32-MOV-CF-F005C/D | 21 | 5 | C74-DTM-FC-DIV3 | 6 | 1 |
| B21-LT -CF-N001ABCD | 18 | 3 | B32-MOV-CF-F005C/D | 42 | 2 | C74-DTM-FC-DIV3 | 28 | 1 |
| B21-LT -CF-N001ABCD | 30 | 1 | B32-MOV-CF-F72HADOPEN | 3 | 5 | C74-DTM-FC-DIV3 | 34 | 1 |
| B21-LT -CF-N010ABCD | 43 | 3 | B32-MOV-CF-F72HBCOPEN | 31 | 5 | C74-DTM-FC-DIV3 | 45 | 3 |
| B21-LT -NO-N001A | 15 | 3 | B32-MOV-OC-F001A | 2 | 3 | C74-DTM-FC-DIV4 | 7 | 1 |
| B21-LT -NO-N001B | 15 | 5 | B32-MOV-OC-F001B | 19 | 1 | C74-DTM-FC-DIV4 | 29 | 1 |
| B21-LT -NO-N001C | 45 | 2 | B32-MOV-OC-F001C | 39 | 1 | C74-DTM-FC-DIV4 | 35 | 1 |
| B21-LT -NO-N001D | 46 | 3 | B32-MOV-OC-F001D | 47 | 1 | C74-DTM-FC-DIV4 | 46 | 1 |
| B21-OR -PG-01A | 15 | 2 | B32-MOV-OC-F002A | 2 | 3 | C74-VLU-CF-ALL | 13 | 2 |
| B21-OR -PG-01A | 26 | 2 | B32-MOV-OC-F002B | 19 | 1 | D11-ACT-CF-3ICAHIGH | 16 | 1 |
| B21-OR -PG-01B | 15 | 4 | B32-MOV-OC-F002C | 39 | 1 | D11-ACT-CF-3ICBHIGH | 20 | 1 |
| B21-OR -PG-01B | 27 | 2 | B32-MOV-OC-F002D | 47 | 1 | D11-ACT-CF-3ICCHIGH | 40 | 1 |
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| B21-PT -CF-02ALLNO | 26 | 5 | B32-MOV-OC-F004A | 2 | 4 | GB32-0004-B1 | 17 | 5 |
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| B32-MOV-CC-F005D | 41 | 4 | C74-DTM-CF-ALL | 4 | 5 | GB32-0038-C1 | 21 | 5 |
| B32-MOV-CC-F72HA | 3 | 3 | C74-DTM-CF-ALL | 9 | 2 | GB32-0044-D1 | 41 | 3 |
| B32-MOV-CC-F72HB | 31 | 3 | C74-DTM-CF-ALL | 26 | 4 | GB32-0045-D1 | 41 | 3 |
| B32-MOV-CC-F72HC | 31 | 6 | C74-DTM-CF-ALL | 32 | 5 | GB32-0046-D1 | 41 | 4 |
| B32-MOV-CC-F72HD | 3 | 6 | C74-DTM-CF-ALL | 37 | 2 | GB32-0046-D1 | 43 | 4 |

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| GB32-0047-D1 | 41 | 2 | GB32-0128-C1 | 21 | 5 | GB32-0318- 1 | 35 | 2 |
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| GB32-0117- 1 | 45 | 2 | GB32-0311- 1 | 32 | 3 | GB32-0605- 1 | 26 | 4 |
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| | | | NEDO-33201 Rev 1 | | | | | |
| GB32-1551-D2 | 41 | 6 | H23-RMU-CF-ALL | 13 | 1 | | | |
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| GC74-0002- 2 | 12 | 6 | H23-RMU-FC-F72HBDIV2 | 33 | 3 | | | |
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| GC74-0003- 1 | 12 | 2 | H23-RMU-FC-F72HD1 | 10 | 3 | | | |
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| GR13-0001- 12 | 11 | 3 | XHOSICLOCAOUT | 1 | 2 | | | |
| GR13-0001- 14 | 17 | 3 | XHOSL2 | 12 | 6 | | | |
| GR13-0001- 16 | 21 | 3 | XHOSL2 | 17 | 5 | | | |
| GR13-0001- 18 | 41 | 3 | XHOSL2 | 30 | 3 | | | |
| GR13-0001- 2 | 4 | 3 | XHOSL2 | 43 | 4 | | | |
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| GR13-0001- 2 | 32 | 3 | XHOSMAN | 10 | 3 | | | |
| GR13-0001- 4 | 5 | 3 | XHOSMAN | 36 | 3 | | | |
| GR13-0001- 4 | 27 | 2 | XHOSMAN | 38 | 3 | | | |
| GR13-0001- 4 | 33 | 2 | XXX-POL-RP-IC/PCC | 1 | 6 | | | |
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| GR13-0001- 9 | 3 | 4 | | | | | | |
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| GU43-0001- 2 | 3 | 5 | | | | | | |
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| H23-EMS-FC-DIV3 | 6 | 4 | | | | | | |
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| H23-EMS-FC-DIV3 | 34 | 4 | | | | | | |
| H23-EMS-FC-DIV4 | 7 | 4 | | | | | | |
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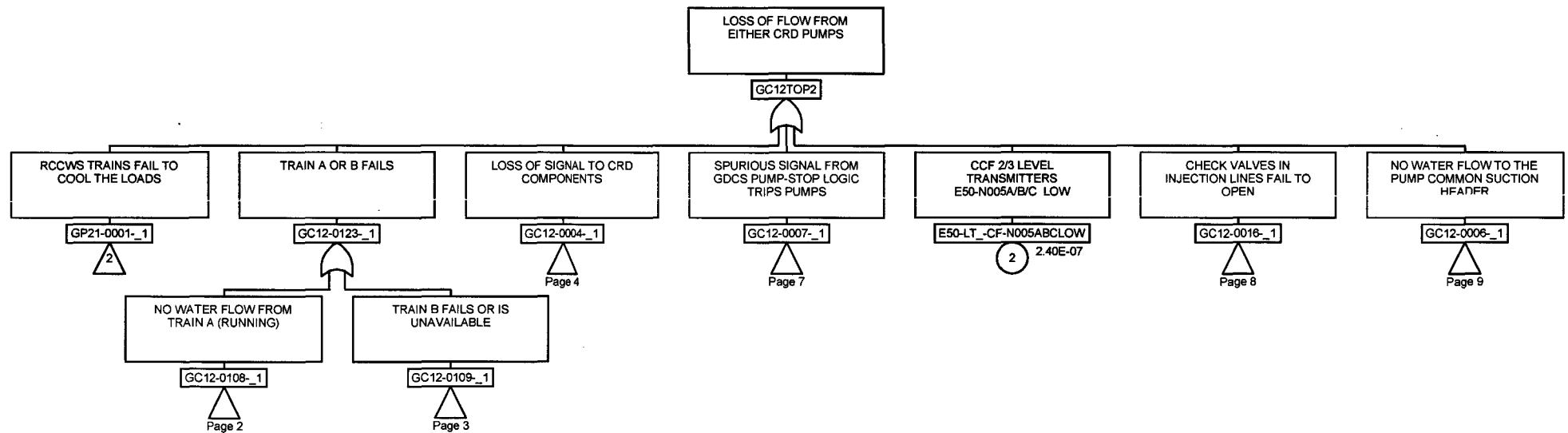
B32 - Isolation Condensers

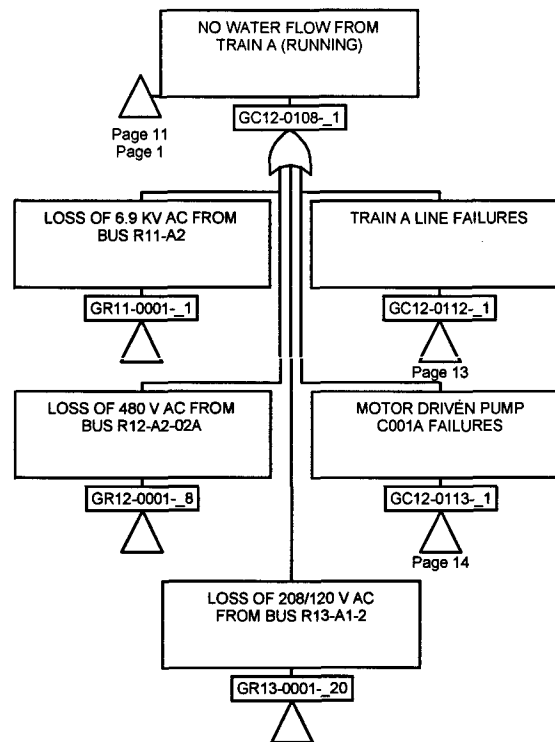
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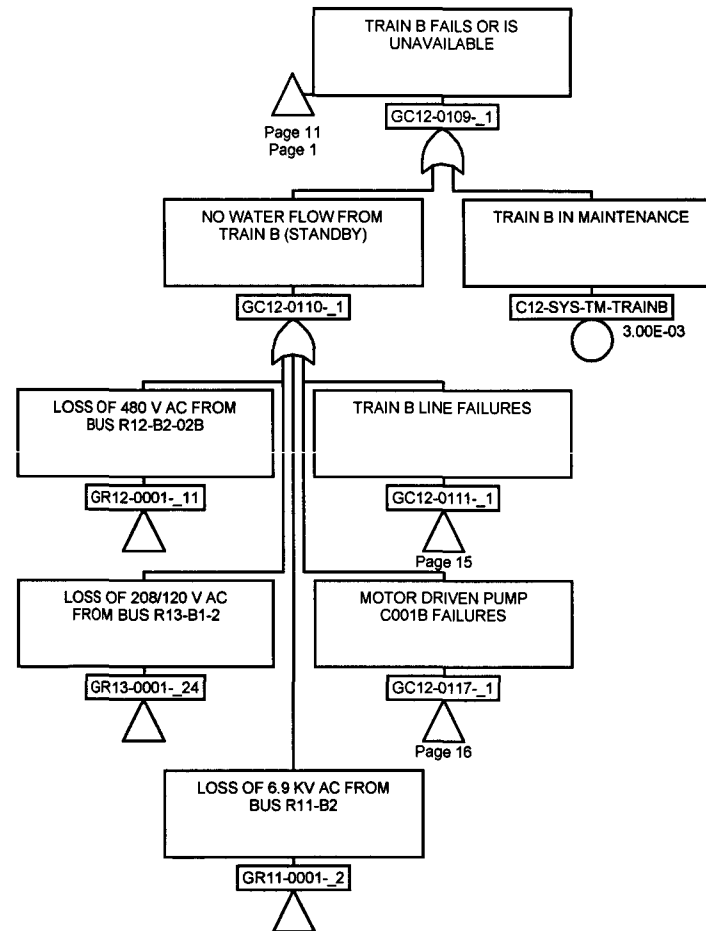
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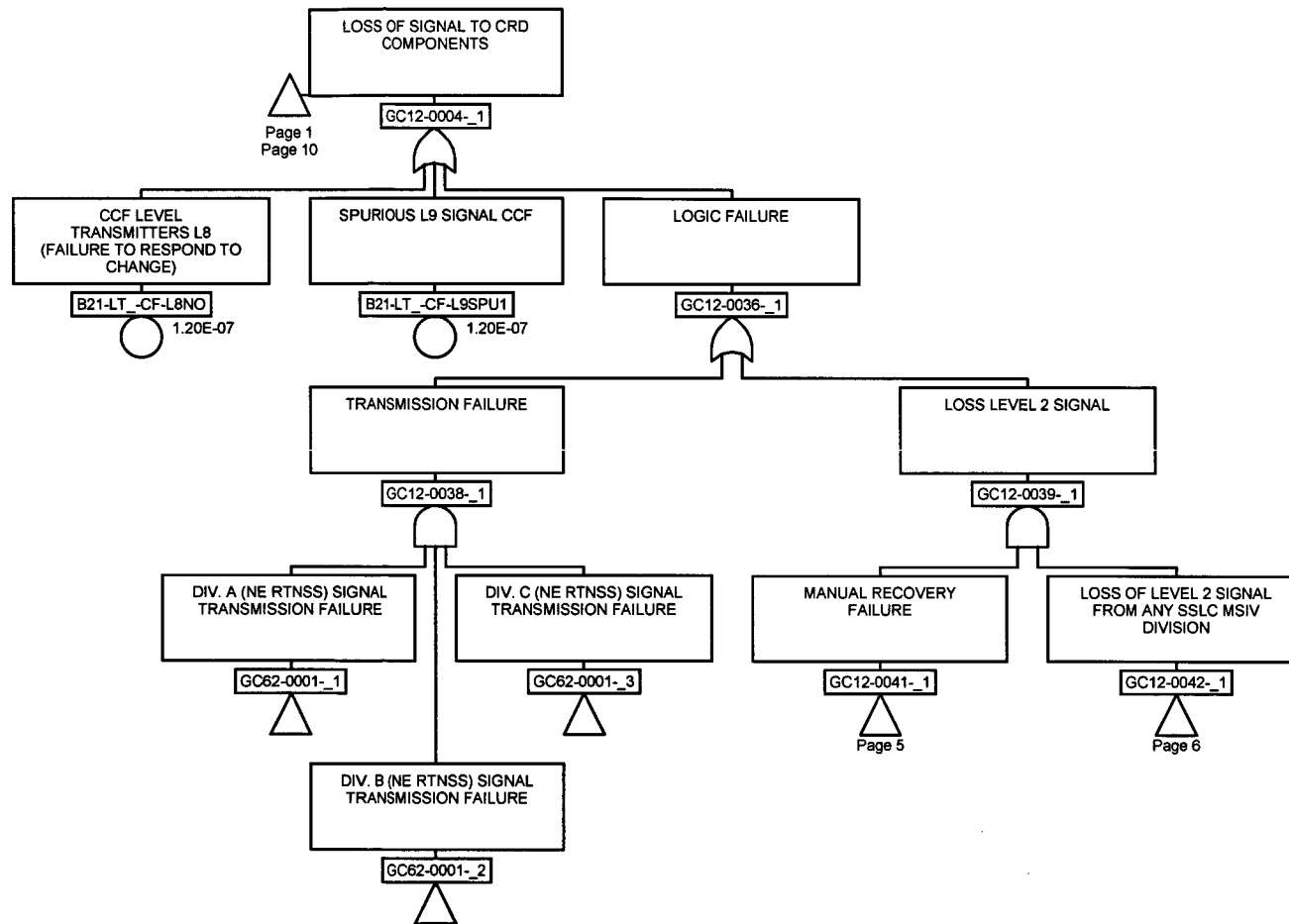
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Appendix B.4.3
Control Rod Drive System
Fault Tree

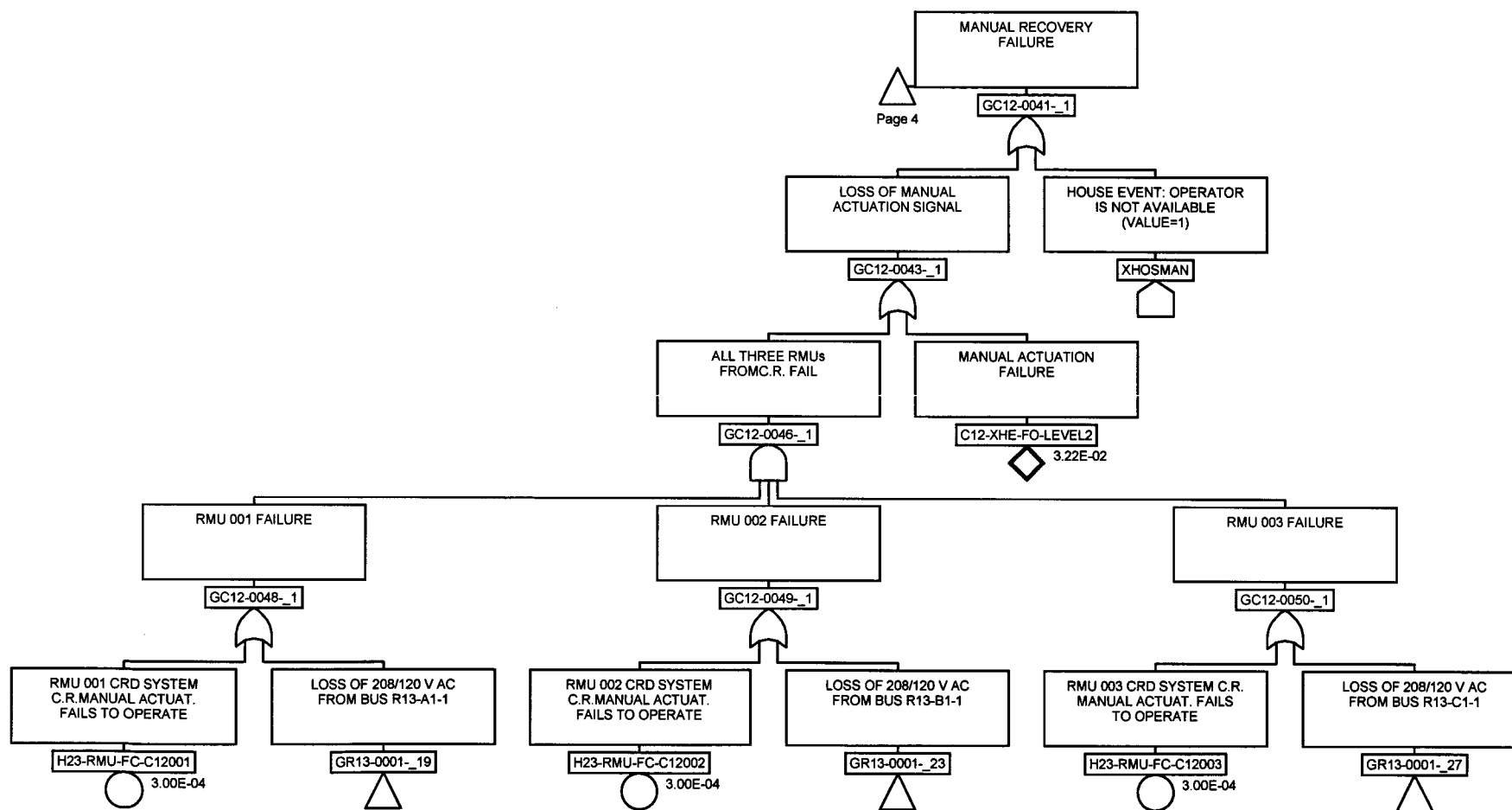


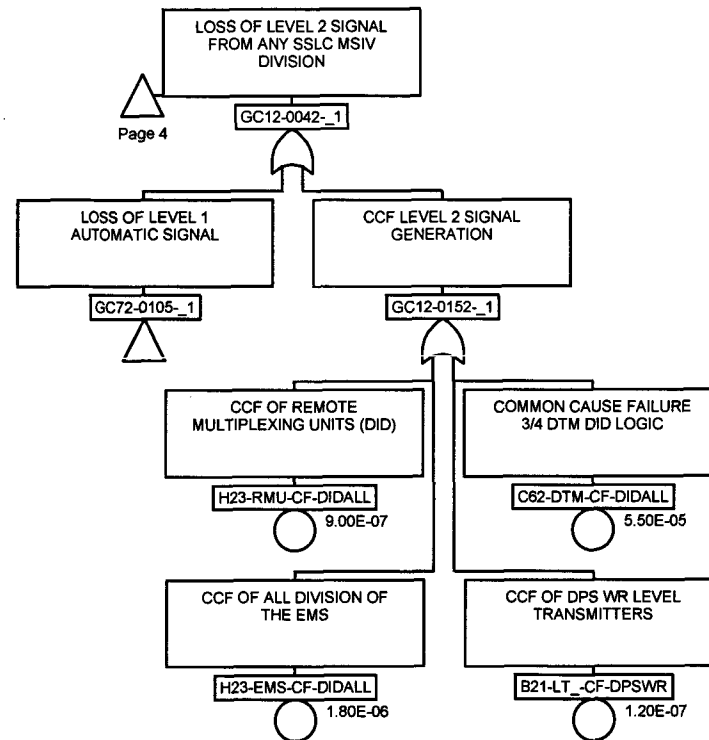


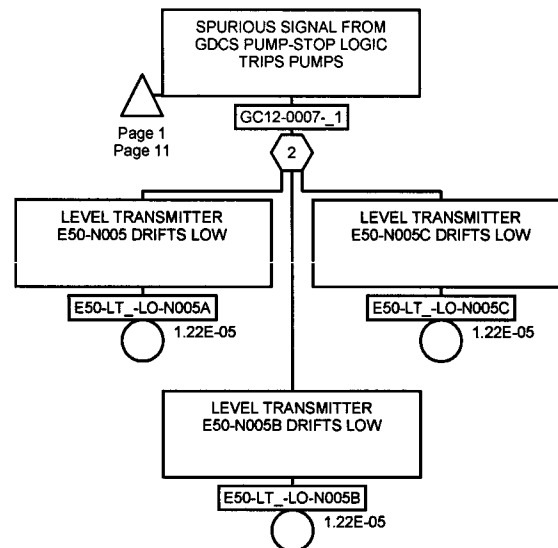


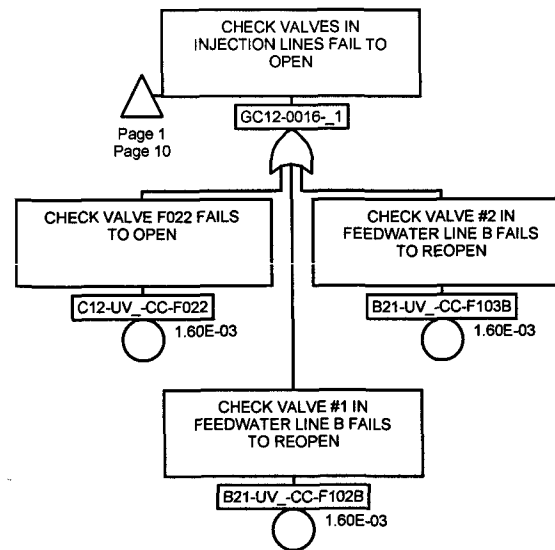


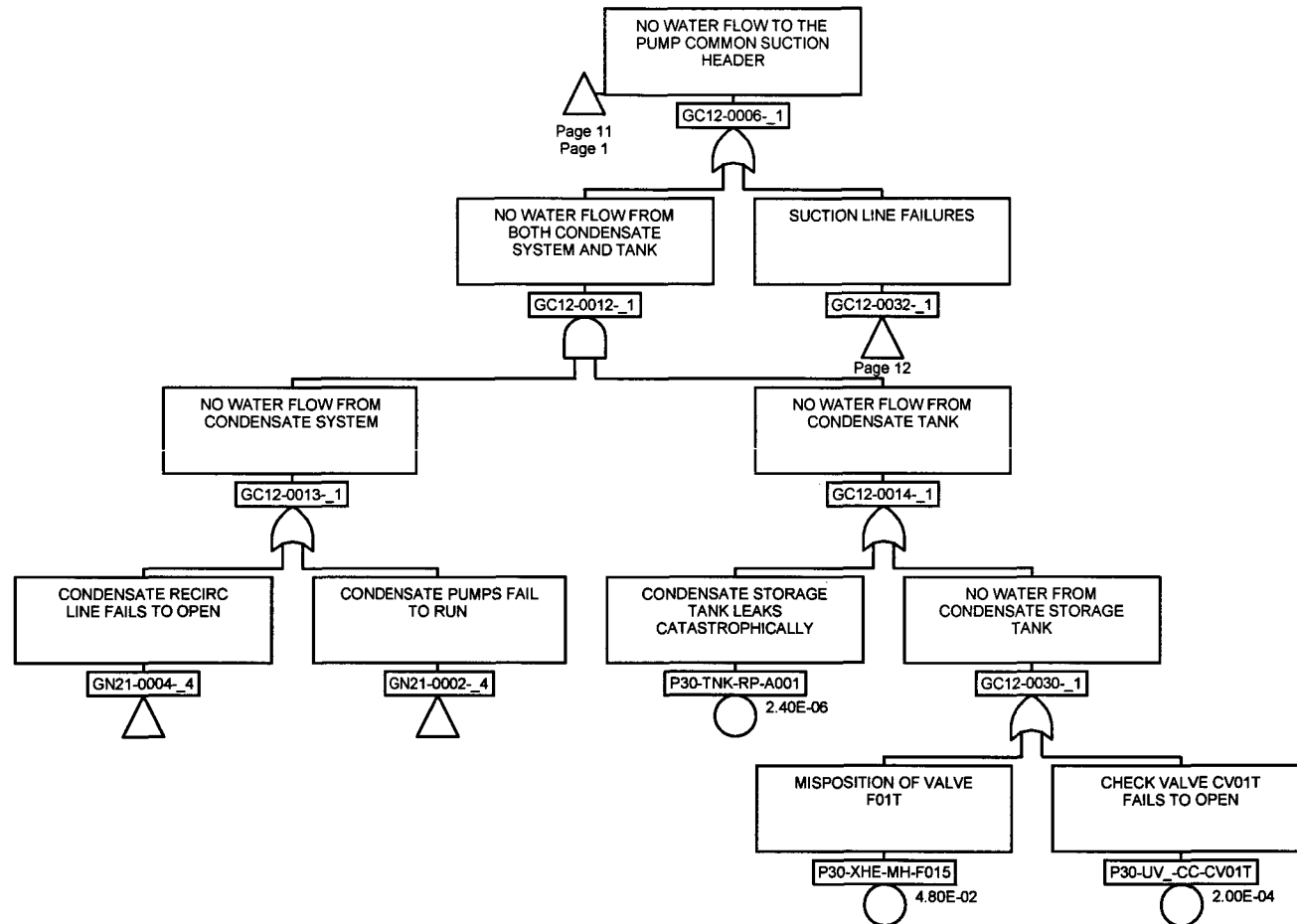
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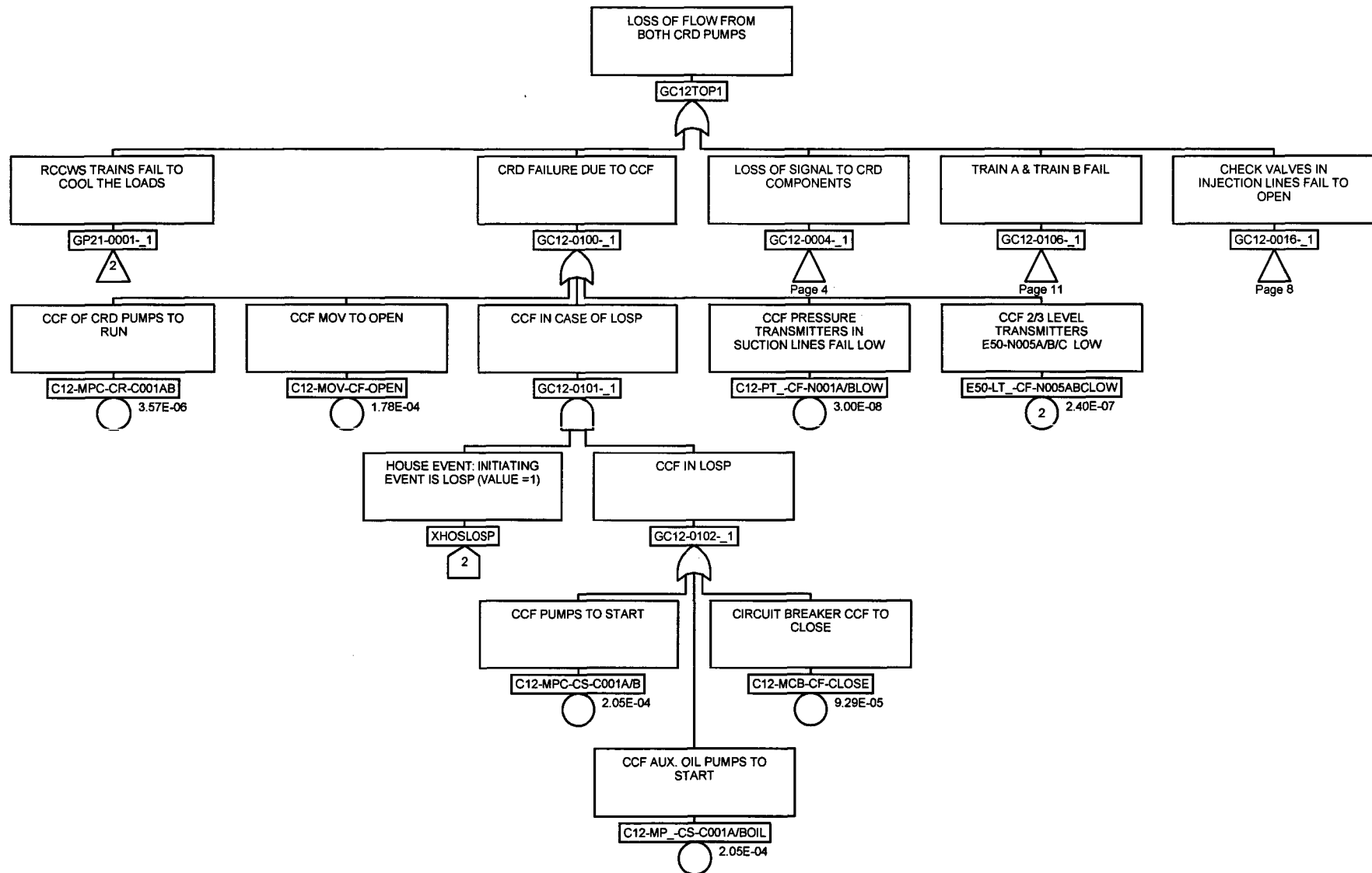


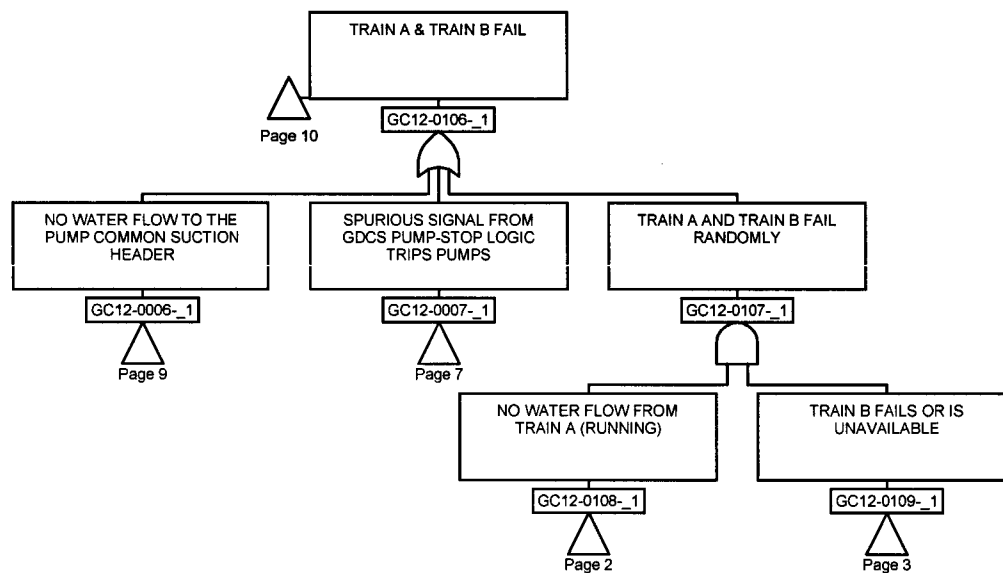


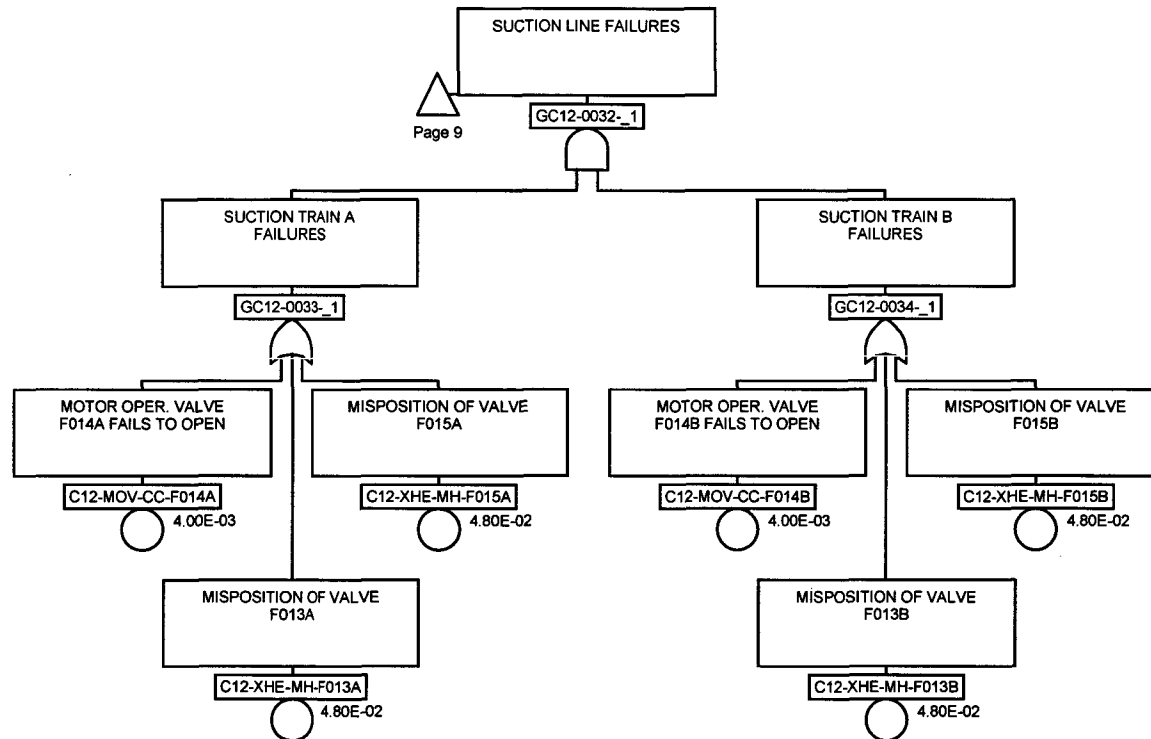


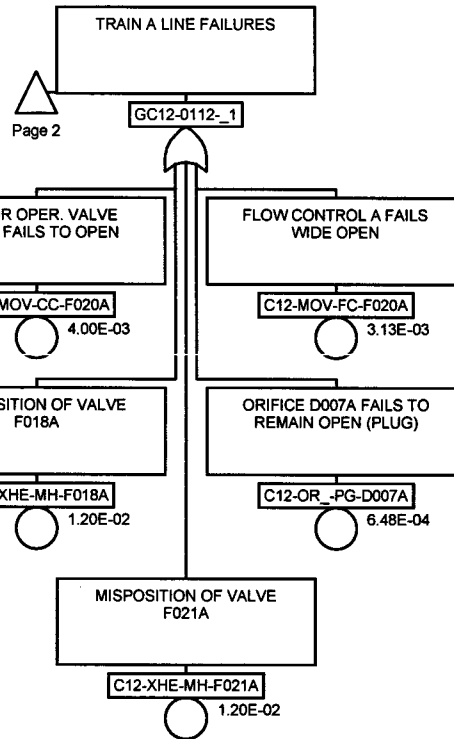


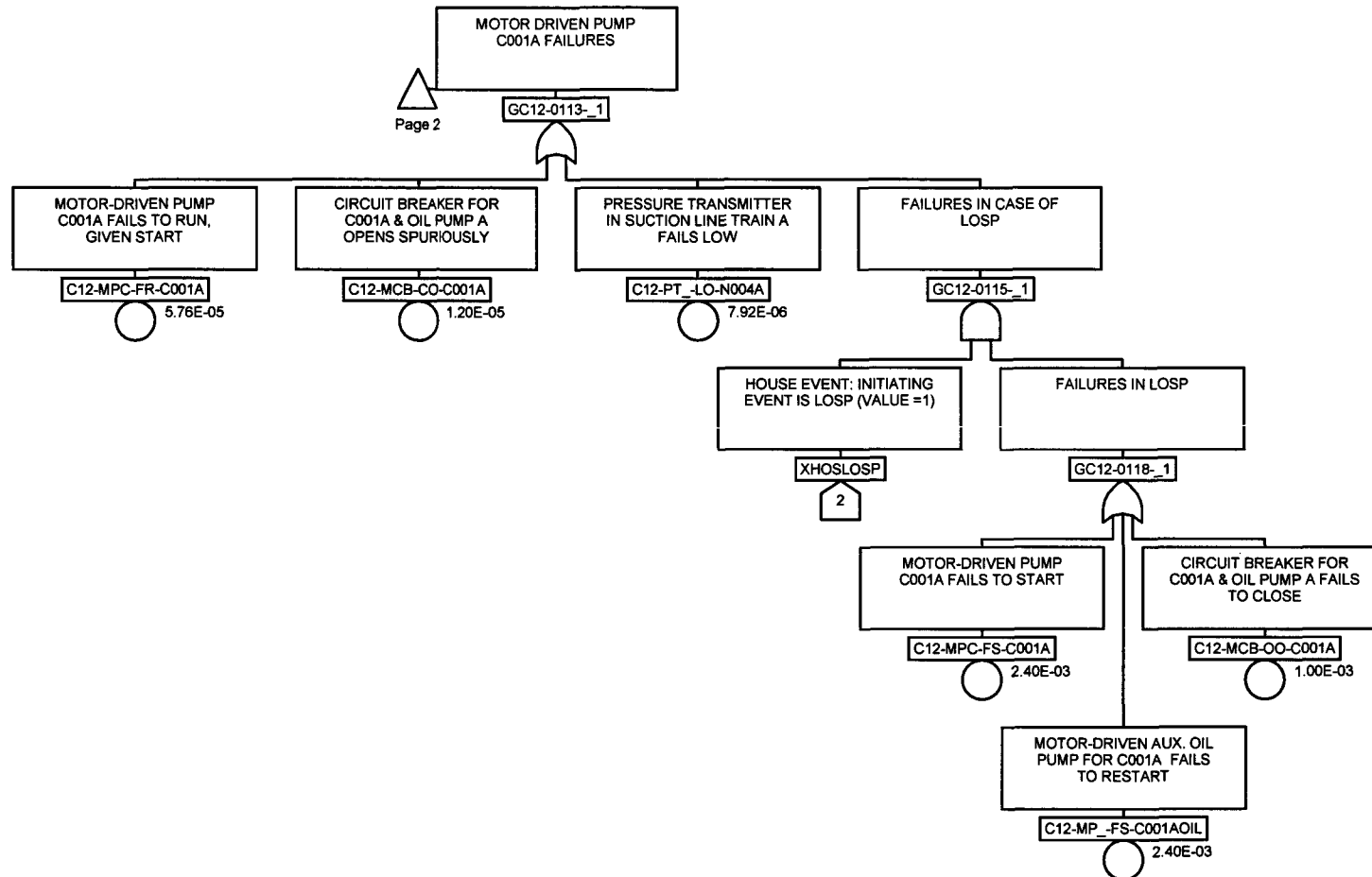
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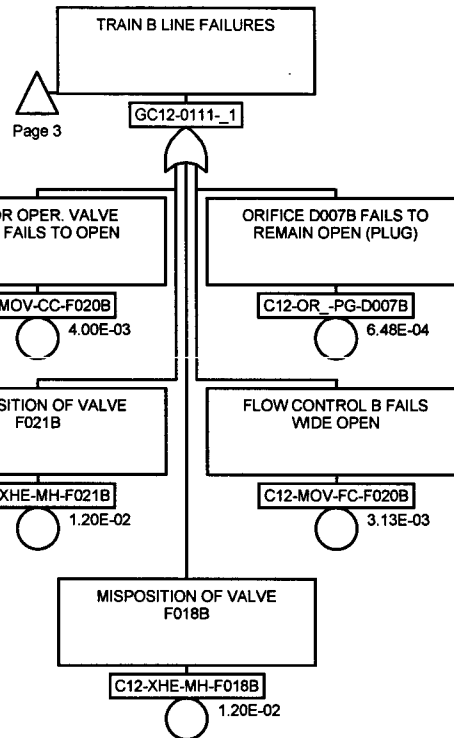


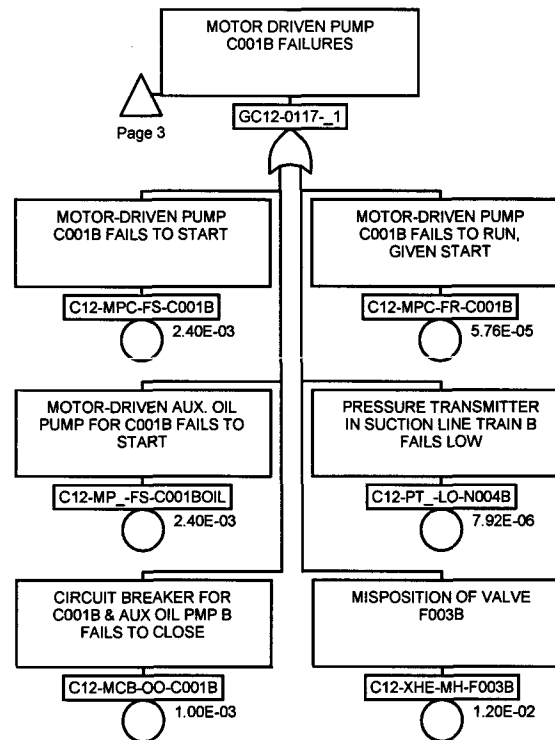






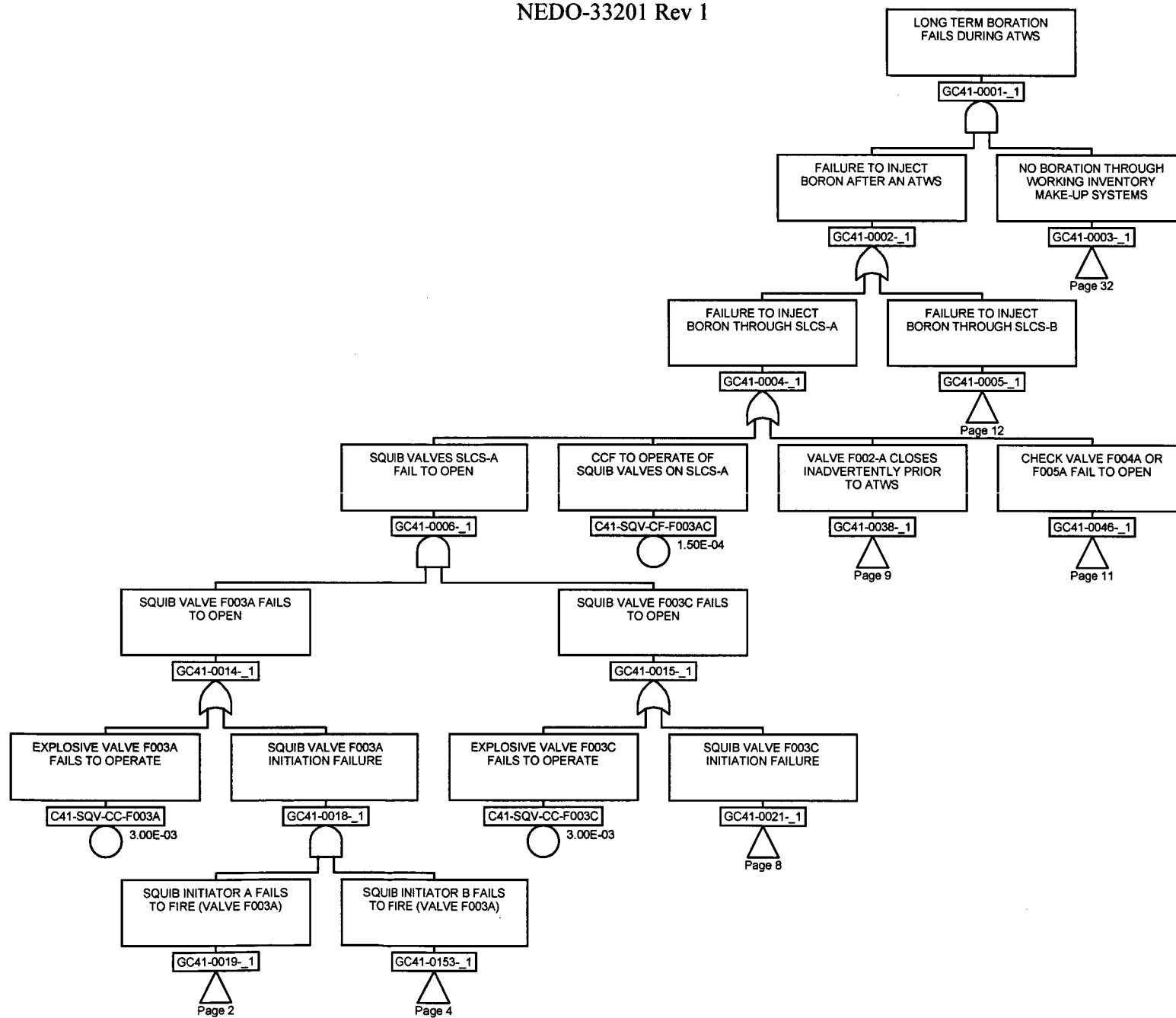


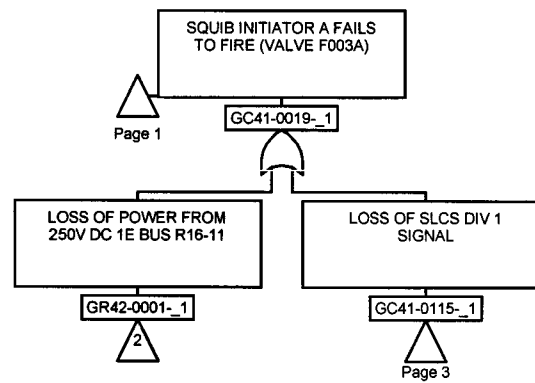


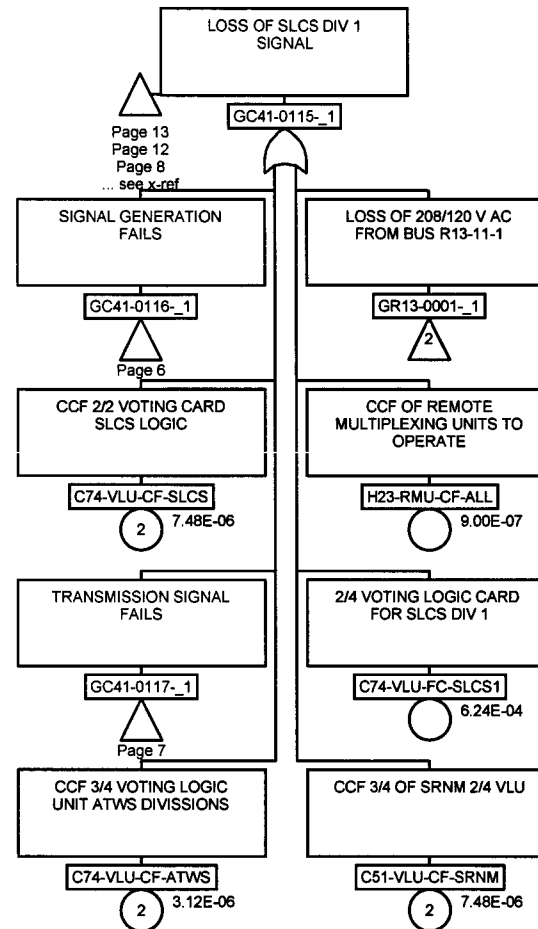


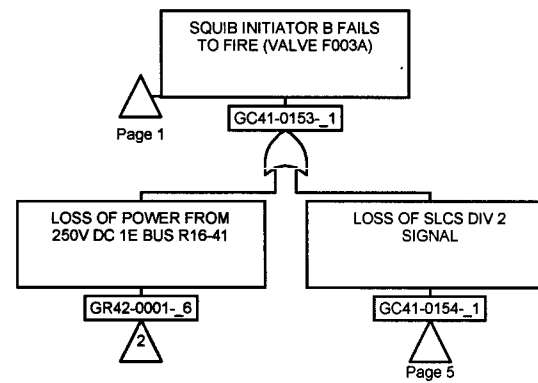
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| B21-LT -CF-L9SPU1 | 4 | 2 | GC12-0007- 1 | 11 | 2 | GC72-0105- 1 | 6 | 1 |
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| B21-UV -CC-F103B | 8 | 2 | GC12-0013- 1 | 9 | 2 | GN21-0004- 4 | 9 | 1 |
| C12-MCB-CF-CLOSE | 10 | 4 | GC12-0014- 1 | 9 | 4 | GP21-0001- 1 | 1 | 1 |
| C12-MCB-CO-C001A | 14 | 2 | GC12-0016- 1 | 1 | 6 | GP21-0001- 1 | 10 | 1 |
| C12-MCB-OO-C001A | 14 | 5 | GC12-0016- 1 | 8 | 2 | GR11-0001- 1 | 2 | 1 |
| C12-MCB-OO-C001B | 16 | 1 | GC12-0016- 1 | 10 | 6 | GR11-0001- 2 | 3 | 2 |
| C12-MOV-CC-F014A | 12 | 1 | GC12-0030- 1 | 9 | 4 | GR12-0001- 11 | 3 | 1 |
| C12-MOV-CC-F014B | 12 | 3 | GC12-0032- 1 | 9 | 3 | GR12-0001- 8 | 2 | 1 |
| C12-MOV-CC-F020A | 13 | 1 | GC12-0032- 1 | 12 | 2 | GR13-0001- 19 | 5 | 2 |
| C12-MOV-CC-F020B | 15 | 1 | GC12-0033- 1 | 12 | 2 | GR13-0001- 20 | 2 | 2 |
| C12-MOV-CF-OPEN | 10 | 2 | GC12-0034- 1 | 12 | 4 | GR13-0001- 23 | 5 | 4 |
| C12-MOV-FC-F020A | 13 | 2 | GC12-0036- 1 | 4 | 3 | GR13-0001- 24 | 3 | 1 |
| C12-MOV-FC-F020B | 15 | 2 | GC12-0038- 1 | 4 | 2 | GR13-0001- 27 | 5 | 6 |
| C12-MPC-CR-C001AB | 10 | 1 | GC12-0039- 1 | 4 | 4 | H23-EMS-CF-DIDALL | 6 | 2 |
| C12-MPC-CS-C001A/B | 10 | 3 | GC12-0041- 1 | 4 | 4 | H23-RMU-CF-DIDALL | 6 | 2 |
| C12-MPC-FR-C001A | 14 | 1 | GC12-0041- 1 | 5 | 4 | H23-RMU-FC-C12001 | 5 | 1 |
| C12-MPC-FR-C001B | 16 | 2 | GC12-0042- 1 | 4 | 5 | H23-RMU-FC-C12002 | 5 | 3 |
| C12-MPC-FS-C001A | 14 | 4 | GC12-0042- 1 | 6 | 2 | H23-RMU-FC-C12003 | 5 | 5 |
| C12-MPC-FS-C001B | 16 | 1 | GC12-0043- 1 | 5 | 4 | P30-TNK-RP-A001 | 9 | 3 |
| C12-MP -CS-C001A/BOIL | 10 | 4 | GC12-0046- 1 | 5 | 3 | P30-UV -CC-CV01T | 9 | 5 |
| C12-MP -FS-C001AOIL | 14 | 5 | GC12-0048- 1 | 5 | 2 | P30-XHE-MH-F015 | 9 | 4 |
| C12-MP -FS-C001BOIL | 16 | 1 | GC12-0049- 1 | 5 | 4 | XHOSLOSP | 10 | 3 |
| C12-OR -PG-D007A | 13 | 2 | GC12-0050- 1 | 5 | 6 | XHOSLOSP | 14 | 4 |
| C12-OR -PG-D007B | 15 | 2 | GC12-0100- 1 | 10 | 3 | XHOSMAN | 5 | 5 |
| C12-PT -CF-N001A/BLOW | 10 | 4 | GC12-0101- 1 | 10 | 3 | | | |
| C12-PT -LO-N004A | 14 | 3 | GC12-0102- 1 | 10 | 4 | | | |
| C12-PT -LO-N004B | 16 | 2 | GC12-0106- 1 | 10 | 5 | | | |
| C12-SYS-TM-TRAINB | 3 | 3 | GC12-0106- 1 | 11 | 2 | | | |
| C12-UV -CC-F022 | 8 | 1 | GC12-0107- 1 | 11 | 3 | | | |
| C12-XHE-FO-LEVEL2 | 5 | 4 | GC12-0108- 1 | 1 | 2 | | | |
| C12-XHE-MH-F003B | 16 | 2 | GC12-0108- 1 | 2 | 2 | | | |
| C12-XHE-MH-F013A | 12 | 2 | GC12-0108- 1 | 11 | 3 | | | |
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| C12-XHE-MH-F018A | 13 | 1 | GC12-0110- 1 | 3 | 2 | | | |
| C12-XHE-MH-F018B | 15 | 2 | GC12-0111- 1 | 3 | 2 | | | |
| C12-XHE-MH-F021A | 13 | 2 | GC12-0111- 1 | 15 | 2 | | | |
| C12-XHE-MH-F021B | 15 | 1 | GC12-0112- 1 | 2 | 2 | | | |
| C62-DTM-CF-DIDALL | 6 | 3 | GC12-0112- 1 | 13 | 2 | | | |
| E50-LT -CF-N005ABCLOW | 1 | 5 | GC12-0113- 1 | 2 | 2 | | | |
| E50-LT -CF-N005ABCLOW | 10 | 5 | GC12-0113- 1 | 14 | 3 | | | |
| E50-LT -LO-N005A | 7 | 1 | GC12-0115- 1 | 14 | 4 | | | |
| E50-LT -LO-N005B | 7 | 2 | GC12-0117- 1 | 3 | 2 | | | |
| E50-LT -LO-N005C | 7 | 2 | GC12-0117- 1 | 16 | 2 | | | |
| GC12-0004- 1 | 1 | 3 | GC12-0118- 1 | 14 | 5 | | | |
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| GC12-0004- 1 | 10 | 4 | GC12-0152- 1 | 6 | 2 | | | |
| GC12-0006- 1 | 1 | 7 | GC12TOP1 | 10 | 4 | | | |
| GC12-0006- 1 | 9 | 3 | GC12TOP2 | 1 | 4 | | | |
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Appendix B.4.4 Standby Liquid Control System Fault Tree

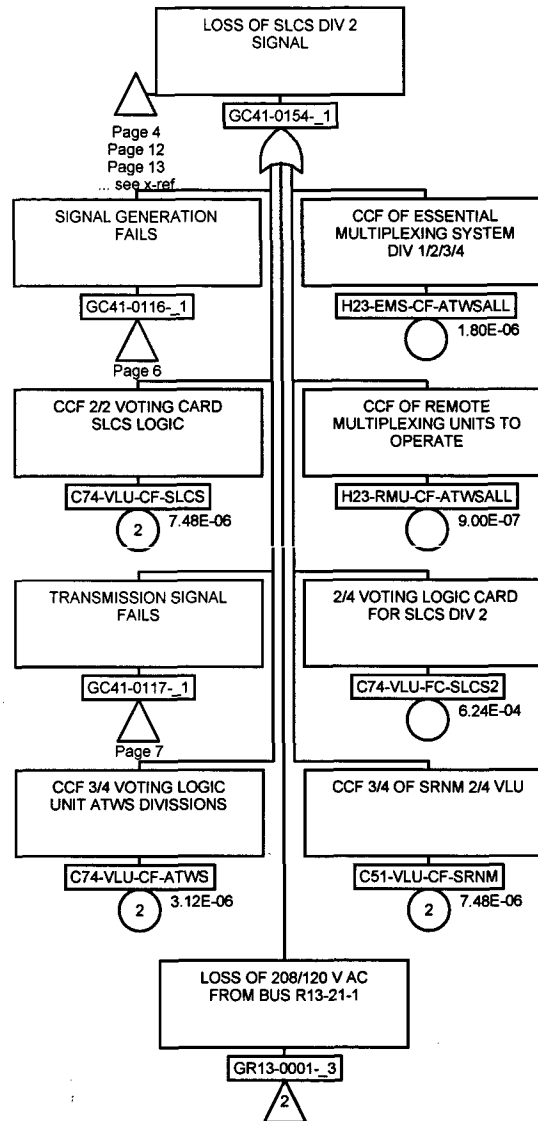


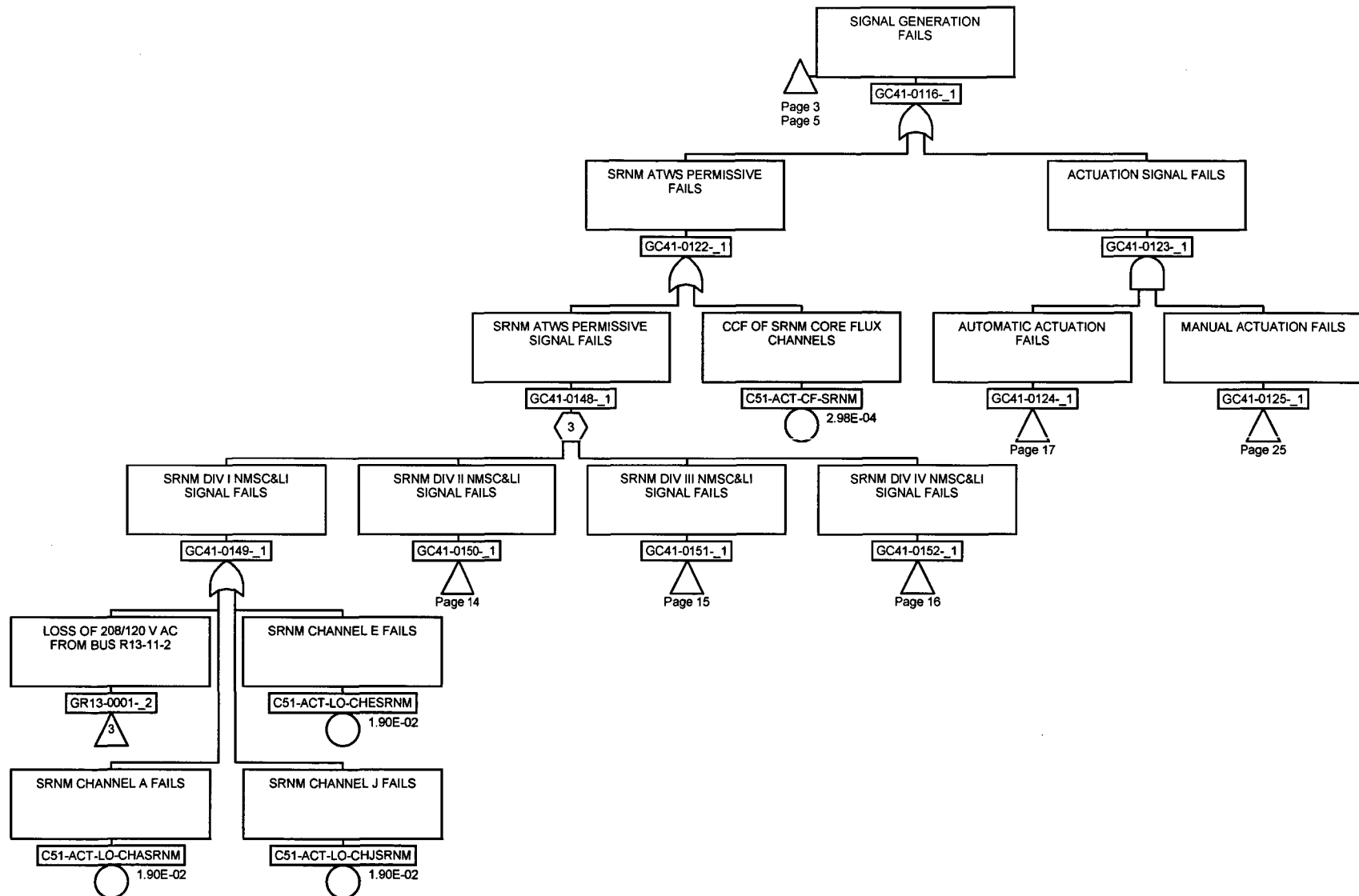


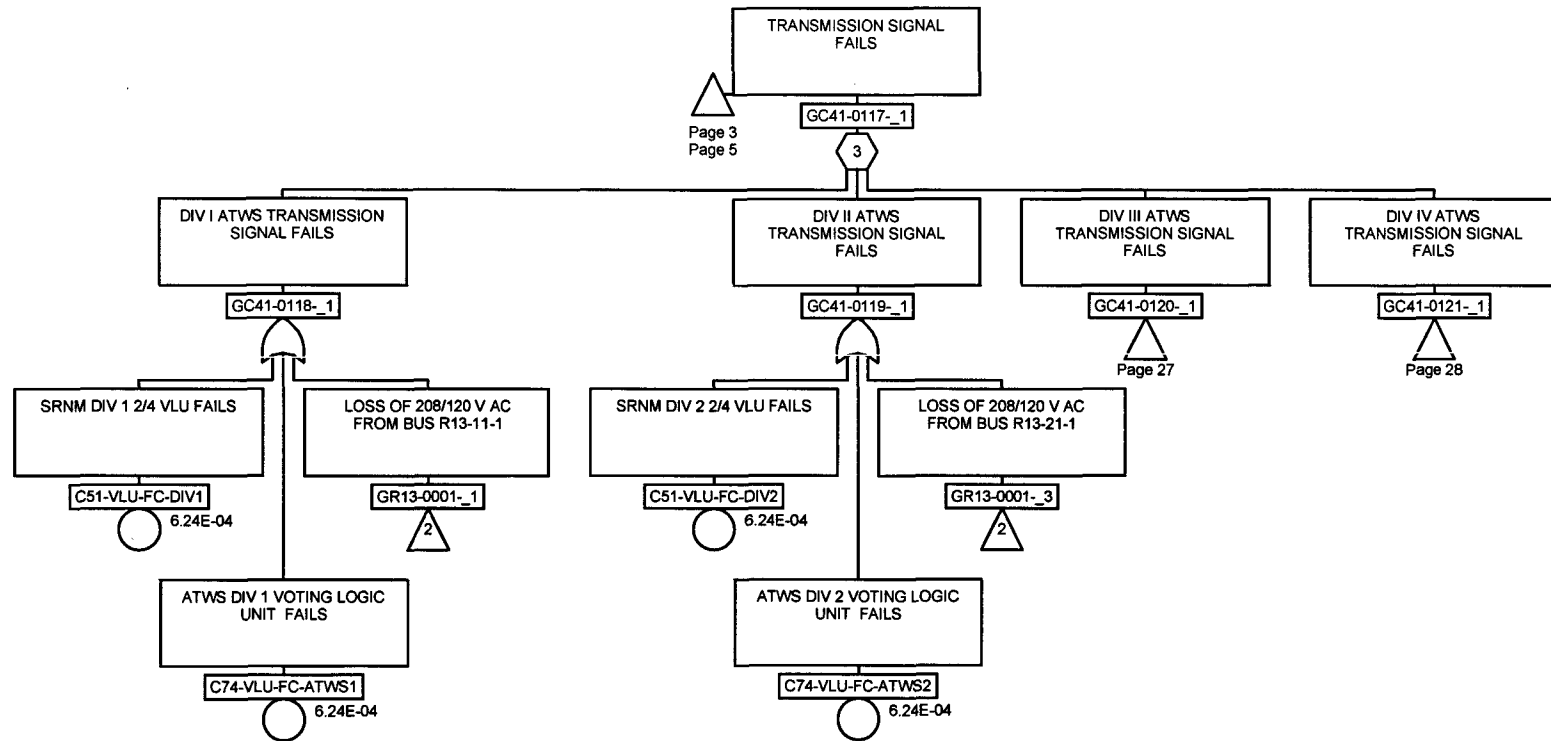


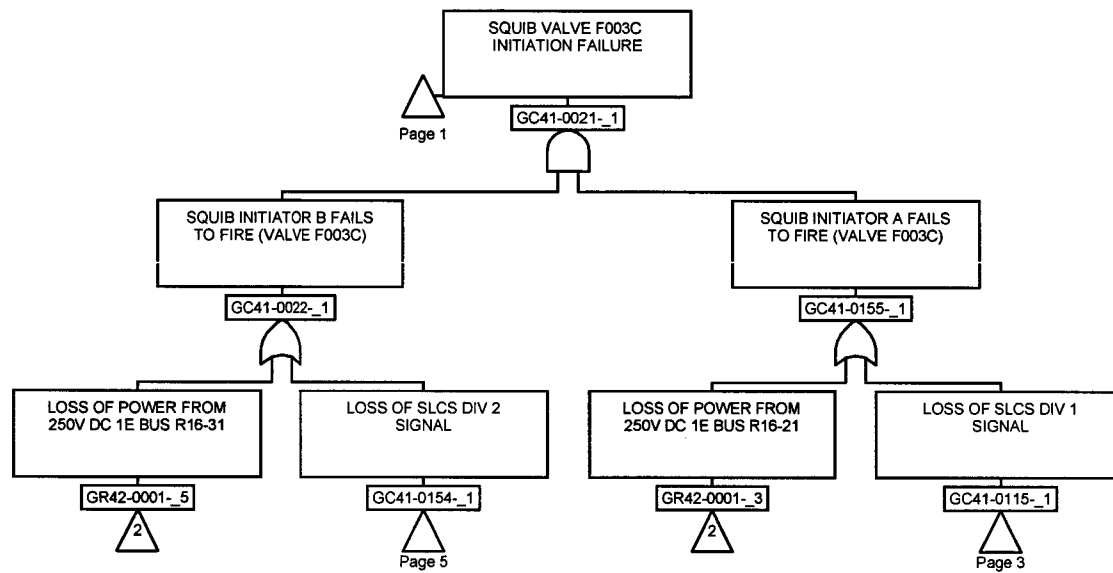


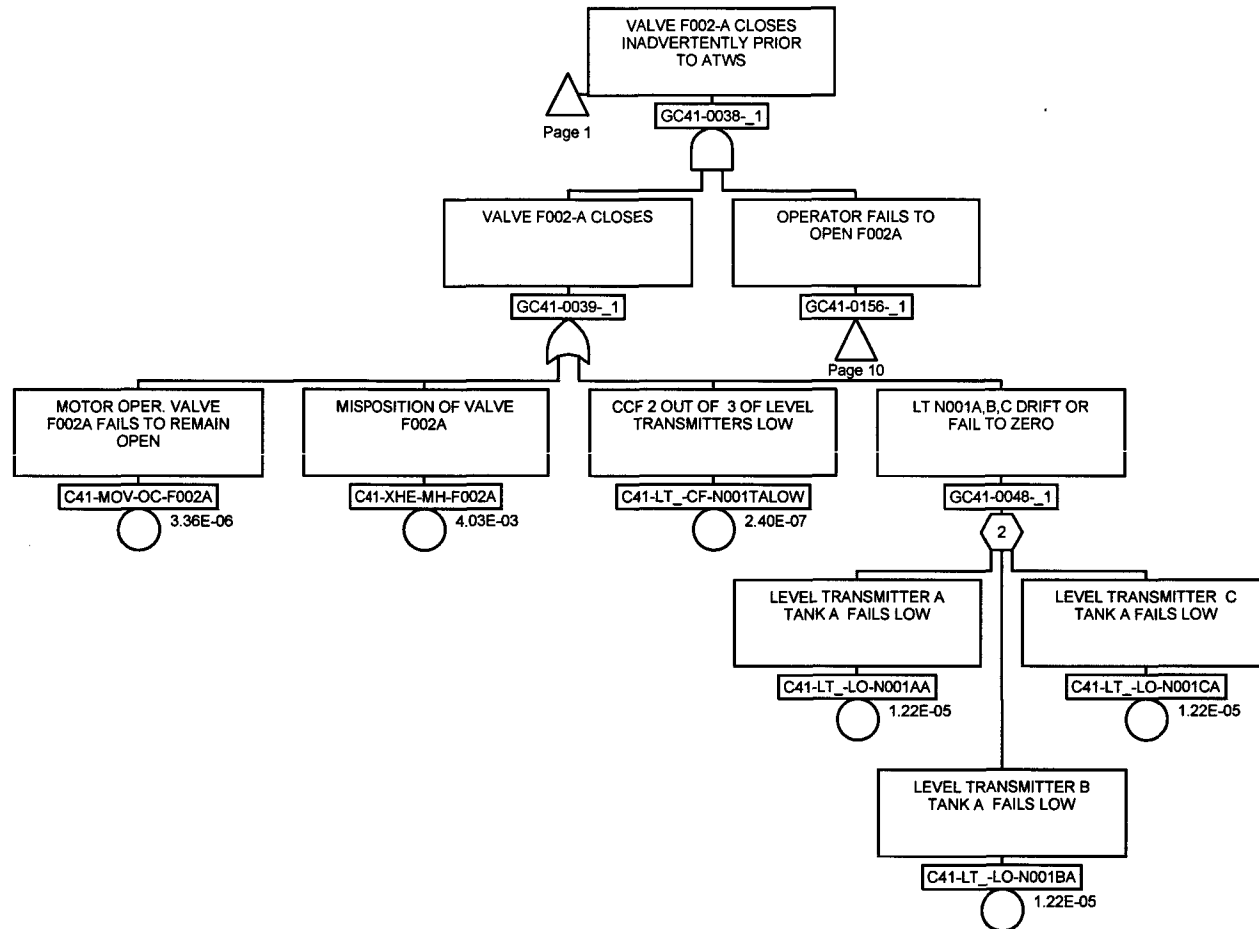
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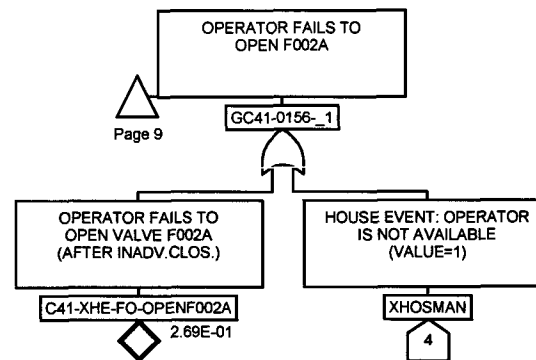


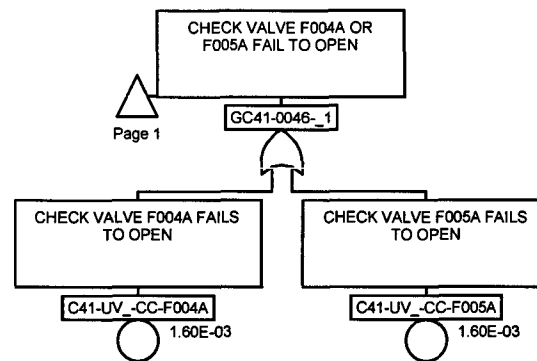






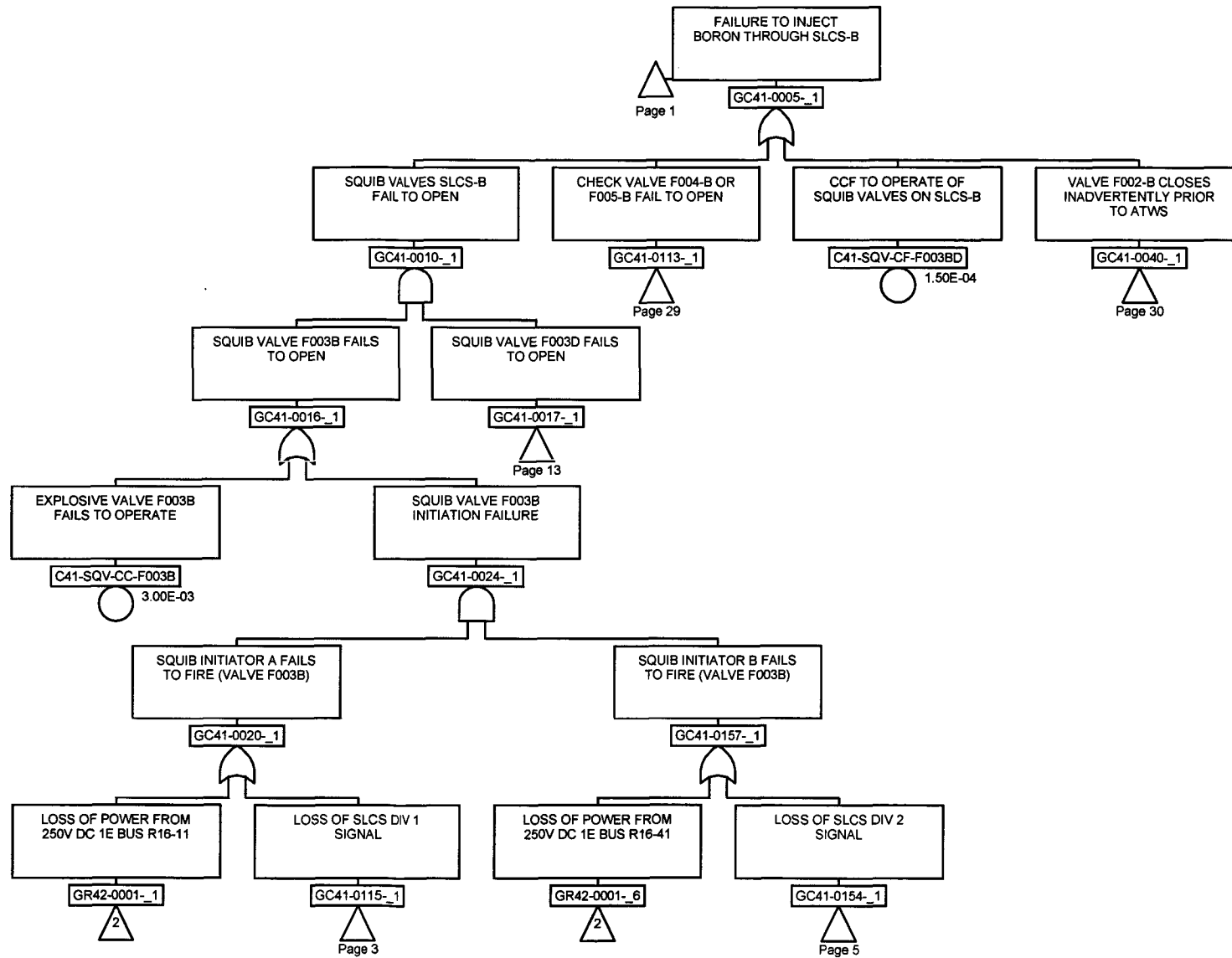


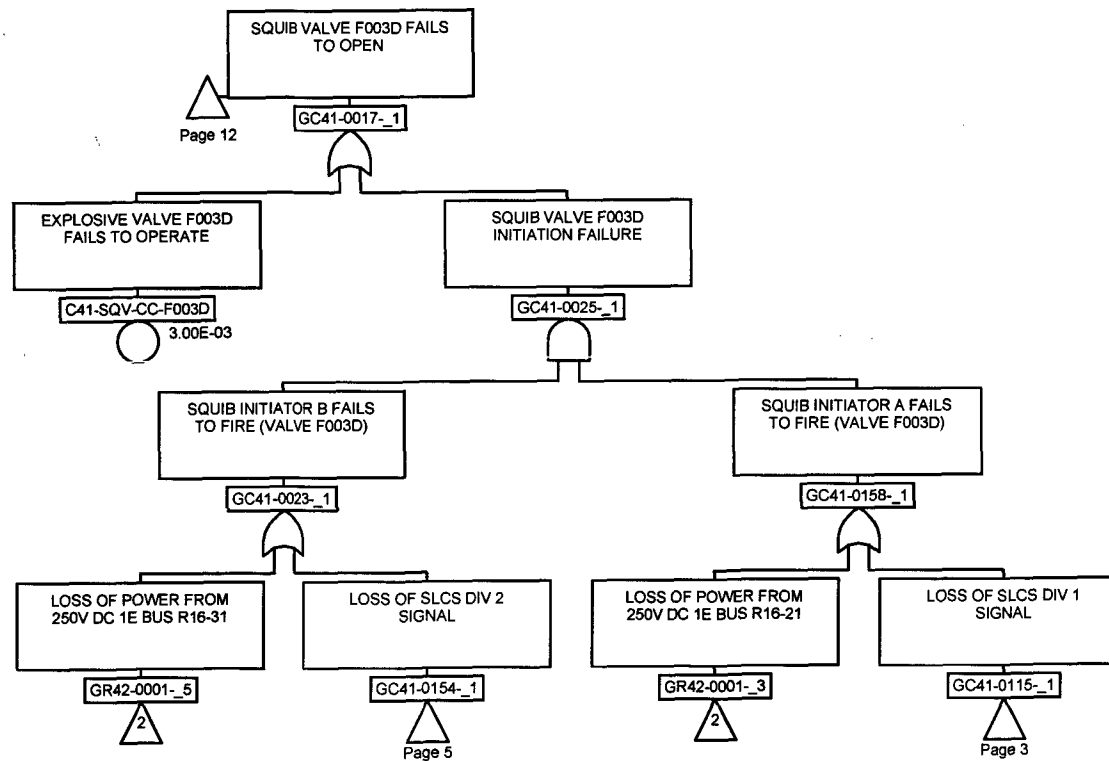


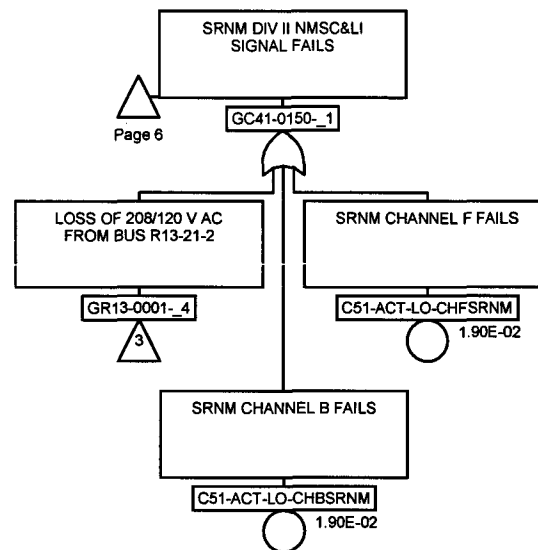


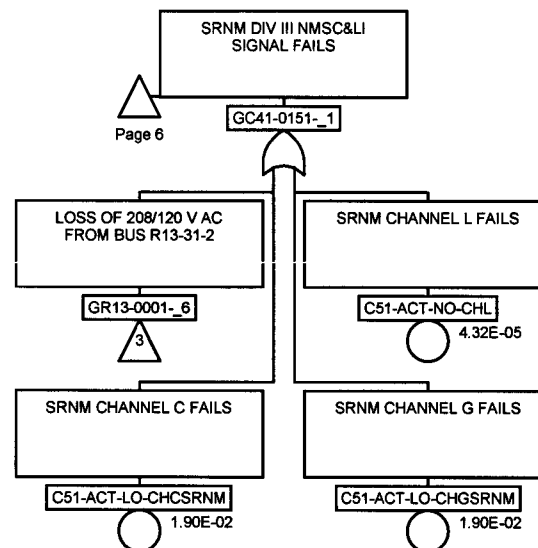
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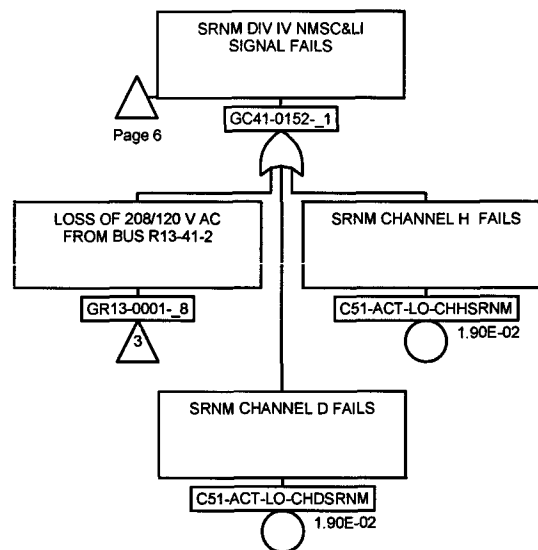
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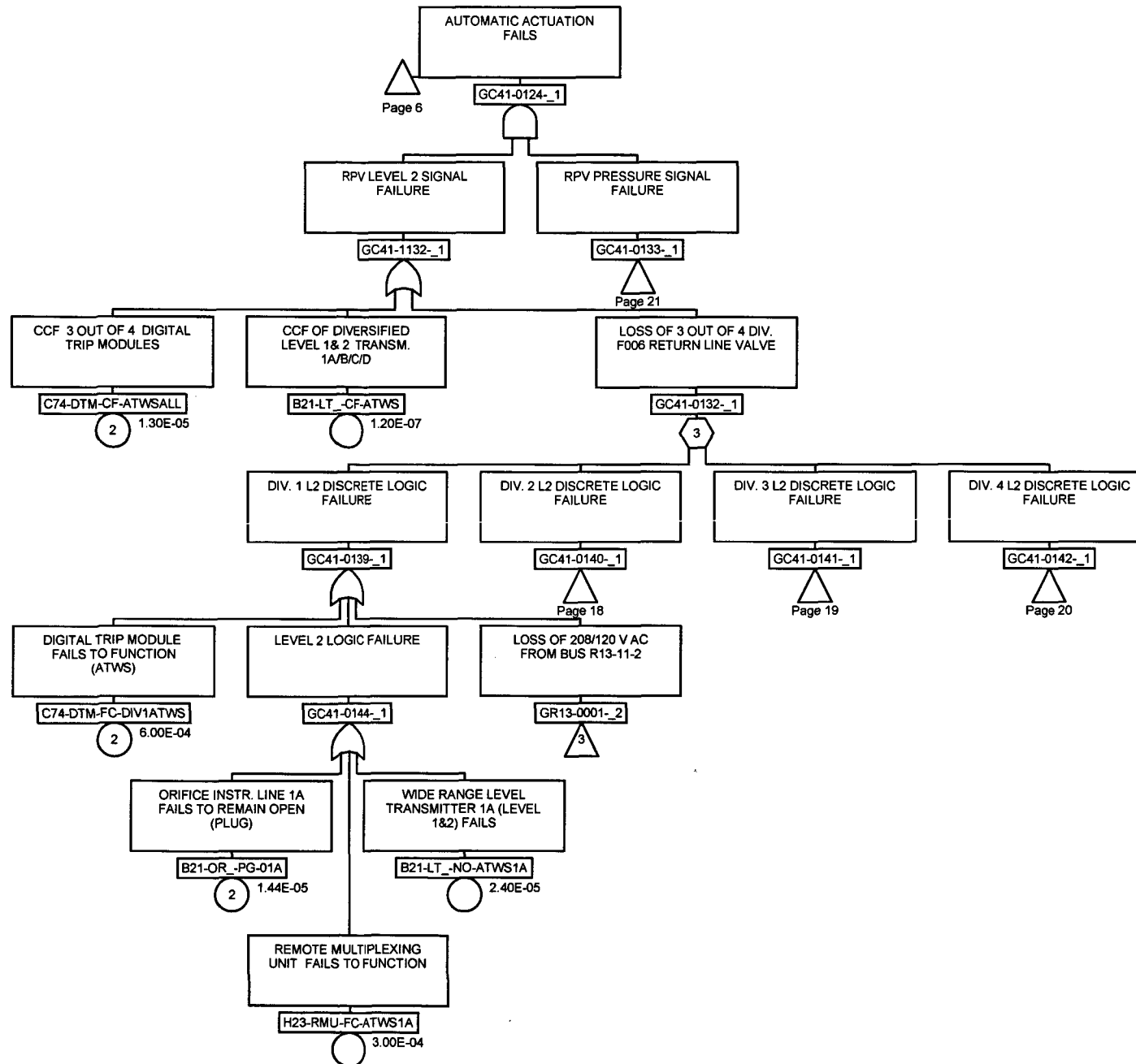


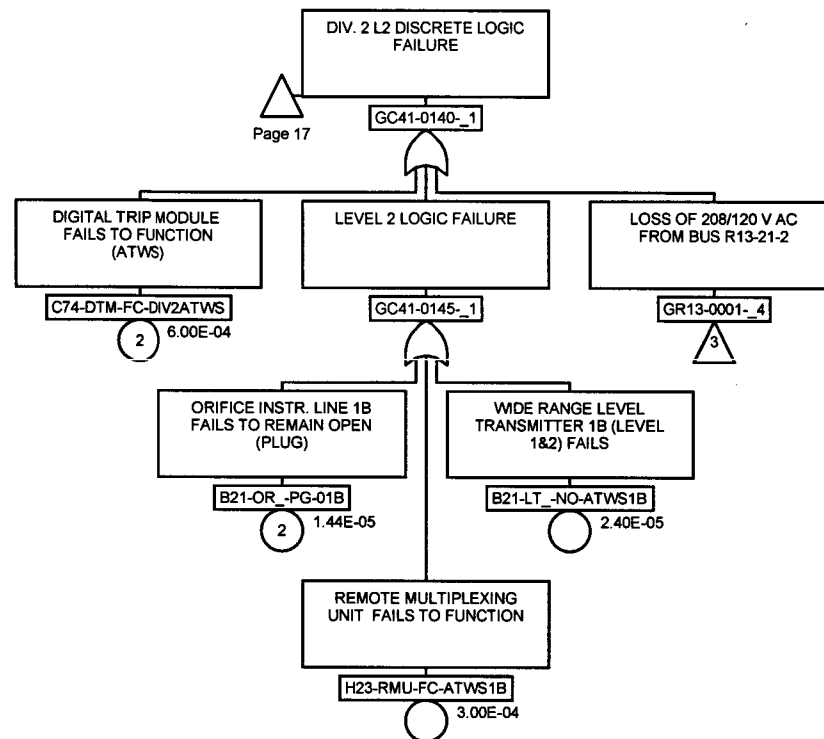


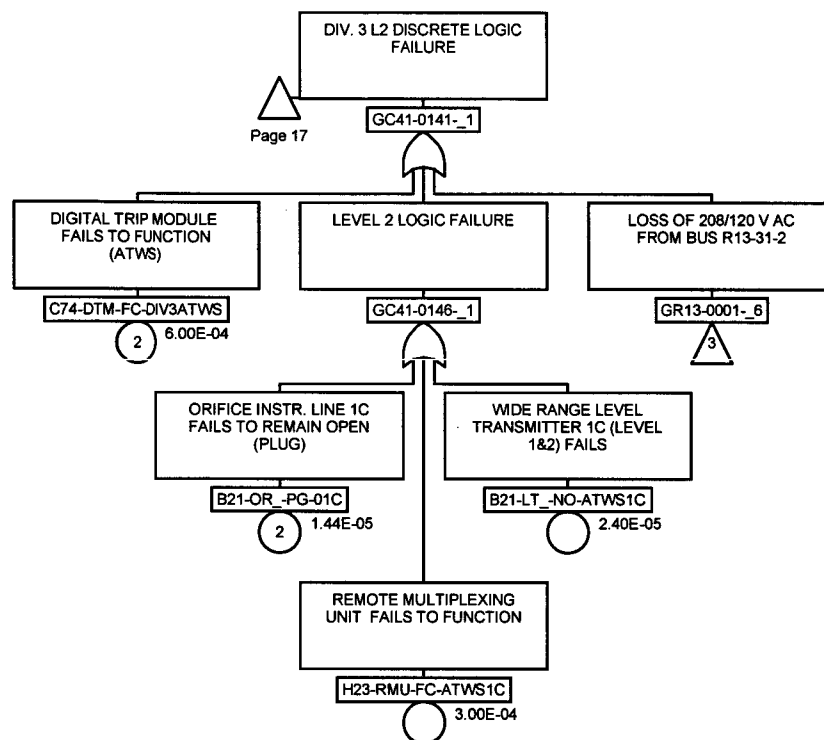


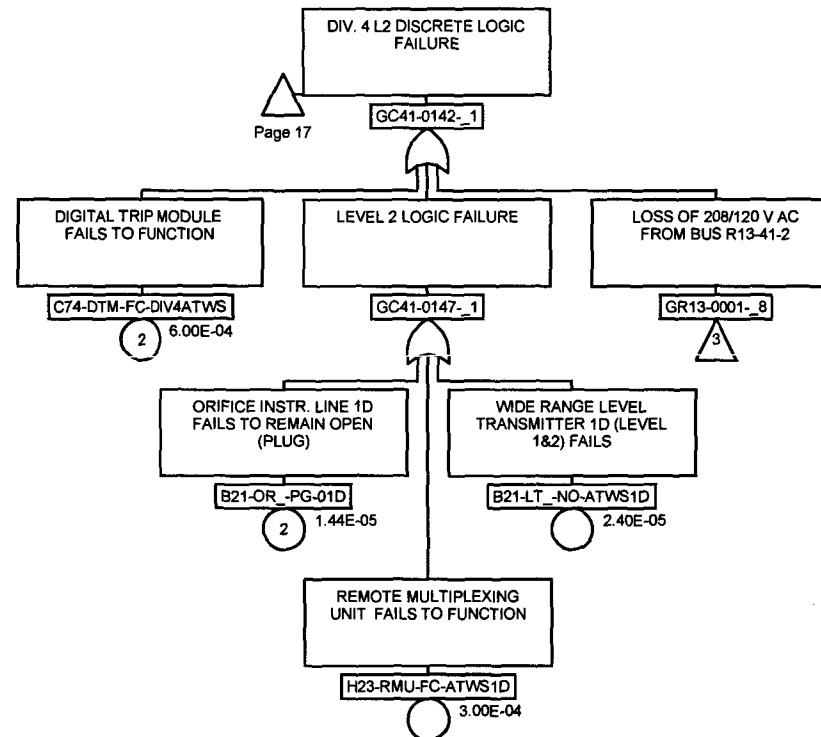


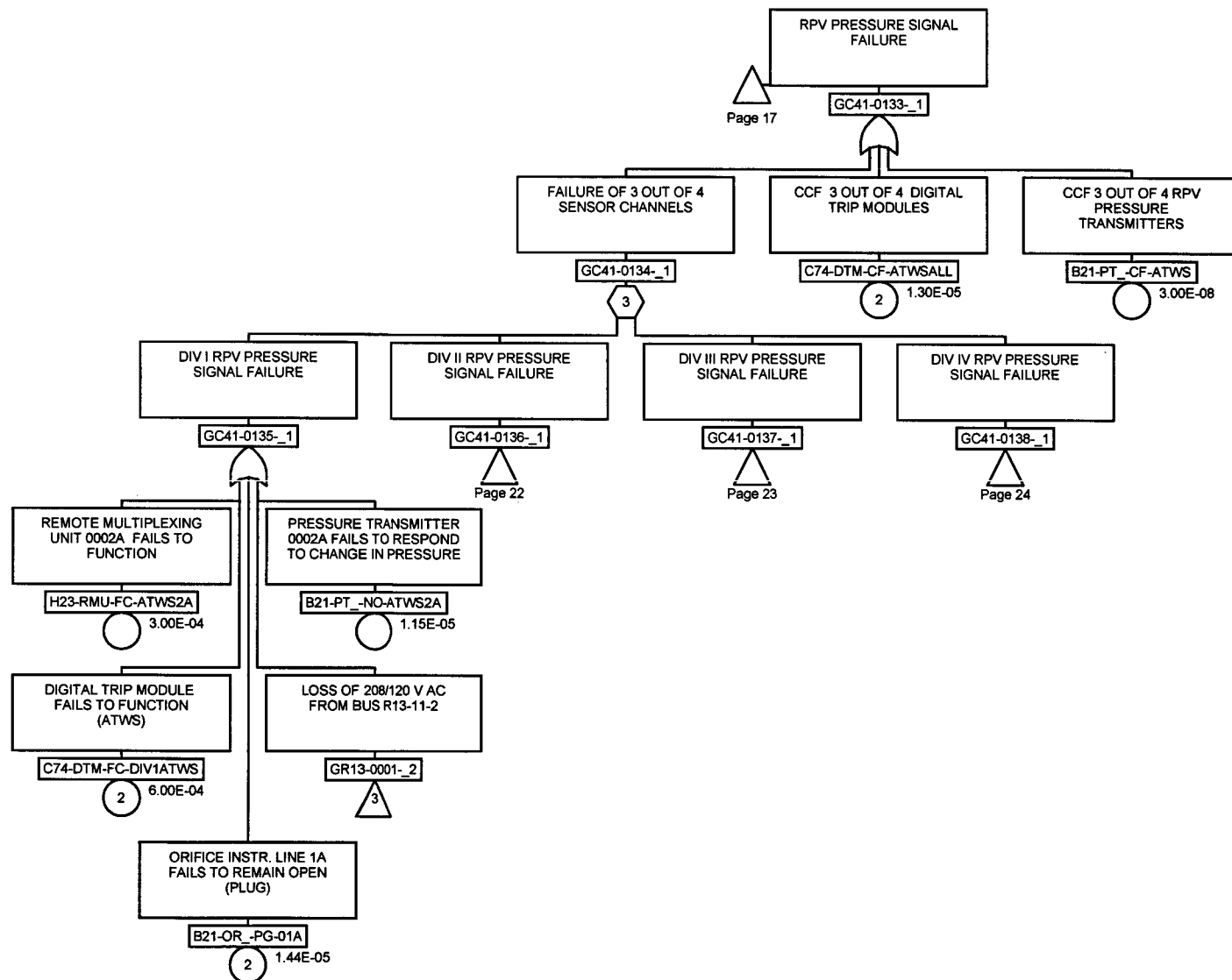
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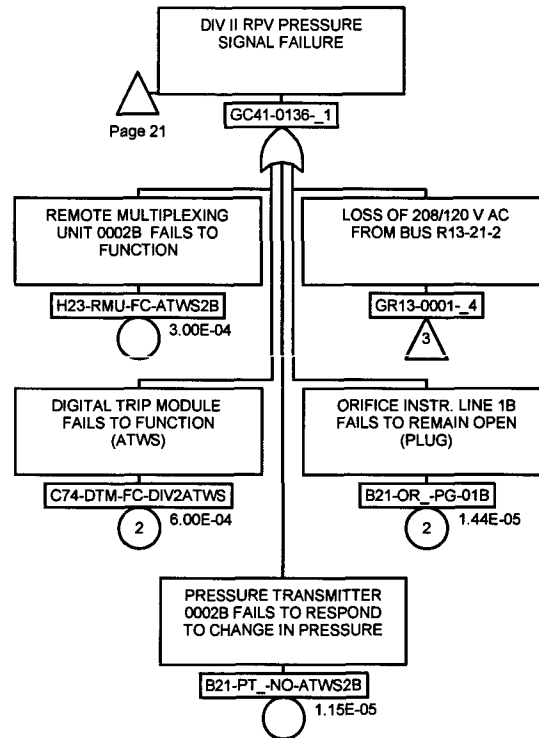


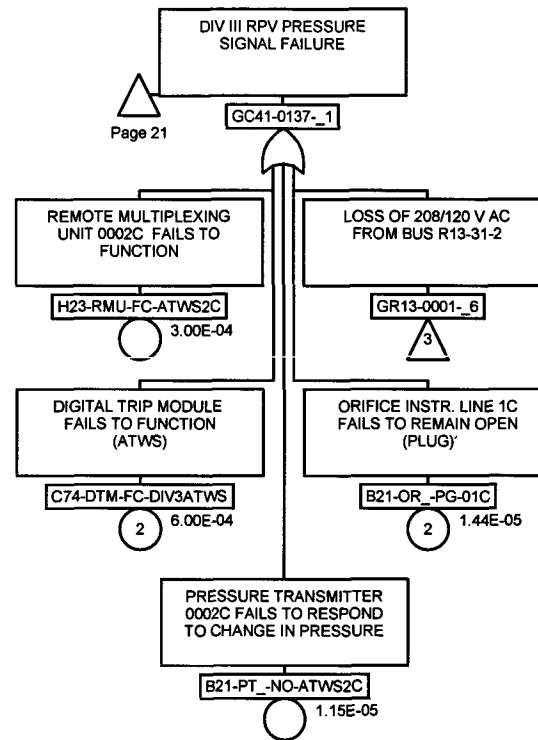


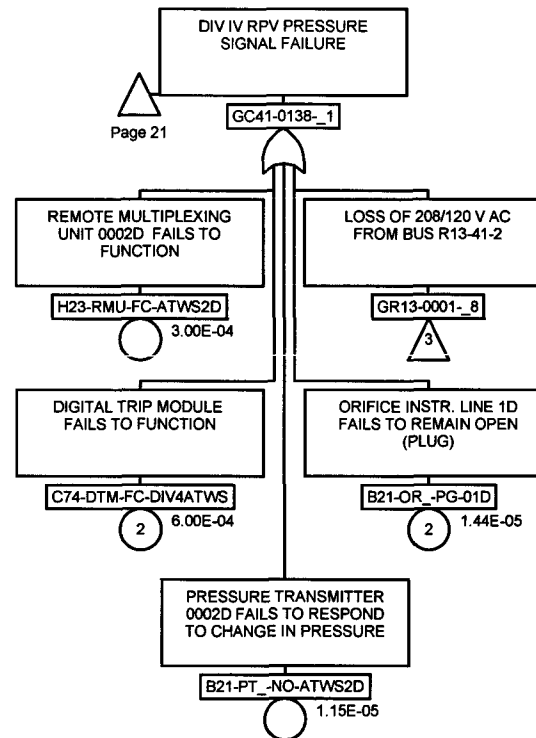


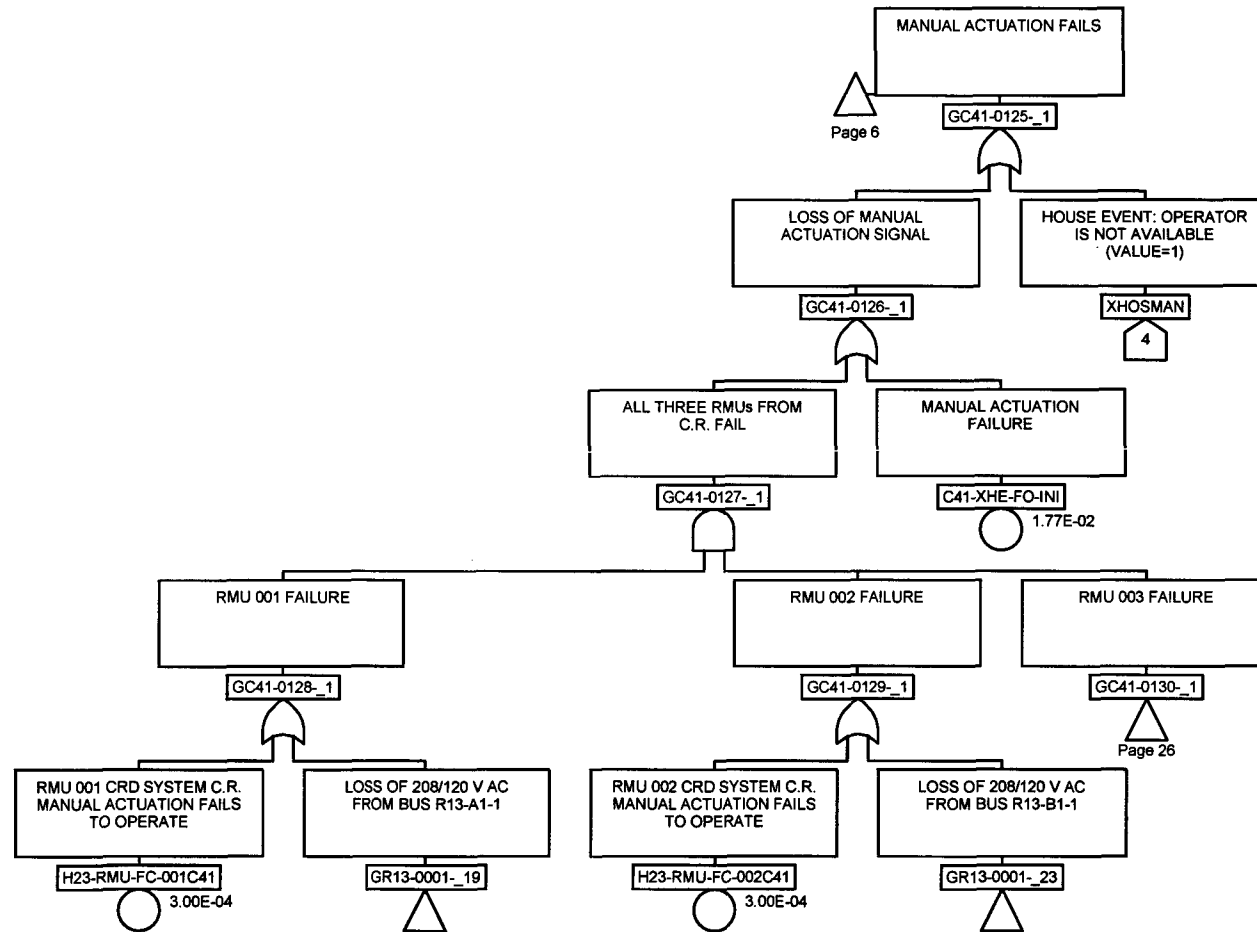


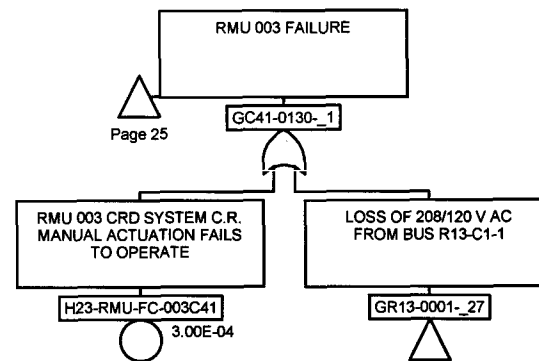


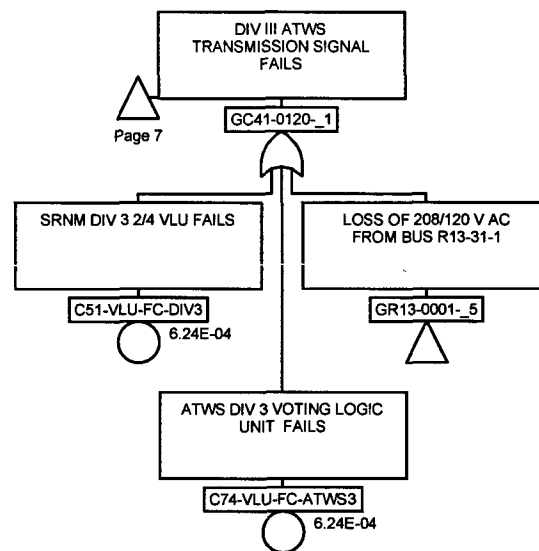


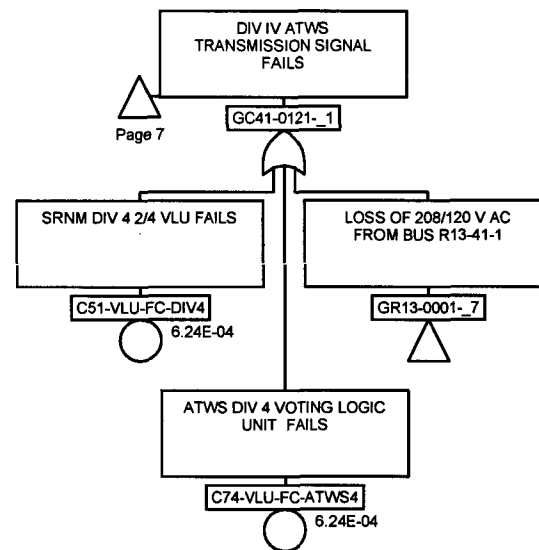


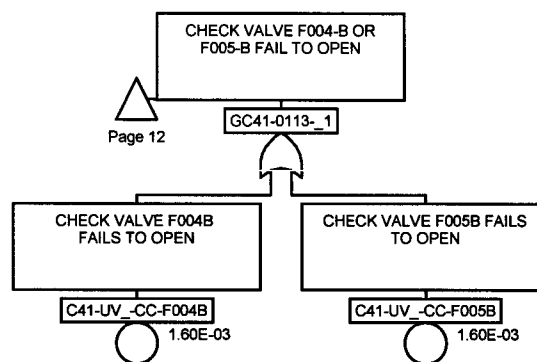


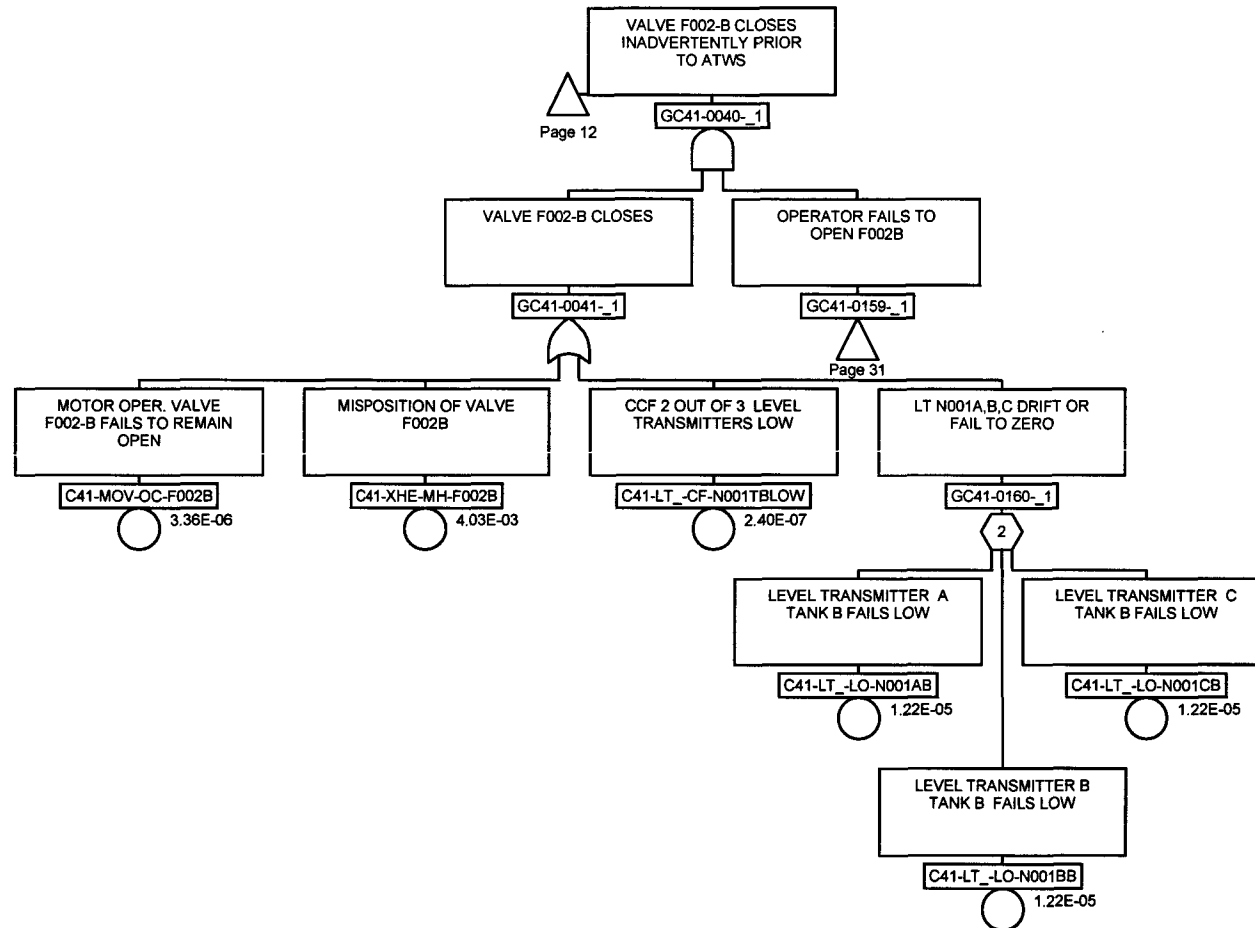


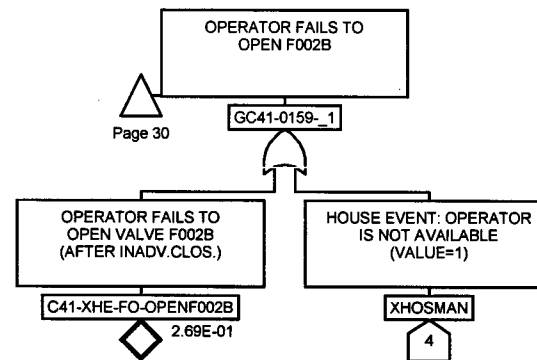


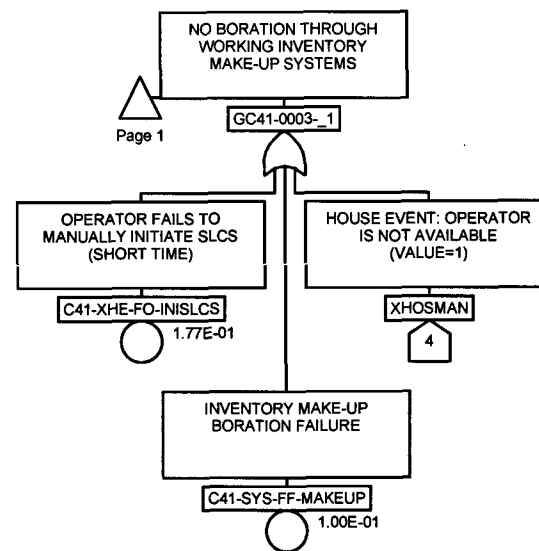












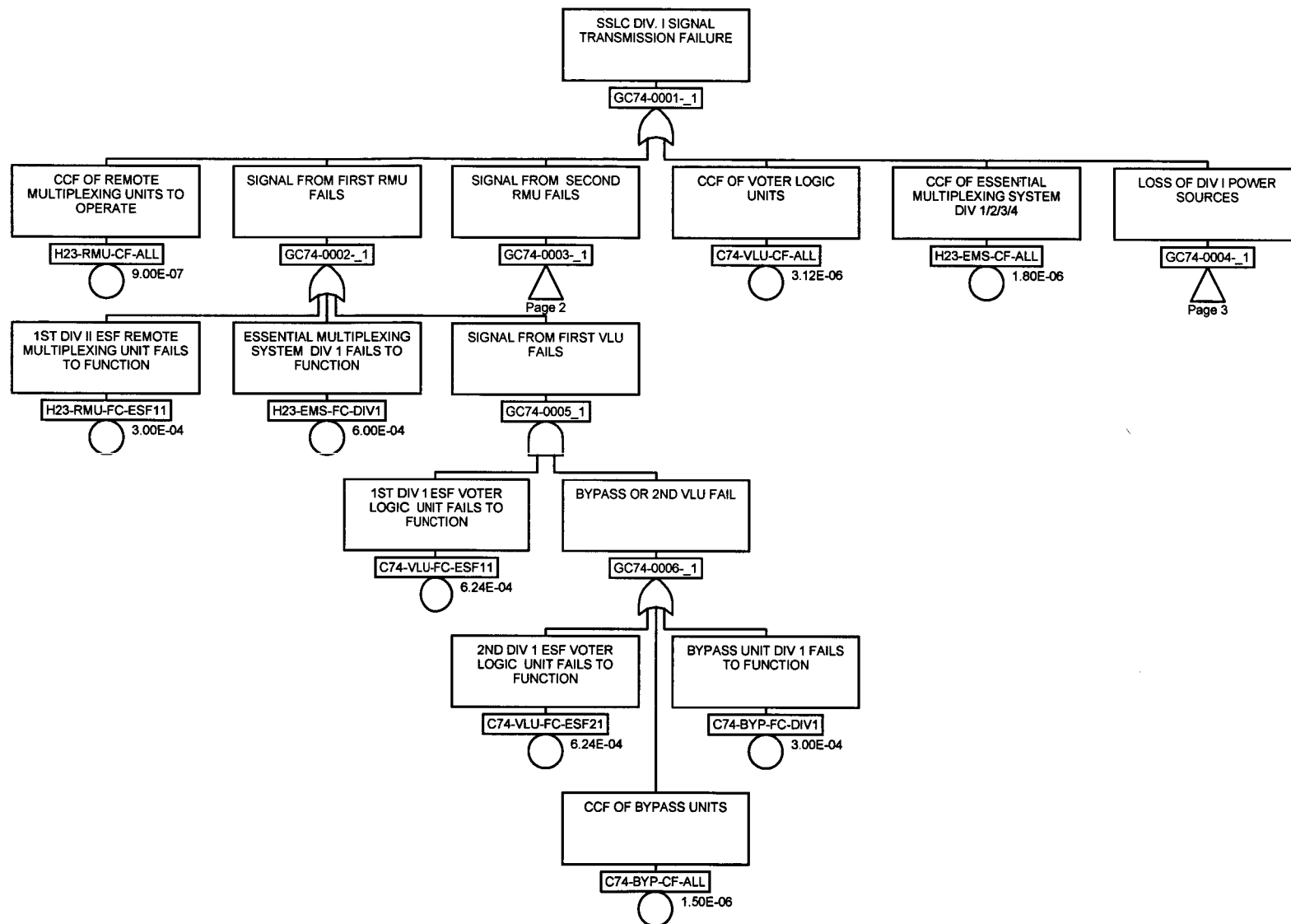
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| B21-LT -NO-ATWS1B | 18 | 3 | C51-VLU-CF-SRNM | 3 | 2 | |
| B21-LT -NO-ATWS1C | 19 | 3 | C51-VLU-CF-SRNM | 5 | 2 | |
| B21-LT -NO-ATWS1D | 20 | 3 | C51-VLU-FC-DIV1 | 7 | 1 | |
| B21-OR -PG-01A | 17 | 2 | C51-VLU-FC-DIV2 | 7 | 3 | |
| B21-OR -PG-01A | 21 | 2 | C51-VLU-FC-DIV3 | 27 | 1 | |
| B21-OR -PG-01B | 18 | 2 | C51-VLU-FC-DIV4 | 28 | 1 | |
| B21-OR -PG-01B | 22 | 2 | C74-DTM-CF-ATWSALL | 17 | 1 | |
| B21-OR -PG-01C | 19 | 2 | C74-DTM-CF-ATWSALL | 21 | 4 | |
| B21-OR -PG-01C | 23 | 2 | C74-DTM-FC-DIV1ATWS | 17 | 1 | |
| B21-OR -PG-01D | 20 | 2 | C74-DTM-FC-DIV1ATWS | 21 | 1 | |
| B21-OR -PG-01D | 24 | 2 | C74-DTM-FC-DIV2ATWS | 18 | 1 | |
| B21-PT -CF-ATWS | 21 | 5 | C74-DTM-FC-DIV2ATWS | 22 | 1 | |
| B21-PT -NO-ATWS2A | 21 | 2 | C74-DTM-FC-DIV3ATWS | 19 | 1 | |
| B21-PT -NO-ATWS2B | 22 | 2 | C74-DTM-FC-DIV3ATWS | 23 | 1 | |
| B21-PT -NO-ATWS2C | 23 | 2 | C74-DTM-FC-DIV4ATWS | 20 | 1 | |
| B21-PT -NO-ATWS2D | 24 | 2 | C74-DTM-FC-DIV4ATWS | 24 | 1 | |
| C41-LT -CF-N001TALOW | 9 | 3 | C74-VLU-CF-ATWS | 3 | 1 | |
| C41-LT -CF-N001TBLOW | 30 | 3 | C74-VLU-CF-ATWS | 5 | 1 | |
| C41-LT -LO-N001AA | 9 | 4 | C74-VLU-CF-SLCS | 3 | 1 | |
| C41-LT -LO-N001AB | 30 | 4 | C74-VLU-CF-SLCS | 5 | 1 | |
| C41-LT -LO-N001BA | 9 | 4 | C74-VLU-FC-ATWS1 | 7 | 2 | |
| C41-LT -LO-N001BB | 30 | 4 | C74-VLU-FC-ATWS2 | 7 | 4 | |
| C41-LT -LO-N001CA | 9 | 5 | C74-VLU-FC-ATWS3 | 27 | 2 | |
| C41-LT -LO-N001CB | 30 | 5 | C74-VLU-FC-ATWS4 | 28 | 2 | |
| C41-MOV-OC-F002A | 9 | 1 | C74-VLU-FC-SLCS1 | 3 | 2 | |
| C41-MOV-OC-F002B | 30 | 1 | C74-VLU-FC-SLCS2 | 5 | 2 | |
| C41-SQV-CC-F003A | 1 | 1 | GC41-0001- 1 | 1 | 5 | |
| C41-SQV-CC-F003B | 12 | 1 | GC41-0002- 1 | 1 | 5 | |
| C41-SQV-CC-F003C | 1 | 3 | GC41-0003- 1 | 1 | 6 | |
| C41-SQV-CC-F003D | 13 | 1 | GC41-0003- 1 | 32 | 2 | |
| C41-SQV-CF-F003AC | 1 | 4 | GC41-0004- 1 | 1 | 4 | |
| C41-SQV-CF-F003BD | 12 | 4 | GC41-0005- 1 | 1 | 5 | |
| C41-SYS-FF-MAKEUP | 32 | 2 | GC41-0005- 1 | 12 | 4 | |
| C41-UV -CC-F004A | 11 | 1 | GC41-0006- 1 | 1 | 3 | |
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| C41-UV -CC-F005B | 29 | 2 | GC41-0015- 1 | 1 | 4 | |
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| C41-XHE-FO-INISLCS | 32 | 1 | GC41-0017- 1 | 12 | 3 | |
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| C41-XHE-FO-OPENF002B | 31 | 1 | GC41-0018- 1 | 1 | 2 | |
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| C51-ACT-LO-CHCSRNM | 15 | 1 | GC41-0022- 1 | 8 | 2 | |
| C51-ACT-LO-CHDSRNM | 16 | 2 | GC41-0023- 1 | 13 | 2 | |
| C51-ACT-LO-CHESRNM | 6 | 2 | GC41-0024- 1 | 12 | 3 | |
| C51-ACT-LO-CHFSRNM | 14 | 2 | GC41-0025- 1 | 13 | 3 | |
| C51-ACT-LO-CHGSRNM | 15 | 2 | GC41-0038- 1 | 1 | 5 | |
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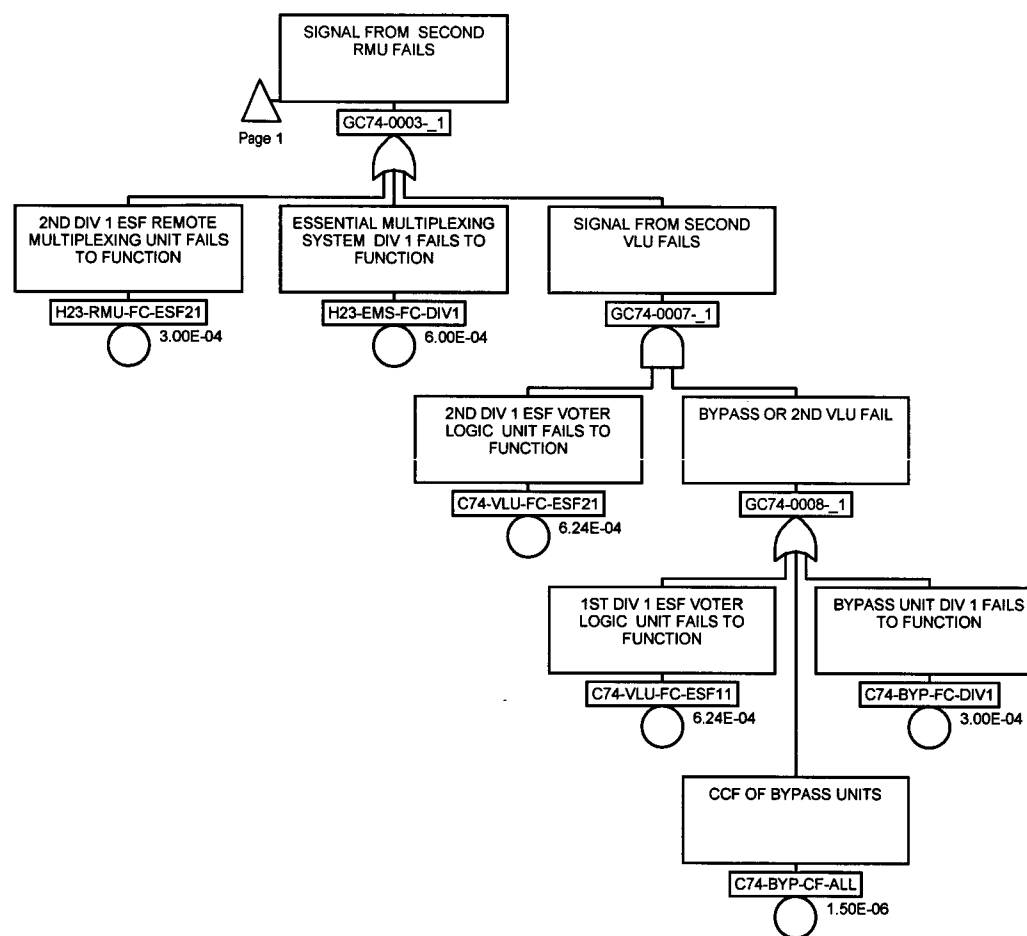
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| GC41-0046- 1 | 1 | 6 | GC41-0146- 1 | 19 | 2 | |
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| GC41-0136- 1 | 22 | 2 | GR13-0001- 6 | 15 | 1 | |
| GC41-0137- 1 | 21 | 4 | GR13-0001- 6 | 19 | 3 | |
| GC41-0137- 1 | 23 | 2 | GR13-0001- 6 | 23 | 2 | |
| GC41-0138- 1 | 21 | 5 | GR13-0001- 7 | 28 | 2 | |
| GC41-0138- 1 | 24 | 2 | GR13-0001- 8 | 16 | 1 | |
| GC41-0139- 1 | 17 | 2 | GR13-0001- 8 | 20 | 3 | |
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| C41 - Standby Liquid Control | | | | | | X-SLC.caf Appendix B.4.4 Page 34 |

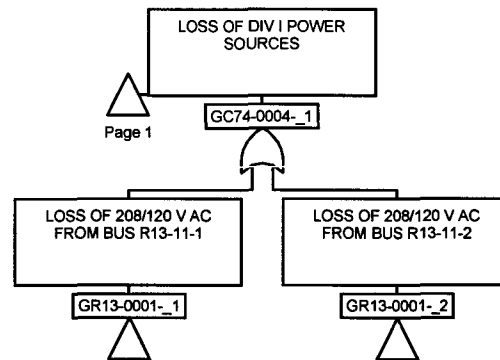
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| H23-RMU-FC-ATWS1B | 18 | 2 | | | | | |
| H23-RMU-FC-ATWS1C | 19 | 2 | | | | | |
| H23-RMU-FC-ATWS1D | 20 | 2 | | | | | |
| H23-RMU-FC-ATWS2A | 21 | 1 | | | | | |
| H23-RMU-FC-ATWS2B | 22 | 1 | | | | | |
| H23-RMU-FC-ATWS2C | 23 | 1 | | | | | |
| H23-RMU-FC-ATWS2D | 24 | 1 | | | | | |
| XHOSMAN | 10 | 2 | | | | | |
| XHOSMAN | 25 | 5 | | | | | |
| XHOSMAN | 31 | 2 | | | | | |
| XHOSMAN | 32 | 2 | | | | | |
| C41 - Standby Liquid Control | | | | | | X-SLC.caf | Appendix B.4.4 |
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Appendix B.4.5-1
Instrument Logic & Control System
Fault Tree

NEDO-33201 Rev 1







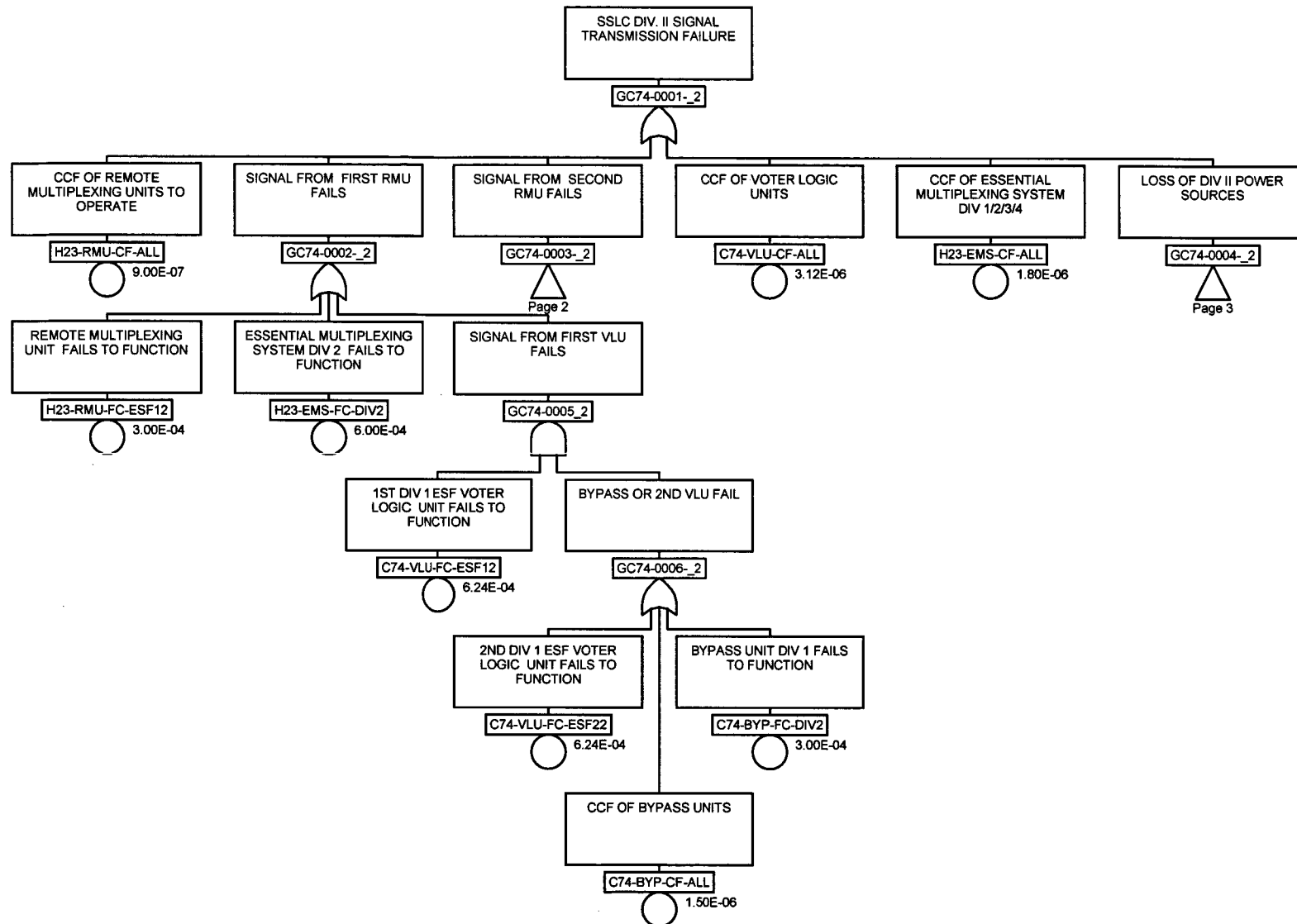
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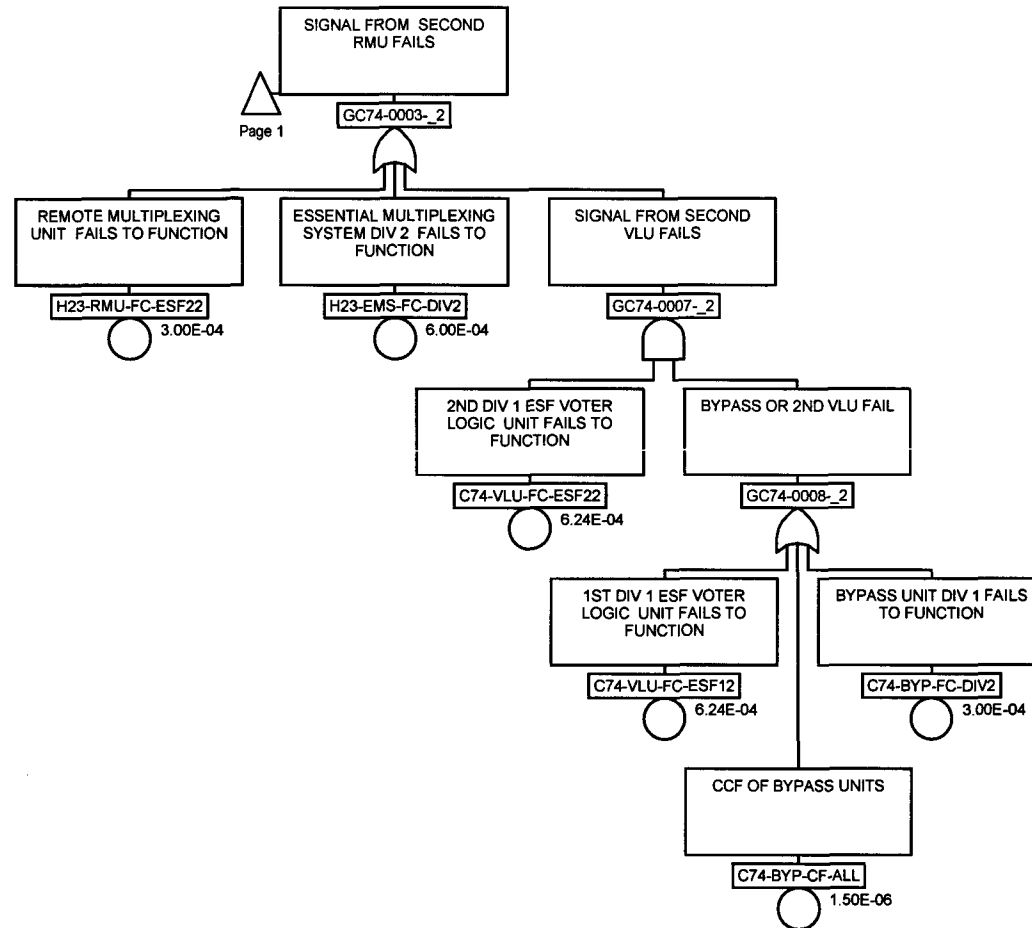
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| C74-VLU-CF-ALL | 1 | 4 | | | | | | | | |
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| C74-VLU-FC-ESF11 | 2 | 3 | | | | | | | | |
| C74-VLU-FC-ESF21 | 1 | 3 | | | | | | | | |
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| GC74-0001- 1 | 1 | 4 | | | | | | | | |
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| GR13-0001- 2 | 3 | 2 | | | | | | | | |
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| H23-EMS-FC-DIV1 | 2 | 2 | | | | | | | | |
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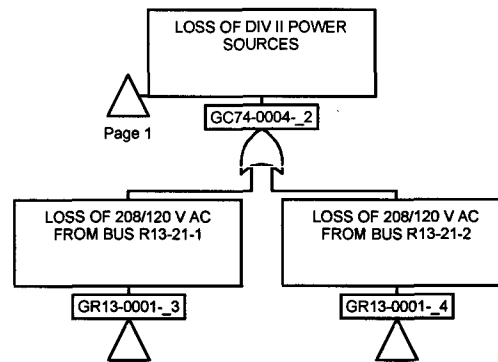
Appendix B.4.5-2

I&C Systems - C74 SSLC - DIV II SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1







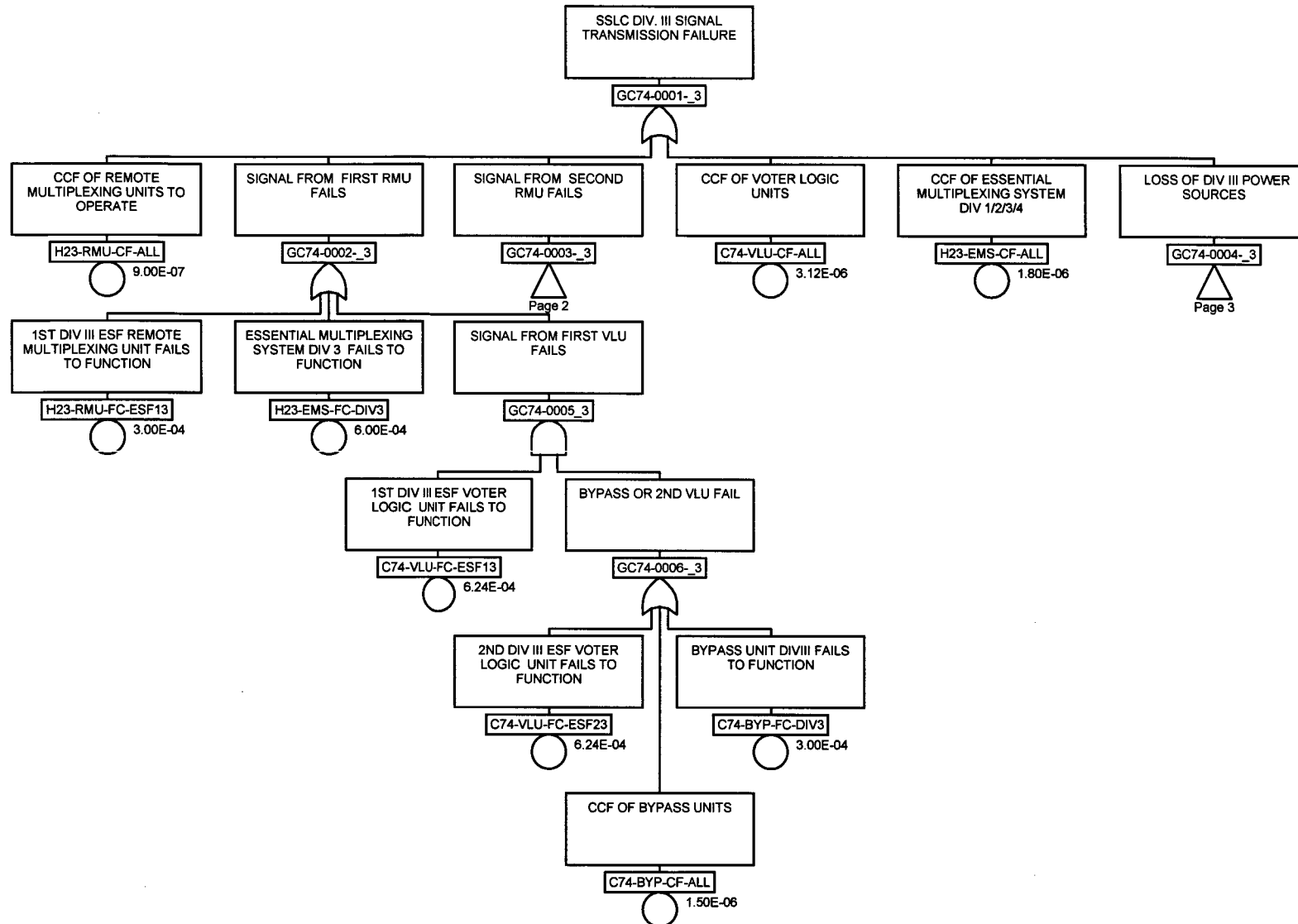
Page 1

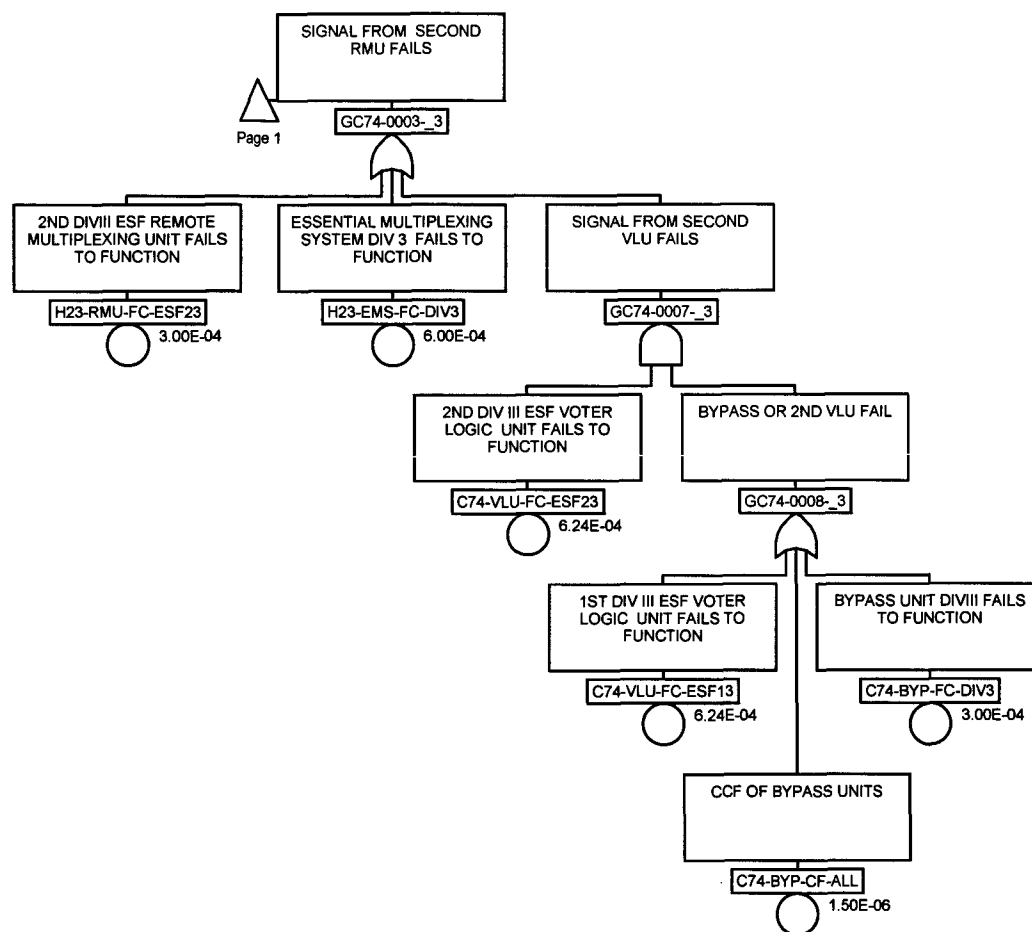
| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone | | | | | | | | |
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| C74-BYP-CF-ALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | | | | | | | | | |
| C74-BYP-CF-ALL | 2 | 4 | | | | | | | | | | | | | | |
| C74-BYP-FC-DIV2 | 1 | 4 | | | | | | | | | | | | | | |
| C74-BYP-FC-DIV2 | 2 | 4 | | | | | | | | | | | | | | |
| C74-VLU-CF-ALL | 1 | 4 | | | | | | | | | | | | | | |
| C74-VLU-FC-ESF12 | 1 | 3 | | | | | | | | | | | | | | |
| C74-VLU-FC-ESF12 | 2 | 3 | | | | | | | | | | | | | | |
| C74-VLU-FC-ESF22 | 1 | 3 | | | | | | | | | | | | | | |
| C74-VLU-FC-ESF22 | 2 | 3 | | | | | | | | | | | | | | |
| GC74-0001- 2 | 1 | 4 | | | | | | | | | | | | | | |
| GC74-0002- 2 | 1 | 2 | | | | | | | | | | | | | | |
| GC74-0003- 2 | 1 | 3 | | | | | | | | | | | | | | |
| GC74-0003- 2 | 2 | 2 | | | | | | | | | | | | | | |
| GC74-0004- 2 | 1 | 6 | | | | | | | | | | | | | | |
| GC74-0004- 2 | 3 | 2 | | | | | | | | | | | | | | |
| GC74-0005 2 | 1 | 3 | | | | | | | | | | | | | | |
| GC74-0006- 2 | 1 | 4 | | | | | | | | | | | | | | |
| GC74-0007- 2 | 2 | 3 | | | | | | | | | | | | | | |
| GC74-0008- 2 | 2 | 4 | | | | | | | | | | | | | | |
| GR13-0001- 3 | 3 | 1 | | | | | | | | | | | | | | |
| GR13-0001- 4 | 3 | 2 | | | | | | | | | | | | | | |
| H23-EMS-CF-ALL | 1 | 5 | | | | | | | | | | | | | | |
| H23-EMS-FC-DIV2 | 1 | 2 | | | | | | | | | | | | | | |
| H23-EMS-FC-DIV2 | 2 | 2 | | | | | | | | | | | | | | |
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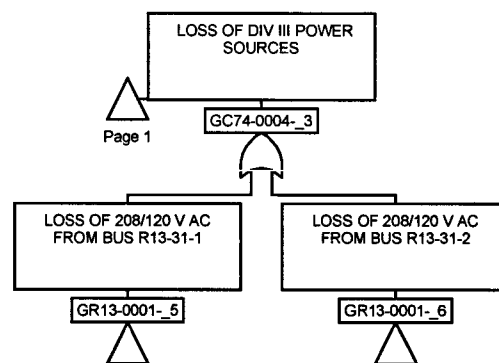
Appendix B.4.5-3

I&C Systems - C74 SSLC - DIV III SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1





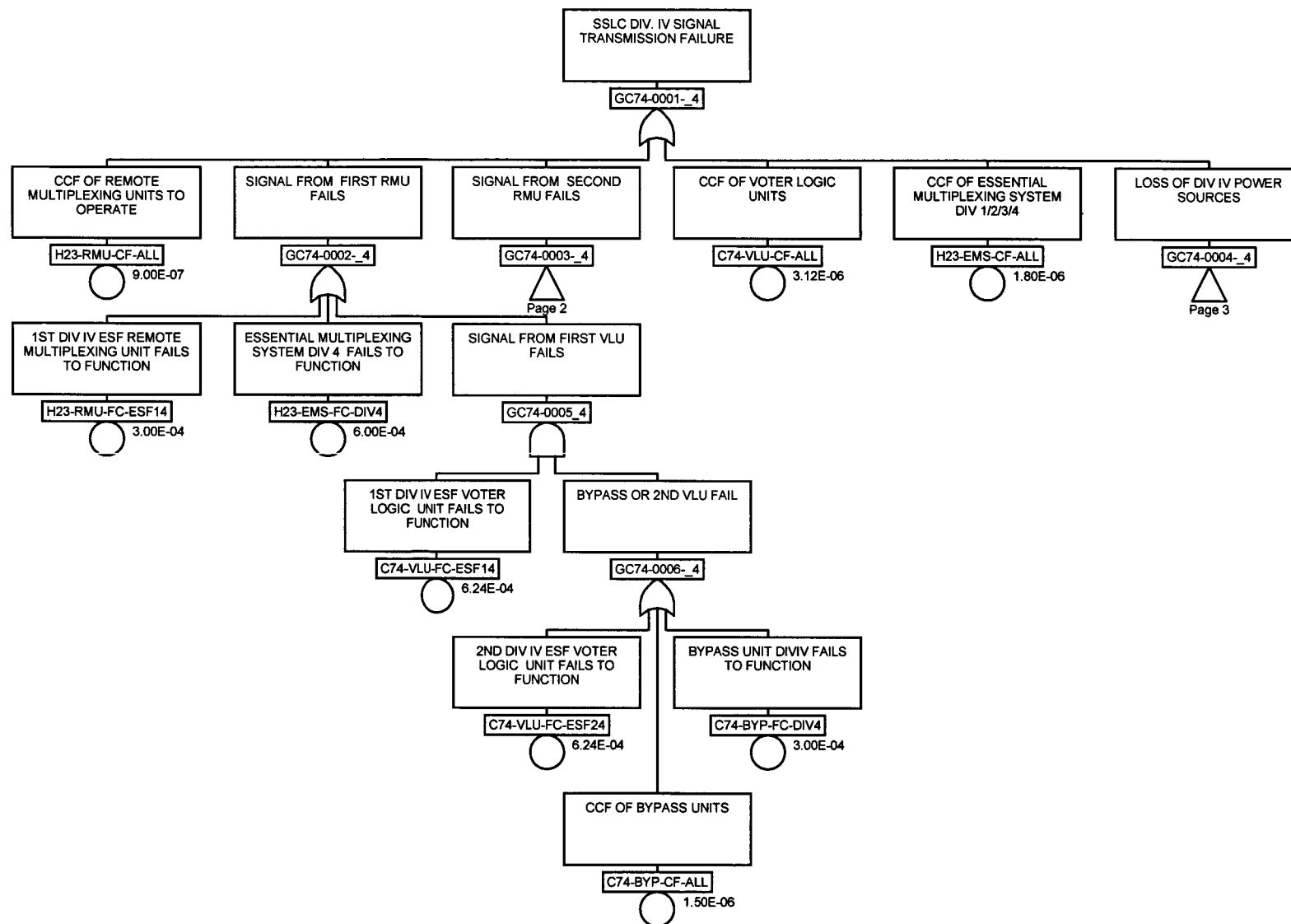


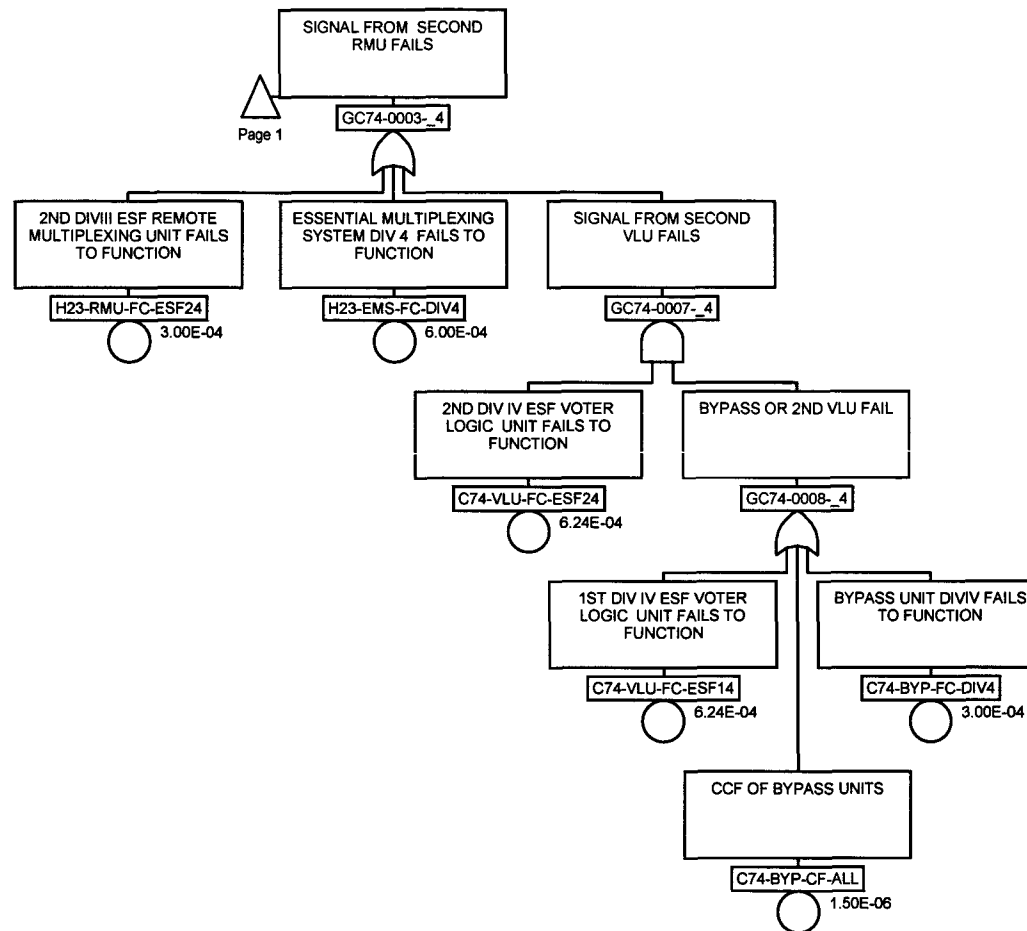
| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone | |
|------------------|------|------|------|------|------|------|------|------|--|
| C74-BYP-CF-ALL | 1 | 4 | | | | | | | |
| C74-BYP-CF-ALL | 2 | 4 | | | | | | | |
| C74-BYP-FC-DIV3 | 1 | 4 | | | | | | | |
| C74-BYP-FC-DIV3 | 2 | 4 | | | | | | | |
| C74-VLU-CF-ALL | 1 | 4 | | | | | | | |
| C74-VLU-FC-ESF13 | 1 | 3 | | | | | | | |
| C74-VLU-FC-ESF13 | 2 | 3 | | | | | | | |
| C74-VLU-FC-ESF23 | 1 | 3 | | | | | | | |
| C74-VLU-FC-ESF23 | 2 | 3 | | | | | | | |
| GC74-0001- 3 | 1 | 4 | | | | | | | |
| GC74-0002- 3 | 1 | 2 | | | | | | | |
| GC74-0003- 3 | 1 | 3 | | | | | | | |
| GC74-0003- 3 | 2 | 2 | | | | | | | |
| GC74-0004- 3 | 1 | 6 | | | | | | | |
| GC74-0004- 3 | 3 | 2 | | | | | | | |
| GC74-0005 3 | 1 | 3 | | | | | | | |
| GC74-0006- 3 | 1 | 4 | | | | | | | |
| GC74-0007- 3 | 2 | 3 | | | | | | | |
| GC74-0008- 3 | 2 | 4 | | | | | | | |
| GR13-0001- 5 | 3 | 1 | | | | | | | |
| GR13-0001- 6 | 3 | 2 | | | | | | | |
| H23-EMS-CF-ALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DIV3 | 1 | 2 | | | | | | | |
| H23-EMS-FC-DIV3 | 2 | 2 | | | | | | | |
| H23-RMU-CF-ALL | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF13 | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF23 | 2 | 1 | | | | | | | |
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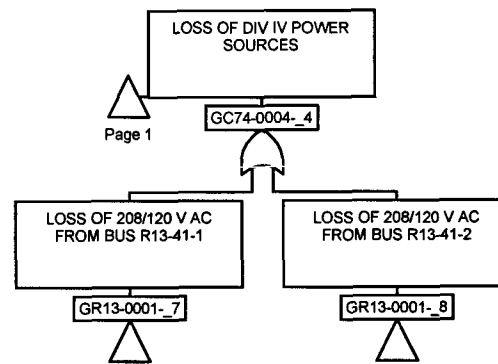
Appendix B.4.5-4

I&C Systems - C74 SSLC - DIV IV SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1



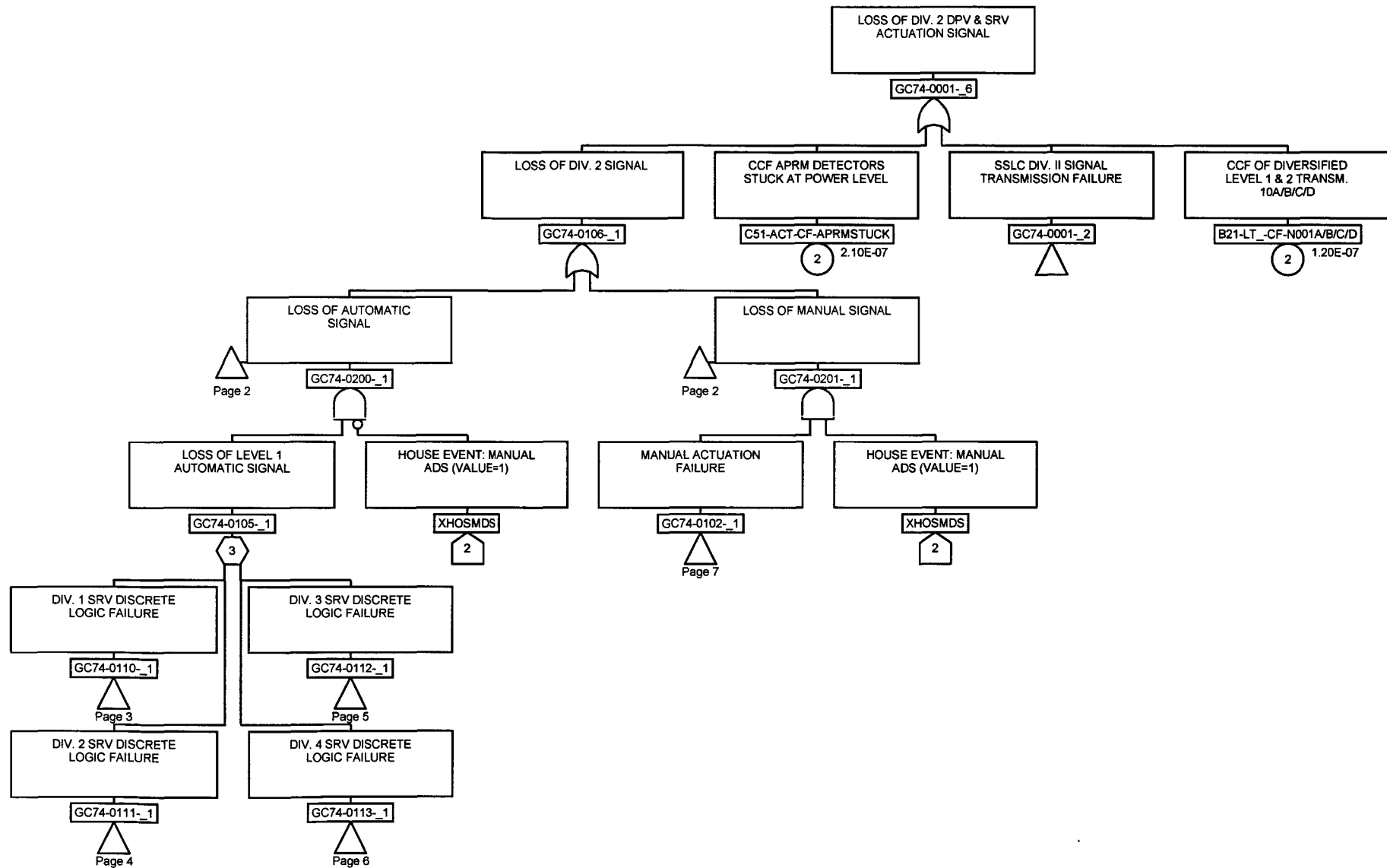


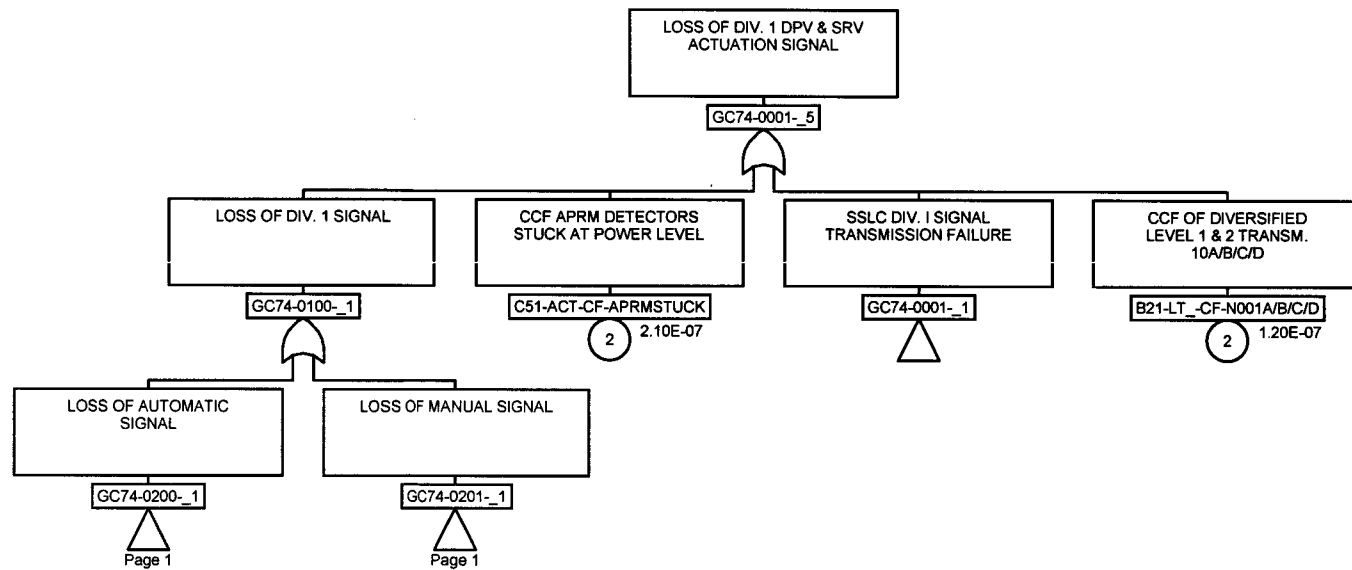


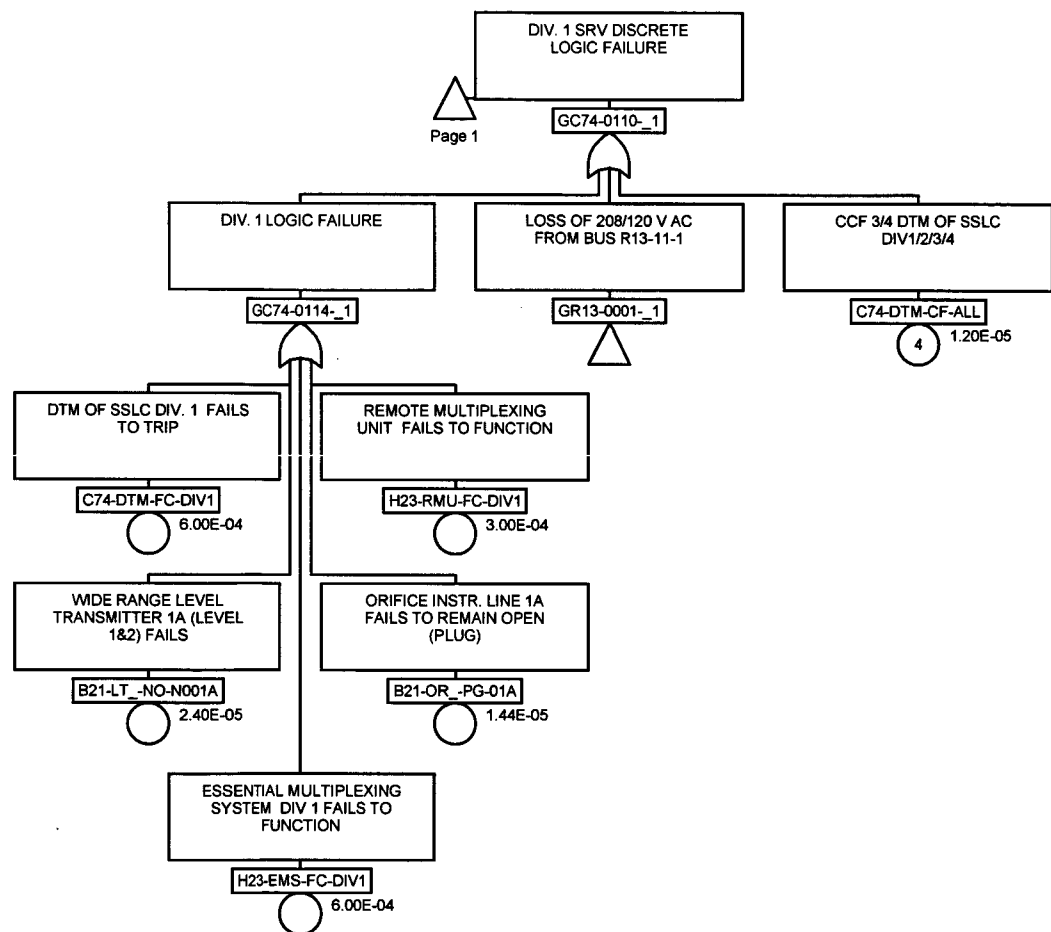
Appendix B.4.5-5

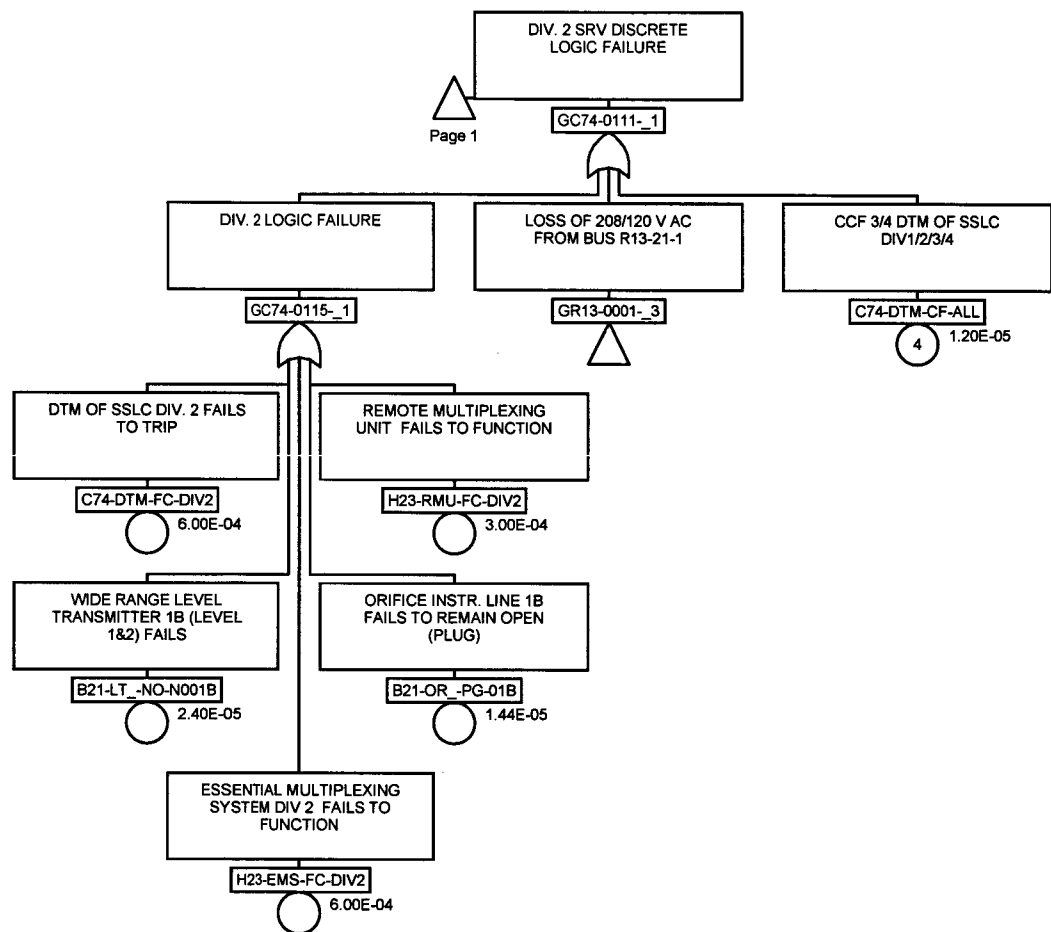
I&C Systems - C74 Auto Depressurization Signal (ADS)

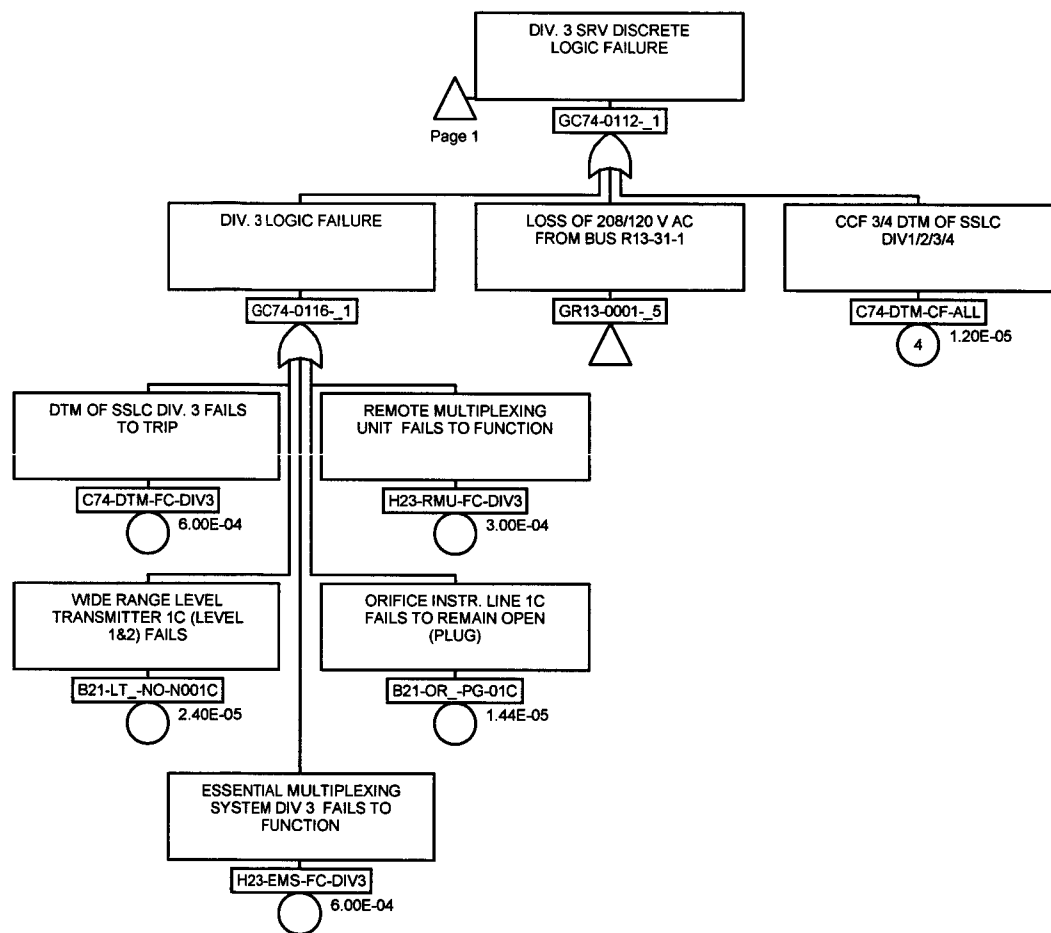
NEDO-33201 Rev 1

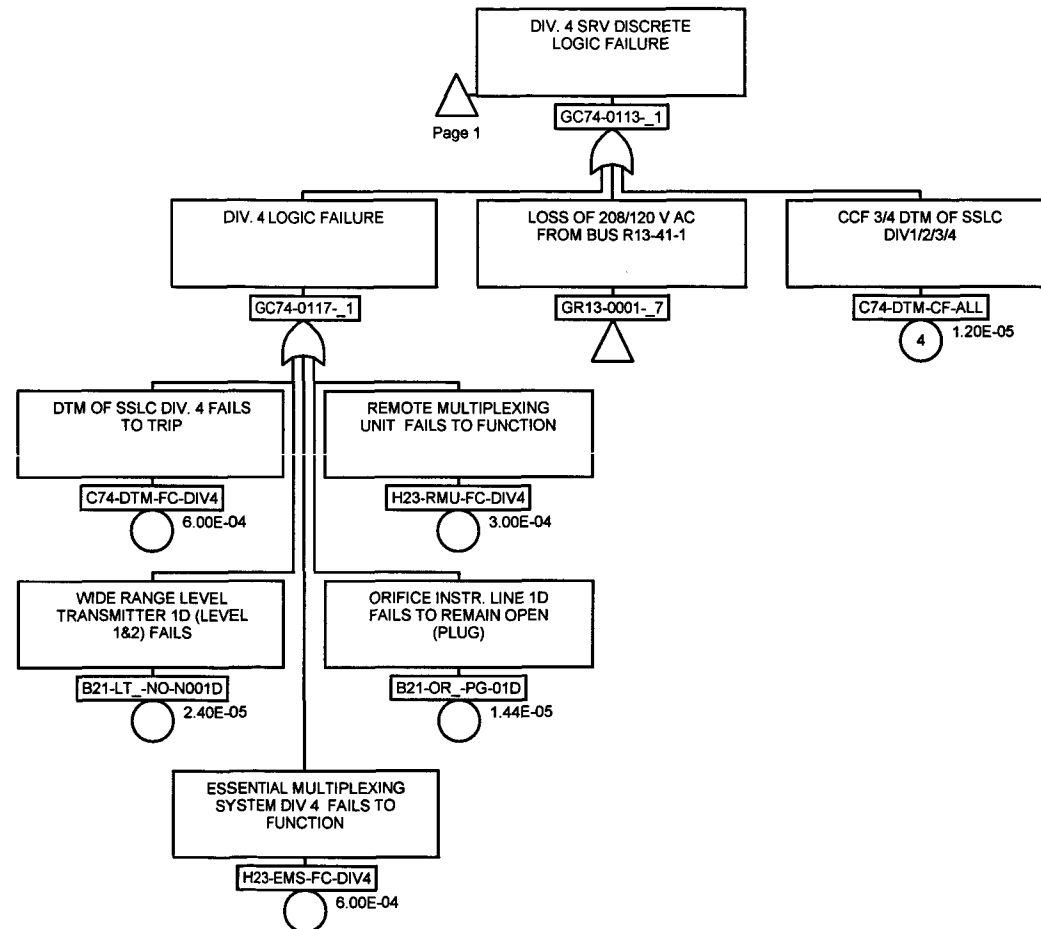


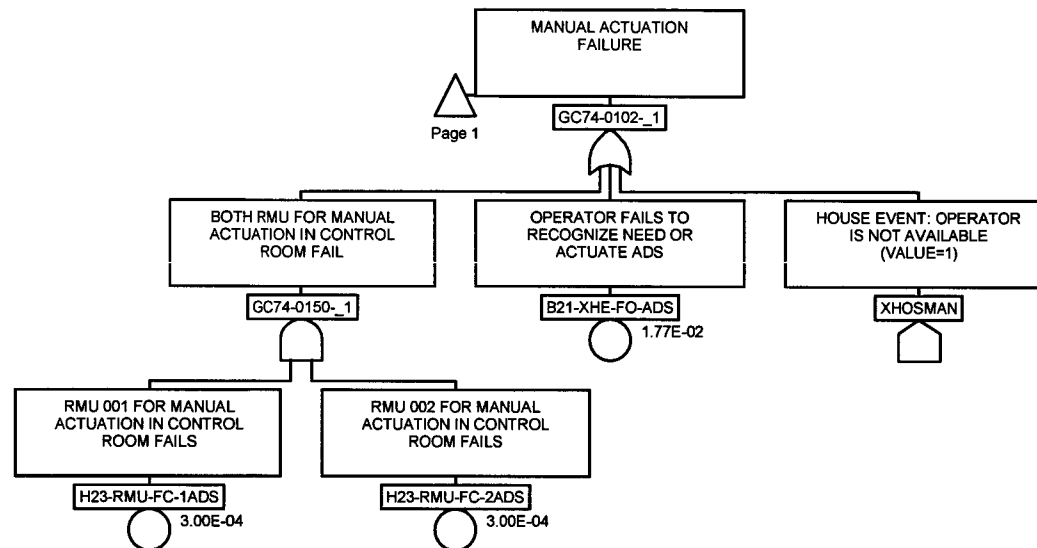










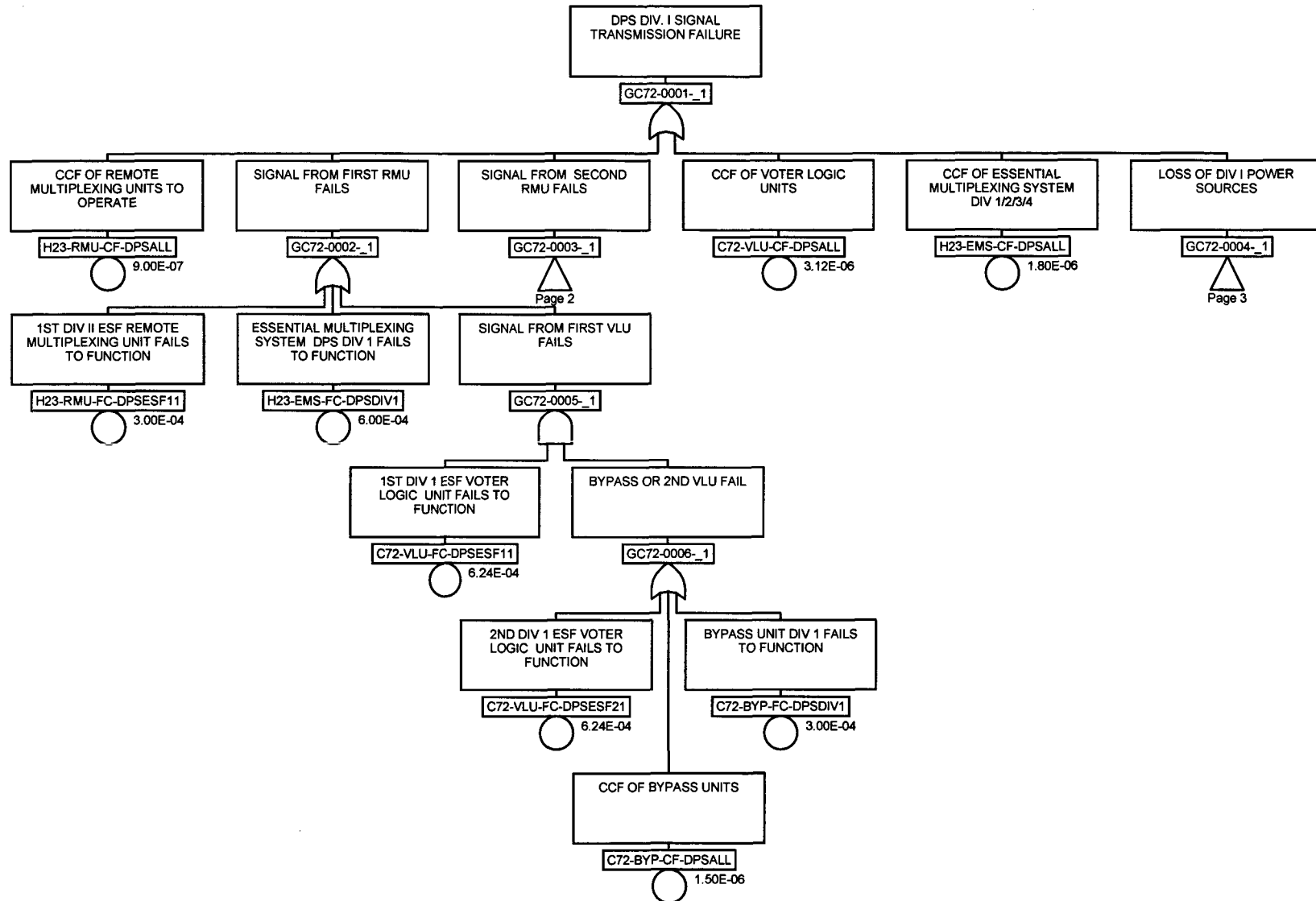


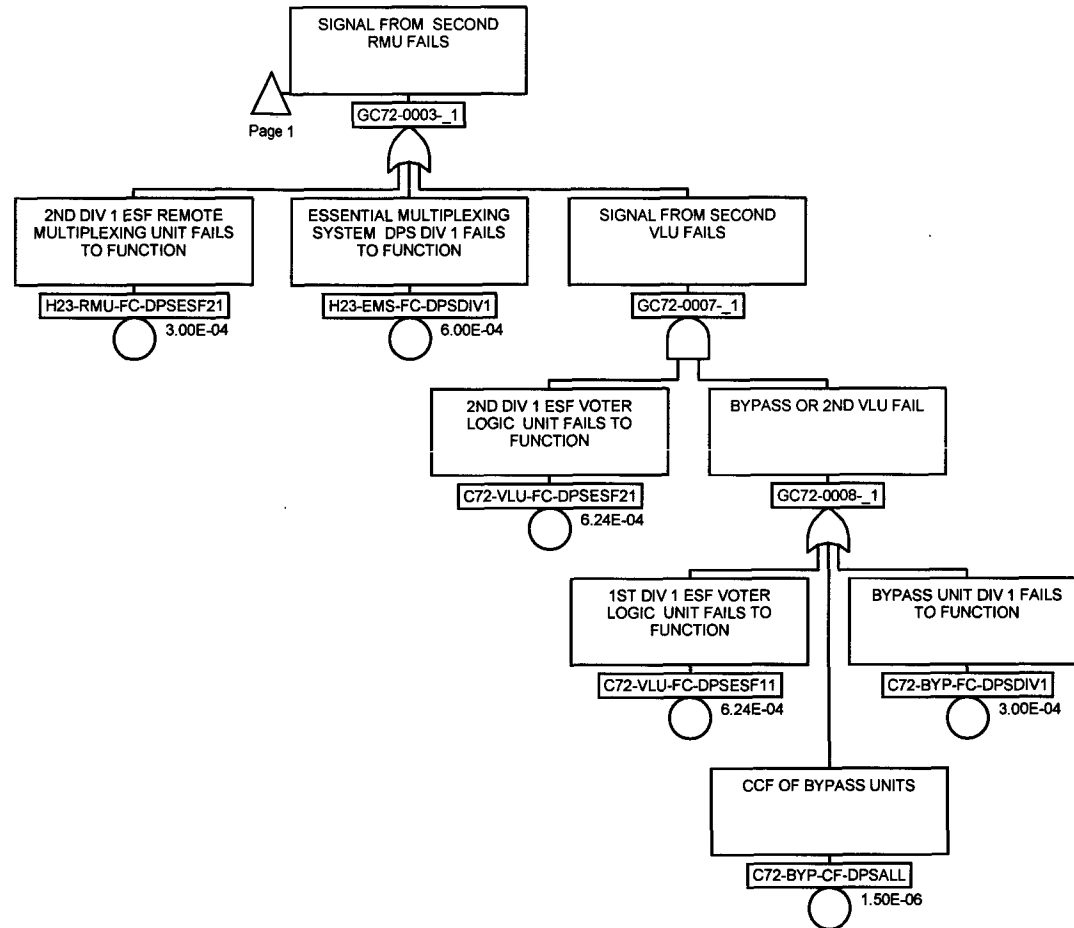
| Name | Page | Zone | Name NEDO-33201 Rev 1 | Page | Zone | |
|--|------|------|---------------------------------------|------|------|--|
| B21-LT -CF-N001A/B/C/D | 1 | 6 | H23-EMS-FC-DIV4 | 6 | 2 | |
| B21-LT -CF-N001A/B/C/D | 2 | 5 | H23-RMU-FC-1ADS | 7 | 1 | |
| B21-LT -NO-N001A | 3 | 1 | H23-RMU-FC-2ADS | 7 | 2 | |
| B21-LT -NO-N001B | 4 | 1 | H23-RMU-FC-DIV1 | 3 | 2 | |
| B21-LT -NO-N001C | 5 | 1 | H23-RMU-FC-DIV2 | 4 | 2 | |
| B21-LT -NO-N001D | 6 | 1 | H23-RMU-FC-DIV3 | 5 | 2 | |
| B21-OR -PG-01A | 3 | 2 | H23-RMU-FC-DIV4 | 6 | 2 | |
| B21-OR -PG-01B | 4 | 2 | XHOSMAN | 7 | 4 | |
| B21-OR -PG-01C | 5 | 2 | XHOSMDS | 1 | 3 | |
| B21-OR -PG-01D | 6 | 2 | XHOSMDS | 1 | 5 | |
| B21-XHE-FO-ADS | 7 | 3 | | | | |
| C51-ACT-CF-APRMSTUCK | 1 | 4 | | | | |
| C51-ACT-CF-APRMSTUCK | 2 | 3 | | | | |
| C74-DTM-CF-ALL | 3 | 4 | | | | |
| C74-DTM-CF-ALL | 4 | 4 | | | | |
| C74-DTM-CF-ALL | 5 | 4 | | | | |
| C74-DTM-CF-ALL | 6 | 4 | | | | |
| C74-DTM-FC-DIV1 | 3 | 1 | | | | |
| C74-DTM-FC-DIV2 | 4 | 1 | | | | |
| C74-DTM-FC-DIV3 | 5 | 1 | | | | |
| C74-DTM-FC-DIV4 | 6 | 1 | | | | |
| GC74-0001- 1 | 2 | 4 | | | | |
| GC74-0001- 2 | 1 | 5 | | | | |
| GC74-0001- 5 | 2 | 3 | | | | |
| GC74-0001- 6 | 1 | 4 | | | | |
| GC74-0100- 1 | 2 | 2 | | | | |
| GC74-0102- 1 | 1 | 4 | | | | |
| GC74-0102- 1 | 7 | 2 | | | | |
| GC74-0105- 1 | 1 | 2 | | | | |
| GC74-0106- 1 | 1 | 3 | | | | |
| GC74-0110- 1 | 1 | 1 | | | | |
| GC74-0110- 1 | 3 | 2 | | | | |
| GC74-0111- 1 | 1 | 1 | | | | |
| GC74-0111- 1 | 4 | 2 | | | | |
| GC74-0112- 1 | 1 | 2 | | | | |
| GC74-0112- 1 | 5 | 2 | | | | |
| GC74-0113- 1 | 1 | 2 | | | | |
| GC74-0113- 1 | 6 | 2 | | | | |
| GC74-0114- 1 | 3 | 2 | | | | |
| GC74-0115- 1 | 4 | 2 | | | | |
| GC74-0116- 1 | 5 | 2 | | | | |
| GC74-0117- 1 | 6 | 2 | | | | |
| GC74-0150- 1 | 7 | 2 | | | | |
| GC74-0200- 1 | 1 | 2 | | | | |
| GC74-0200- 1 | 2 | 1 | | | | |
| GC74-0201- 1 | 1 | 4 | | | | |
| GC74-0201- 1 | 2 | 2 | | | | |
| GR13-0001- 1 | 3 | 3 | | | | |
| GR13-0001- 3 | 4 | 3 | | | | |
| GR13-0001- 5 | 5 | 3 | | | | |
| GR13-0001- 7 | 6 | 3 | | | | |
| H23-EMS-FC-DIV1 | 3 | 2 | | | | |
| H23-EMS-FC-DIV2 | 4 | 2 | | | | |
| H23-EMS-FC-DIV3 | 5 | 2 | | | | |
| I&C Systems - C74 Auto Depressurization Signal (ADS) | | | AD Signal.caf Appendix B.4.5-5 Page 8 | | | |

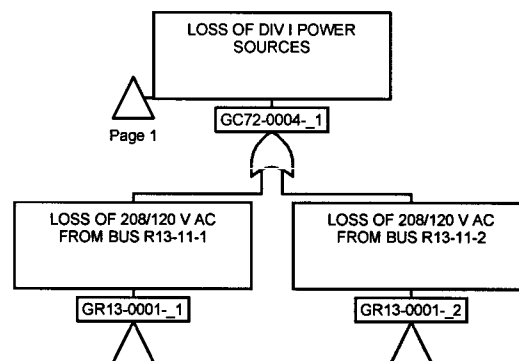
Appendix B.4.5-6

I&C Systems - C72 - DPS DIV I SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1





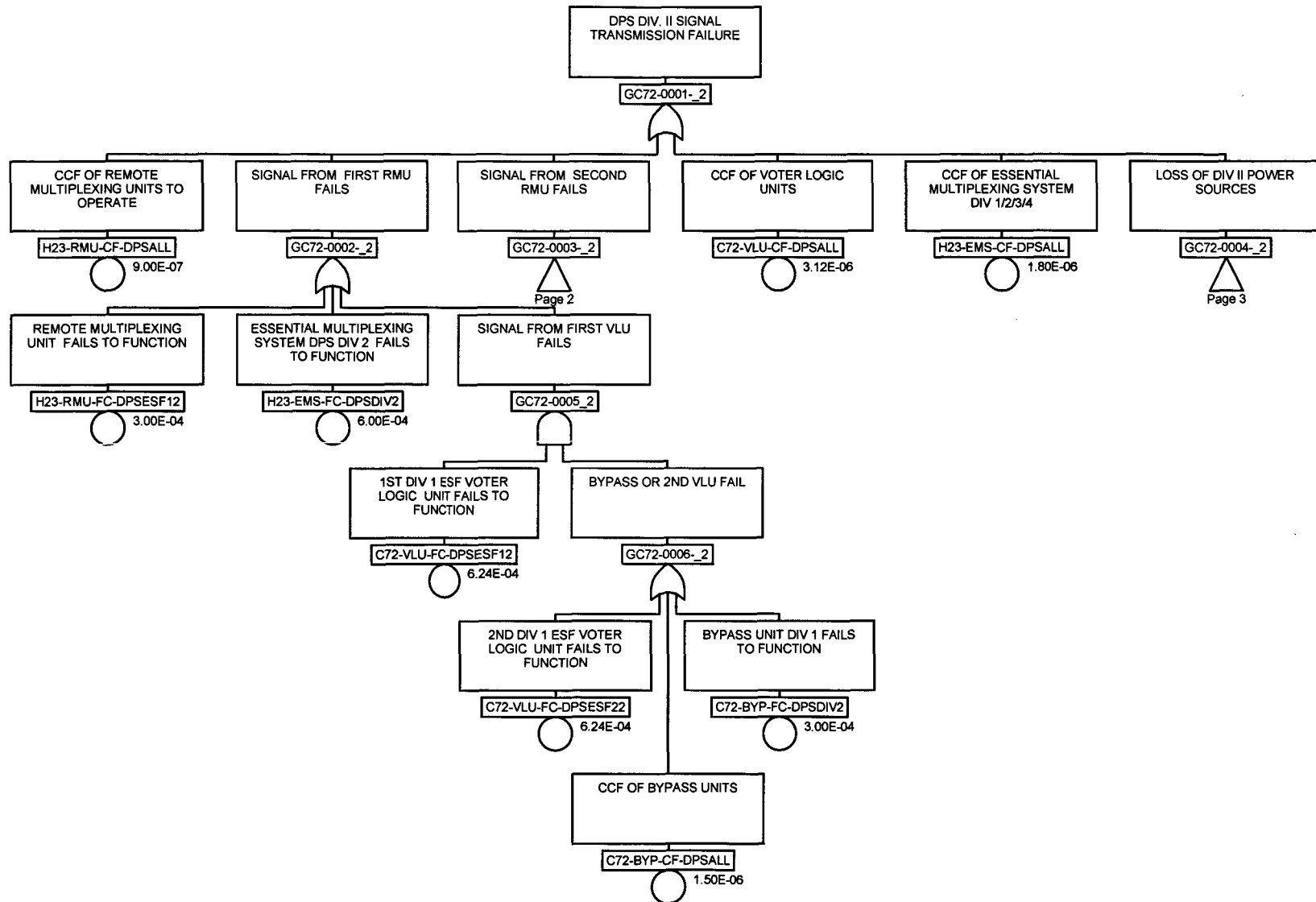


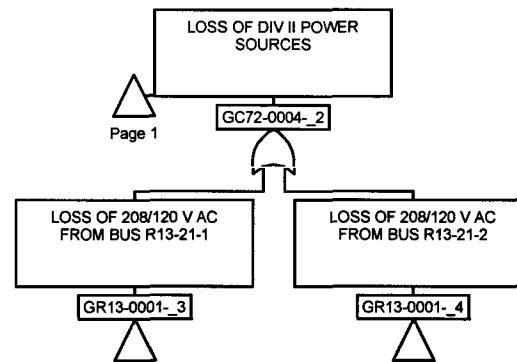
| Name | Page | Zone | Name | NEDO-33201 Rev 1 | Page | Zone | Name | Page | Zone |
|---|------|------|------------------|------------------|------|------|------------------|------|--------|
| C72-BYP-CF-DPSALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | | |
| C72-BYP-CF-DPSALL | 2 | 4 | | | | | | | |
| C72-BYP-FC-DPSDIV1 | 1 | 4 | | | | | | | |
| C72-BYP-FC-DPSDIV1 | 2 | 4 | | | | | | | |
| C72-VLU-CF-DPSALL | 1 | 4 | | | | | | | |
| C72-VLU-FC-DPSESF11 | 1 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF11 | 2 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF21 | 1 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF21 | 2 | 3 | | | | | | | |
| GC72-0001- 1 | 1 | 4 | | | | | | | |
| GC72-0002- 1 | 1 | 2 | | | | | | | |
| GC72-0003- 1 | 1 | 3 | | | | | | | |
| GC72-0003- 1 | 2 | 2 | | | | | | | |
| GC72-0004- 1 | 1 | 6 | | | | | | | |
| GC72-0004- 1 | 3 | 2 | | | | | | | |
| GC72-0005- 1 | 1 | 3 | | | | | | | |
| GC72-0006- 1 | 1 | 4 | | | | | | | |
| GC72-0007- 1 | 2 | 3 | | | | | | | |
| GC72-0008- 1 | 2 | 4 | | | | | | | |
| GR13-0001- 1 | 3 | 1 | | | | | | | |
| GR13-0001- 2 | 3 | 2 | | | | | | | |
| H23-EMS-CF-DPSALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DPSDIV1 | 1 | 2 | | | | | | | |
| H23-EMS-FC-DPSDIV1 | 2 | 2 | | | | | | | |
| H23-RMU-CF-DPSALL | 1 | 1 | | | | | | | |
| H23-RMU-FC-DPSESF11 | 1 | 1 | | | | | | | |
| H23-RMU-FC-DPSESF21 | 2 | 1 | | | | | | | |
| I&C Systems - C72 - DPS DIV I SIGNAL TRANSMISSION FAILURE | | | | | | | | | |
| C72(1).CAF | | | | | | | Appendix B.4.5-6 | | Page 4 |

Appendix B.4.5-7

I&C Systems - C72 - DPS DIV II SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1





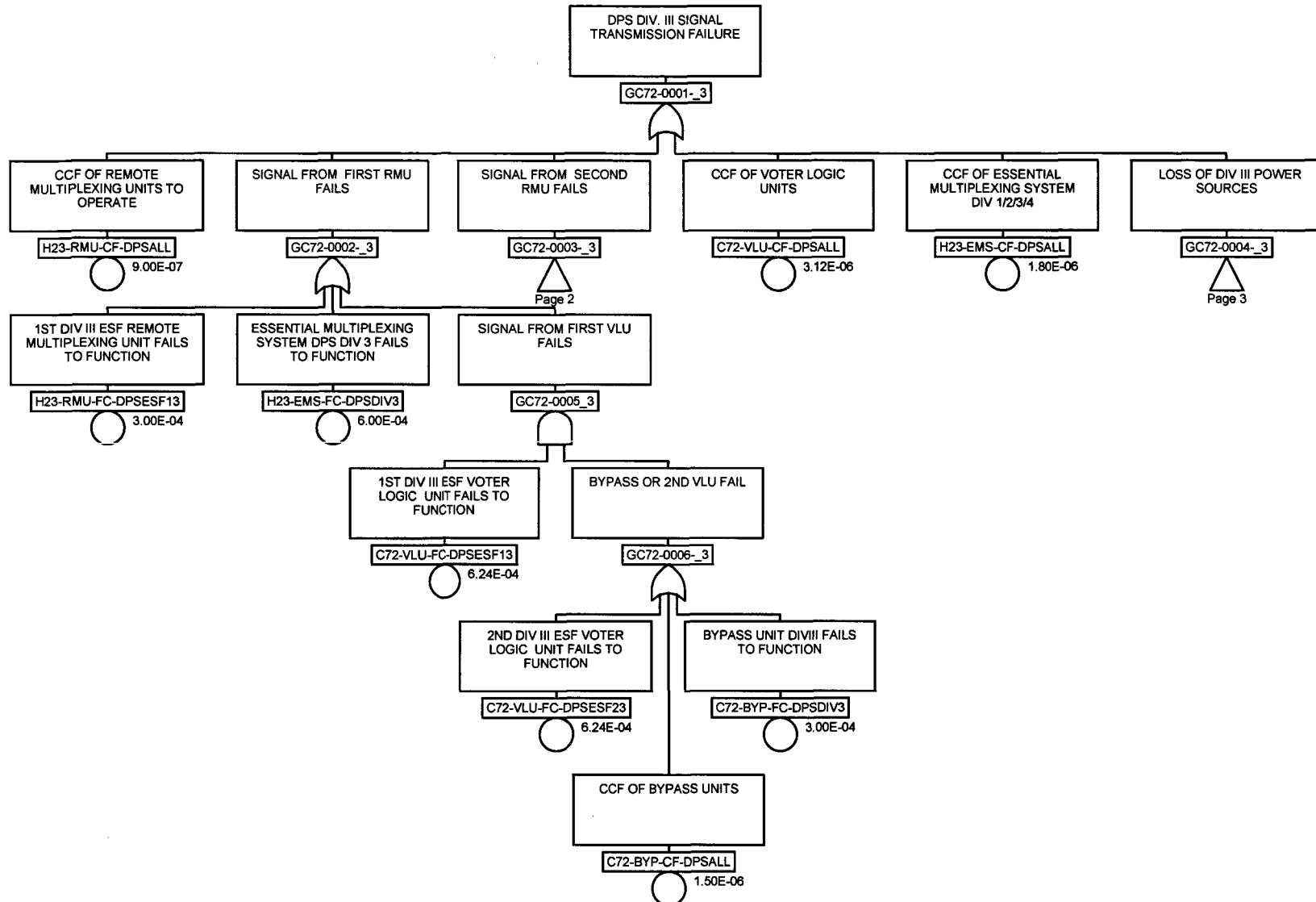
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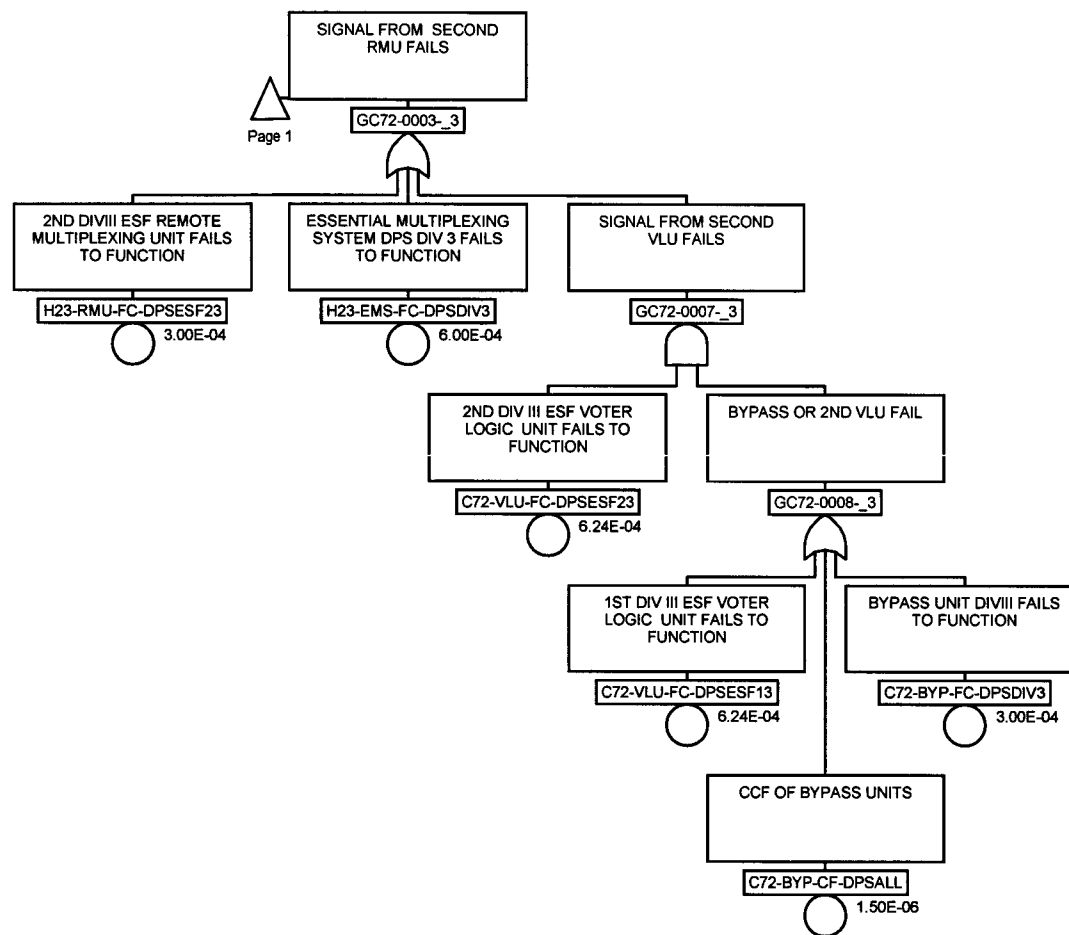
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| C72-BYP-CF-DPSALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | |
| C72-BYP-CF-DPSALL | 2 | 4 | | | | | | |
| C72-BYP-FC-DPSDIV2 | 1 | 4 | | | | | | |
| C72-BYP-FC-DPSDIV2 | 2 | 4 | | | | | | |
| C72-VLU-CF-DPSALL | 1 | 4 | | | | | | |
| C72-VLU-FC-DPSESF12 | 1 | 3 | | | | | | |
| C72-VLU-FC-DPSESF12 | 2 | 3 | | | | | | |
| C72-VLU-FC-DPSESF22 | 1 | 3 | | | | | | |
| C72-VLU-FC-DPSESF22 | 2 | 3 | | | | | | |
| GC72-0001- 2 | 1 | 4 | | | | | | |
| GC72-0002- 2 | 1 | 2 | | | | | | |
| GC72-0003- 2 | 1 | 3 | | | | | | |
| GC72-0003- 2 | 2 | 2 | | | | | | |
| GC72-0004- 2 | 1 | 6 | | | | | | |
| GC72-0004- 2 | 3 | 2 | | | | | | |
| GC72-0005 2 | 1 | 3 | | | | | | |
| GC72-0006- 2 | 1 | 4 | | | | | | |
| GC72-0007- 2 | 2 | 3 | | | | | | |
| GC72-0008- 2 | 2 | 4 | | | | | | |
| GR13-0001- 3 | 3 | 1 | | | | | | |
| GR13-0001- 4 | 3 | 2 | | | | | | |
| H23-EMS-CF-DPSALL | 1 | 5 | | | | | | |
| H23-EMS-FC-DPSDIV2 | 1 | 2 | | | | | | |
| H23-EMS-FC-DPSDIV2 | 2 | 2 | | | | | | |
| H23-RMU-CF-DPSALL | 1 | 1 | | | | | | |
| H23-RMU-FC-DPSESF12 | 1 | 1 | | | | | | |
| H23-RMU-FC-DPSESF22 | 2 | 1 | | | | | | |
| I&C Systems - C72 - DPS DIV II SIGNAL TRANSMISSION FAILURE | | | C72(2).CAF | | | Appendix B.4.5-7 | | |
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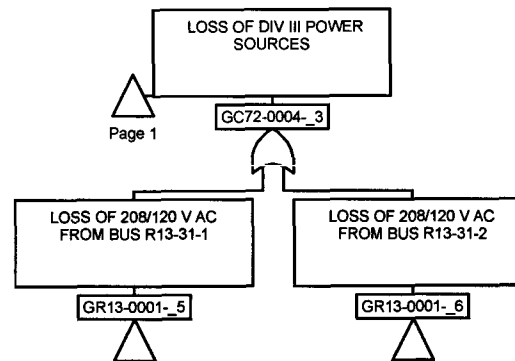
Appendix B.4.5-8

I&C Systems - C72 - DPS DIV III SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1



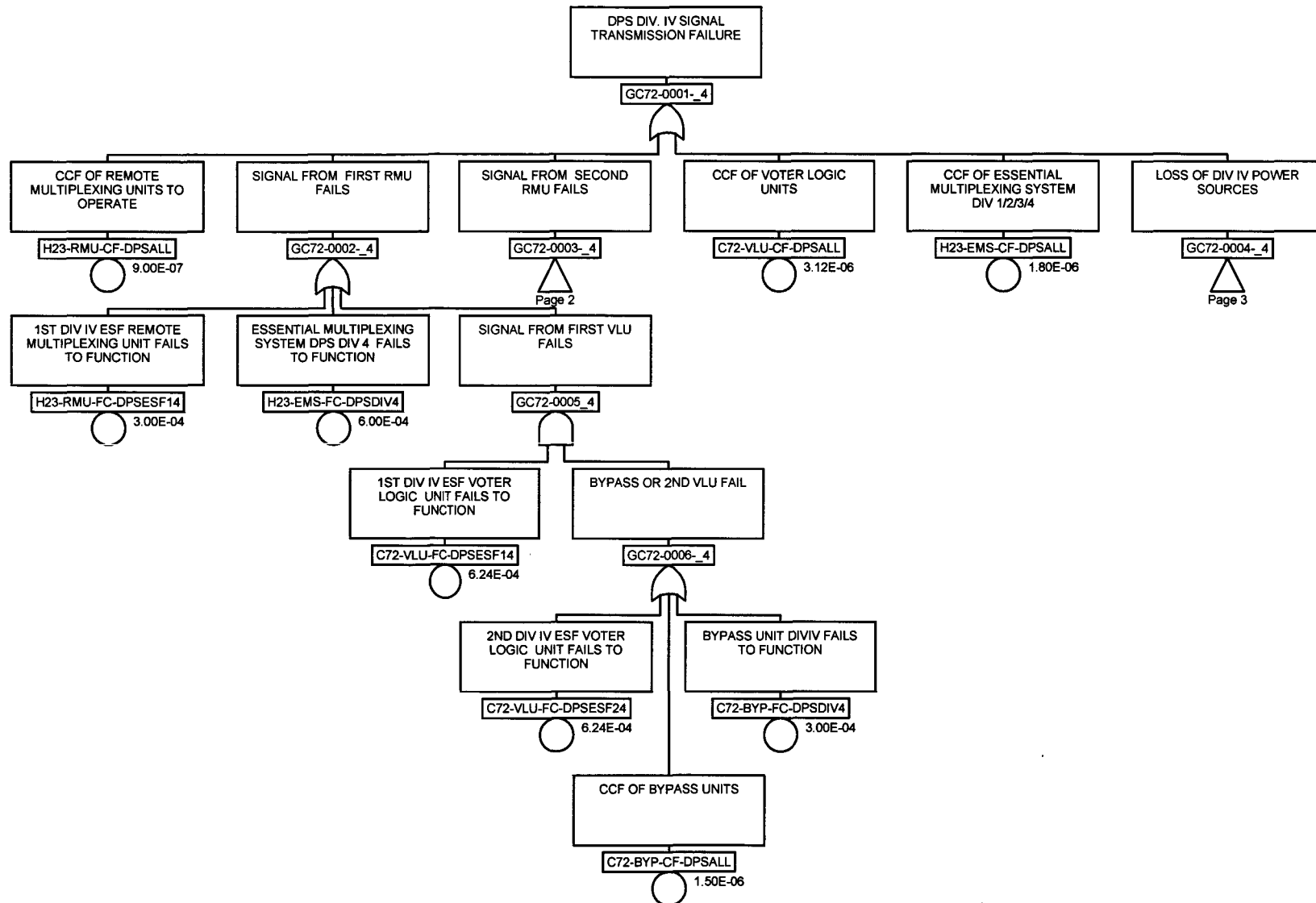


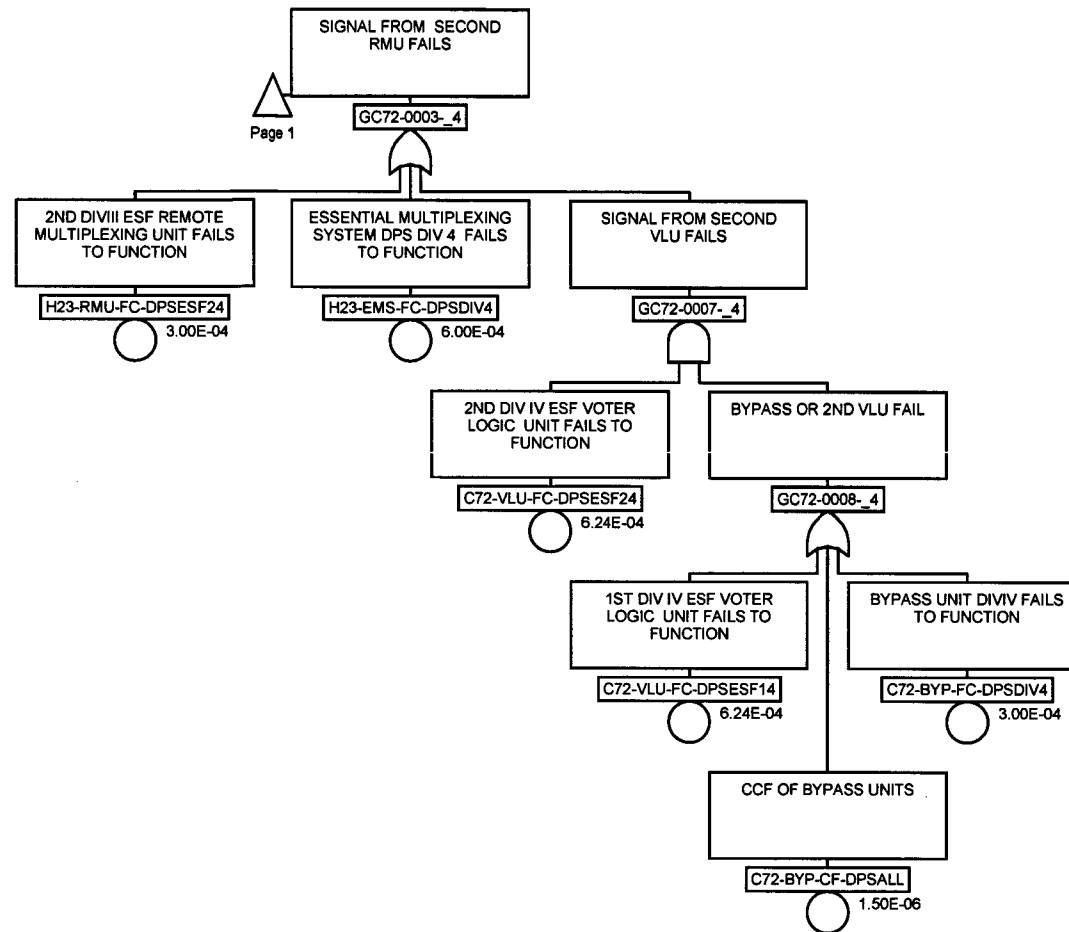


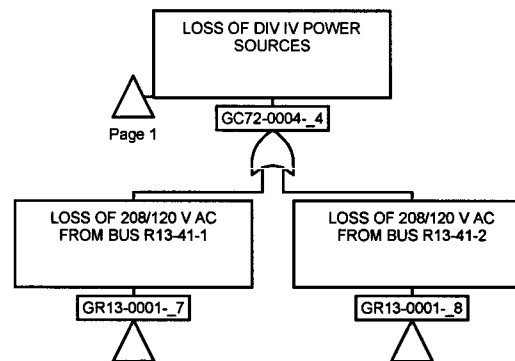
Appendix B.4.5-9

I&C Systems - C72 - DPS DIV IV SIGNAL TRANSMISSION FAILURE

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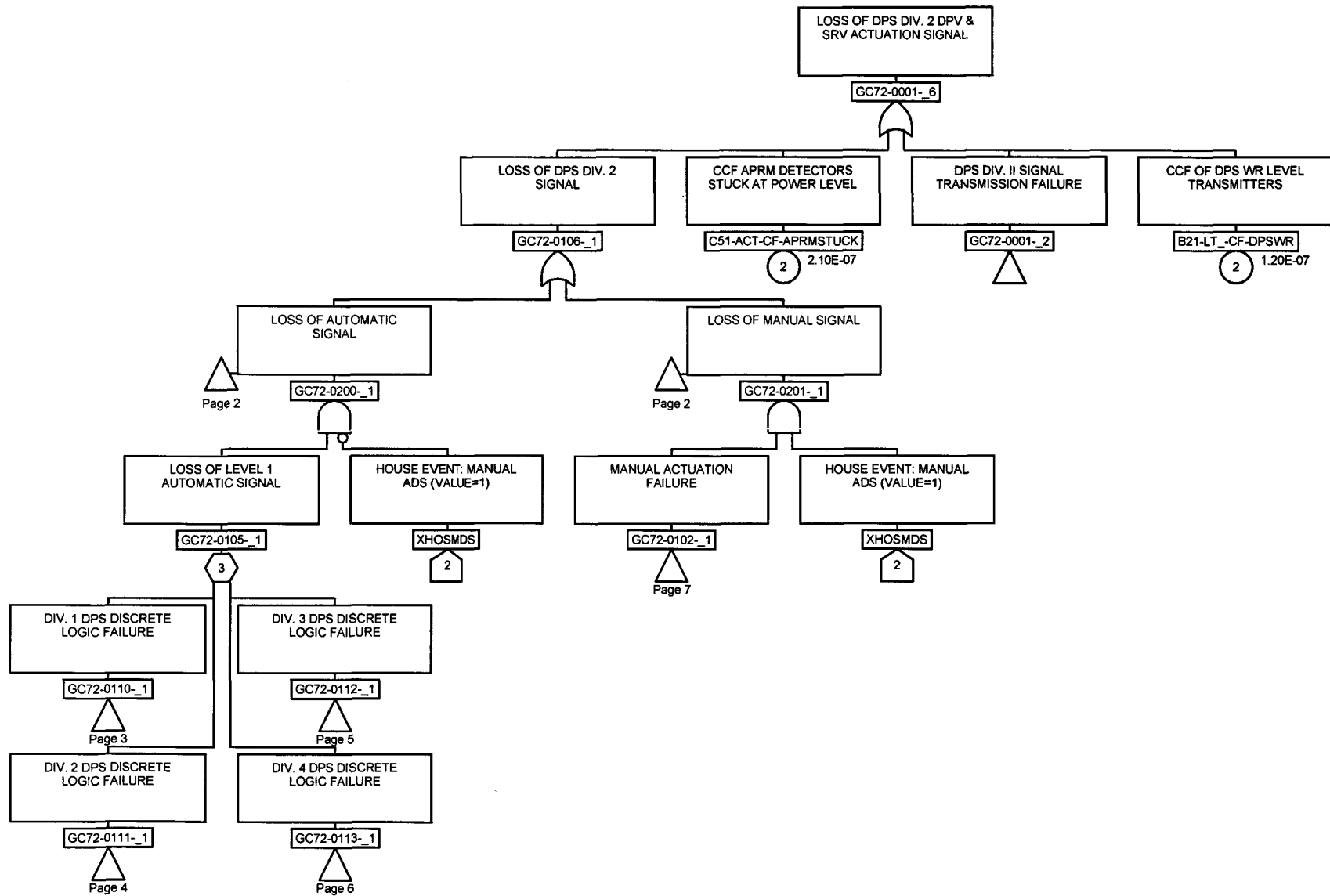
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| C72-BYP-CF-DPSALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | | |
| C72-BYP-CF-DPSALL | 2 | 4 | | | | | | | |
| C72-BYP-FC-DPSDIV4 | 1 | 4 | | | | | | | |
| C72-BYP-FC-DPSDIV4 | 2 | 4 | | | | | | | |
| C72-VLU-CF-DPSALL | 1 | 4 | | | | | | | |
| C72-VLU-FC-DPSESF14 | 1 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF14 | 2 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF24 | 1 | 3 | | | | | | | |
| C72-VLU-FC-DPSESF24 | 2 | 3 | | | | | | | |
| GC72-0001- 4 | 1 | 4 | | | | | | | |
| GC72-0002- 4 | 1 | 2 | | | | | | | |
| GC72-0003- 4 | 1 | 3 | | | | | | | |
| GC72-0003- 4 | 2 | 2 | | | | | | | |
| GC72-0004- 4 | 1 | 6 | | | | | | | |
| GC72-0004- 4 | 3 | 2 | | | | | | | |
| GC72-0005 4 | 1 | 3 | | | | | | | |
| GC72-0006- 4 | 1 | 4 | | | | | | | |
| GC72-0007- 4 | 2 | 3 | | | | | | | |
| GC72-0008- 4 | 2 | 4 | | | | | | | |
| GR13-0001- 7 | 3 | 1 | | | | | | | |
| GR13-0001- 8 | 3 | 2 | | | | | | | |
| H23-EMS-CF-DPSALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DPSDIV4 | 1 | 2 | | | | | | | |
| H23-EMS-FC-DPSDIV4 | 2 | 2 | | | | | | | |
| H23-RMU-CF-DPSALL | 1 | 1 | | | | | | | |
| H23-RMU-FC-DPSESF14 | 1 | 1 | | | | | | | |
| H23-RMU-FC-DPSESF24 | 2 | 1 | | | | | | | |
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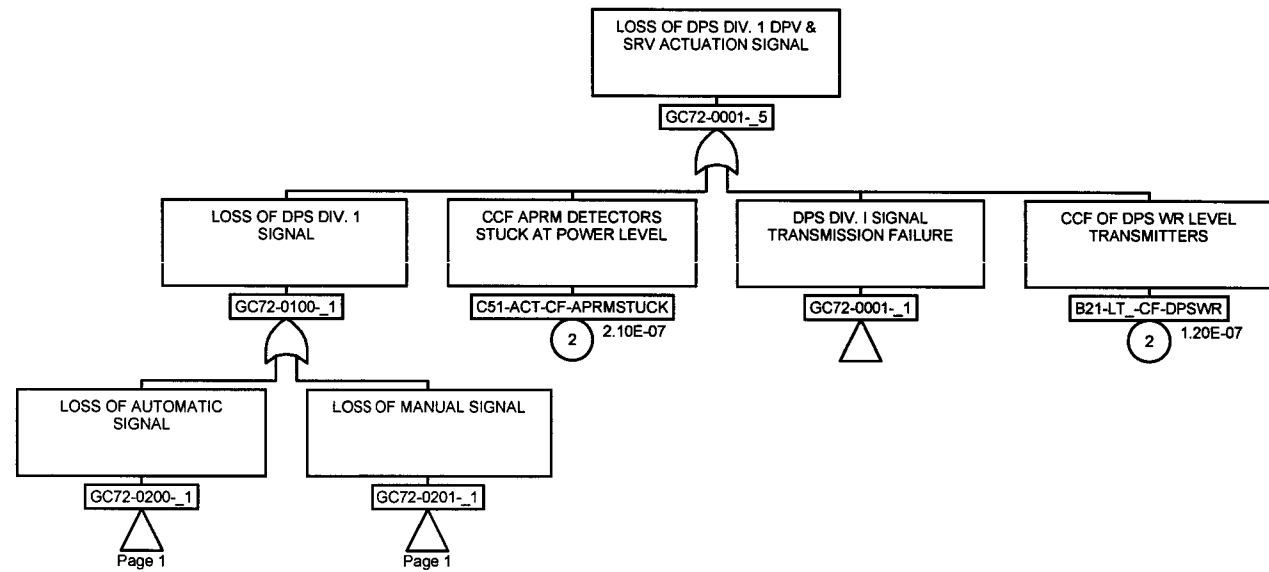
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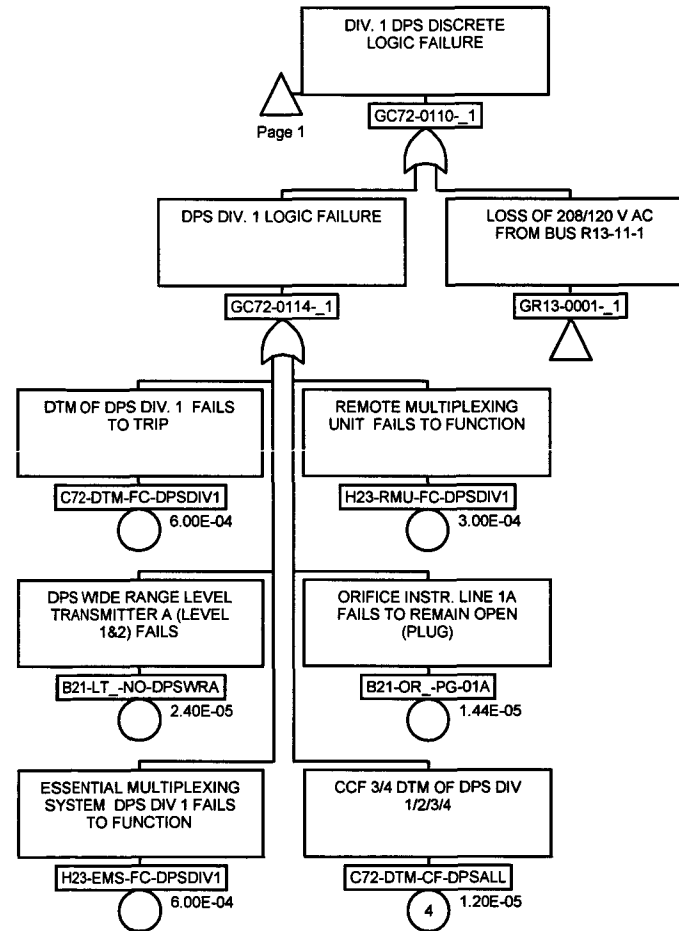
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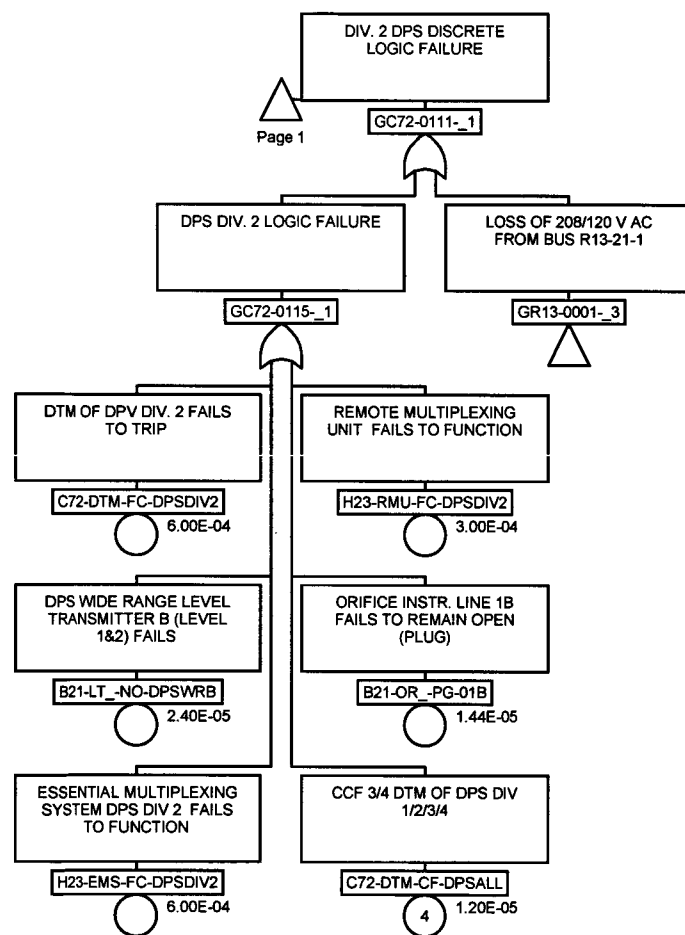
NEDO-33201 Rev 1

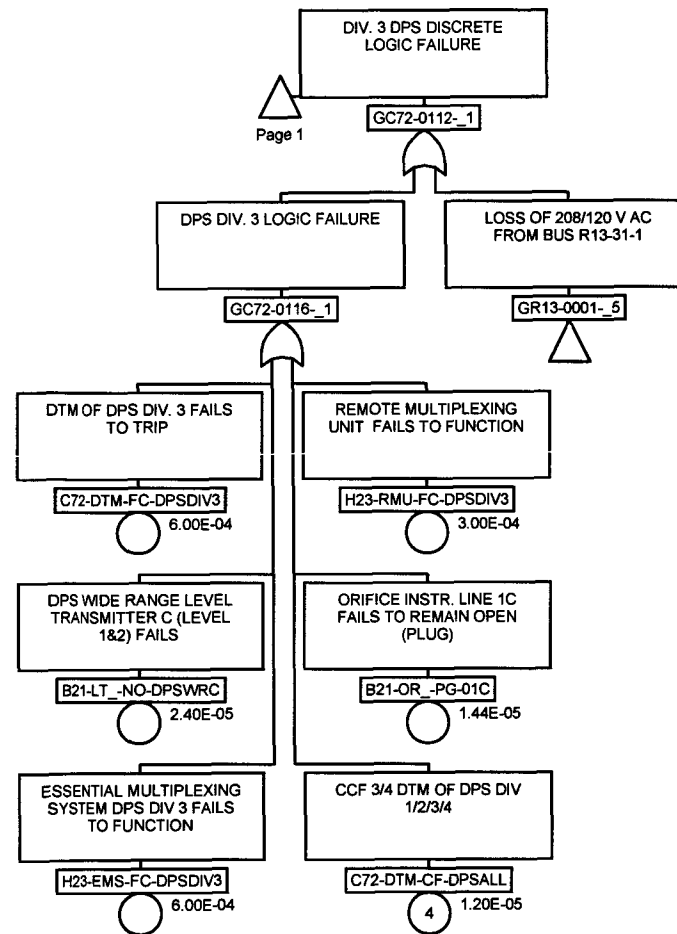


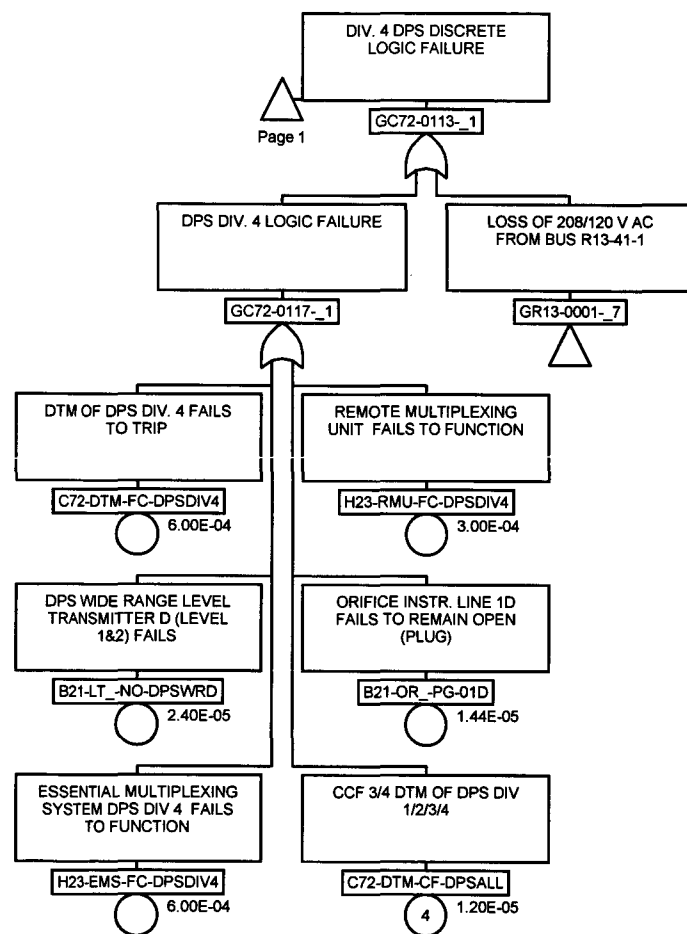
NEDO-33201 Rev 1

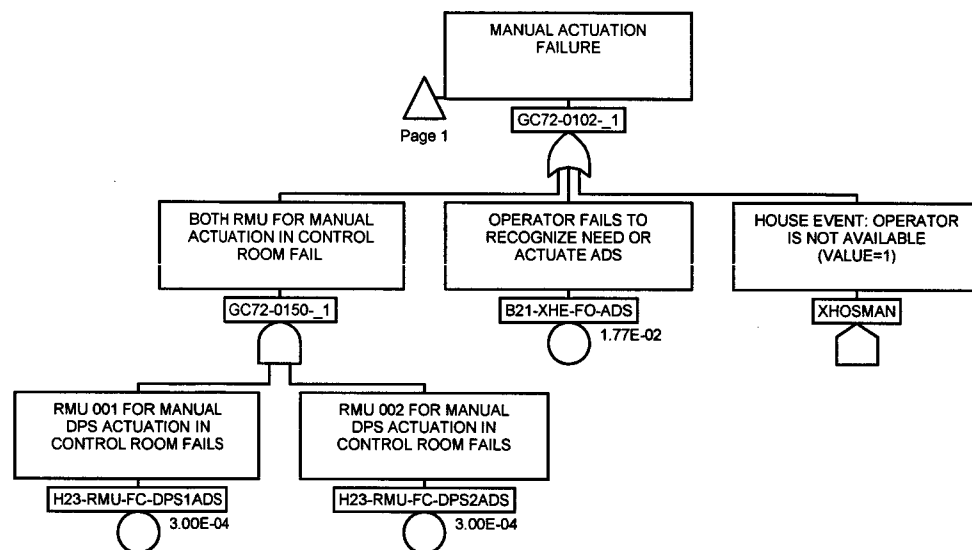








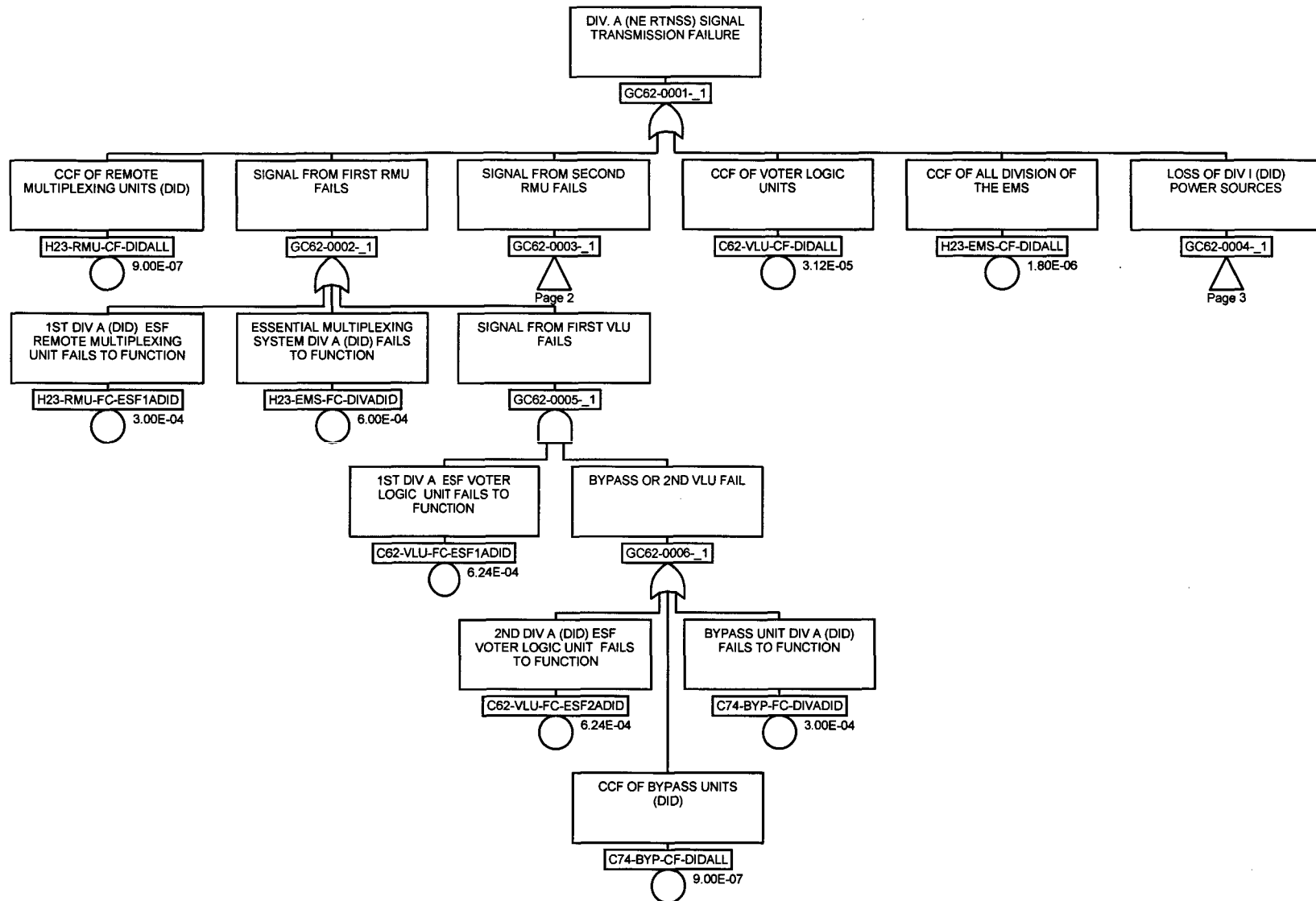


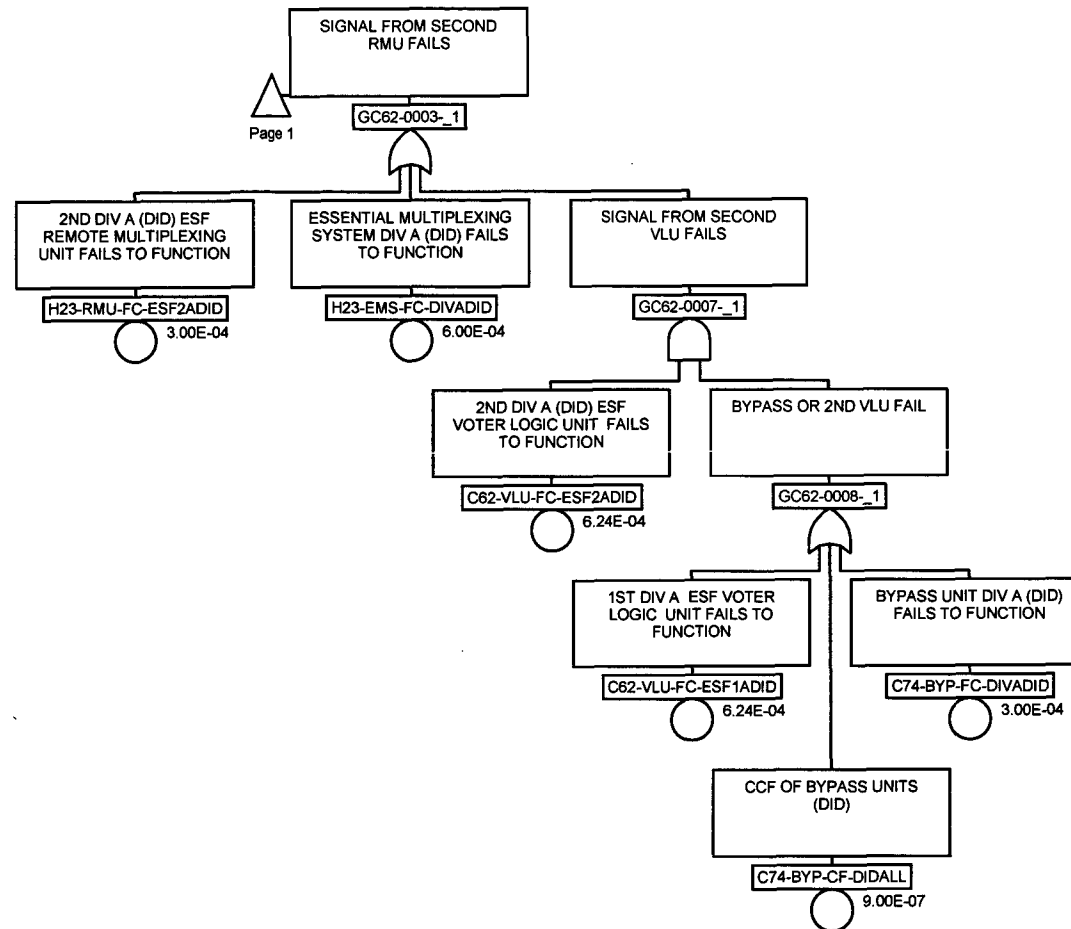


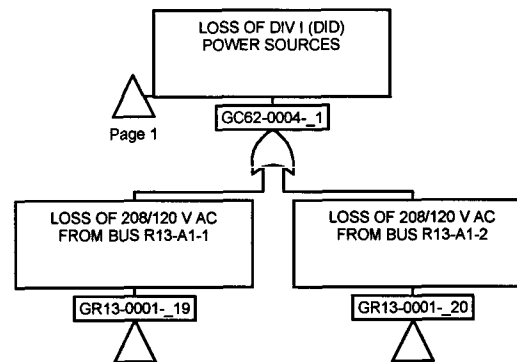
Appendix B.4.5-11

I&C Systems - C62 - DIV A SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1







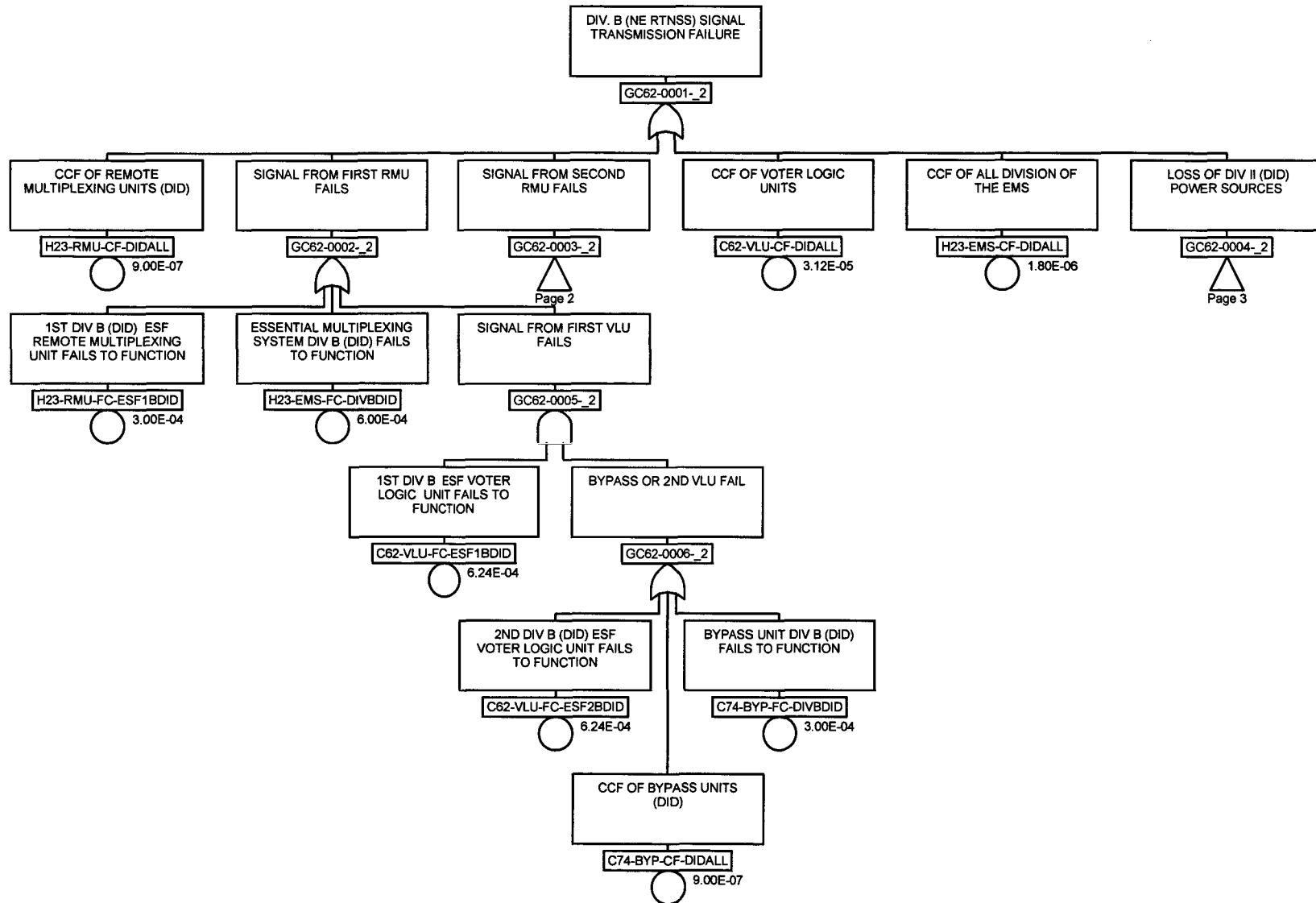
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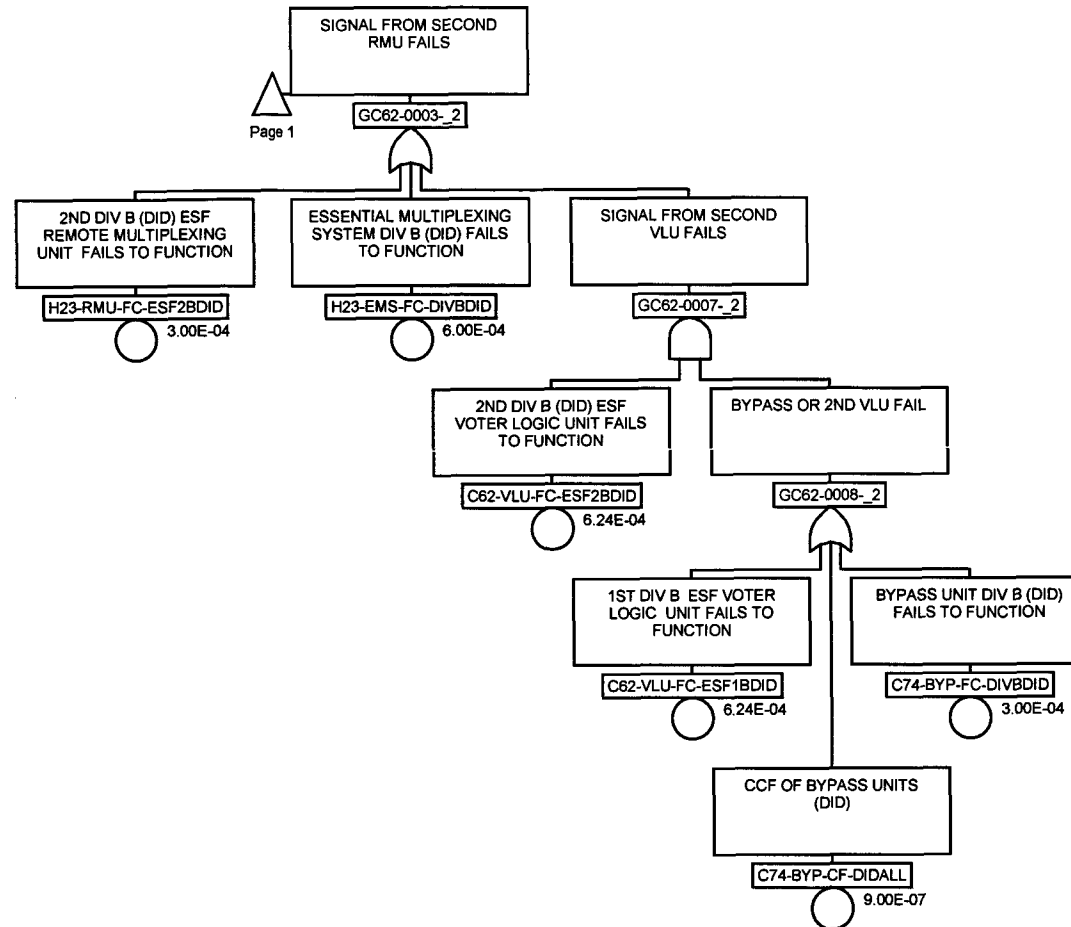
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| C62-VLU-CF-DIDALL | 1 | 4 | | | | | | | |
| C62-VLU-FC-ESF1ADID | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF1ADID | 2 | 3 | | | | | | | |
| C62-VLU-FC-ESF2ADID | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF2ADID | 2 | 3 | | | | | | | |
| C74-BYP-CF-DIDALL | 1 | 4 | | | | | | | |
| C74-BYP-CF-DIDALL | 2 | 4 | | | | | | | |
| C74-BYP-FC-DIVADID | 1 | 4 | | | | | | | |
| C74-BYP-FC-DIVADID | 2 | 4 | | | | | | | |
| GC62-0001- 1 | 1 | 4 | | | | | | | |
| GC62-0002- 1 | 1 | 2 | | | | | | | |
| GC62-0003- 1 | 1 | 3 | | | | | | | |
| GC62-0003- 1 | 2 | 2 | | | | | | | |
| GC62-0004- 1 | 1 | 6 | | | | | | | |
| GC62-0004- 1 | 3 | 2 | | | | | | | |
| GC62-0005- 1 | 1 | 3 | | | | | | | |
| GC62-0006- 1 | 1 | 4 | | | | | | | |
| GC62-0007- 1 | 2 | 3 | | | | | | | |
| GC62-0008- 1 | 2 | 4 | | | | | | | |
| GR13-0001- 19 | 3 | 1 | | | | | | | |
| GR13-0001- 20 | 3 | 2 | | | | | | | |
| H23-EMS-CF-DIDALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DIVADID | 1 | 2 | | | | | | | |
| H23-EMS-FC-DIVADID | 2 | 2 | | | | | | | |
| H23-RMU-CF-DIDALL | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF1ADID | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF2ADID | 2 | 1 | | | | | | | |
| I&C Systems - C62 - DIV A SIGNAL TRANSMISSION FAILURE | | | C62- DIVA.CAF | | | Appendix B.4.5-11 | | Page 4 | |

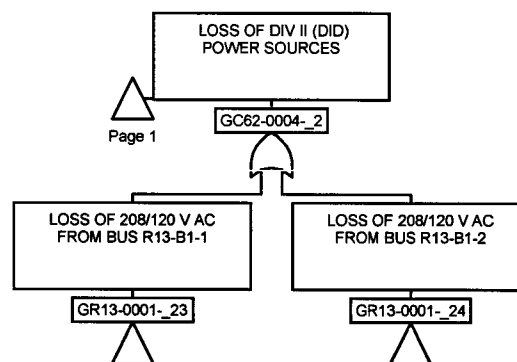
Appendix B.4.5-12

I&C Systems - C62 - DIV B SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1







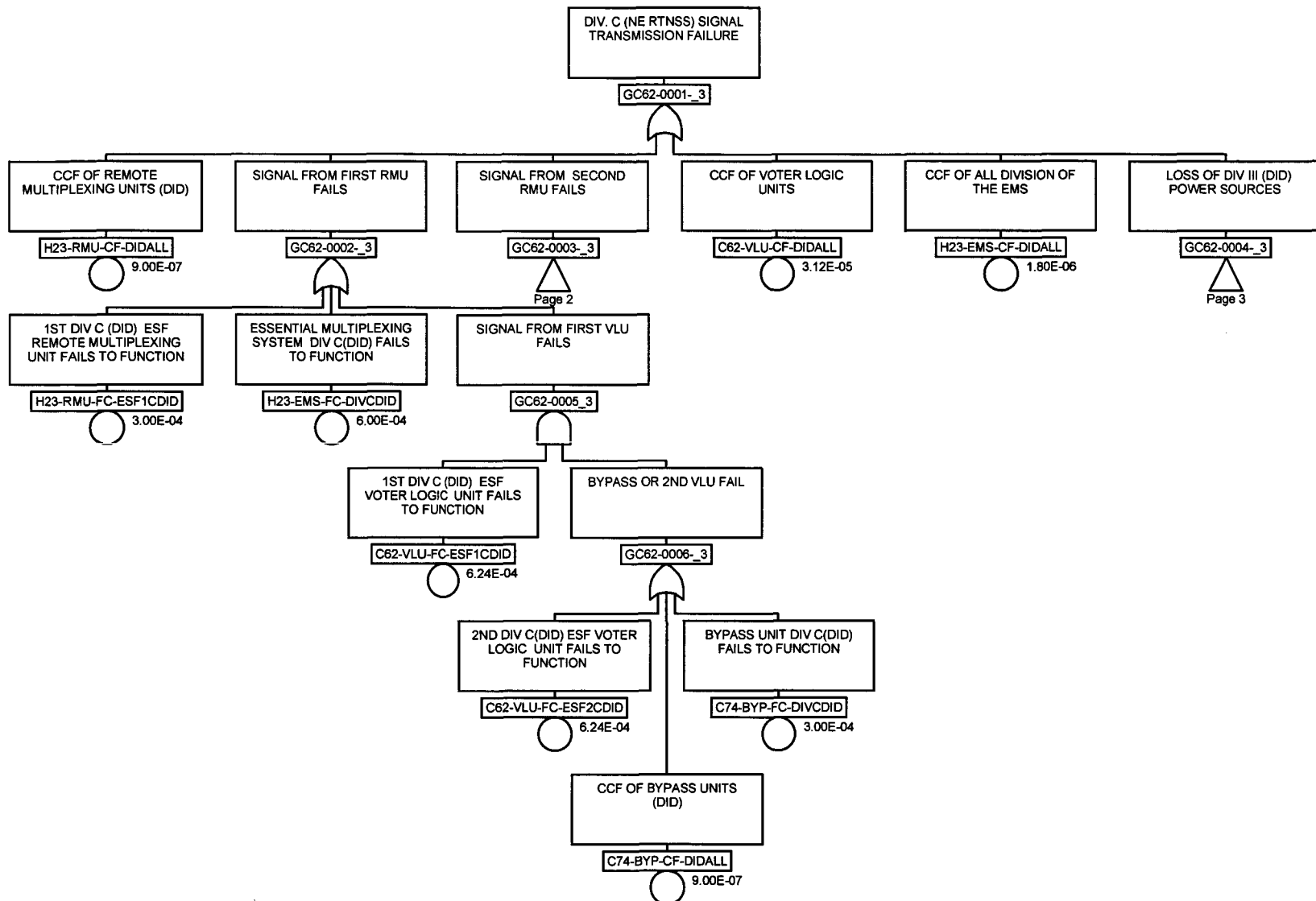
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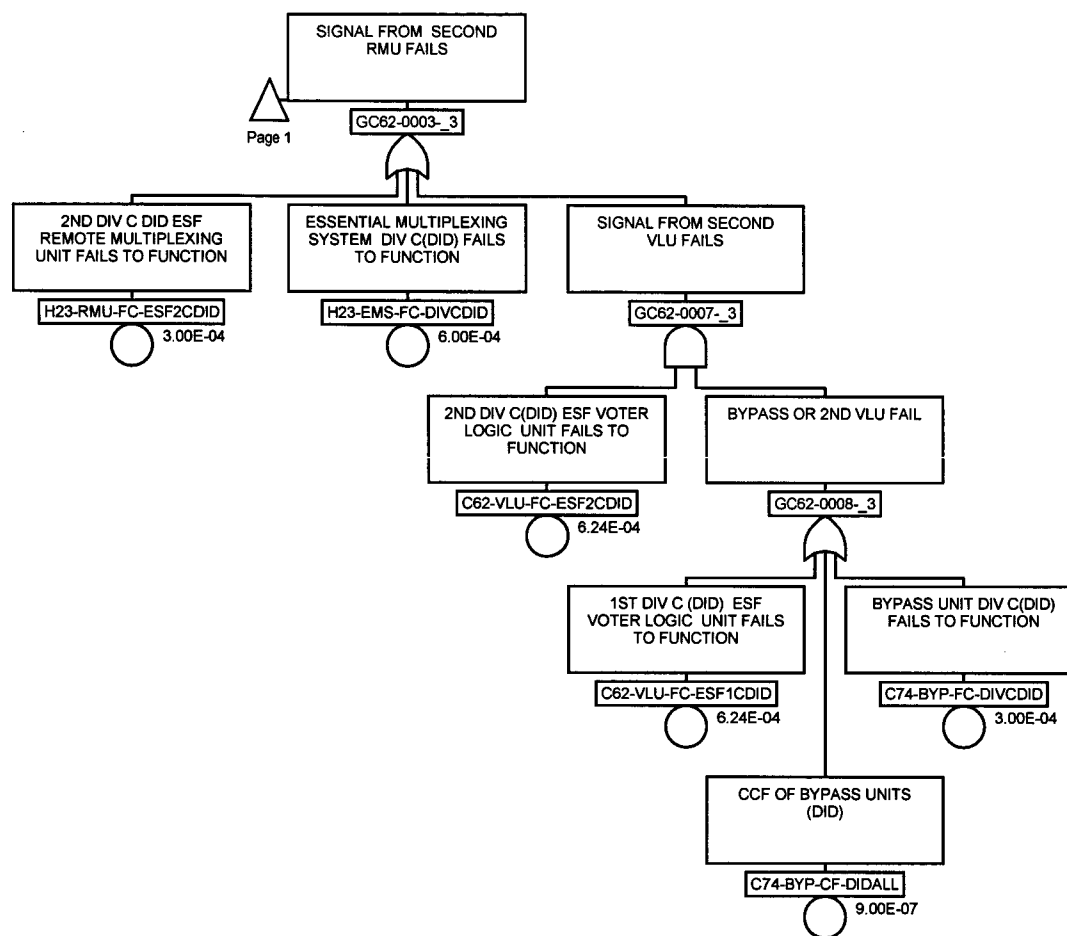
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| C62-VLU-CF-DIDALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | |
| C62-VLU-FC-ESF1BDID | 1 | 3 | | | | | | |
| C62-VLU-FC-ESF1BDID | 2 | 3 | | | | | | |
| C62-VLU-FC-ESF2BDID | 1 | 3 | | | | | | |
| C62-VLU-FC-ESF2BDID | 2 | 3 | | | | | | |
| C74-BYP-CF-DIDALL | 1 | 4 | | | | | | |
| C74-BYP-CF-DIDALL | 2 | 4 | | | | | | |
| C74-BYP-FC-DIVBDID | 1 | 4 | | | | | | |
| C74-BYP-FC-DIVBDID | 2 | 4 | | | | | | |
| GC62-0001- 2 | 1 | 4 | | | | | | |
| GC62-0002- 2 | 1 | 2 | | | | | | |
| GC62-0003- 2 | 1 | 3 | | | | | | |
| GC62-0003- 2 | 2 | 2 | | | | | | |
| GC62-0004- 2 | 1 | 6 | | | | | | |
| GC62-0004- 2 | 3 | 2 | | | | | | |
| GC62-0005- 2 | 1 | 3 | | | | | | |
| GC62-0006- 2 | 1 | 4 | | | | | | |
| GC62-0007- 2 | 2 | 3 | | | | | | |
| GC62-0008- 2 | 2 | 4 | | | | | | |
| GR13-0001- 23 | 3 | 1 | | | | | | |
| GR13-0001- 24 | 3 | 2 | | | | | | |
| H23-EMS-CF-DIDALL | 1 | 5 | | | | | | |
| H23-EMS-FC-DIVBDID | 1 | 2 | | | | | | |
| H23-EMS-FC-DIVBDID | 2 | 2 | | | | | | |
| H23-RMU-CF-DIDALL | 1 | 1 | | | | | | |
| H23-RMU-FC-ESF1BDID | 1 | 1 | | | | | | |
| H23-RMU-FC-ESF2BDID | 2 | 1 | | | | | | |
| I&C Systems - C62 - DIV B SIGNAL TRANSMISSION FAILURE | | | C62- DIVB.CAF | | | Appendix B.4.5-12 | | |
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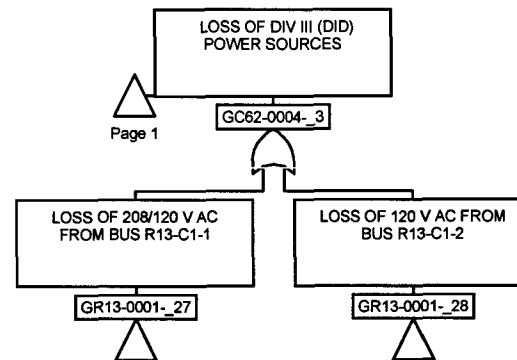
Appendix B.4.5-13

I&C Systems - C62 - DIV C SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1







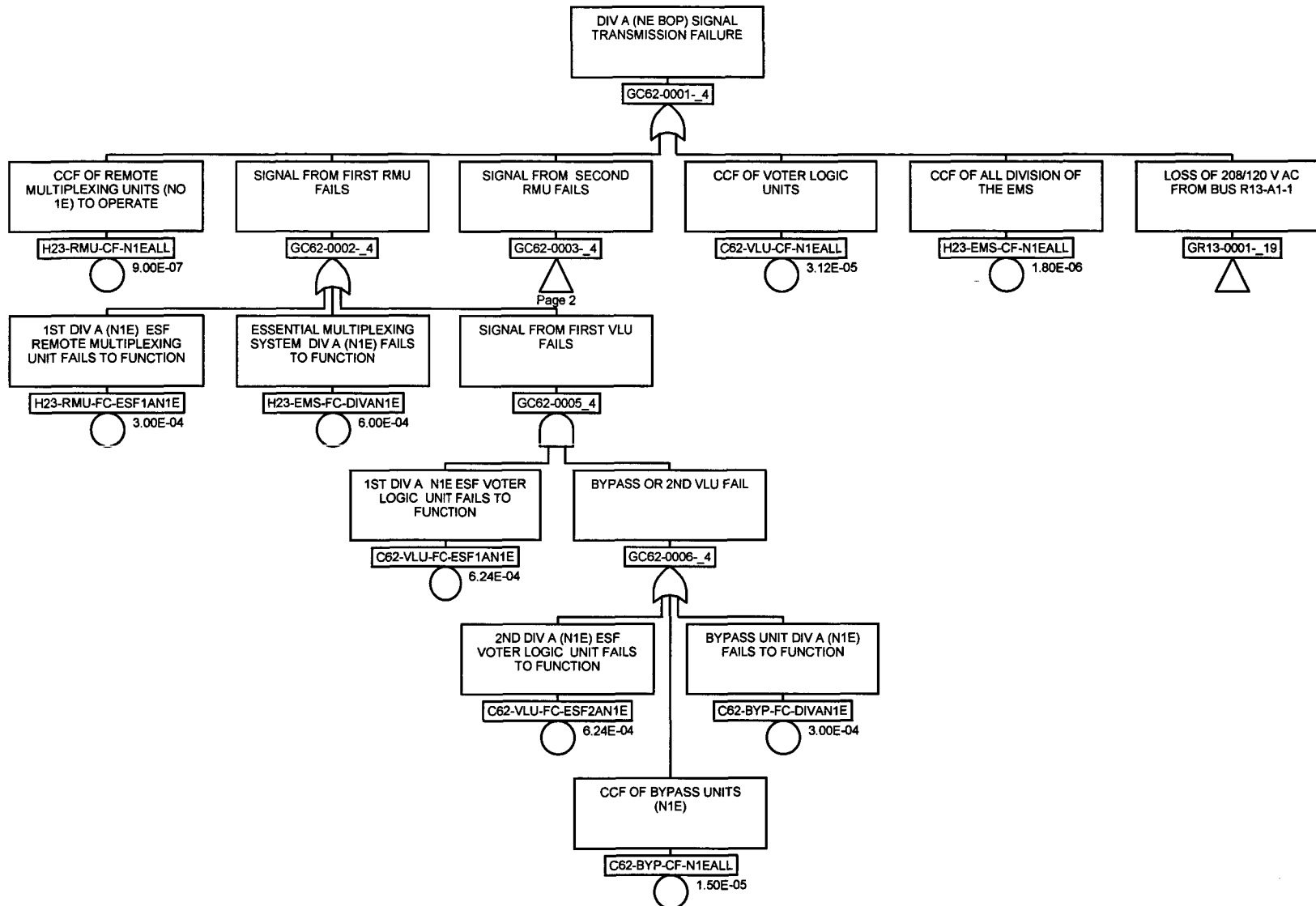
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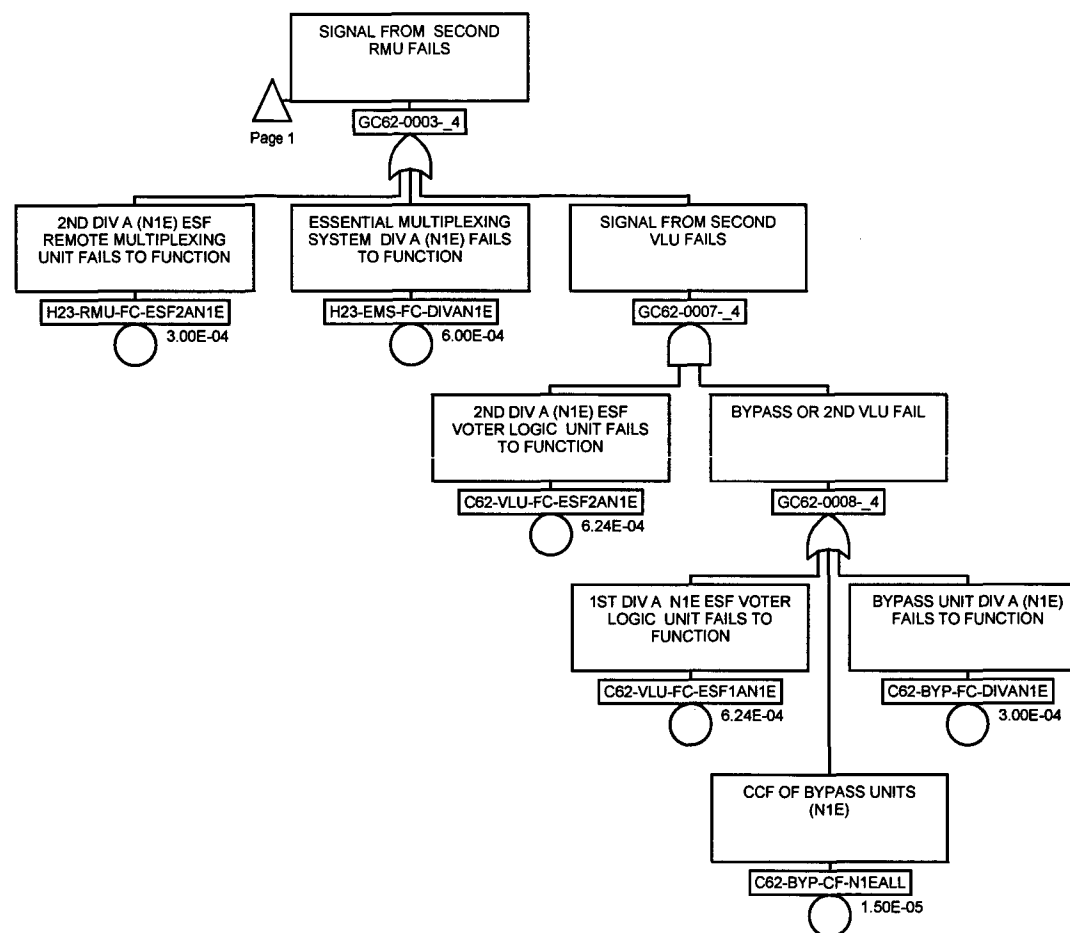
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| C62-VLU-CF-DIDALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | | |
| C62-VLU-FC-ESF1CDID | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF1CDID | 2 | 3 | | | | | | | |
| C62-VLU-FC-ESF2CDID | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF2CDID | 2 | 3 | | | | | | | |
| C74-BYP-CF-DIDALL | 1 | 4 | | | | | | | |
| C74-BYP-CF-DIDALL | 2 | 4 | | | | | | | |
| C74-BYP-FC-DIVCDID | 1 | 4 | | | | | | | |
| C74-BYP-FC-DIVCDID | 2 | 4 | | | | | | | |
| GC62-0001- 3 | 1 | 4 | | | | | | | |
| GC62-0002- 3 | 1 | 2 | | | | | | | |
| GC62-0003- 3 | 1 | 3 | | | | | | | |
| GC62-0003- 3 | 2 | 2 | | | | | | | |
| GC62-0004- 3 | 1 | 6 | | | | | | | |
| GC62-0004- 3 | 3 | 2 | | | | | | | |
| GC62-0005 3 | 1 | 3 | | | | | | | |
| GC62-0006- 3 | 1 | 4 | | | | | | | |
| GC62-0007- 3 | 2 | 3 | | | | | | | |
| GC62-0008- 3 | 2 | 4 | | | | | | | |
| GR13-0001- 27 | 3 | 1 | | | | | | | |
| GR13-0001- 28 | 3 | 2 | | | | | | | |
| H23-EMS-CF-DIDALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DIVCDID | 1 | 2 | | | | | | | |
| H23-EMS-FC-DIVCDID | 2 | 2 | | | | | | | |
| H23-RMU-CF-DIDALL | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF1CDID | 1 | 1 | | | | | | | |
| H23-RMU-FC-ESF2CDID | 2 | 1 | | | | | | | |
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Appendix B.4.5-14

I&C - C62 - DIV A NE-BOP SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1

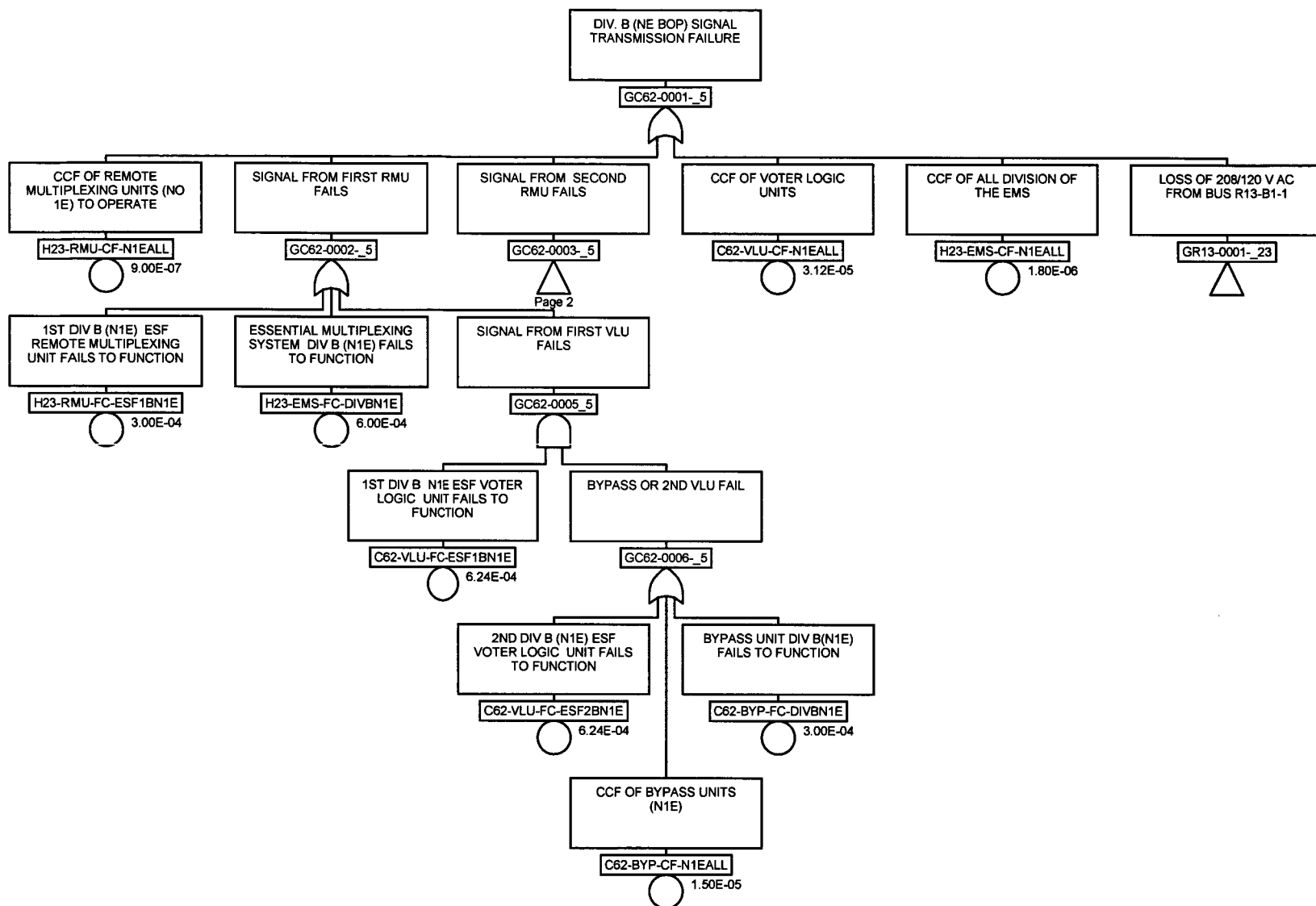


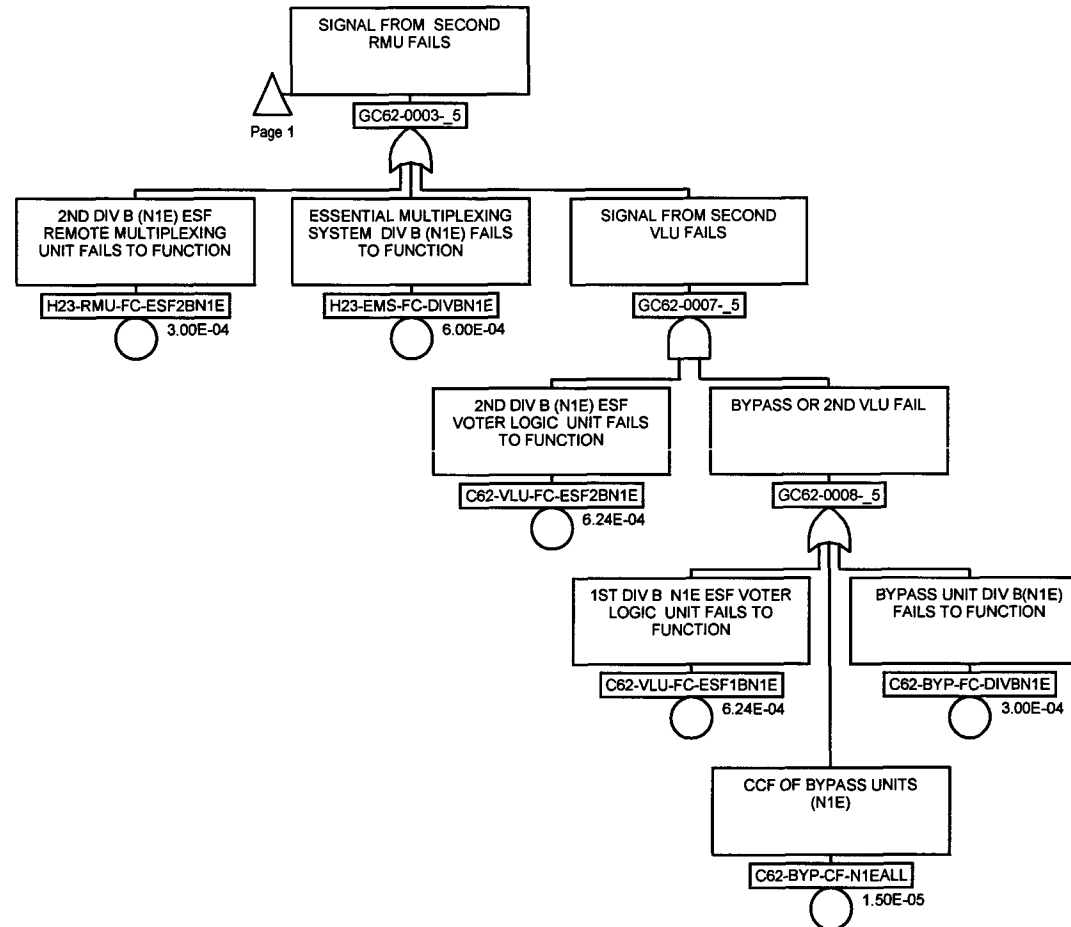


Appendix B.4.5-15

I&C - C62 - DIV B NE-BOP SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1



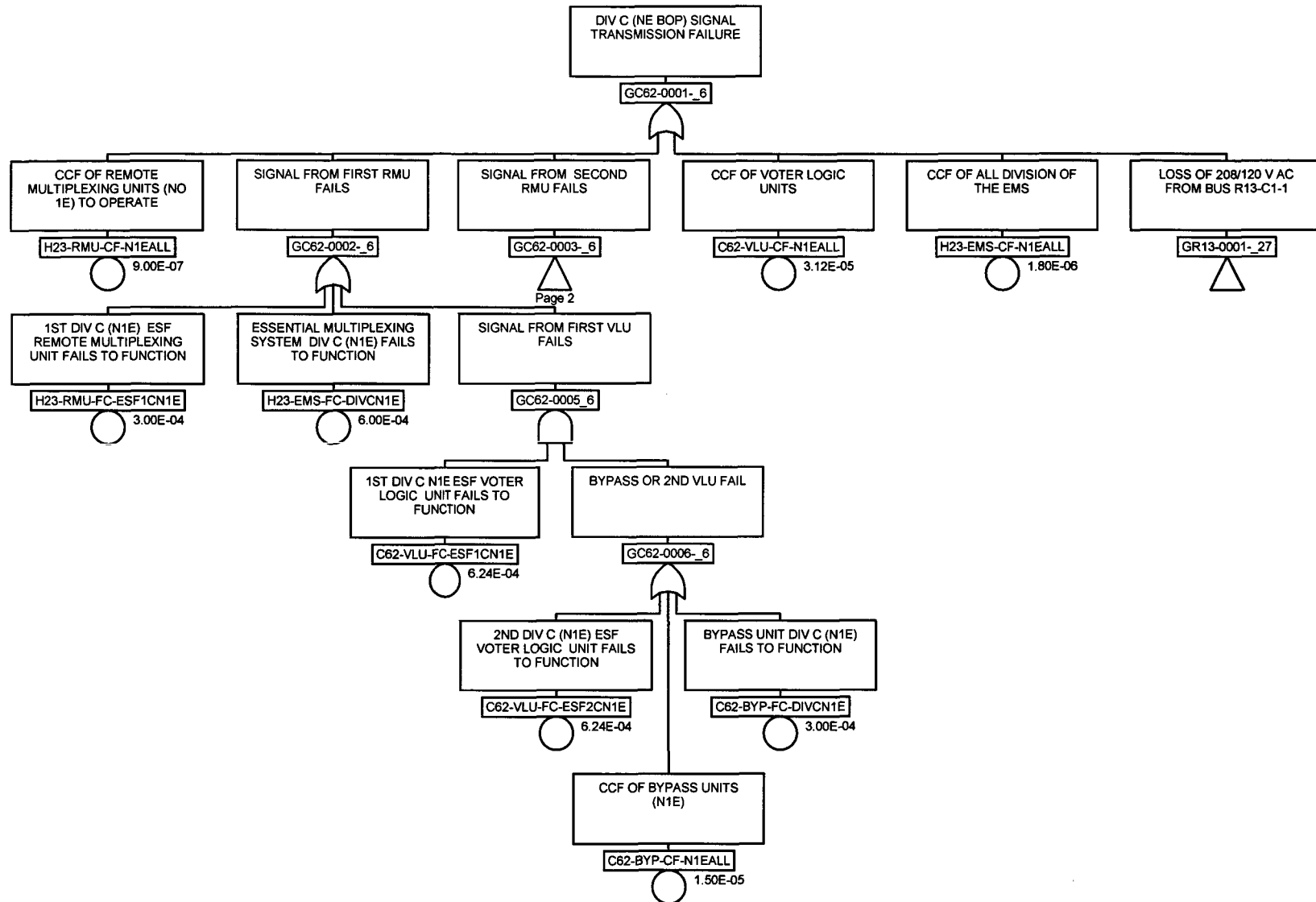


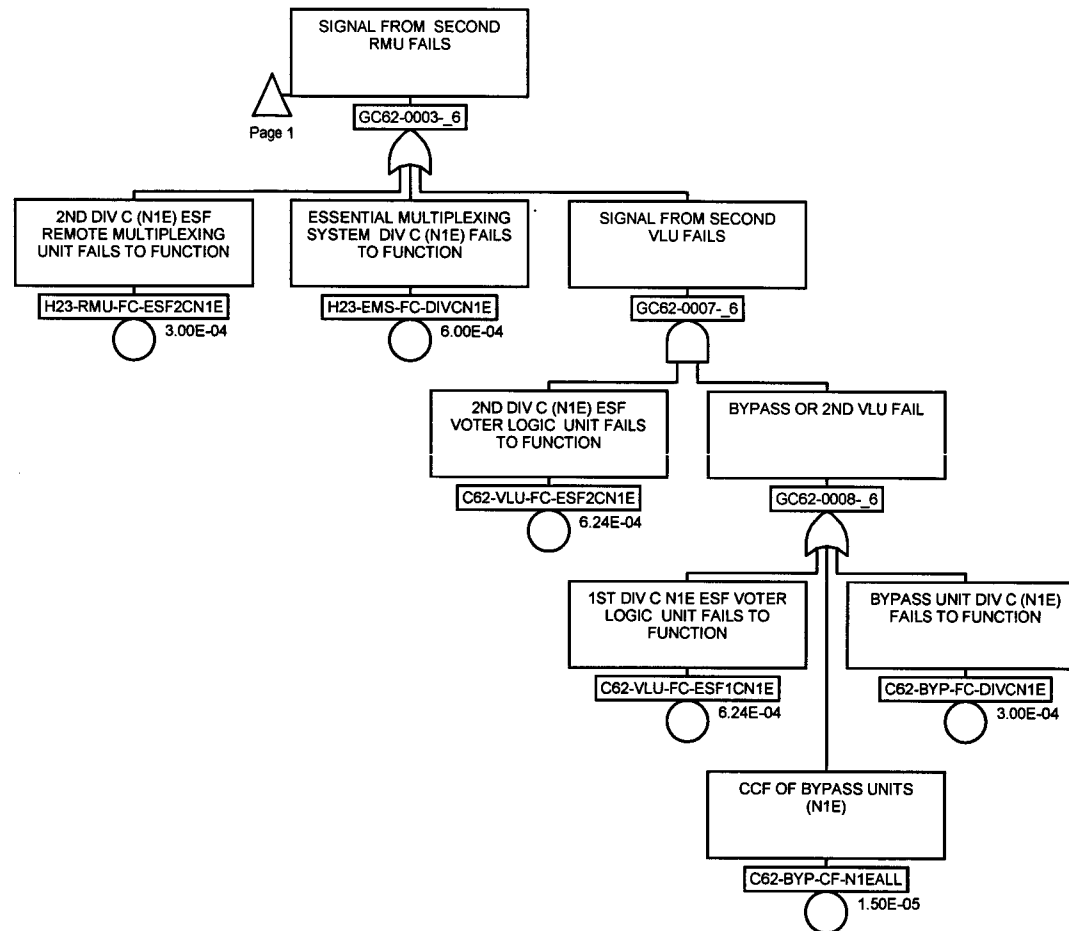
| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone | |
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| C62-BYP-CF-N1EALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | | |
| C62-BYP-CF-N1EALL | 2 | 4 | | | | | | | |
| C62-BYP-FC-DIVBN1E | 1 | 4 | | | | | | | |
| C62-BYP-FC-DIVBN1E | 2 | 4 | | | | | | | |
| C62-VLU-CF-N1EALL | 1 | 4 | | | | | | | |
| C62-VLU-FC-ESF1BN1E | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF1BN1E | 2 | 3 | | | | | | | |
| C62-VLU-FC-ESF2BN1E | 1 | 3 | | | | | | | |
| C62-VLU-FC-ESF2BN1E | 2 | 3 | | | | | | | |
| GC62-0001- 5 | 1 | 4 | | | | | | | |
| GC62-0002- 5 | 1 | 2 | | | | | | | |
| GC62-0003- 5 | 1 | 3 | | | | | | | |
| GC62-0003- 5 | 2 | 2 | | | | | | | |
| GC62-0005 5 | 1 | 3 | | | | | | | |
| GC62-0006- 5 | 1 | 4 | | | | | | | |
| GC62-0007- 5 | 2 | 3 | | | | | | | |
| GC62-0008- 5 | 2 | 4 | | | | | | | |
| GR13-0001- 23 | 1 | 6 | | | | | | | |
| H23-EMS-CF-N1EALL | 1 | 5 | | | | | | | |
| H23-EMS-FC-DIVBN1E | 1 | 2 | | | | | | | |
| H23-EMS-FC-DIVBN1E | 2 | 2 | | | | | | | |
| H23-RMU-CF-N1EALL | 1 | 1 | | | | | | | |
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Appendix B.4.5-16

I&C - C62 - DIV C NE-BOP SIGNAL TRANSMISSION FAILURE

NEDO-33201 Rev 1

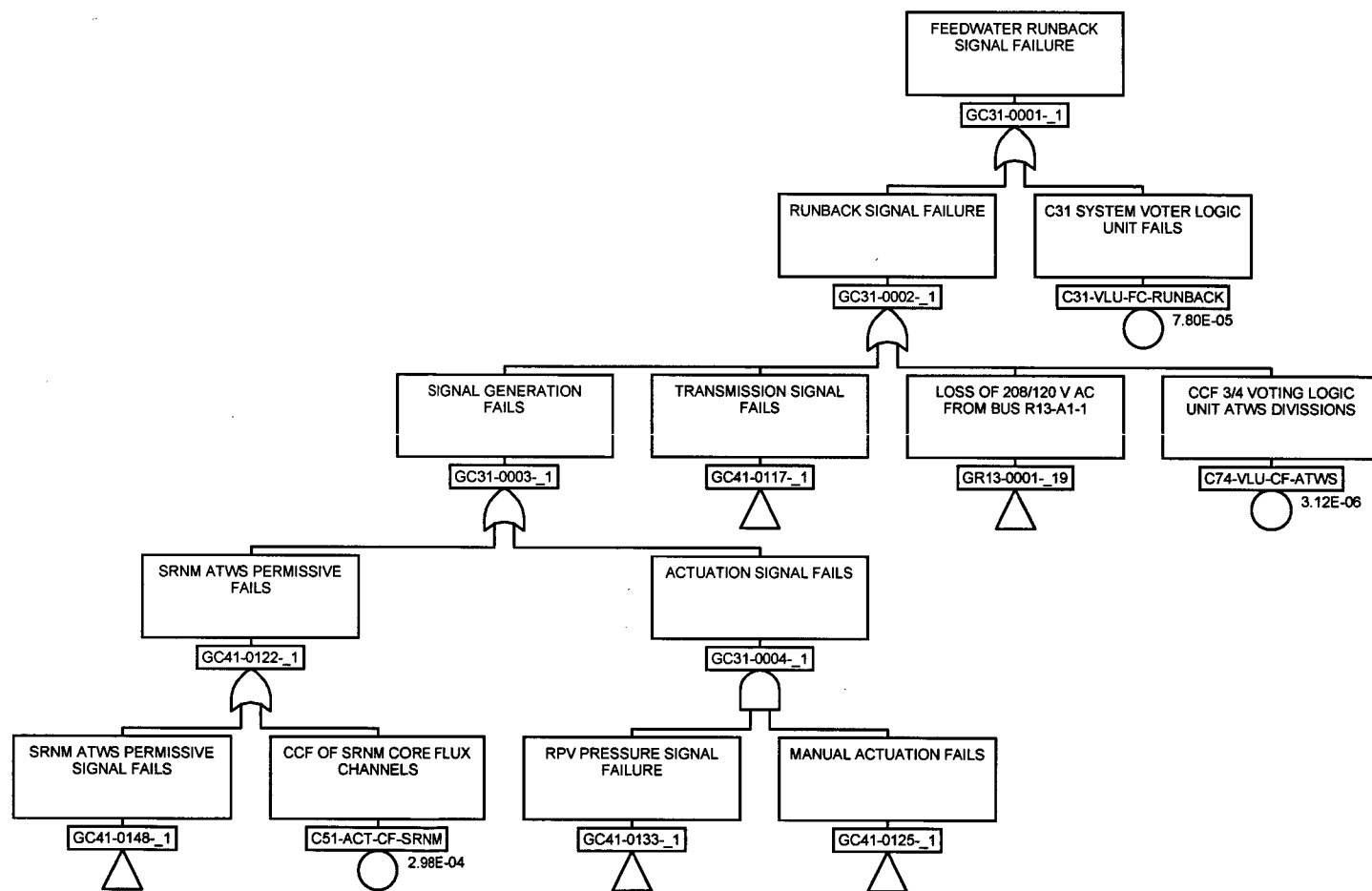




| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone |
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| C62-BYP-CF-N1EALL | 1 | 4 | NEDO-33201 Rev 1 | | | | | |
| C62-BYP-CF-N1EALL | 2 | 4 | | | | | | |
| C62-BYP-FC-DIVCN1E | 1 | 4 | | | | | | |
| C62-BYP-FC-DIVCN1E | 2 | 4 | | | | | | |
| C62-VLU-CF-N1EALL | 1 | 4 | | | | | | |
| C62-VLU-FC-ESF1CN1E | 1 | 3 | | | | | | |
| C62-VLU-FC-ESF1CN1E | 2 | 3 | | | | | | |
| C62-VLU-FC-ESF2CN1E | 1 | 3 | | | | | | |
| C62-VLU-FC-ESF2CN1E | 2 | 3 | | | | | | |
| GC62-0001- 6 | 1 | 4 | | | | | | |
| GC62-0002- 6 | 1 | 2 | | | | | | |
| GC62-0003- 6 | 1 | 3 | | | | | | |
| GC62-0003- 6 | 2 | 2 | | | | | | |
| GC62-0005 6 | 1 | 3 | | | | | | |
| GC62-0006- 6 | 1 | 4 | | | | | | |
| GC62-0007- 6 | 2 | 3 | | | | | | |
| GC62-0008- 6 | 2 | 4 | | | | | | |
| GR13-0001- 27 | 1 | 6 | | | | | | |
| H23-EMS-CF-N1EALL | 1 | 5 | | | | | | |
| H23-EMS-FC-DIVCN1E | 1 | 2 | | | | | | |
| H23-EMS-FC-DIVCN1E | 2 | 2 | | | | | | |
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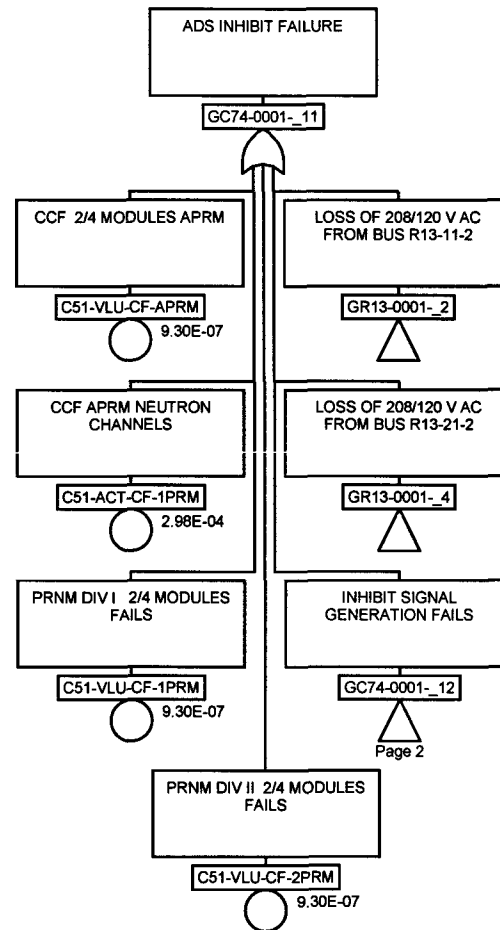
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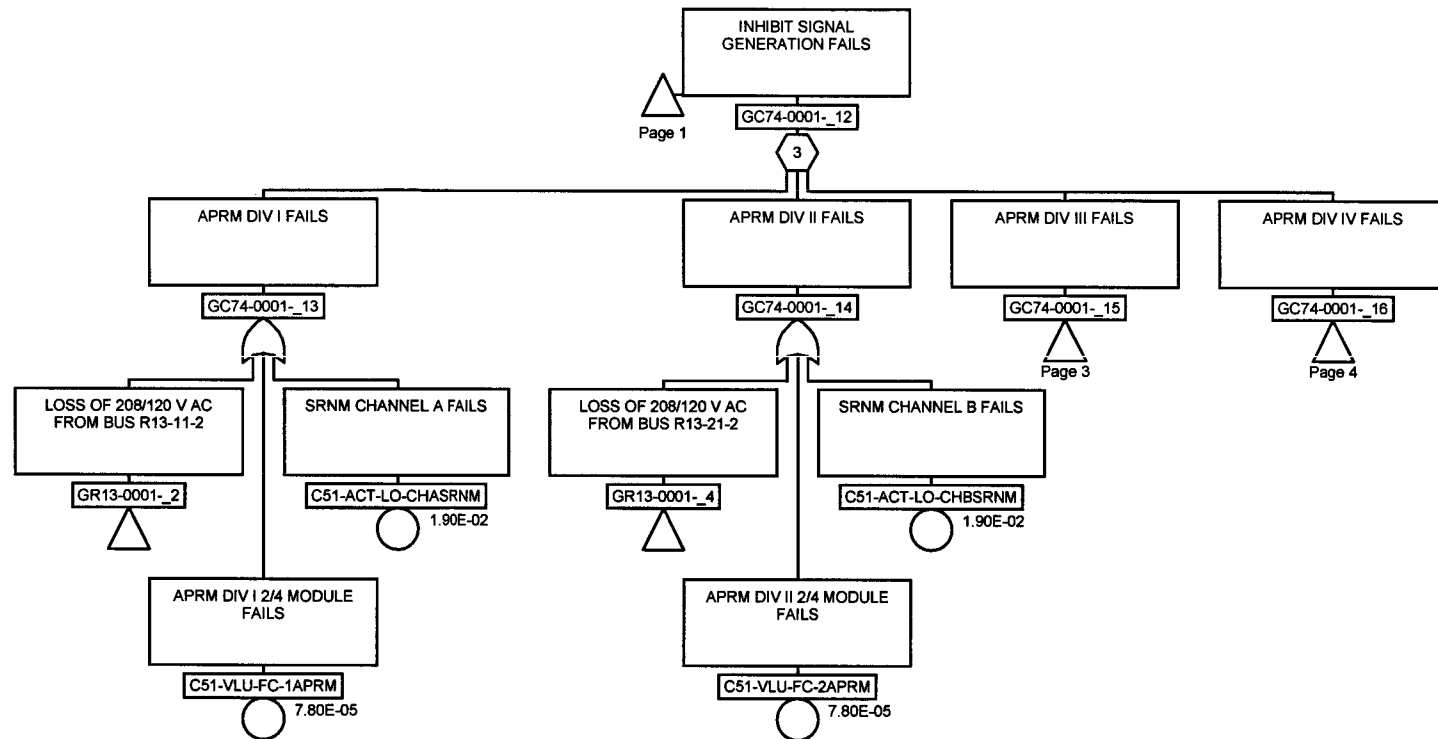
I&C Systems - C31 - FEEDWATER RUNBACK SIGNAL FAILURE

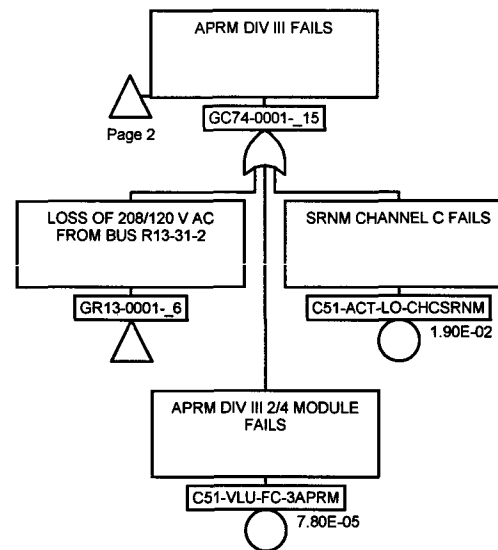


Appendix B.4.5-18

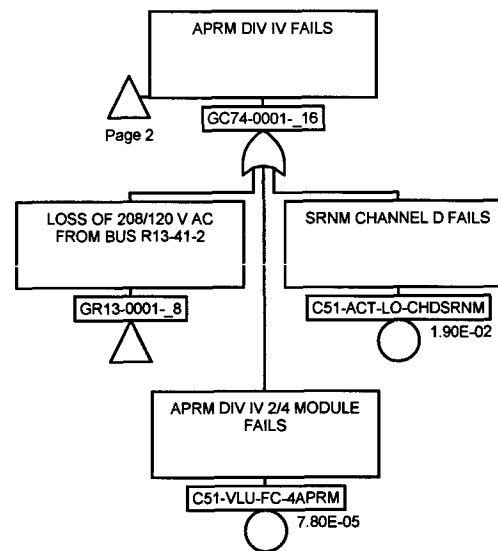
I&C System - C74 - ADS INHIBIT FAILURE





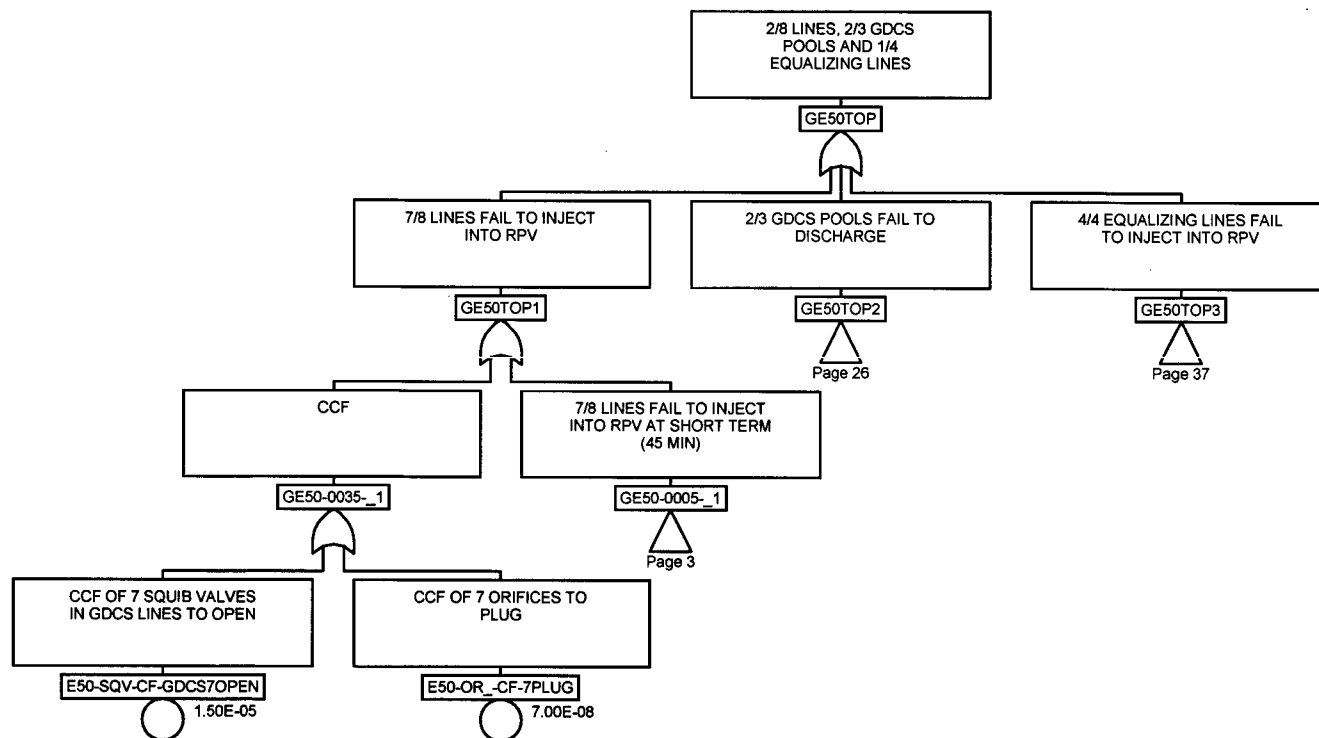


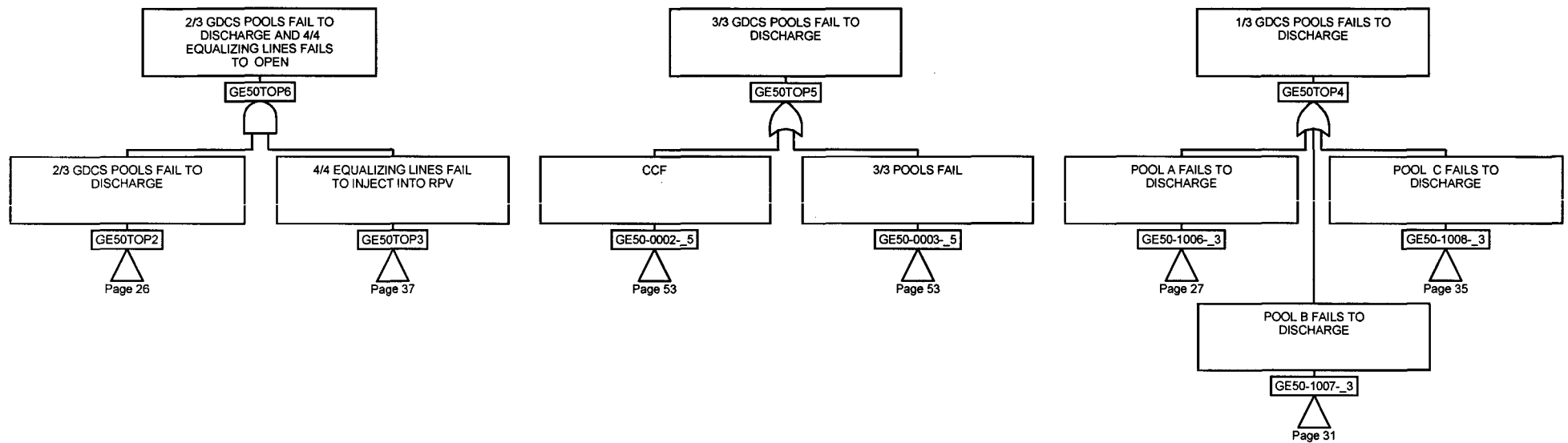
Page 2

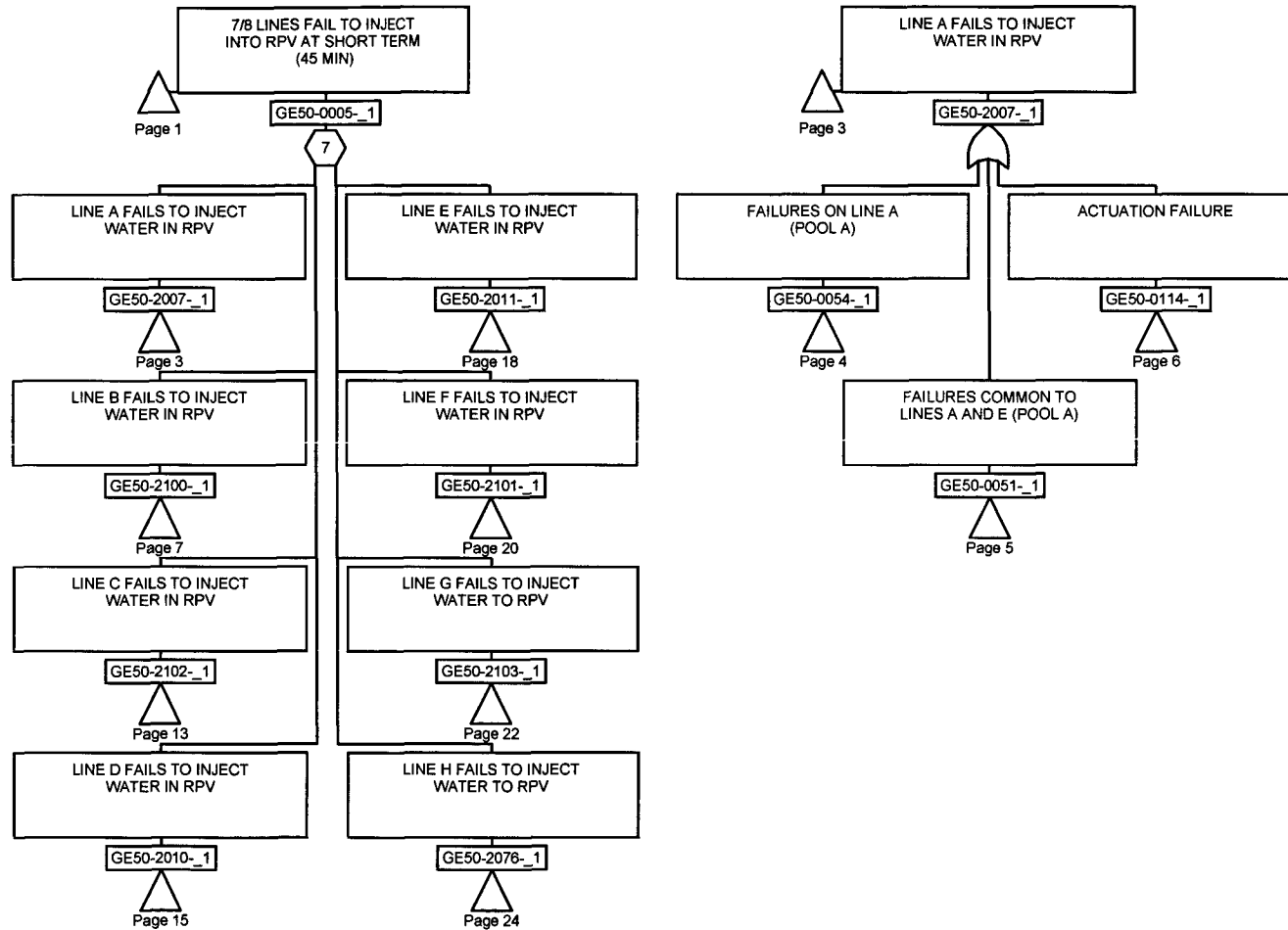


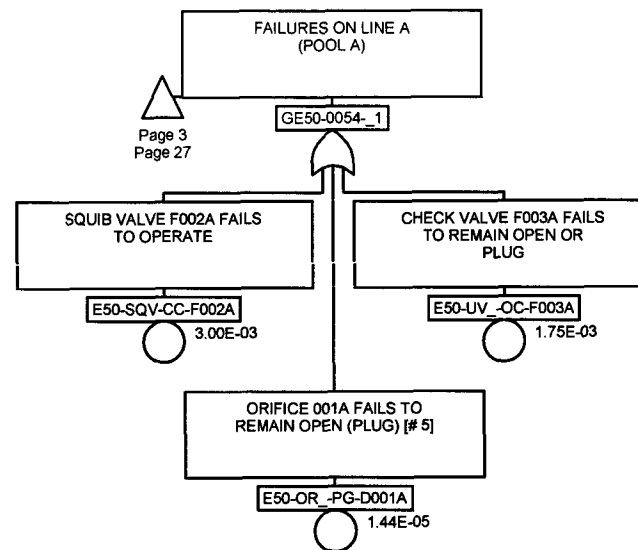
| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone | |
|--|------|------|------------------|------|------|--------------------|-------------------|--------|--|
| C51-ACT-CF-1PRM | 1 | 1 | NEDO-33201 Rev 1 | | | | | | |
| C51-ACT-LO-CHASRNM | 2 | 2 | | | | | | | |
| C51-ACT-LO-CHBSRNM | 2 | 4 | | | | | | | |
| C51-ACT-LO-CHCSRNM | 3 | 2 | | | | | | | |
| C51-ACT-LO-CHDSRNM | 4 | 2 | | | | | | | |
| C51-VLU-CF-1PRM | 1 | 1 | | | | | | | |
| C51-VLU-CF-2PRM | 1 | 2 | | | | | | | |
| C51-VLU-CF-APRM | 1 | 1 | | | | | | | |
| C51-VLU-FC-1APRM | 2 | 2 | | | | | | | |
| C51-VLU-FC-2APRM | 2 | 4 | | | | | | | |
| C51-VLU-FC-3APRM | 3 | 2 | | | | | | | |
| C51-VLU-FC-4APRM | 4 | 2 | | | | | | | |
| GC74-0001- 11 | 1 | 2 | | | | | | | |
| GC74-0001- 12 | 1 | 2 | | | | | | | |
| GC74-0001- 12 | 2 | 3 | | | | | | | |
| GC74-0001- 13 | 2 | 2 | | | | | | | |
| GC74-0001- 14 | 2 | 4 | | | | | | | |
| GC74-0001- 15 | 2 | 5 | | | | | | | |
| GC74-0001- 15 | 3 | 2 | | | | | | | |
| GC74-0001- 16 | 2 | 6 | | | | | | | |
| GC74-0001- 16 | 4 | 2 | | | | | | | |
| GR13-0001- 2 | 1 | 2 | | | | | | | |
| GR13-0001- 2 | 2 | 1 | | | | | | | |
| GR13-0001- 4 | 1 | 2 | | | | | | | |
| GR13-0001- 4 | 2 | 3 | | | | | | | |
| GR13-0001- 6 | 3 | 1 | | | | | | | |
| GR13-0001-_8 | 4 | 1 | | | | | | | |
| I&C System - C74 - ADS INHIBIT FAILURE | | | | | | inhibicion ads.CAF | Appendix B.4.5-18 | Page 5 | |

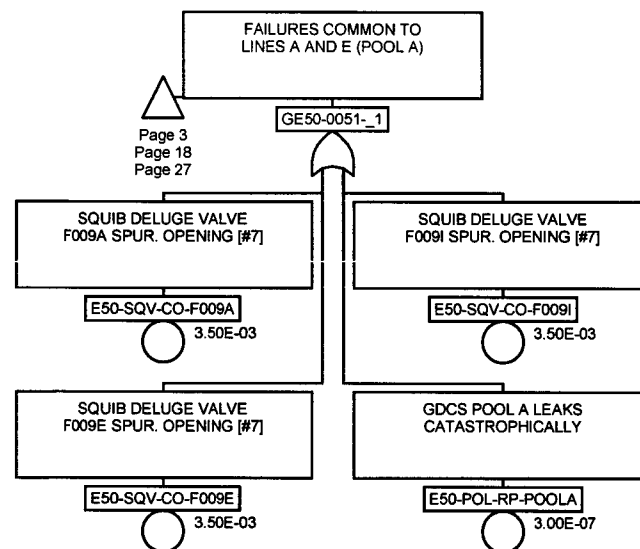
Appendix B.4.6 Gravity Driven Cooling System Fault Tree

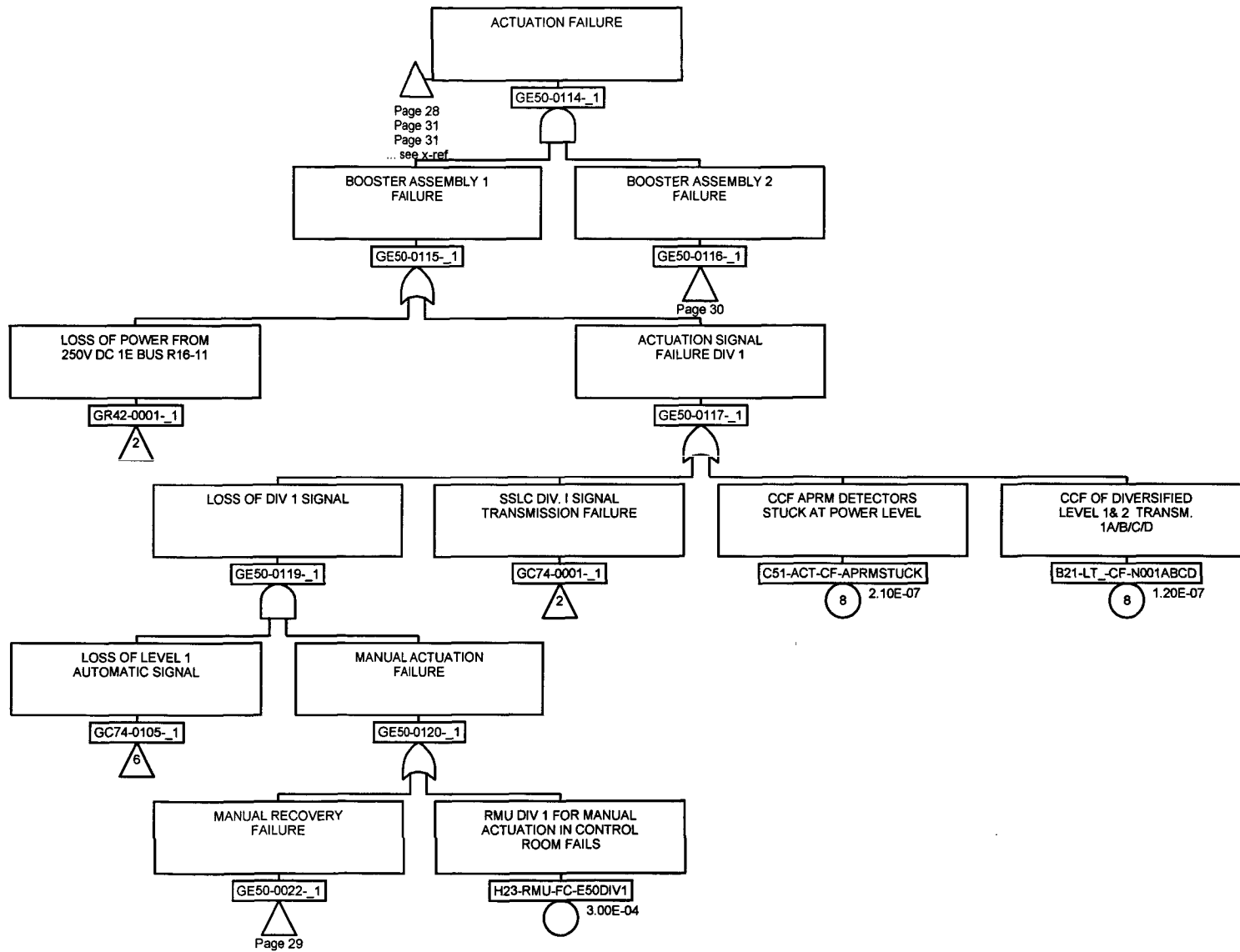


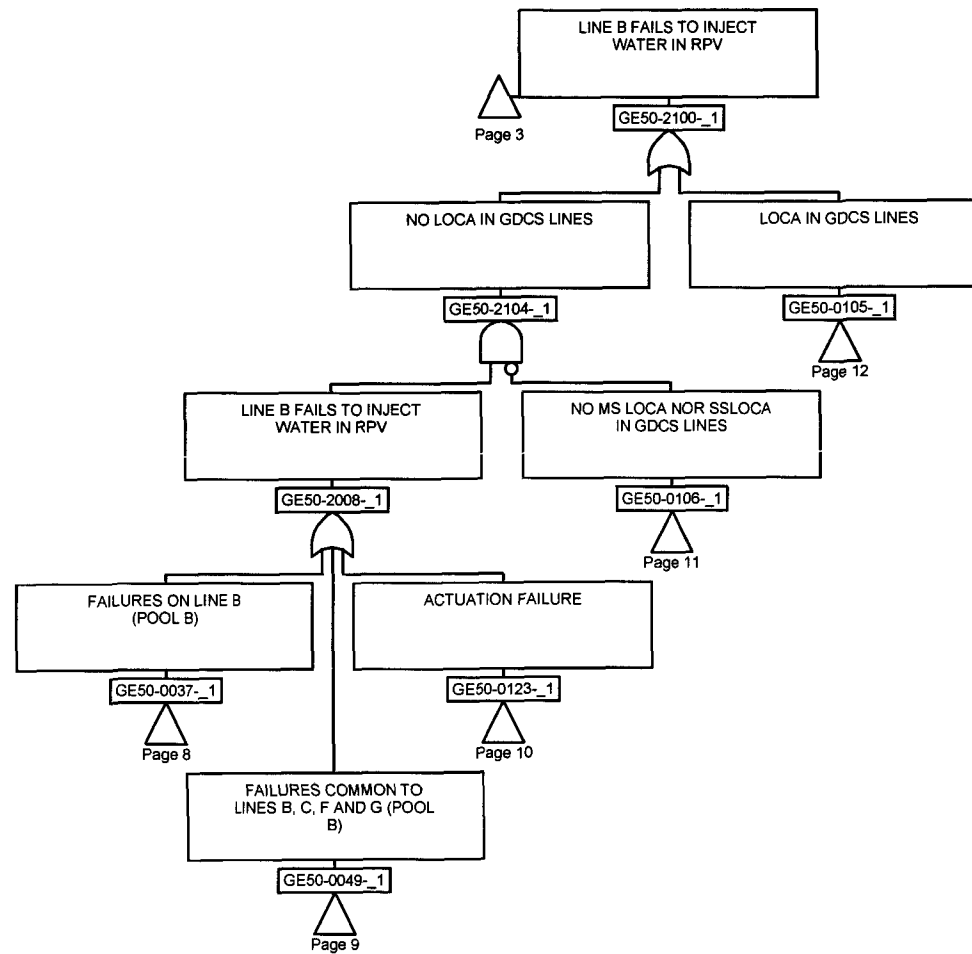


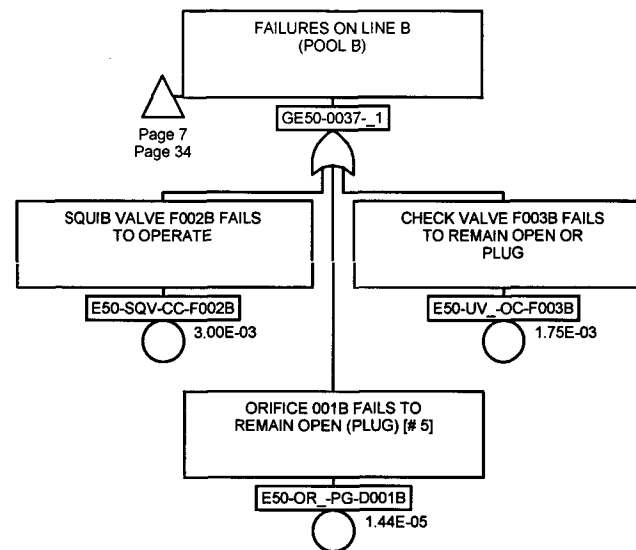


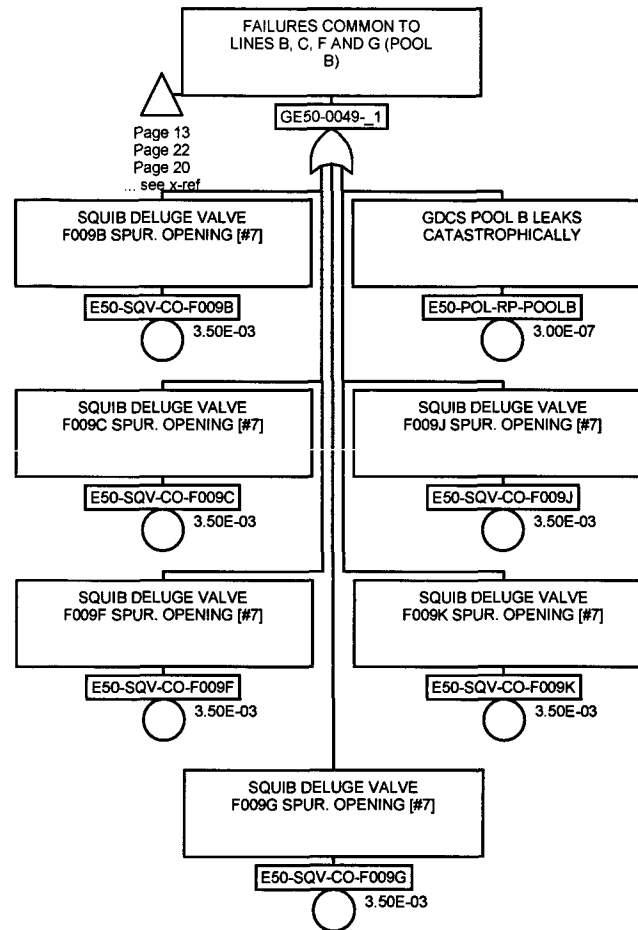


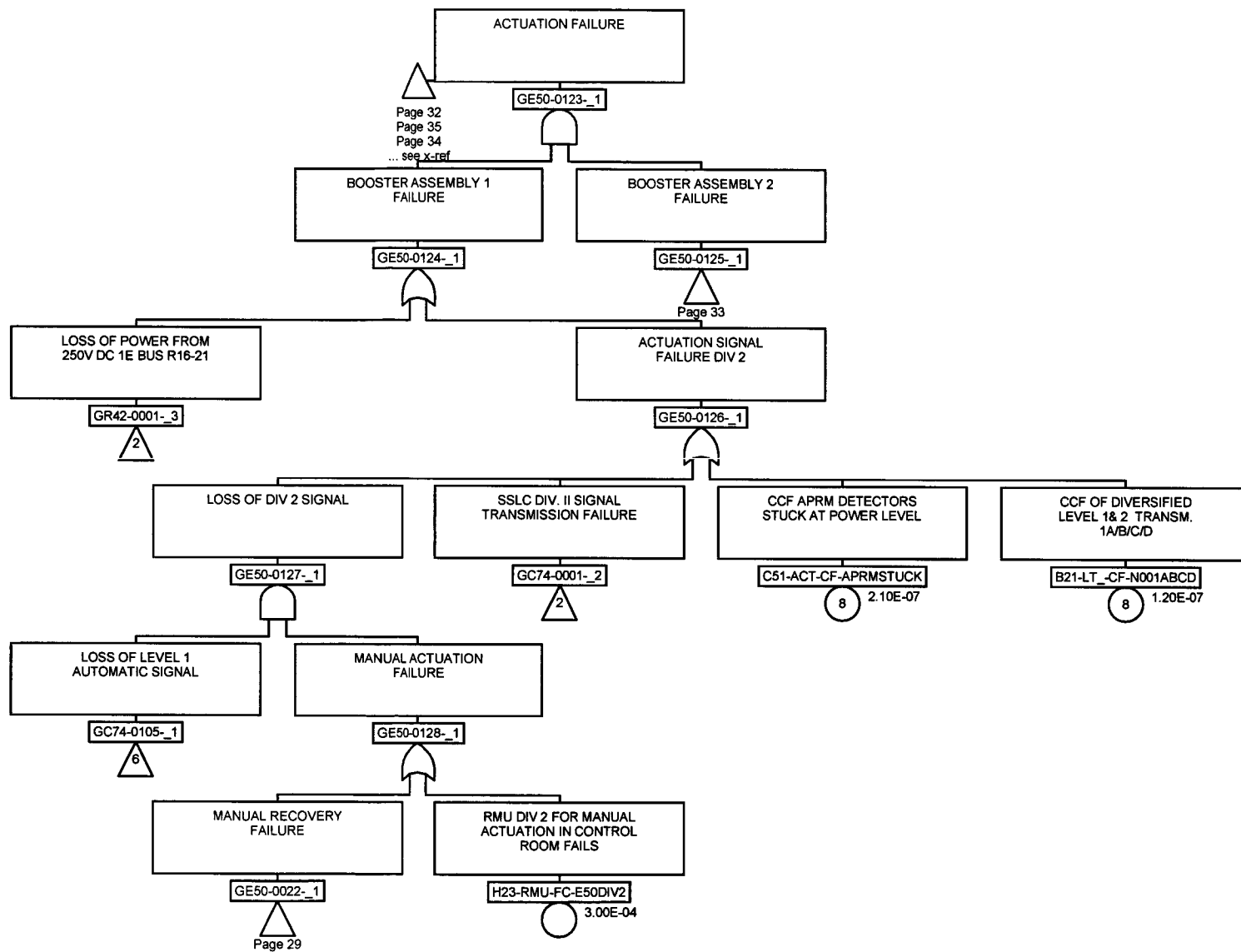


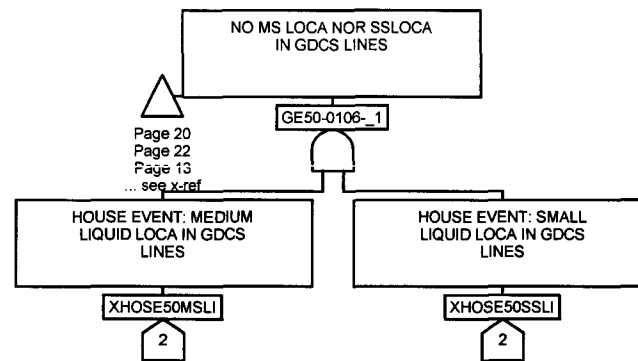


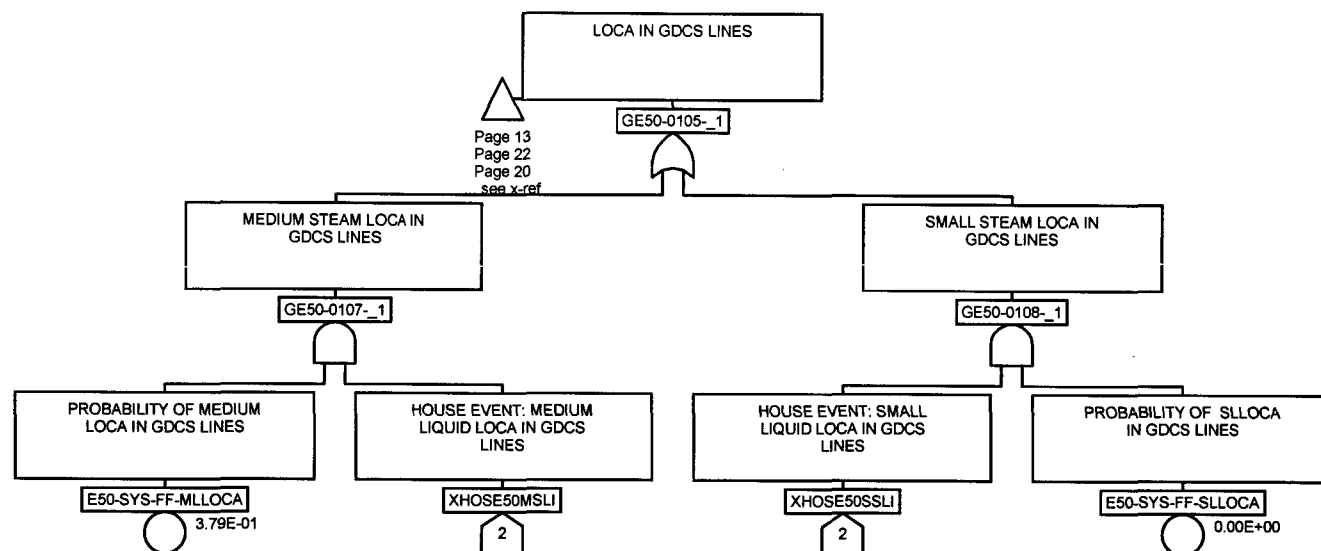


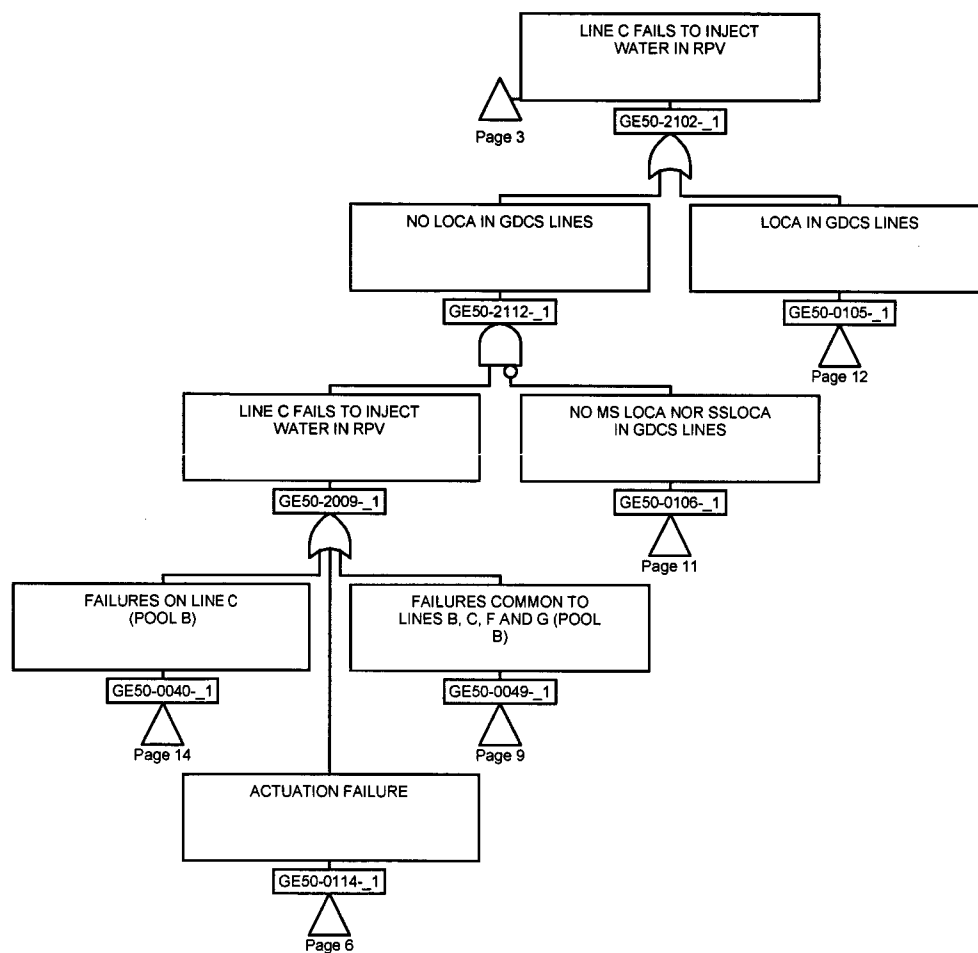


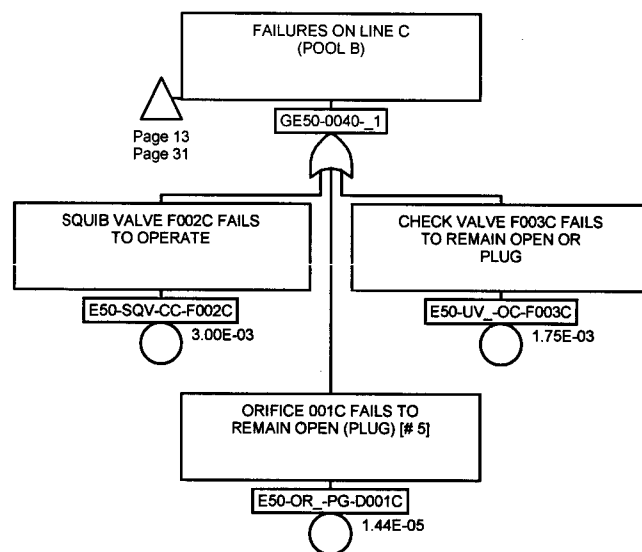


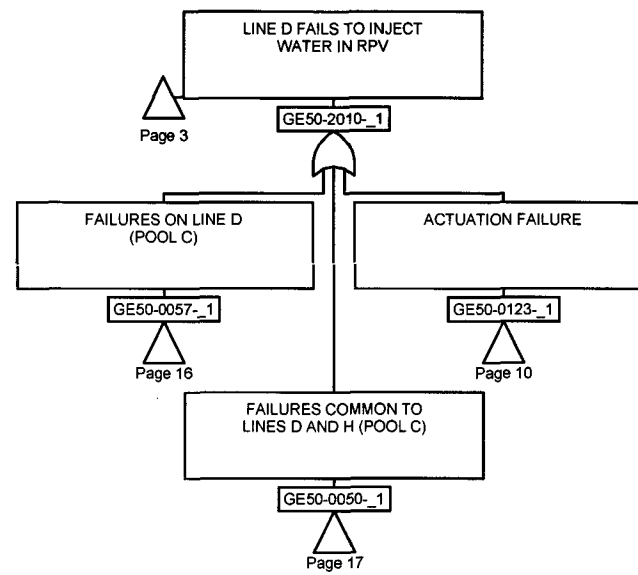


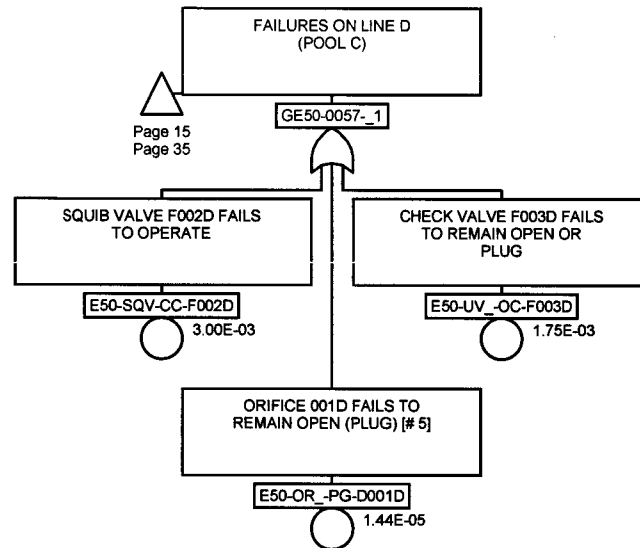


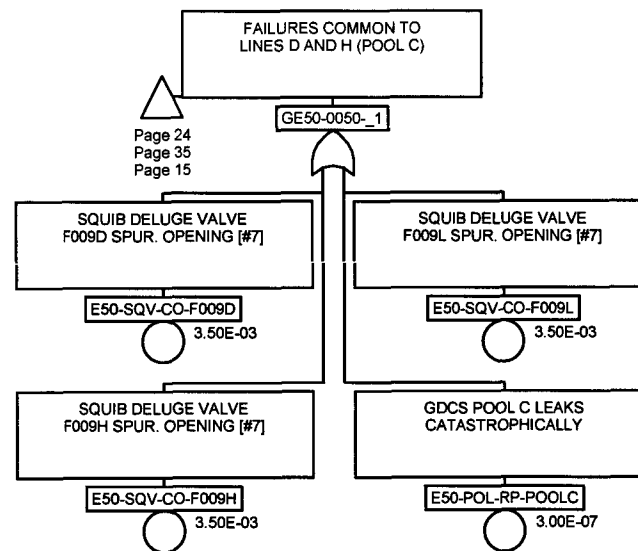


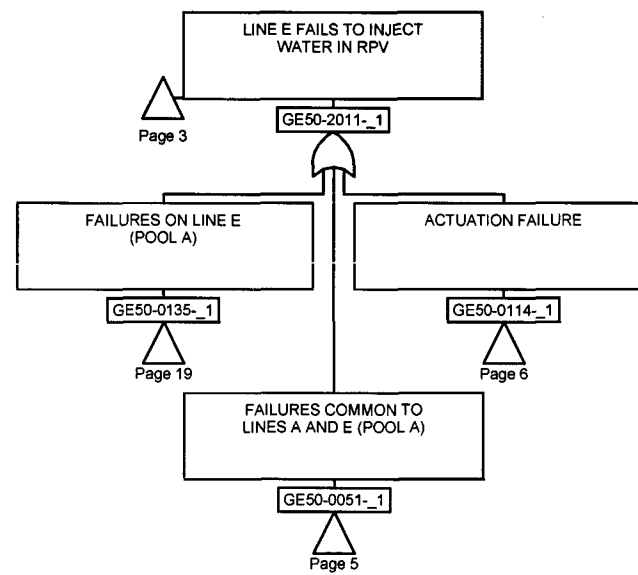


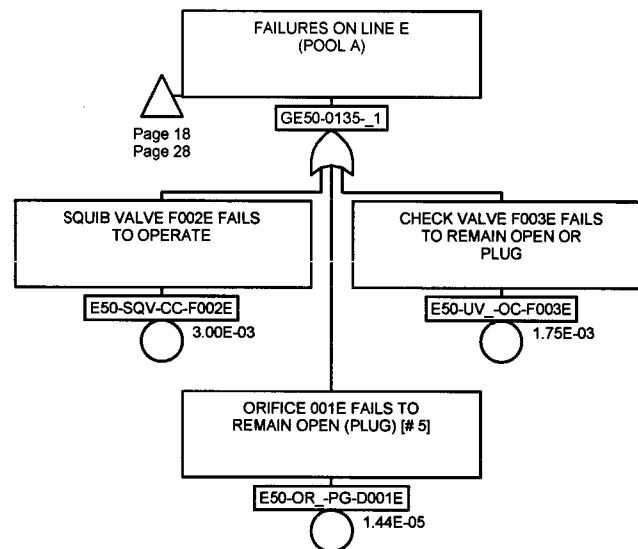


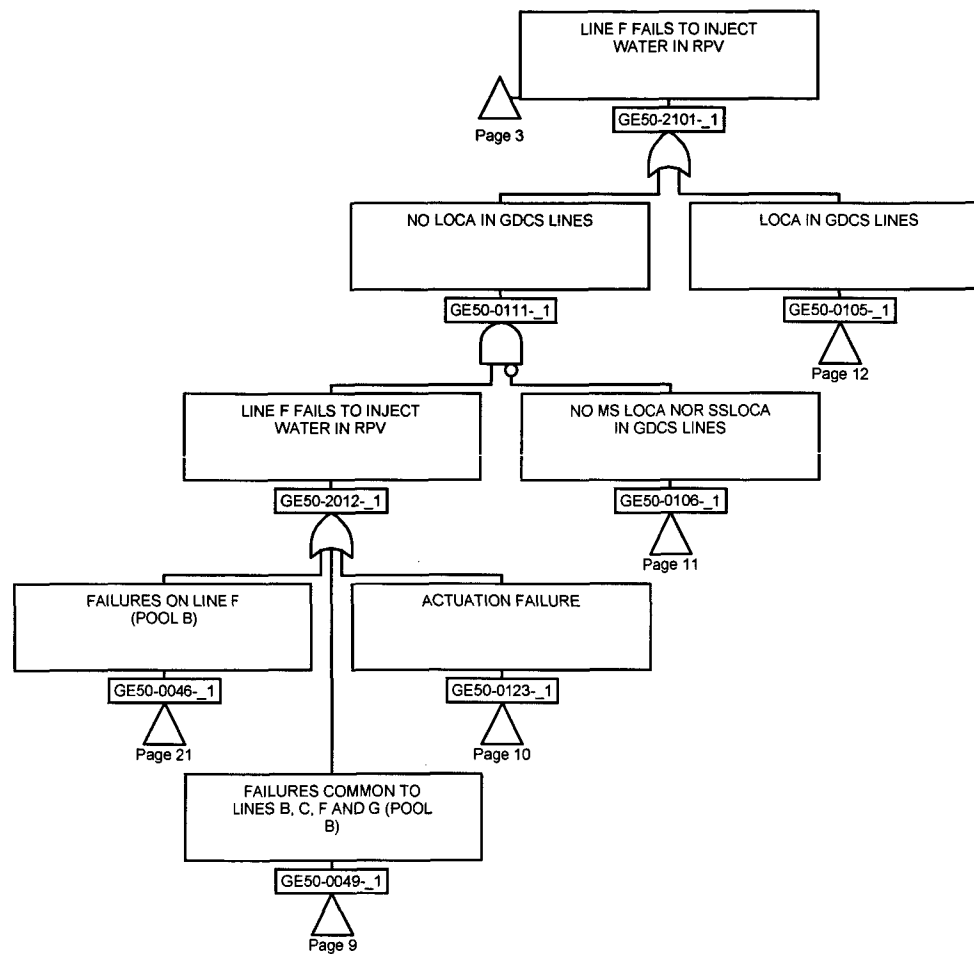


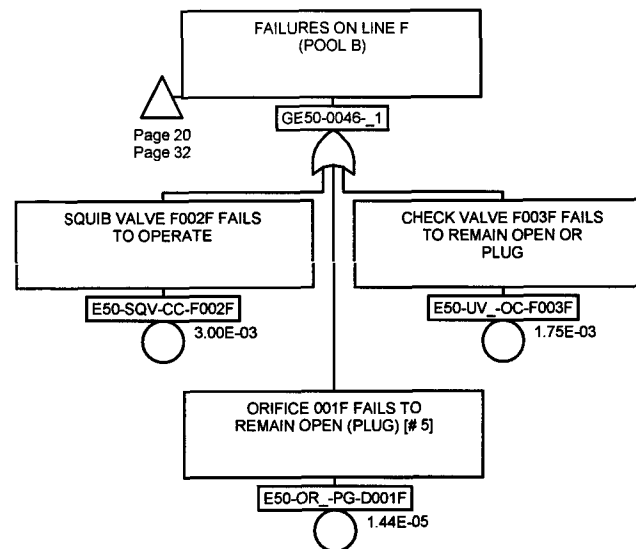


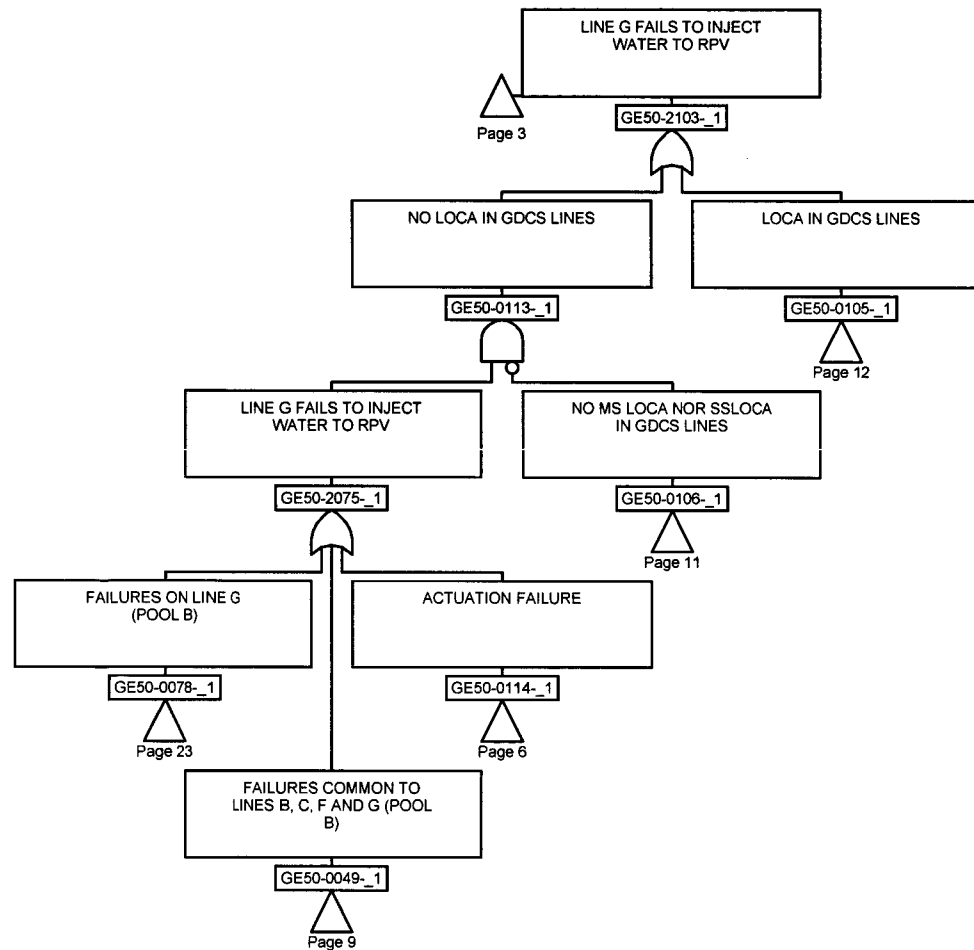


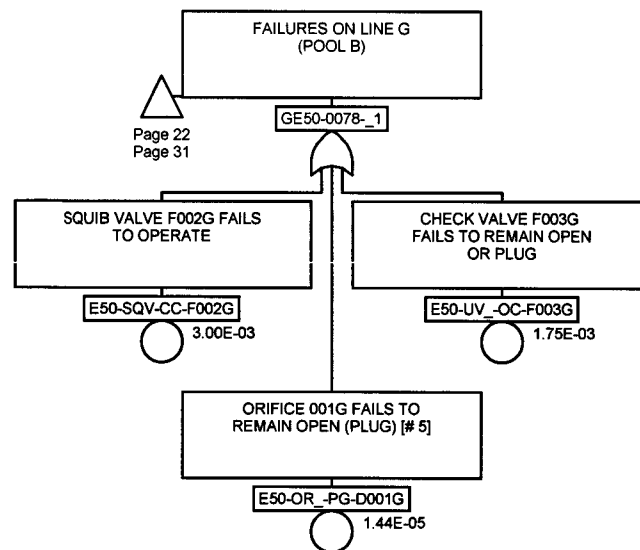


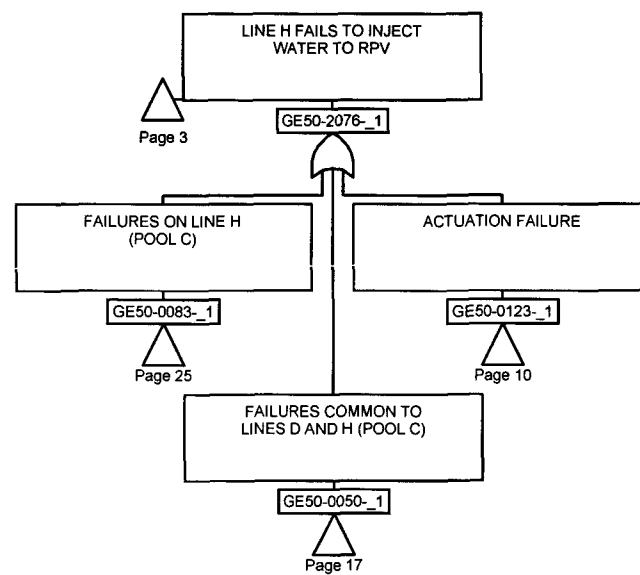


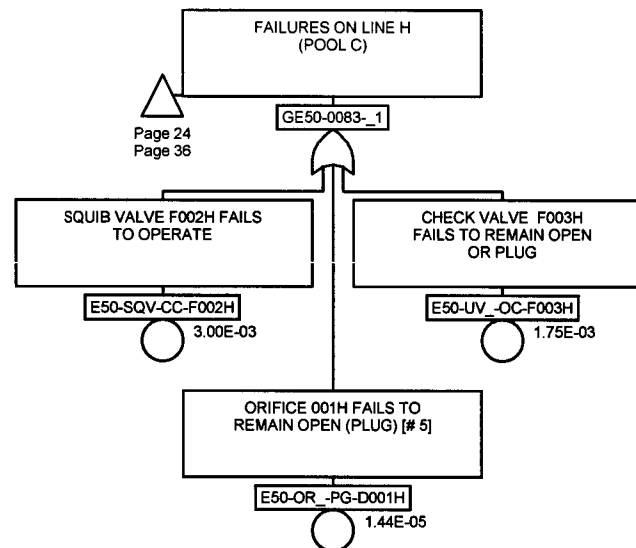


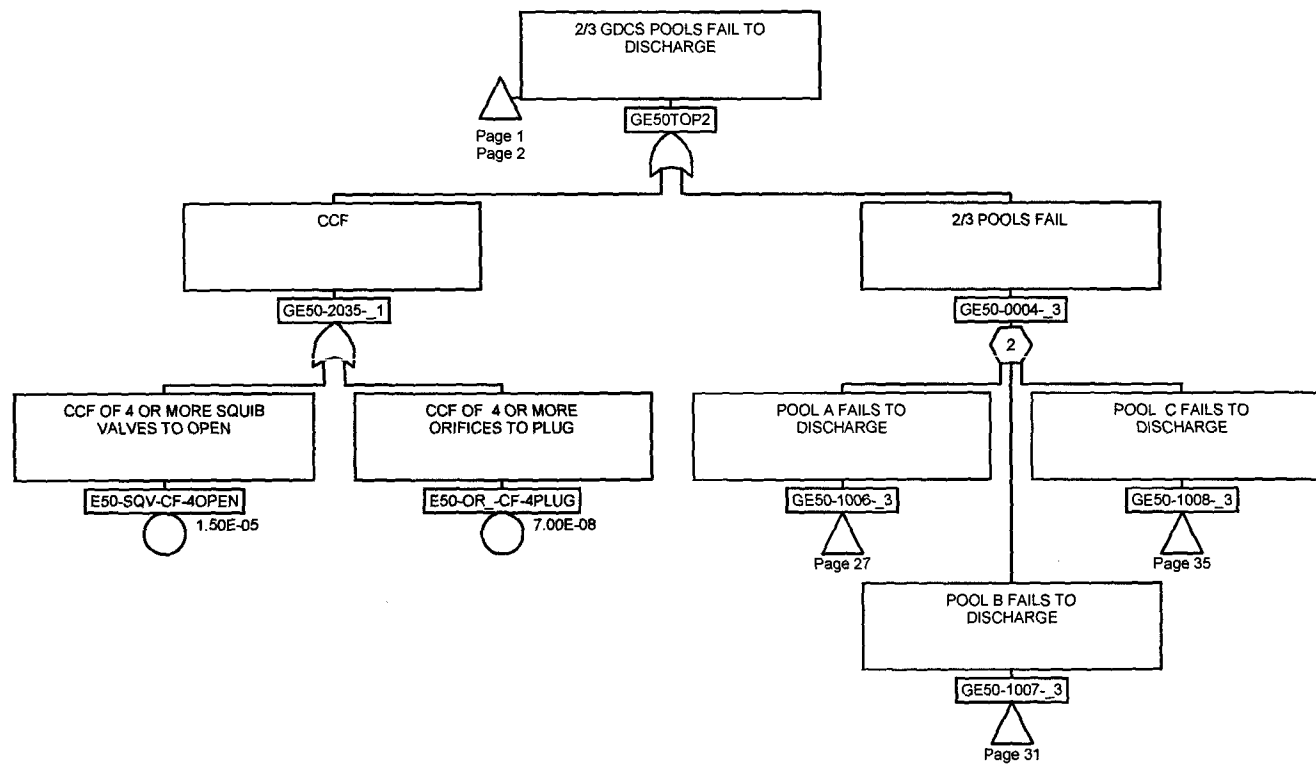


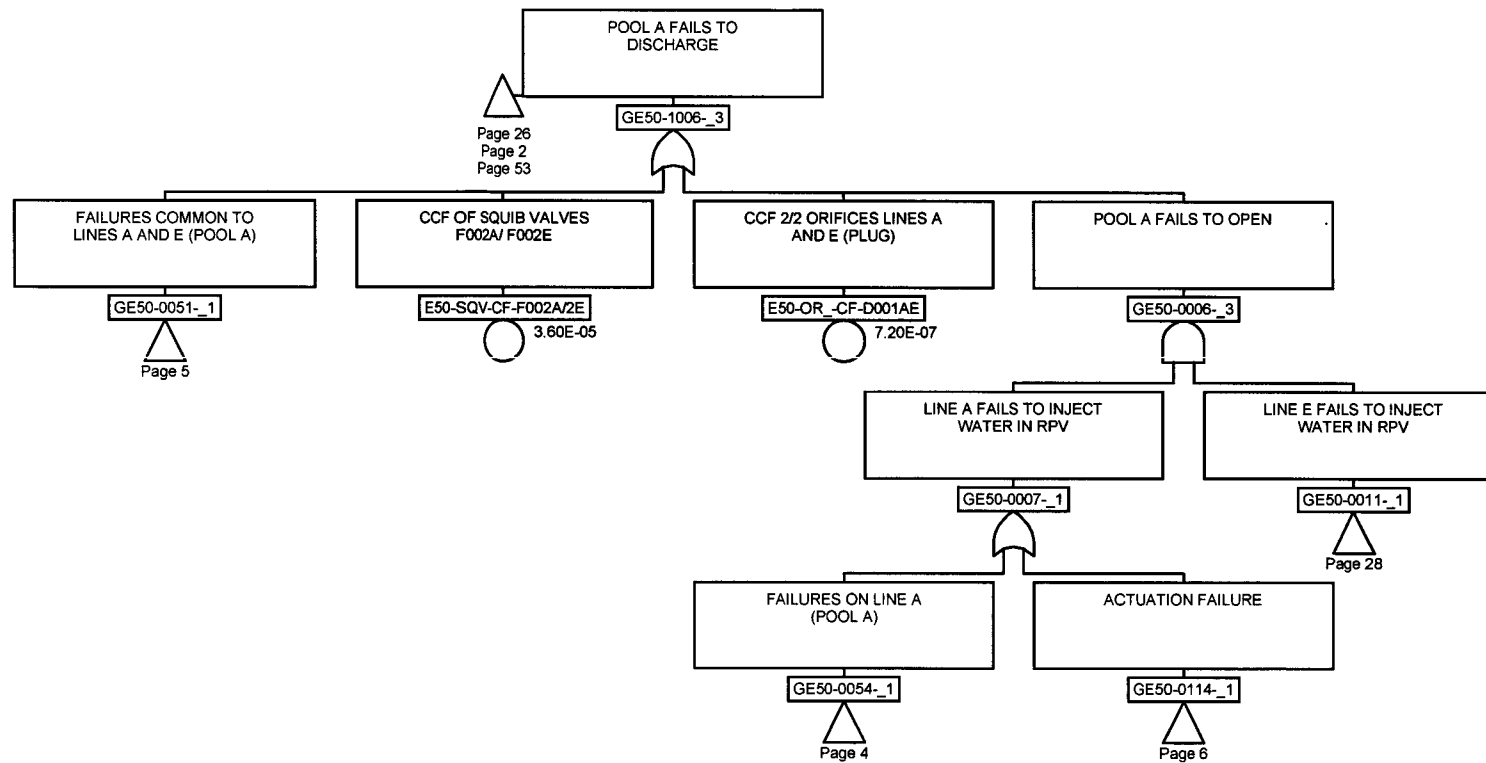


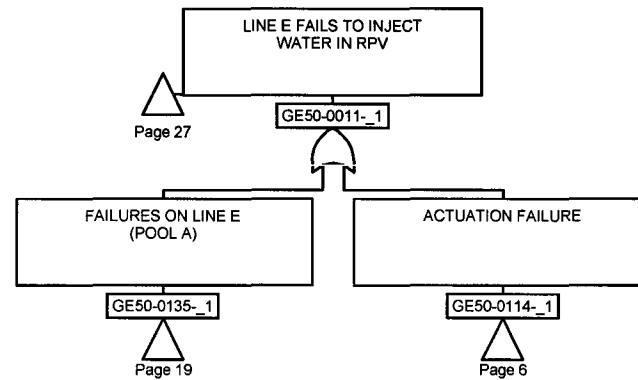


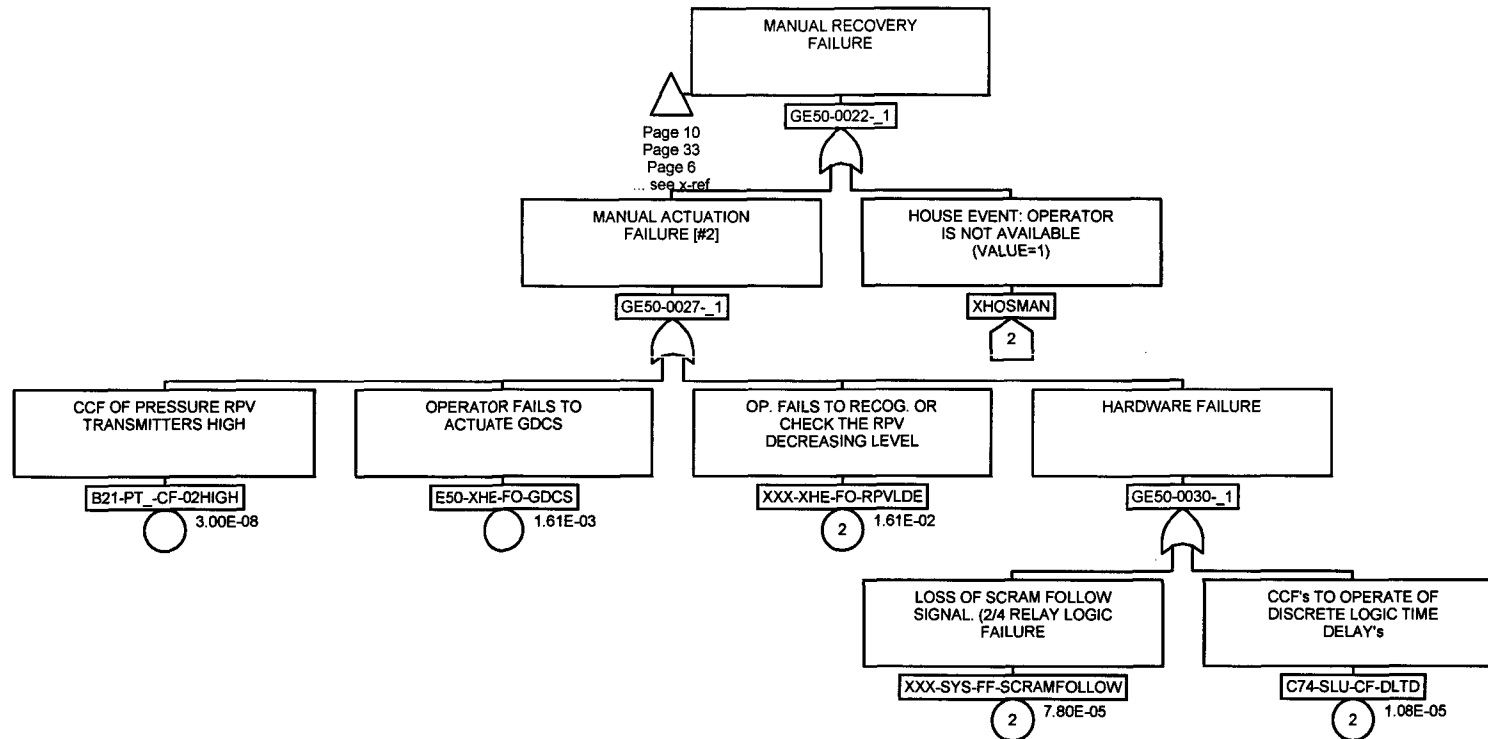


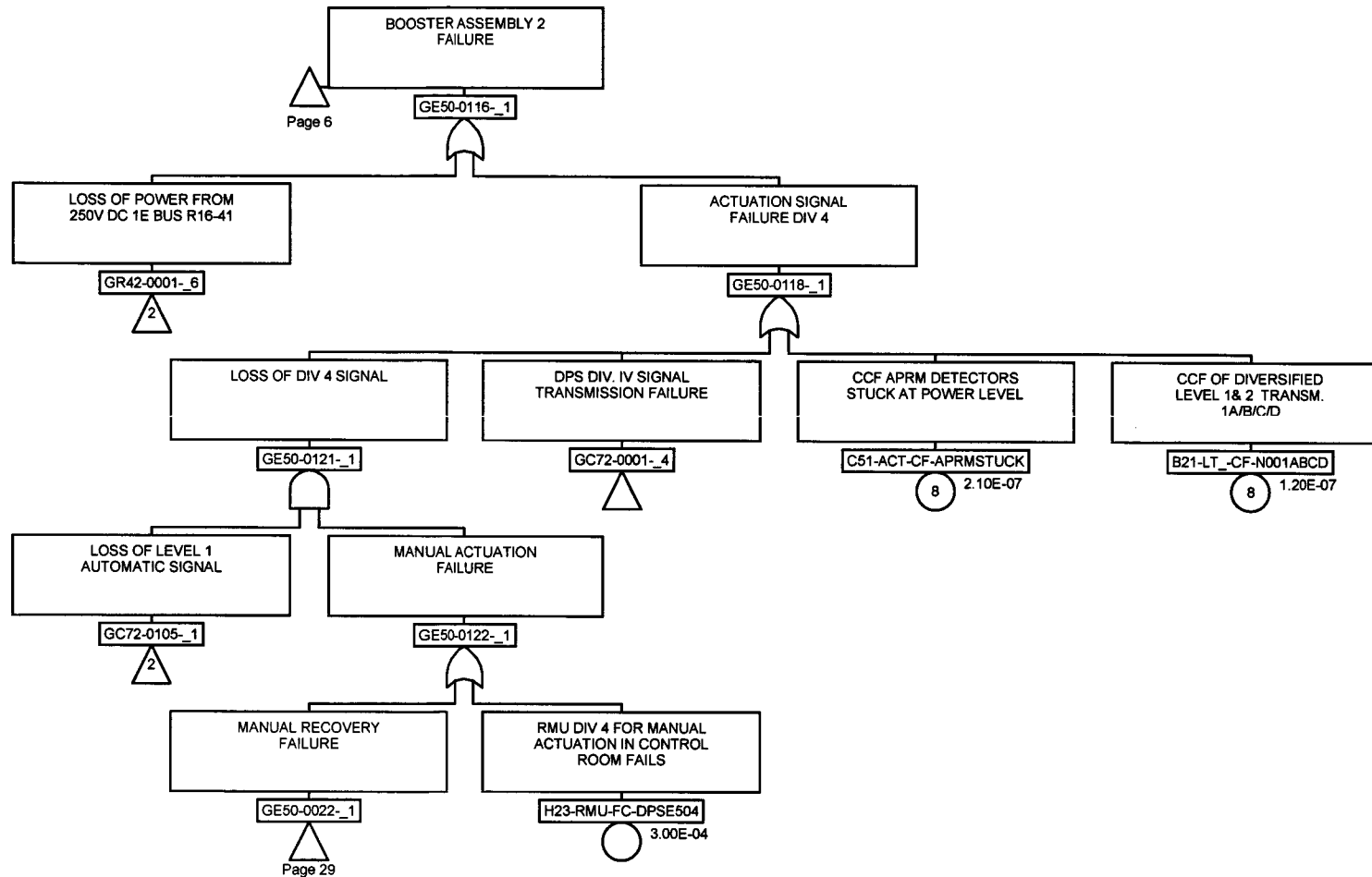


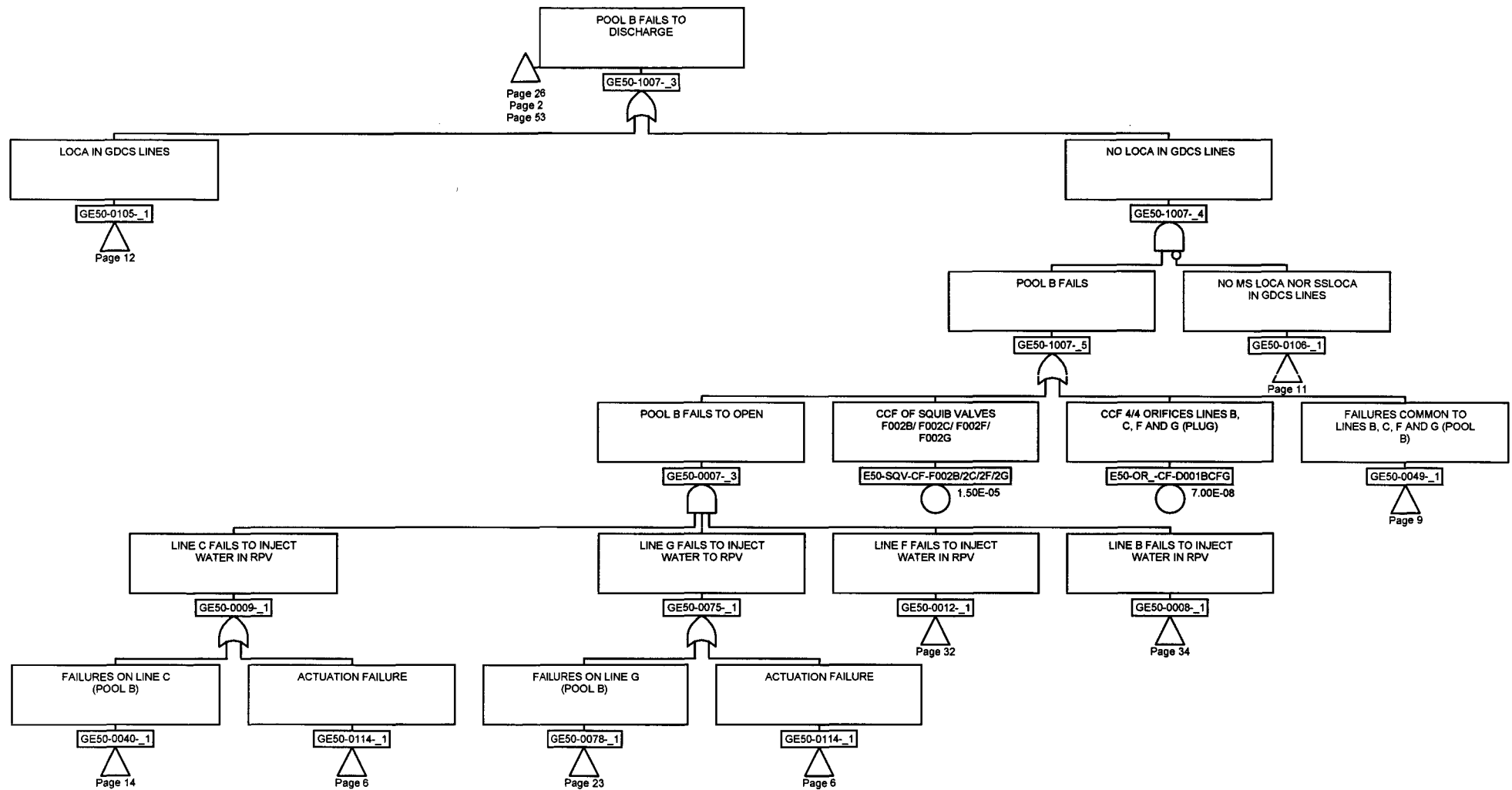


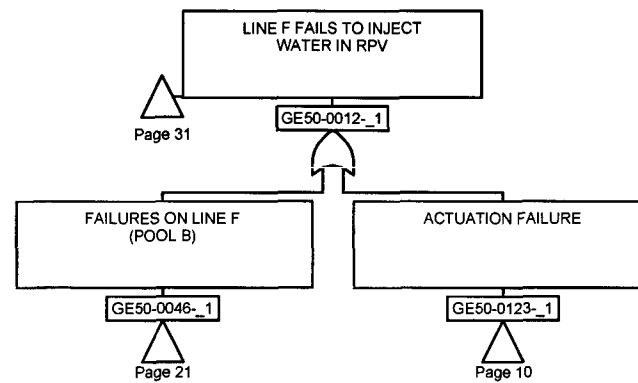


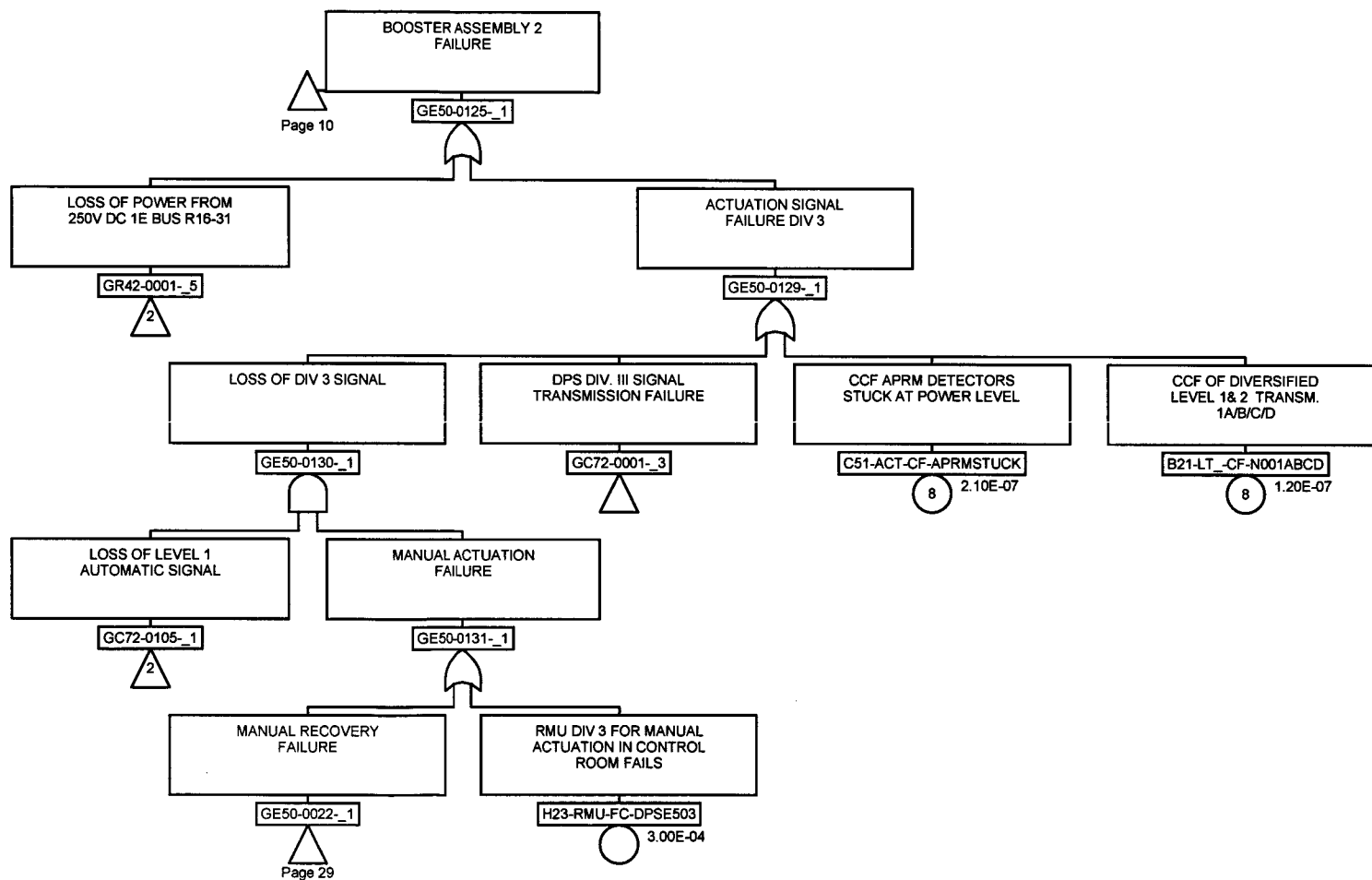


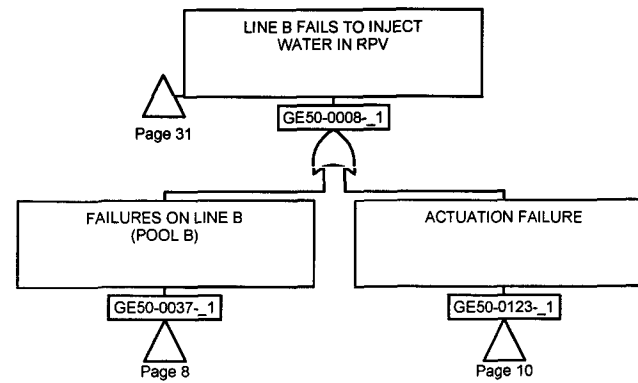


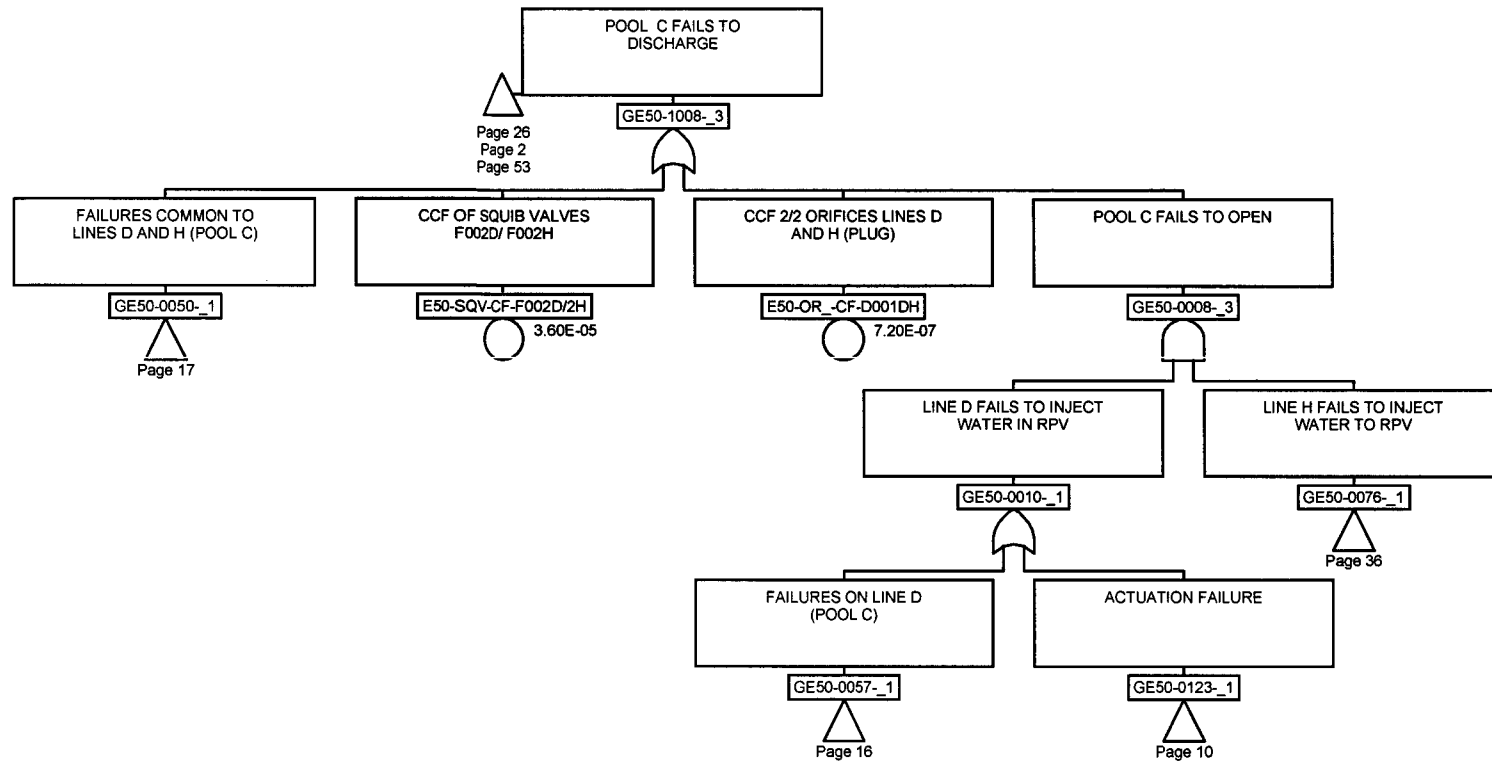


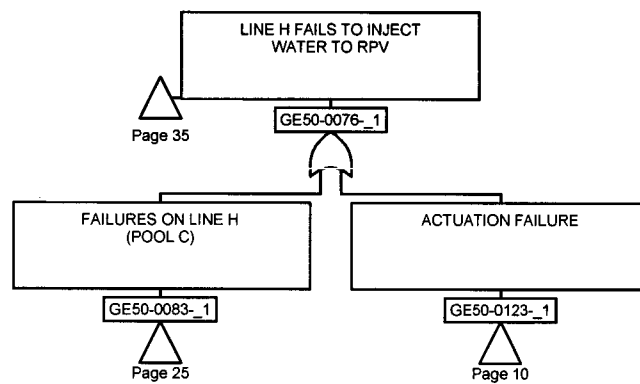


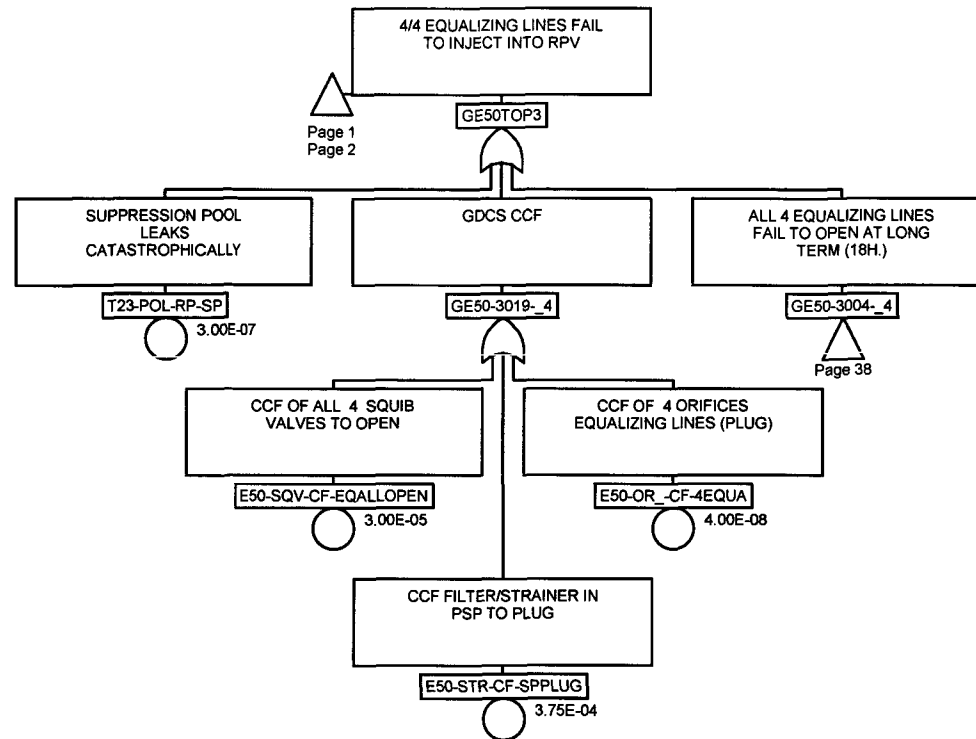


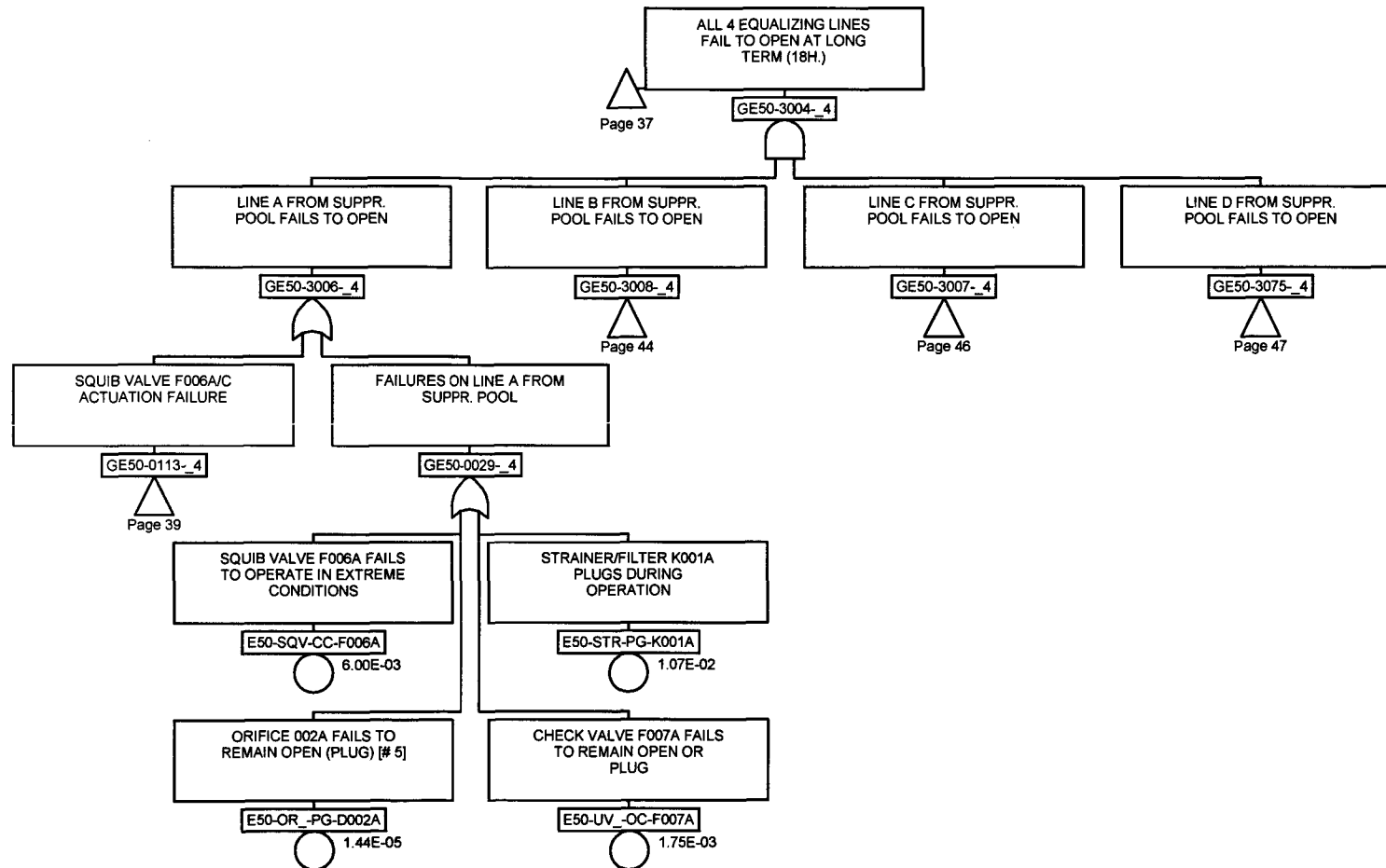


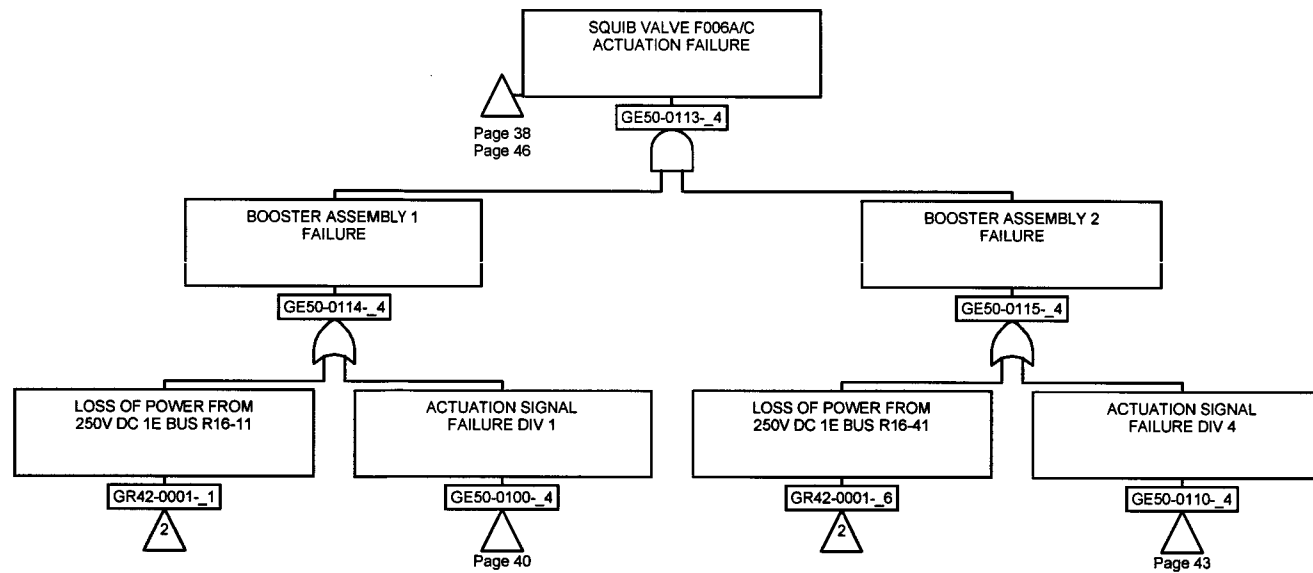


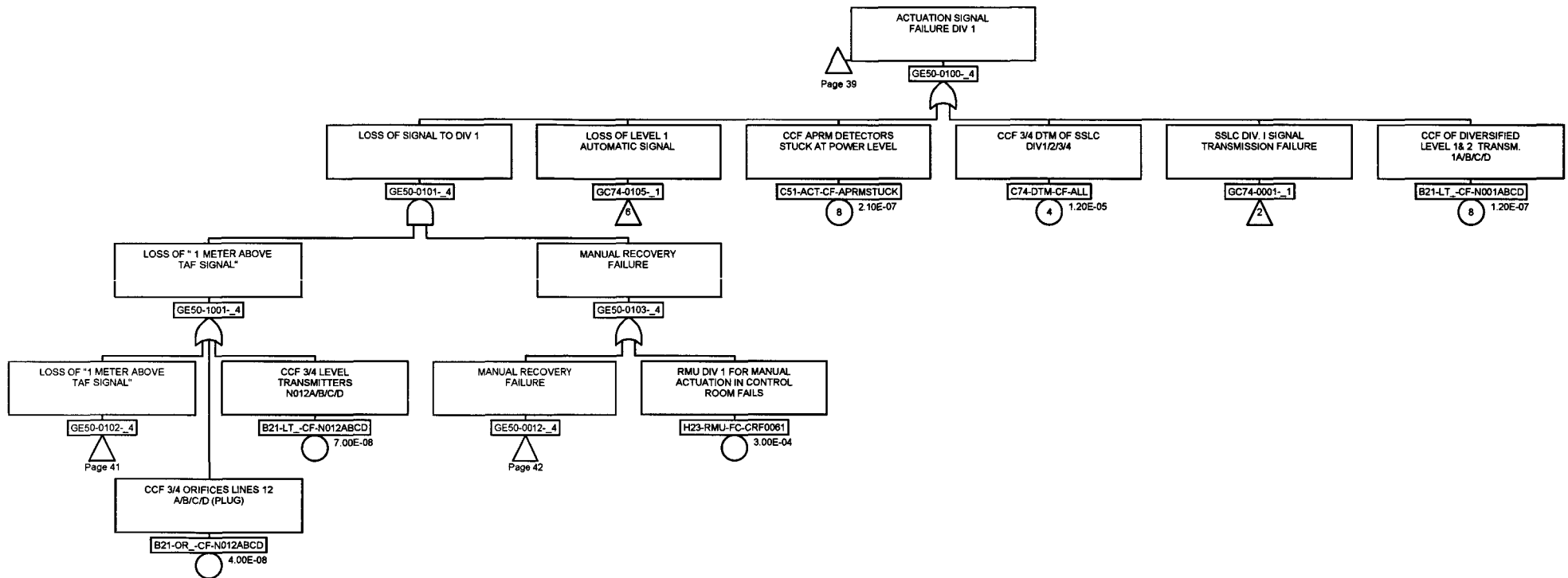




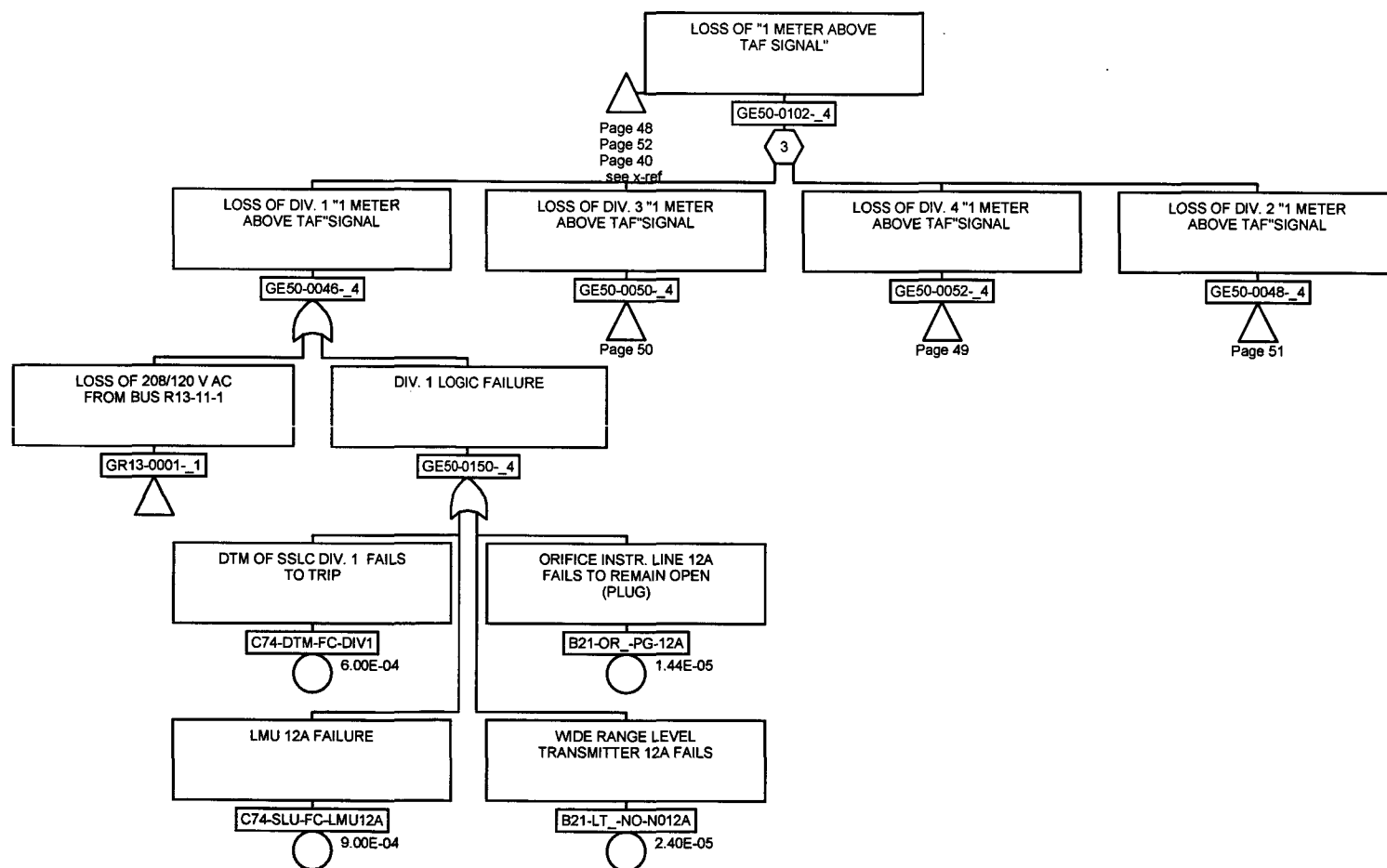


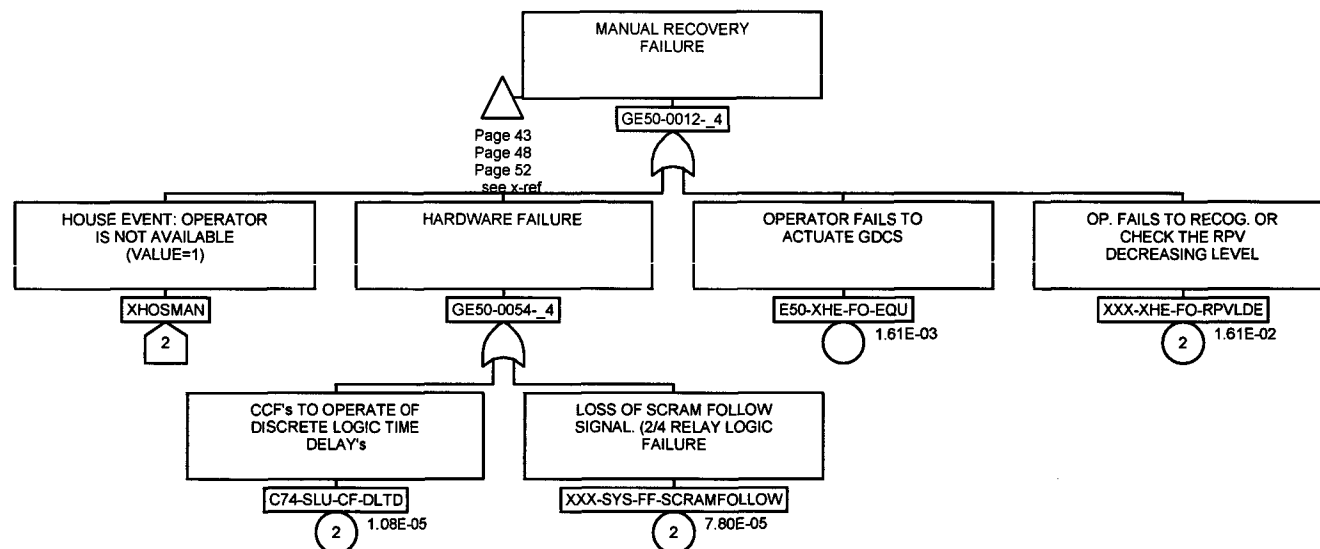




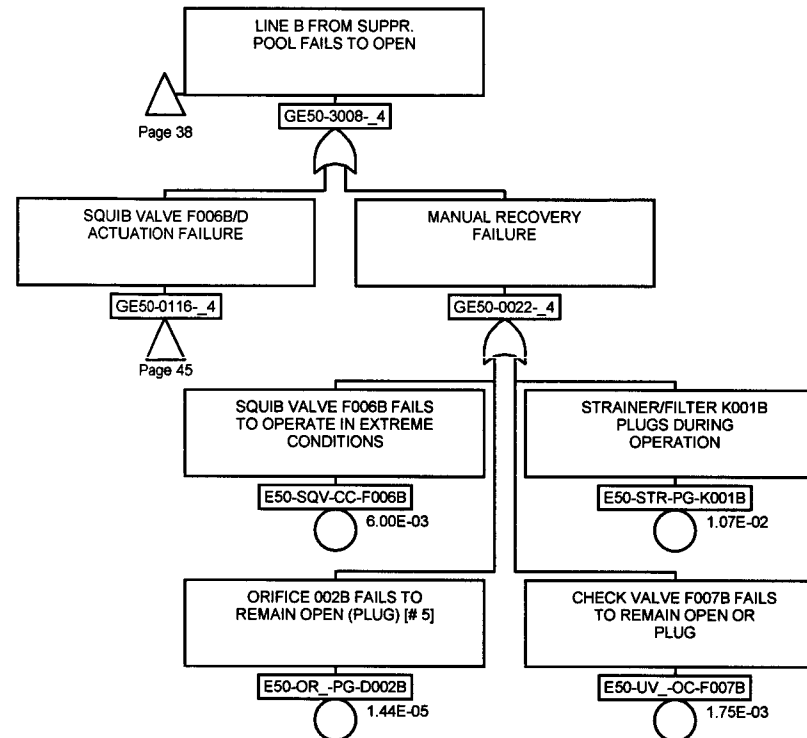


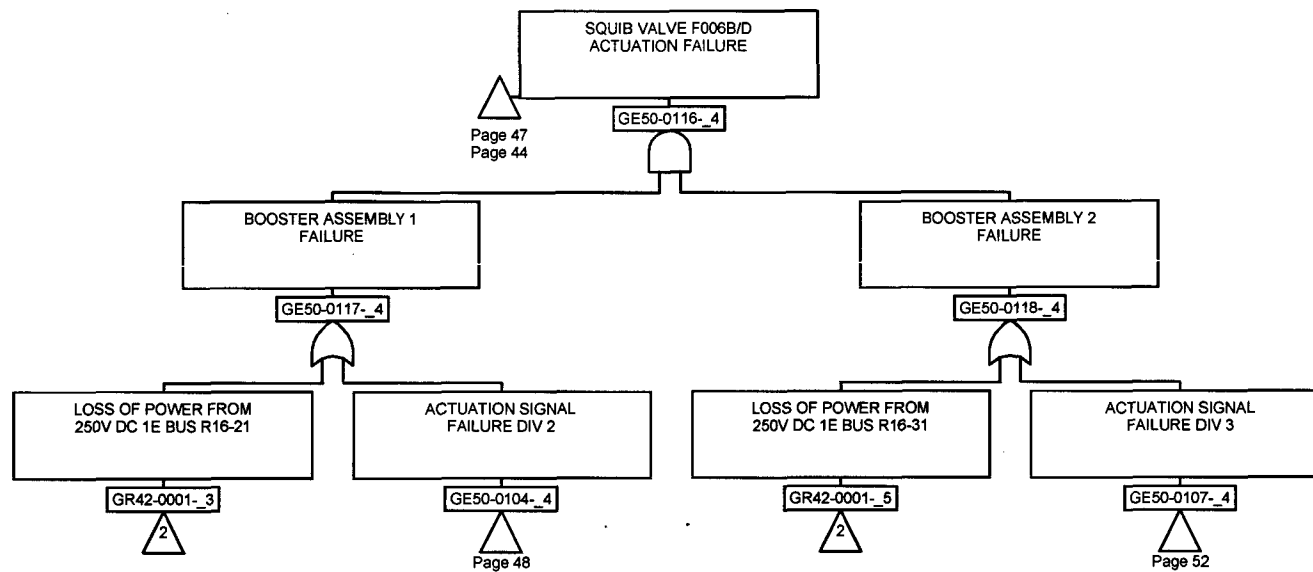
NEDO-33201 Rev 1

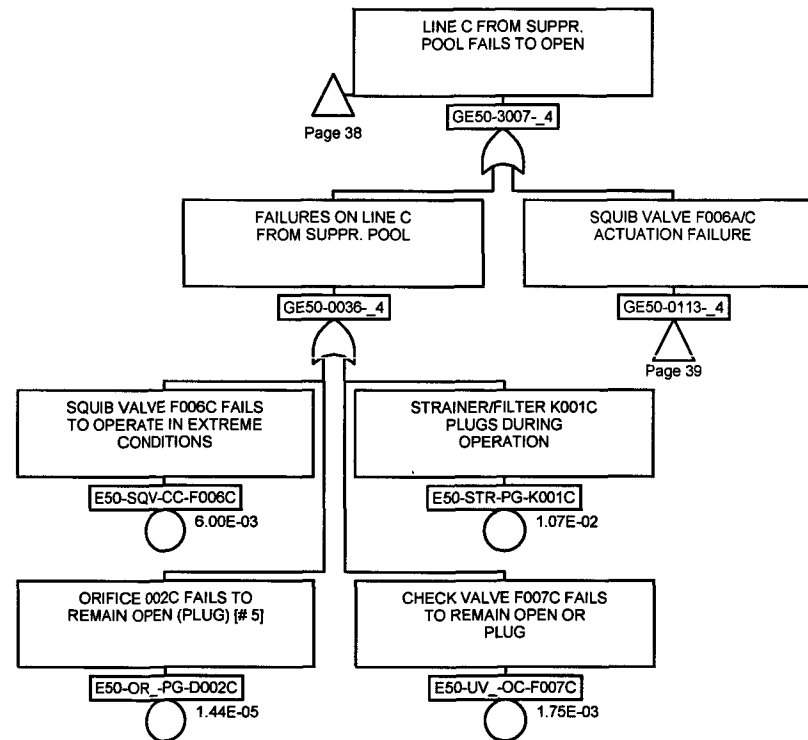


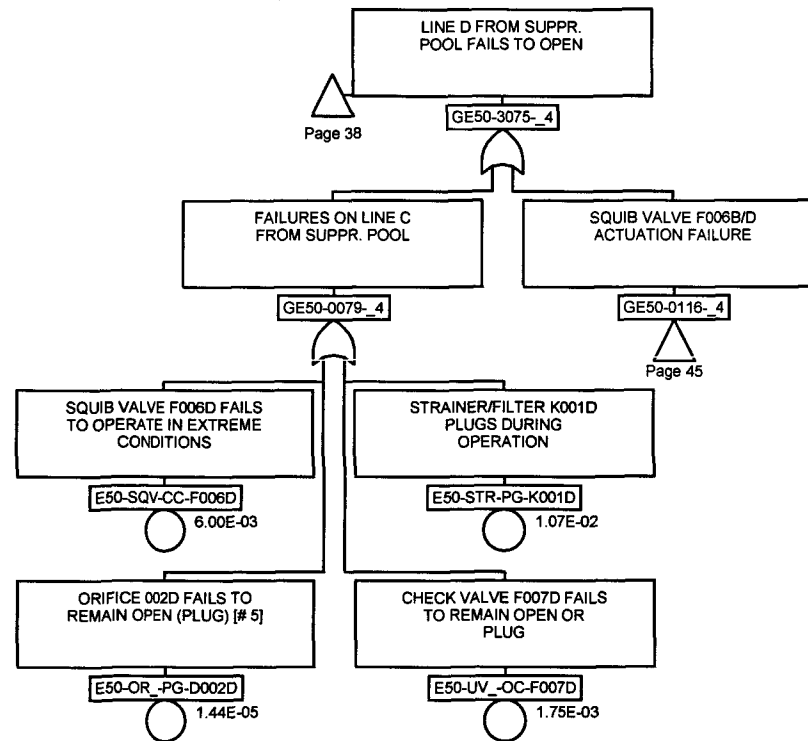


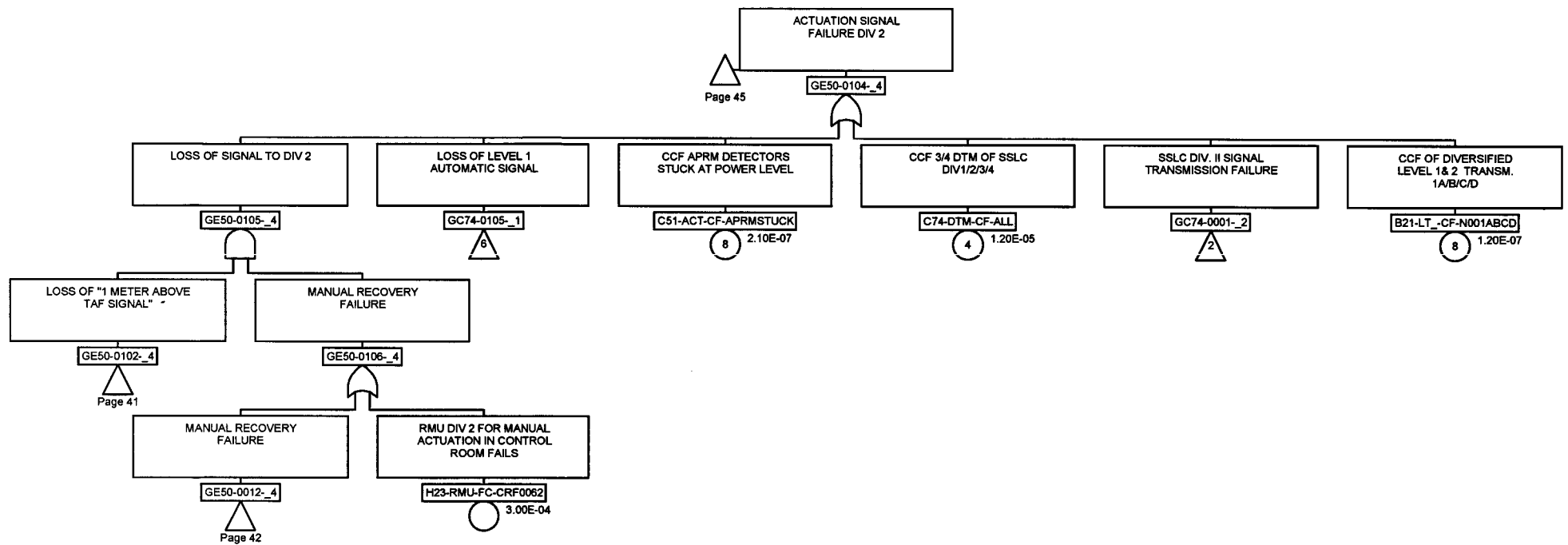


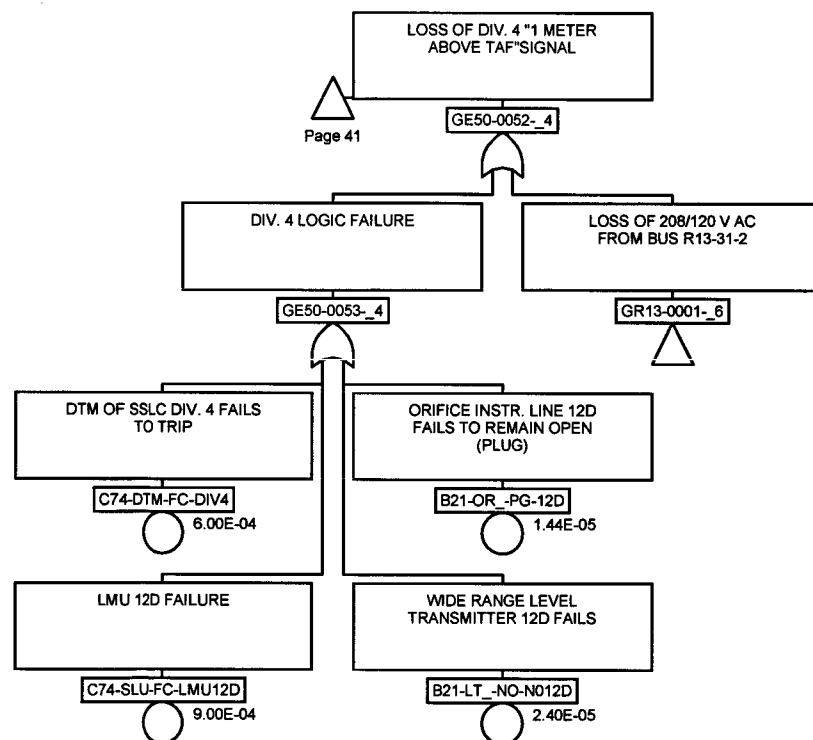


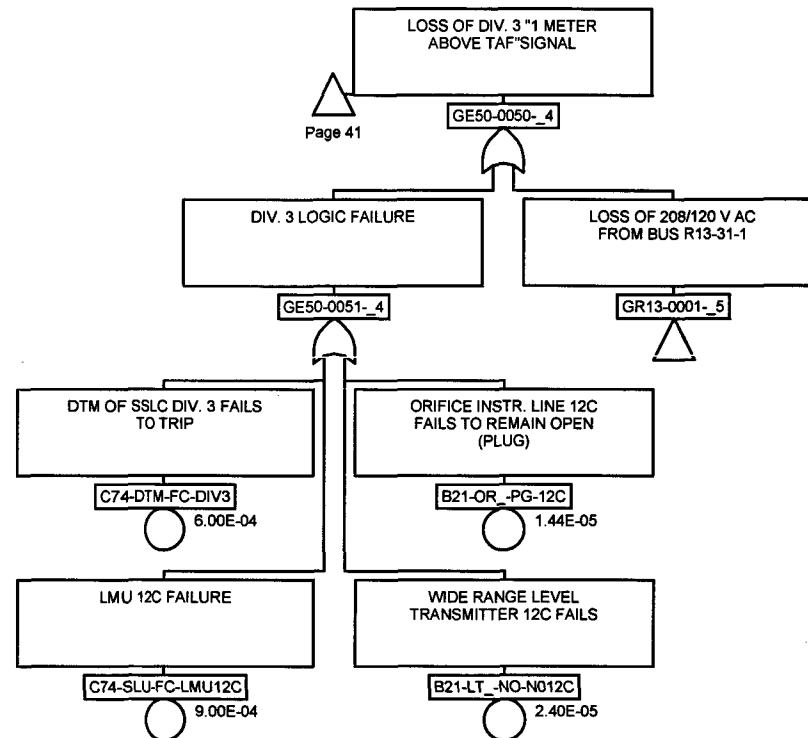


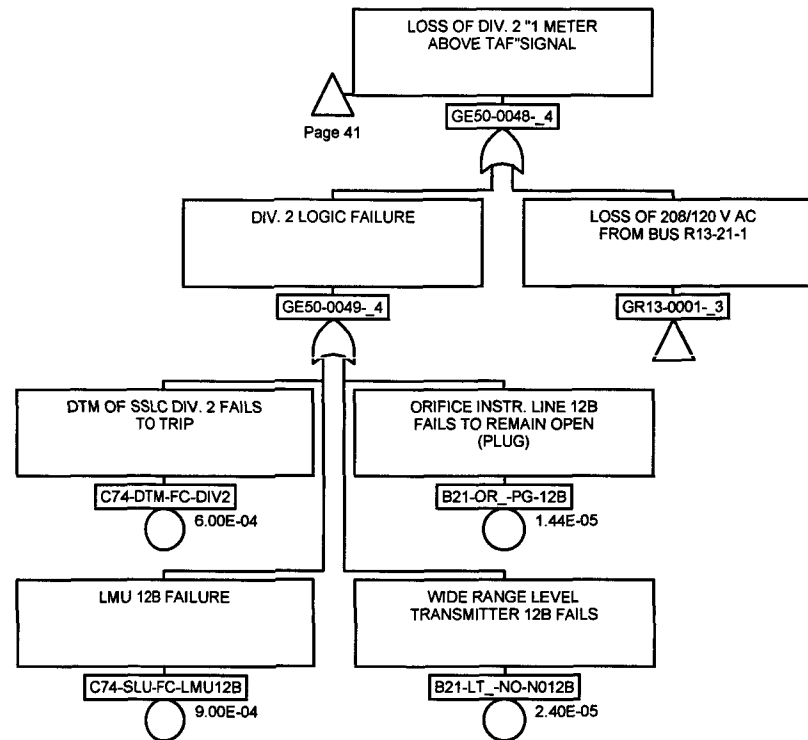


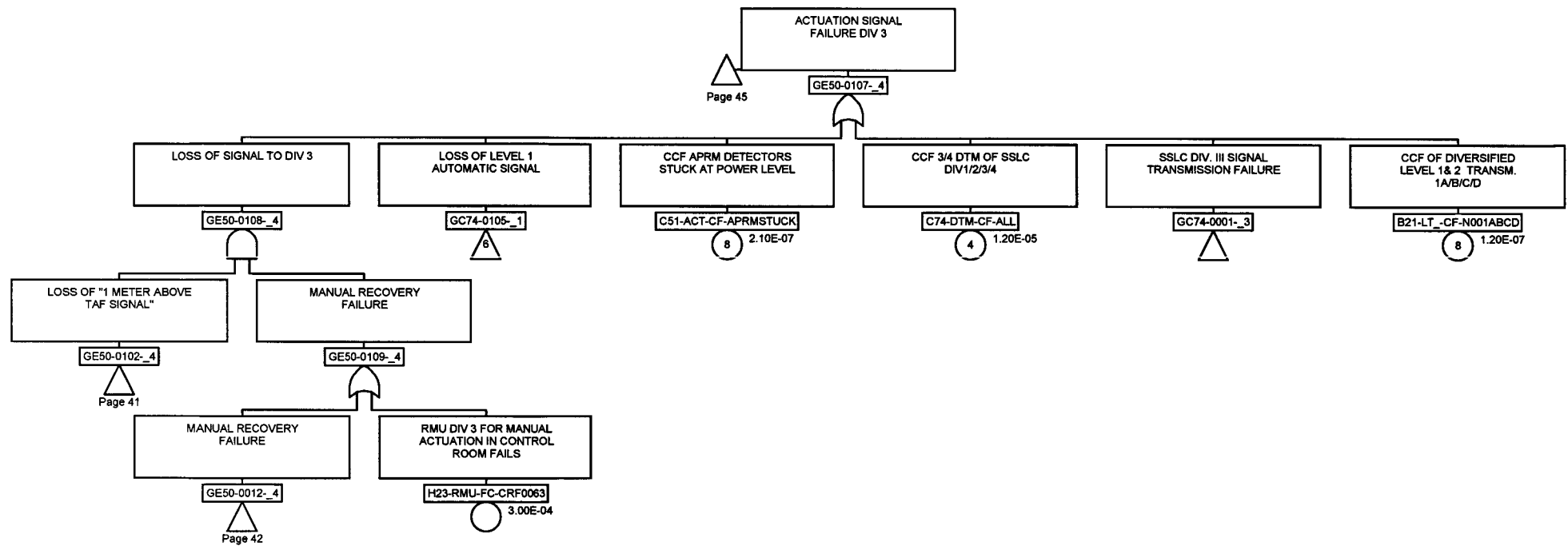


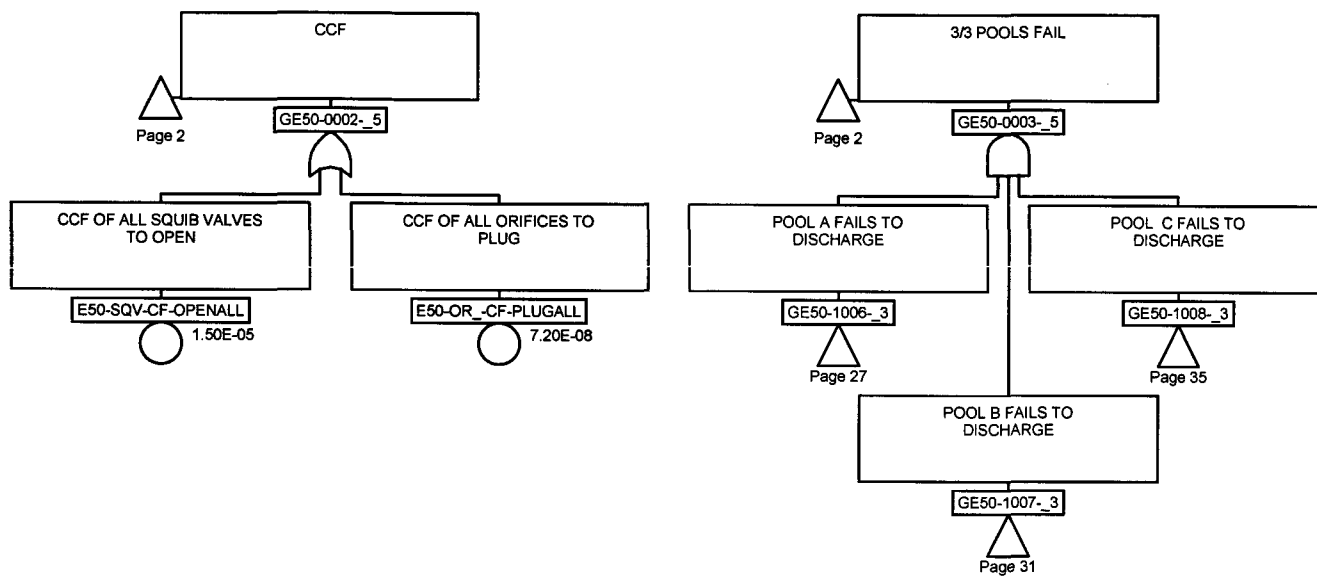










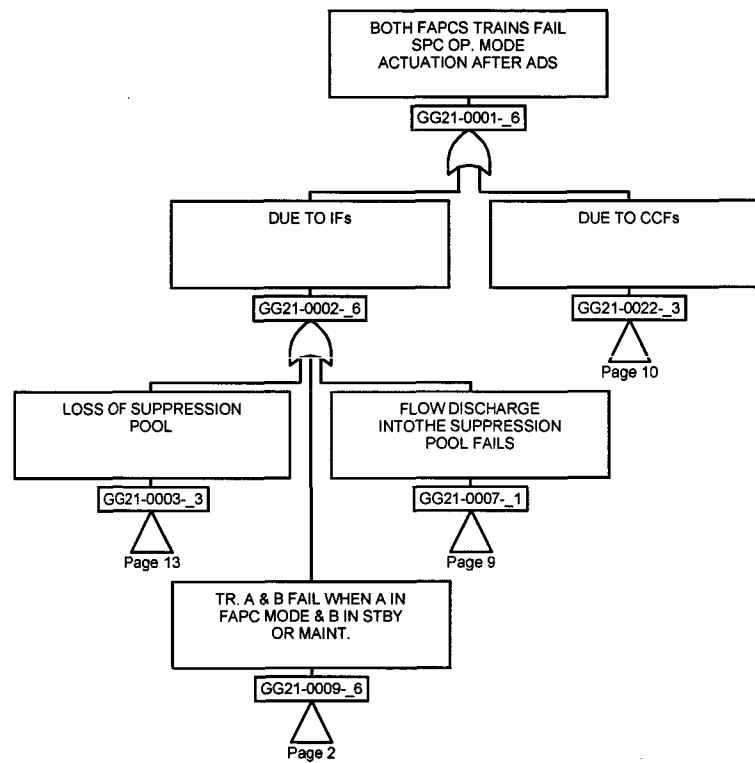


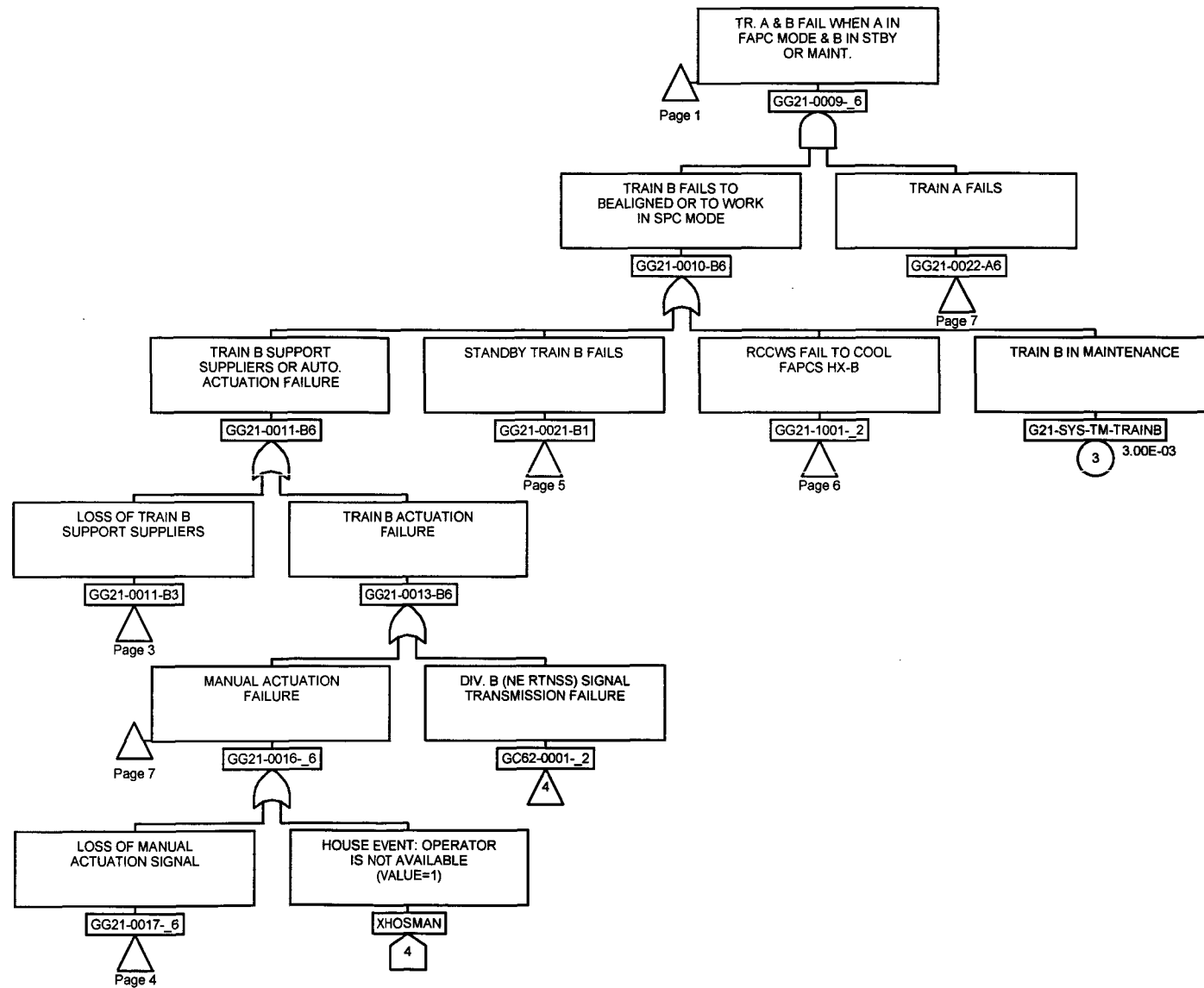
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| B21-LT -CF-N001ABCD | 6 | 5 | E50-OR -PG-D001G | 23 | 2 | E50-UV -OC-F003H | 25 | 2 |
| B21-LT -CF-N001ABCD | 10 | 5 | E50-OR -PG-D001H | 25 | 2 | E50-UV -OC-F007A | 38 | 3 |
| B21-LT -CF-N001ABCD | 30 | 5 | E50-OR -PG-D002A | 38 | 2 | E50-UV -OC-F007B | 44 | 3 |
| B21-LT -CF-N001ABCD | 33 | 5 | E50-OR -PG-D002B | 44 | 2 | E50-UV -OC-F007C | 46 | 2 |
| B21-LT -CF-N001ABCD | 40 | 7 | E50-OR -PG-D002C | 46 | 1 | E50-UV -OC-F007D | 47 | 2 |
| B21-LT -CF-N001ABCD | 43 | 7 | E50-OR -PG-D002D | 47 | 1 | E50-XHE-FO-EQU | 42 | 3 |
| B21-LT -CF-N001ABCD | 48 | 7 | E50-POL-RP-POOLA | 5 | 2 | E50-XHE-FO-GDCS | 29 | 2 |
| B21-LT -CF-N001ABCD | 52 | 7 | E50-POL-RP-POOLB | 9 | 2 | GC72-0001- 3 | 33 | 3 |
| B21-LT -CF-N012ABCD | 40 | 2 | E50-POL-RP-POOLC | 17 | 2 | GC72-0001- 4 | 30 | 3 |
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| B21-OR -CF-N012ABCD | 40 | 2 | E50-SQV-CC-F002E | 19 | 1 | GC74-0001- 2 | 10 | 3 |
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| B21-OR -PG-12B | 51 | 2 | E50-SQV-CC-F002G | 23 | 1 | GC74-0001- 3 | 52 | 6 |
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| B21-OR -PG-12D | 49 | 2 | E50-SQV-CC-F006A | 38 | 2 | GC74-0105- 1 | 6 | 1 |
| B21-PT -CF-02HIGH | 29 | 1 | E50-SQV-CC-F006B | 44 | 2 | GC74-0105- 1 | 10 | 1 |
| C51-ACT-CF-APRMSTUCK | 6 | 4 | E50-SQV-CC-F006C | 46 | 1 | GC74-0105- 1 | 40 | 3 |
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| C51-ACT-CF-APRMSTUCK | 33 | 4 | E50-SQV-CF-EQALLOPEN | 37 | 2 | GC74-0105- 1 | 52 | 3 |
| C51-ACT-CF-APRMSTUCK | 40 | 4 | E50-SQV-CF-F002A/2E | 27 | 2 | GE50-0002- 5 | 2 | 3 |
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| E50-OR -CF-D001BCFG | 31 | 5 | E50-SYS-FF-MLLOCA | 12 | 1 | GE50-0012- 4 | 48 | 2 |
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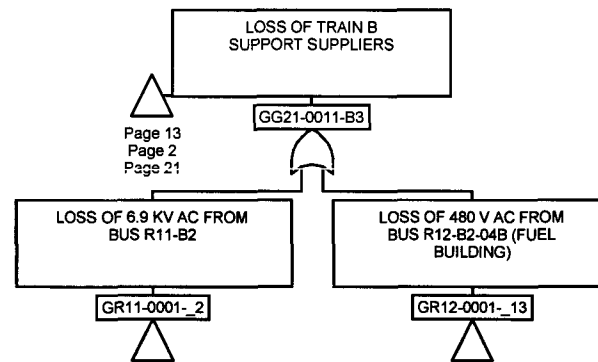
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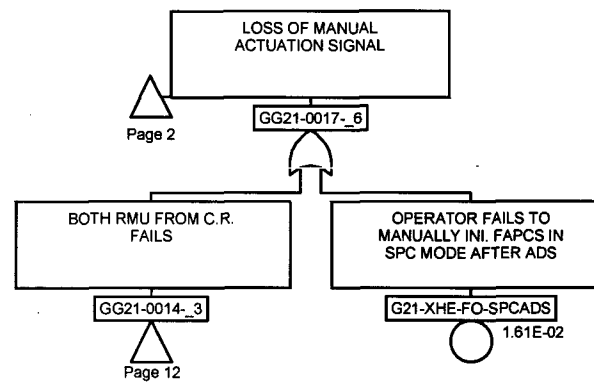
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| GE50-2076- 1 | 3 | 2 | XHOSE50MSLI | 11 | 1 | | | |
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| GE50-2101- 1 | 3 | 2 | XHOSMAN | 29 | 4 | | | |
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| GE50-2103- 1 | 3 | 2 | XXX-XHE-FO-RPVLDE | 29 | 3 | | | |
| GE50-2103- 1 | 22 | 2 | XXX-XHE-FO-RPVLDE | 42 | 4 | | | |
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| GE50TOP5 | 2 | 4 | | | | | | |
| GE50TOP6 | 2 | 2 | | | | | | |
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| H23-RMU-FC-CRF0063 | 52 | 3 | | | | | | |
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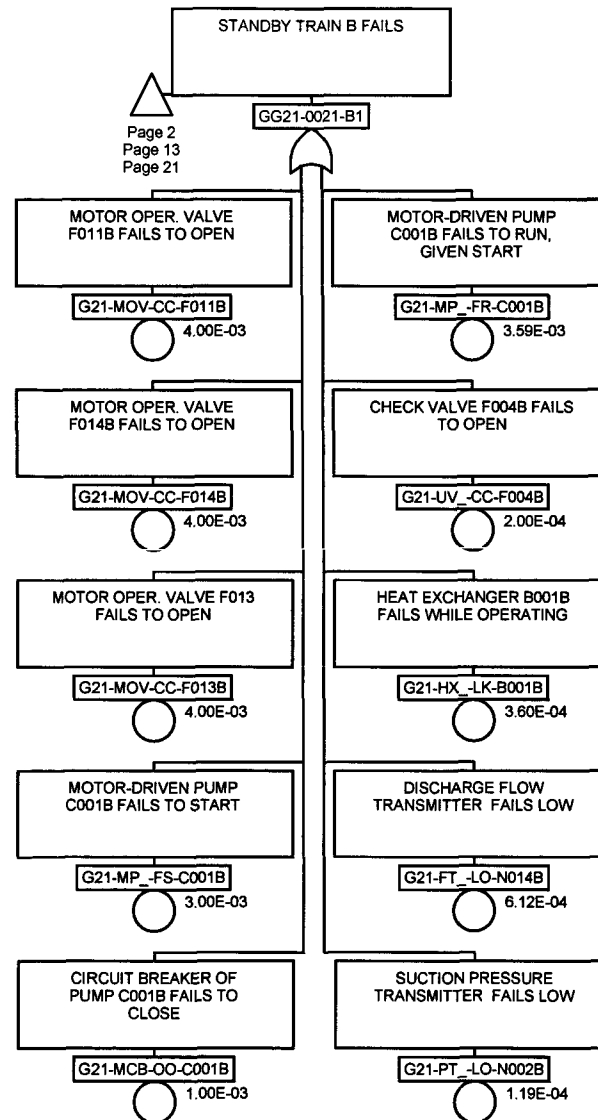
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Fault Tree

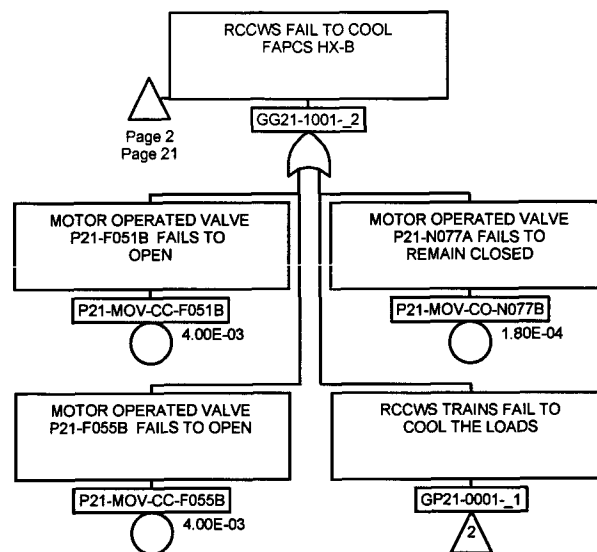


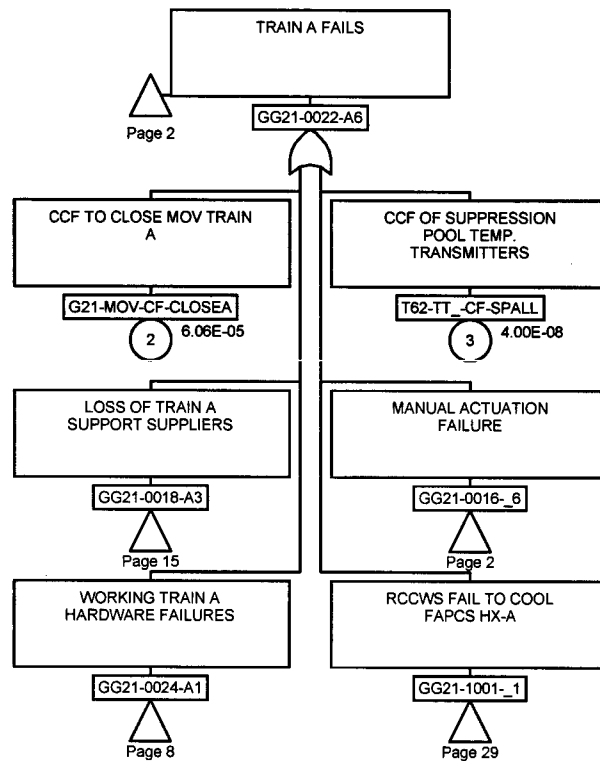


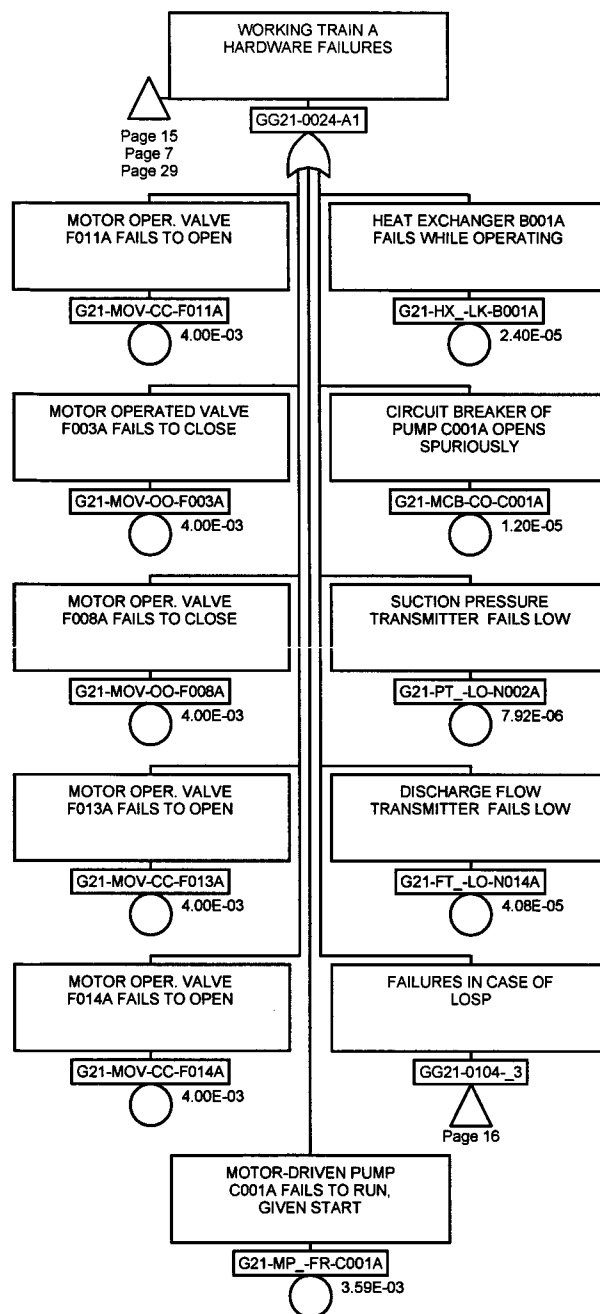


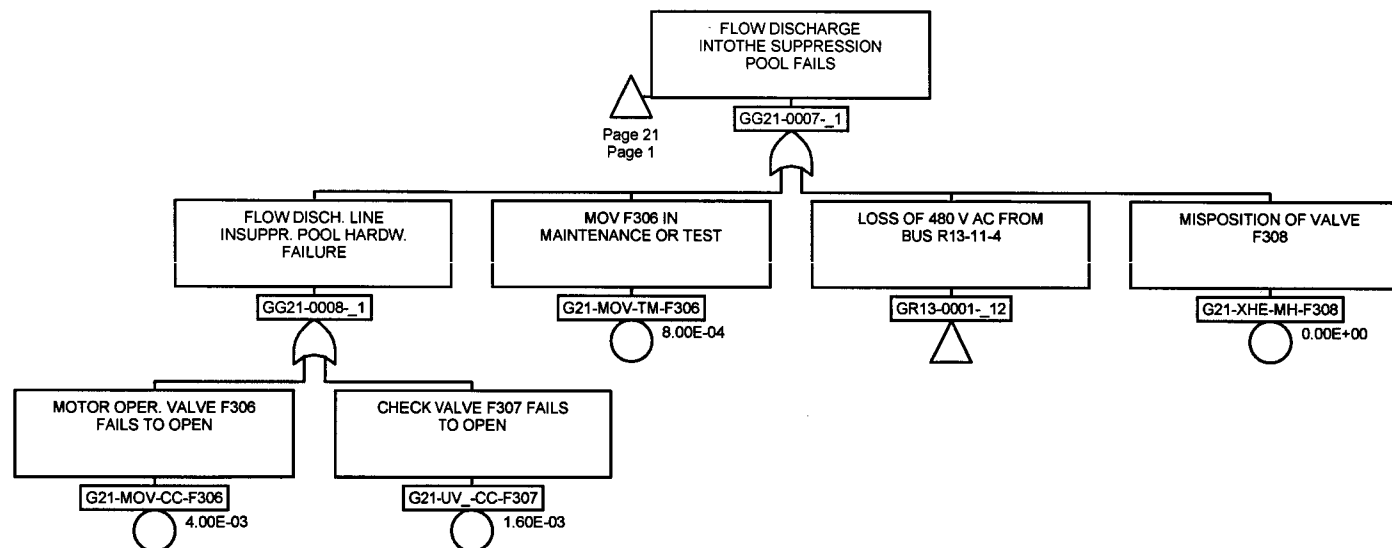


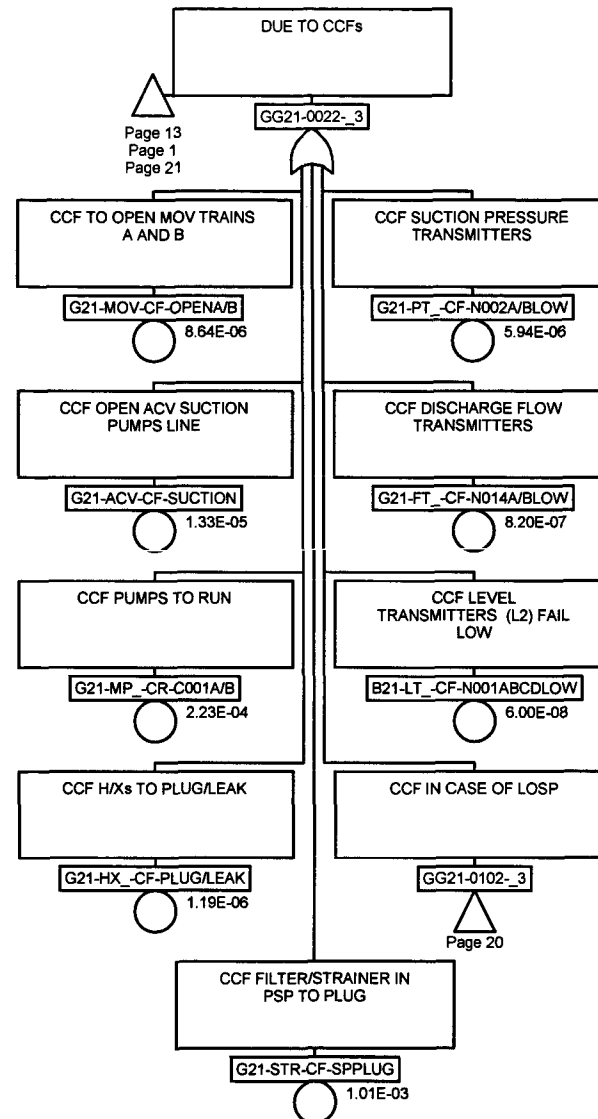


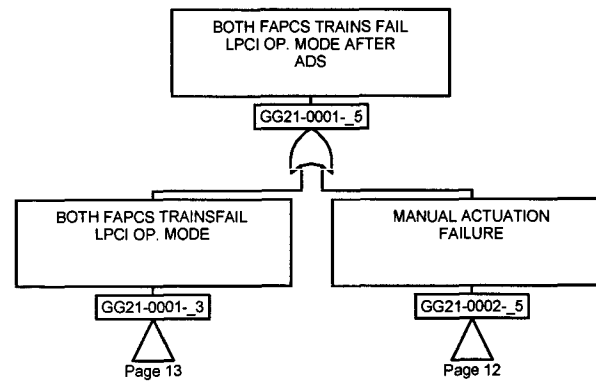


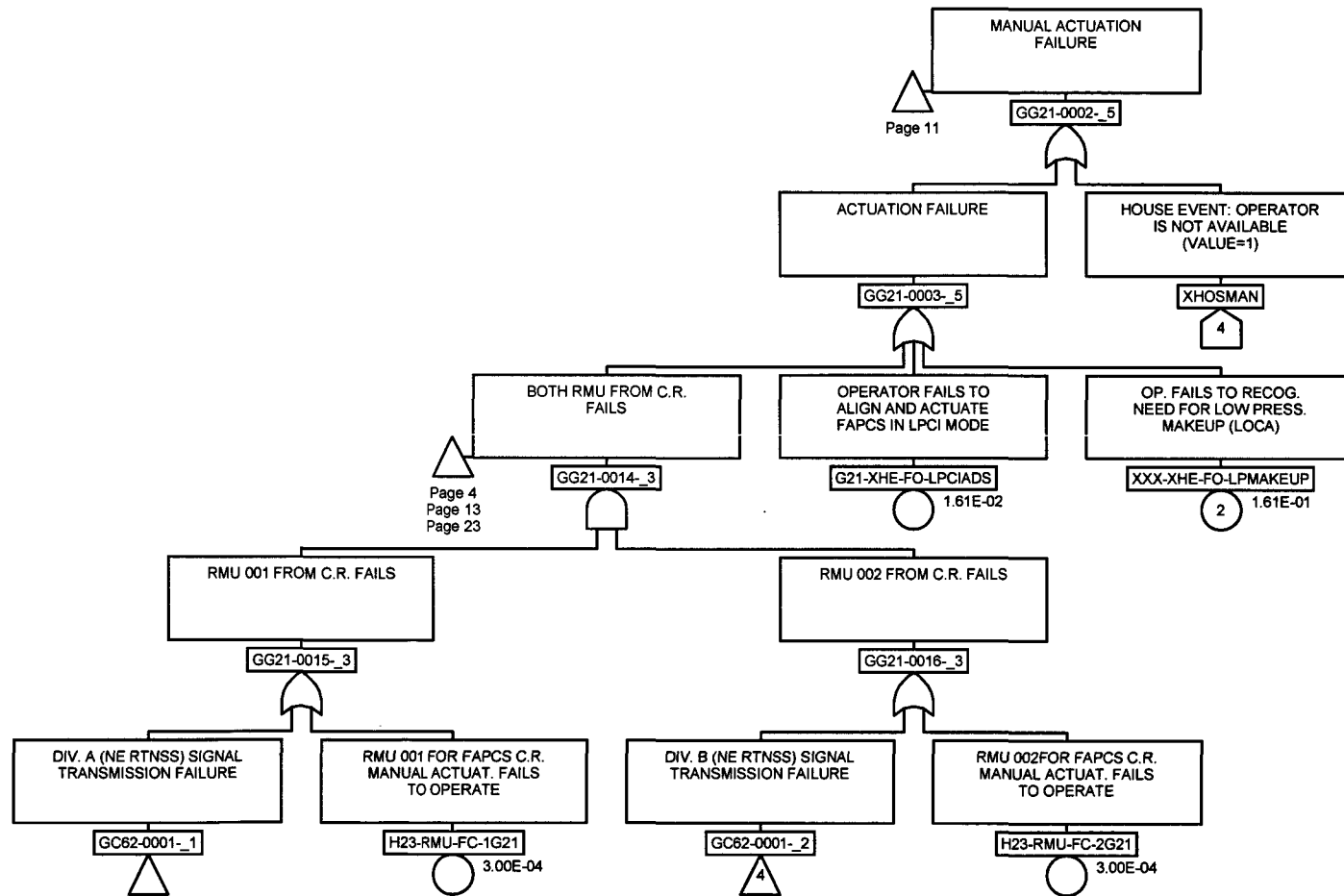


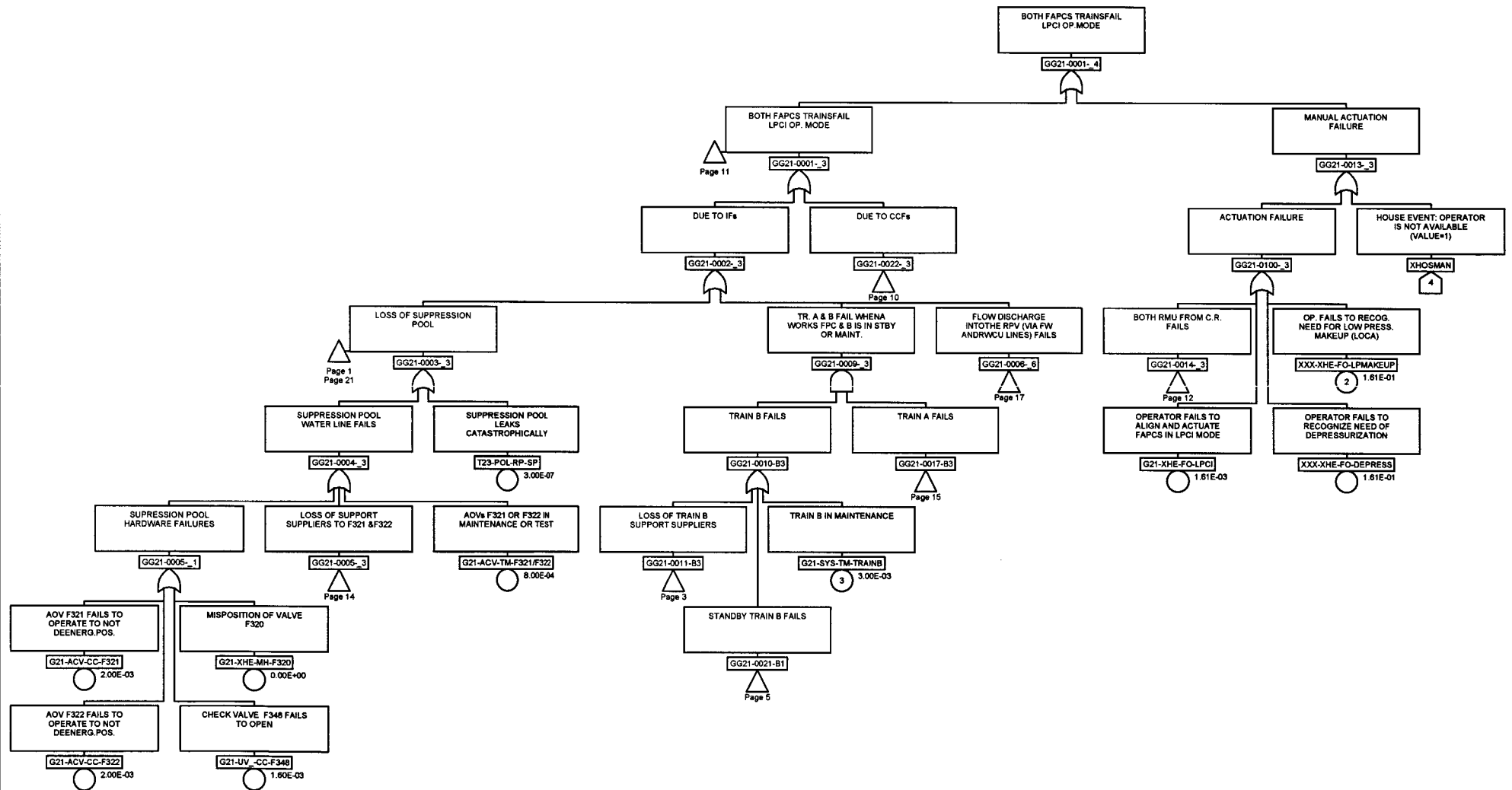


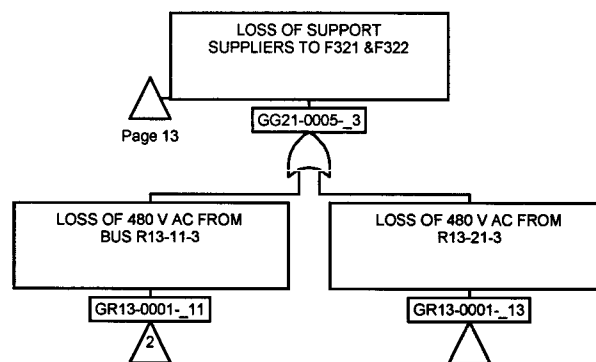


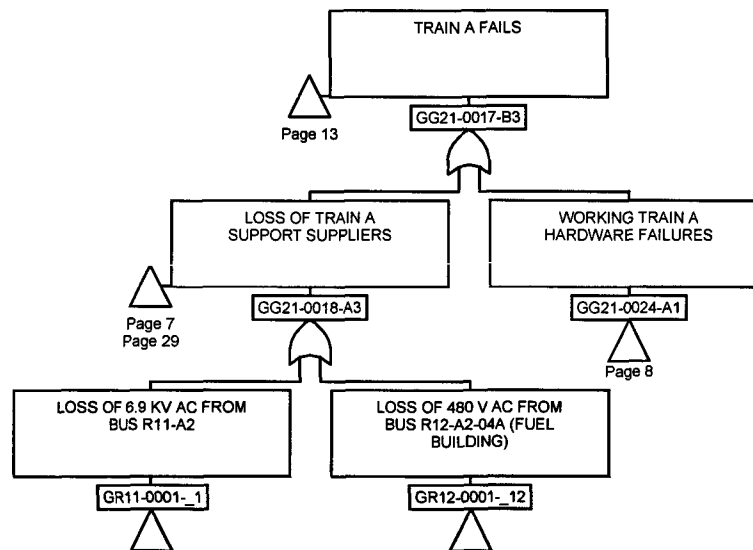


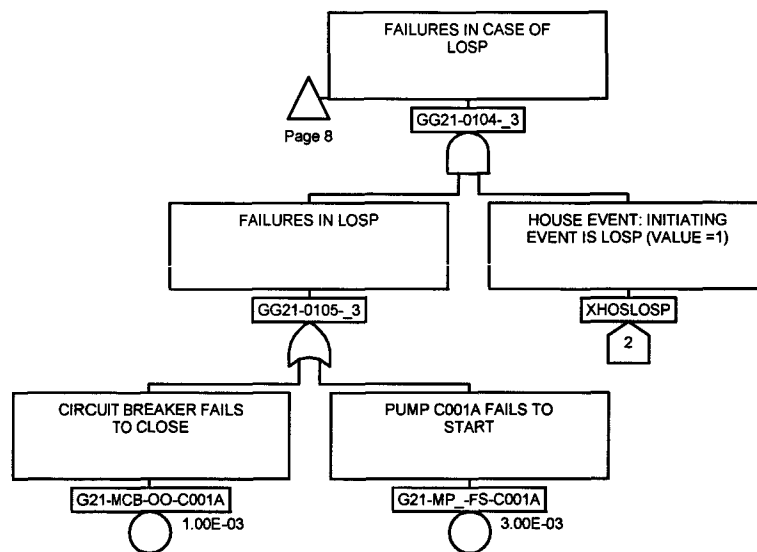


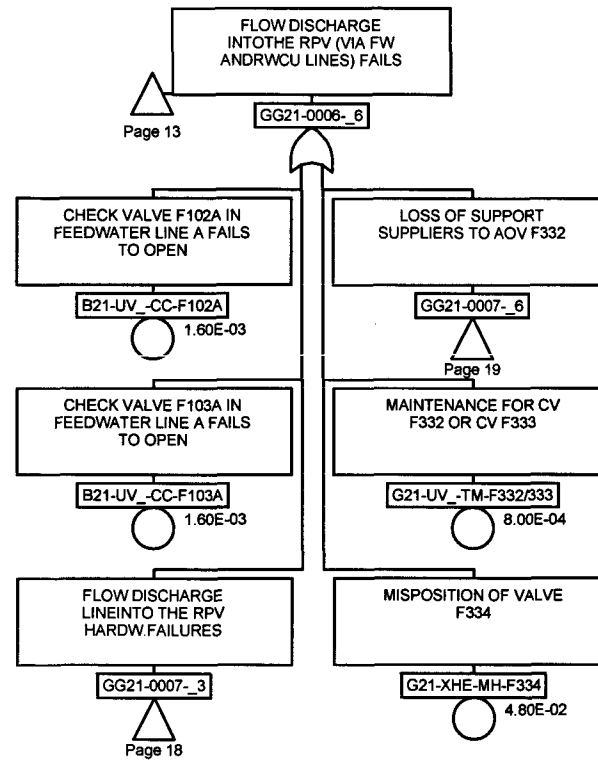


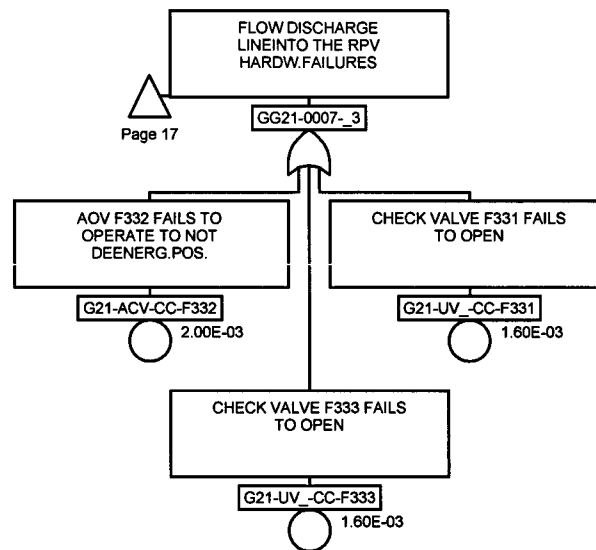


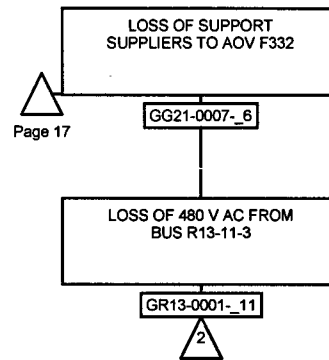


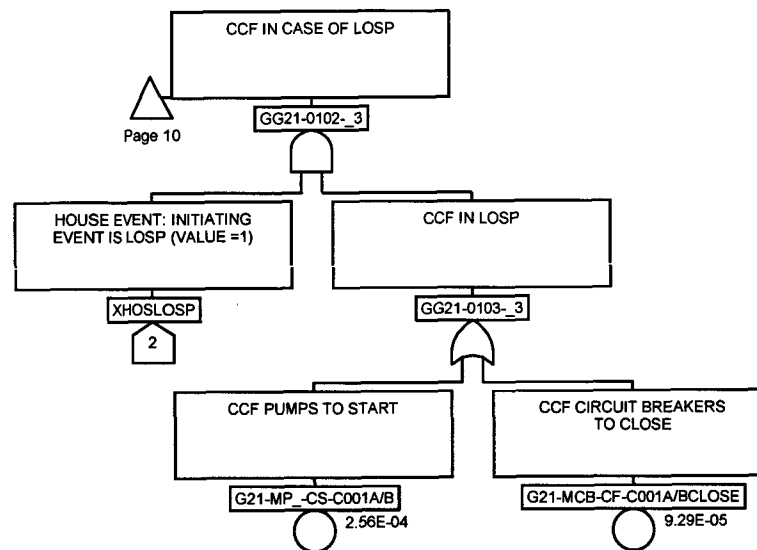


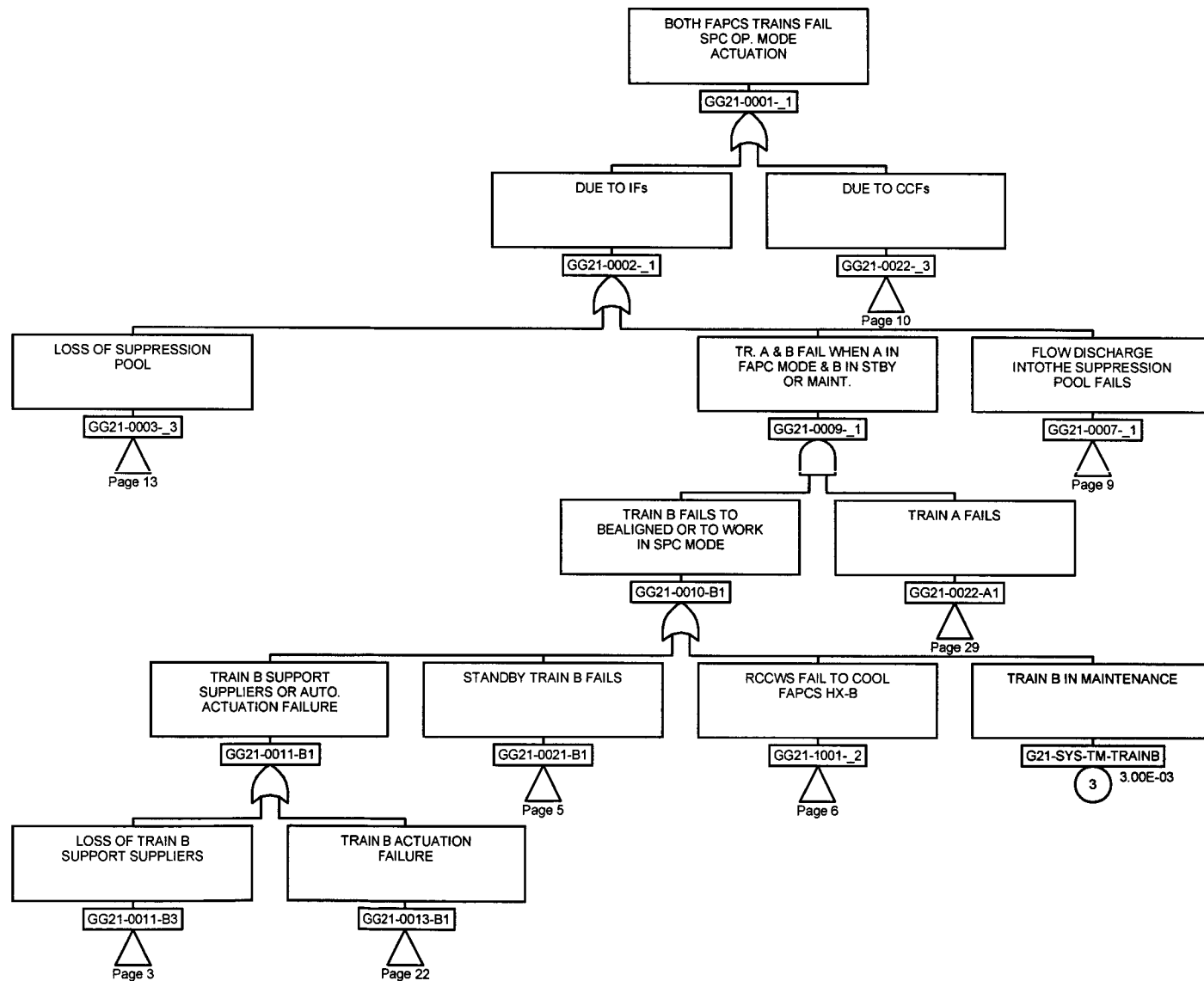


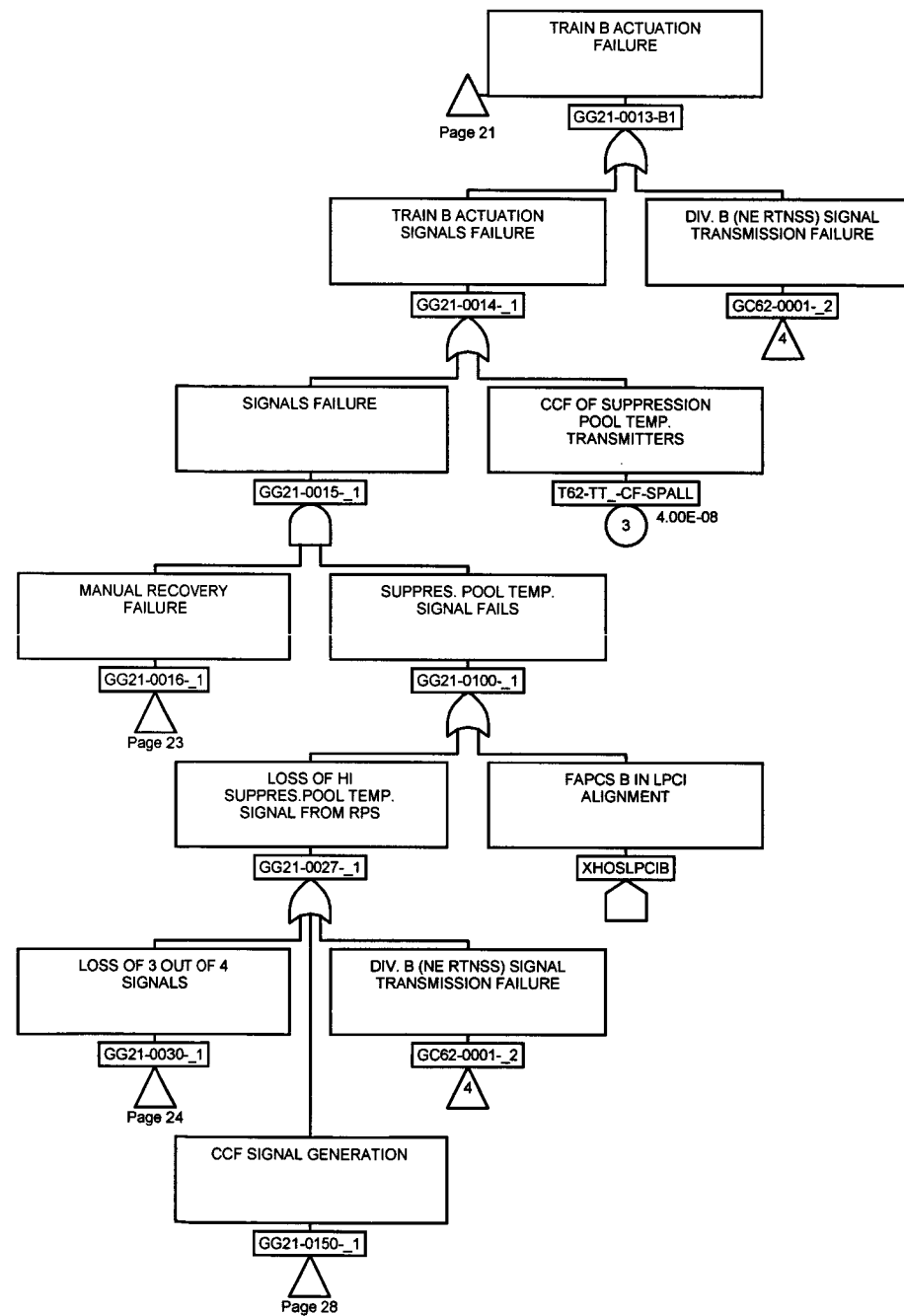


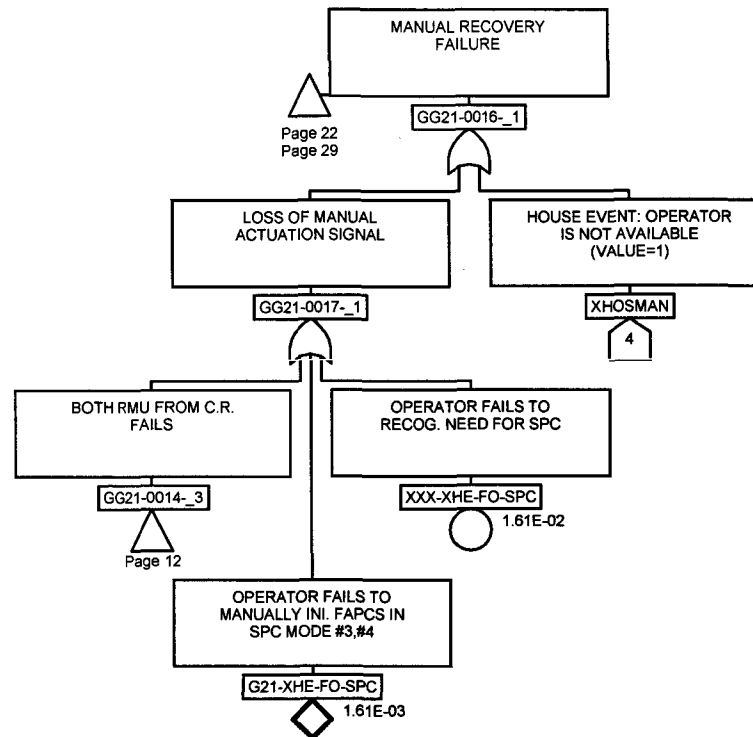


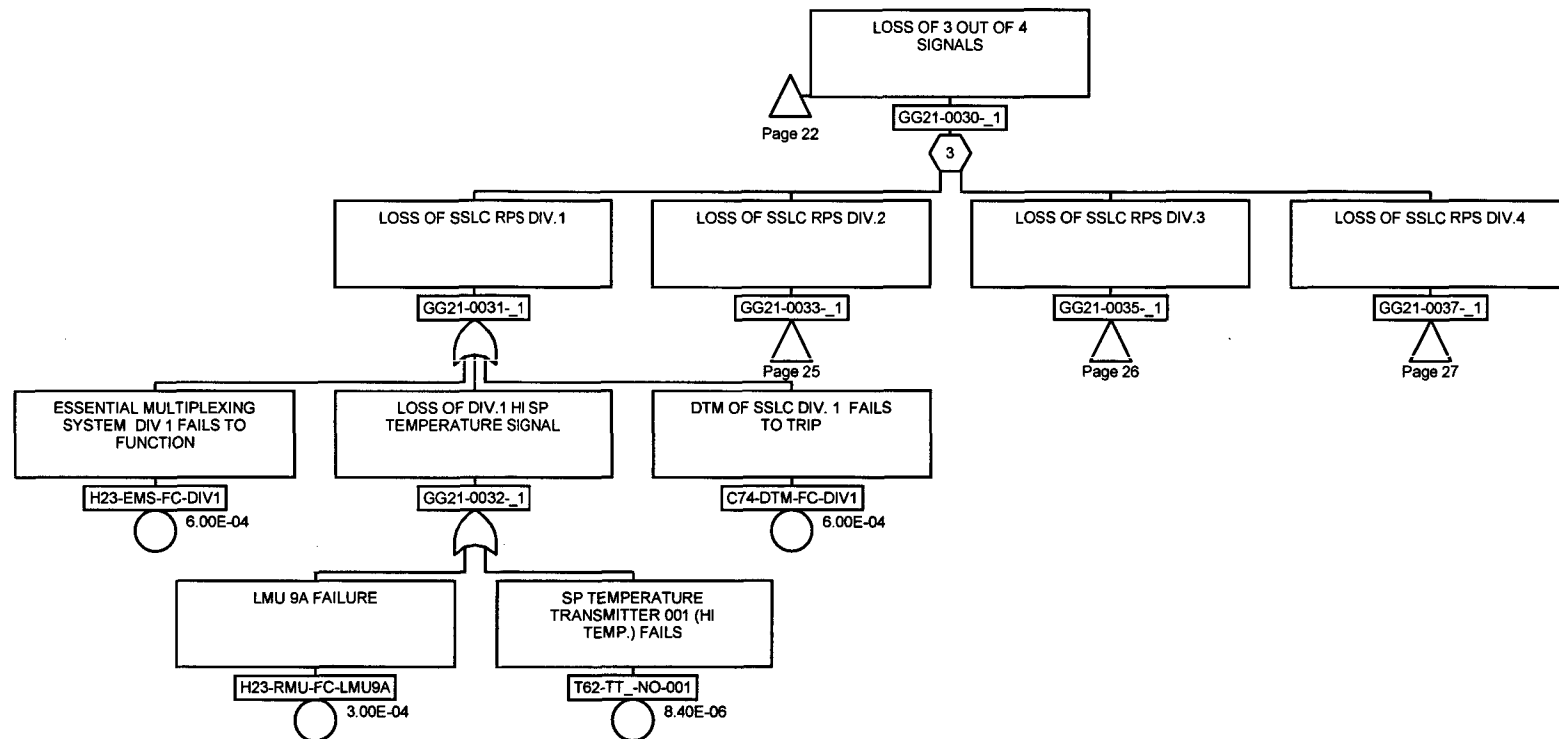


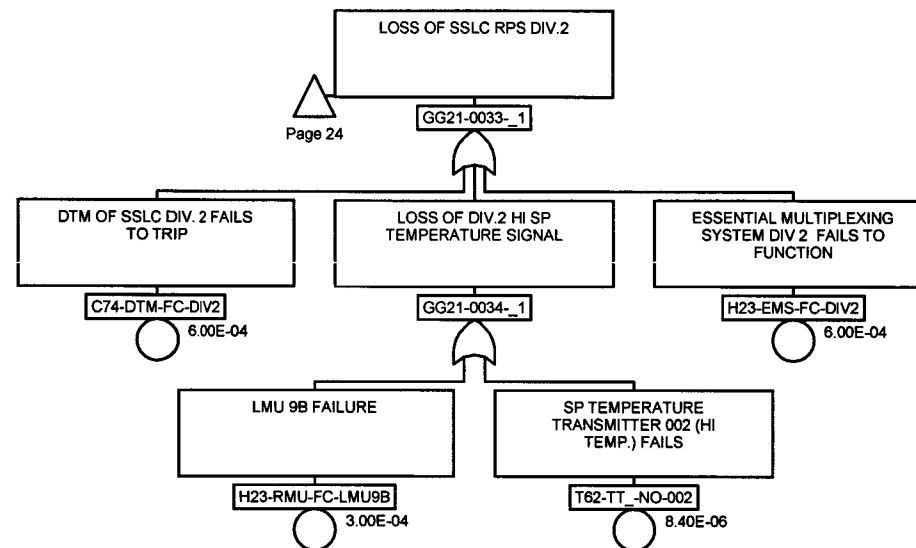


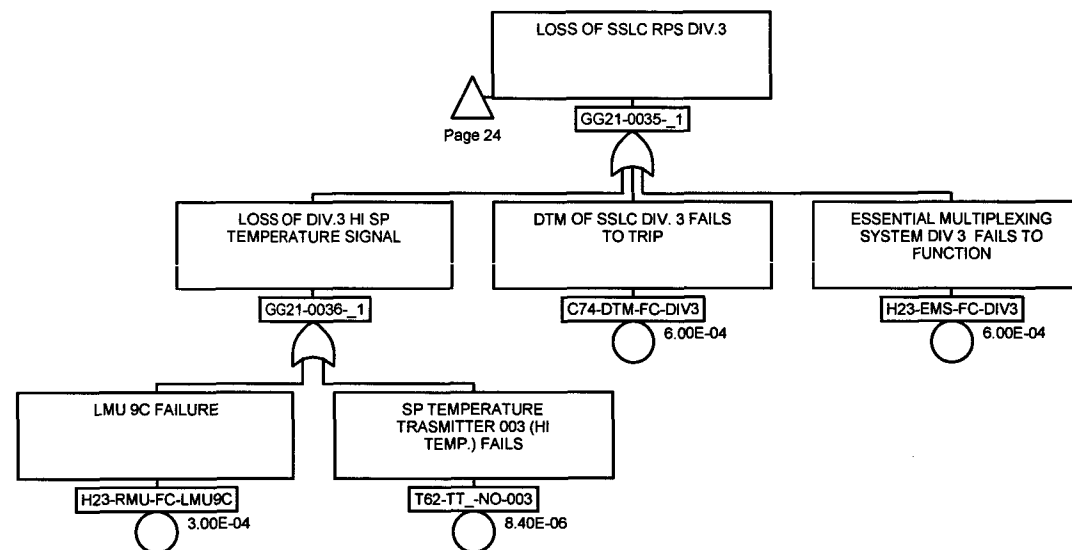


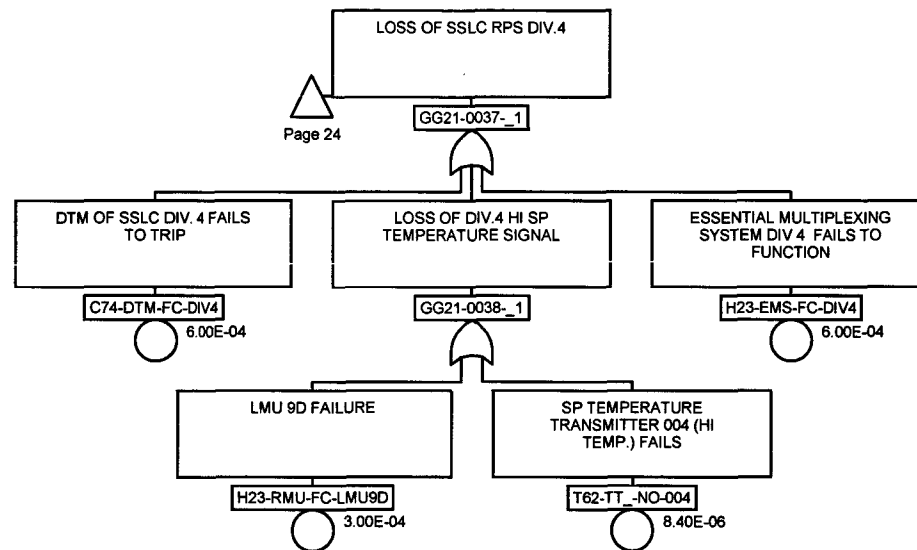


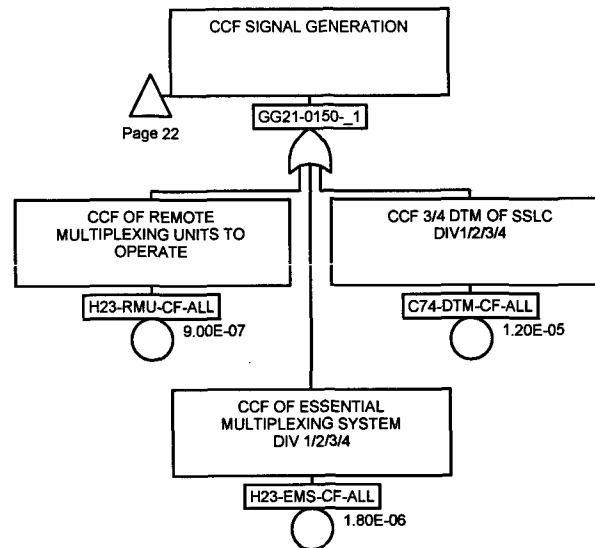




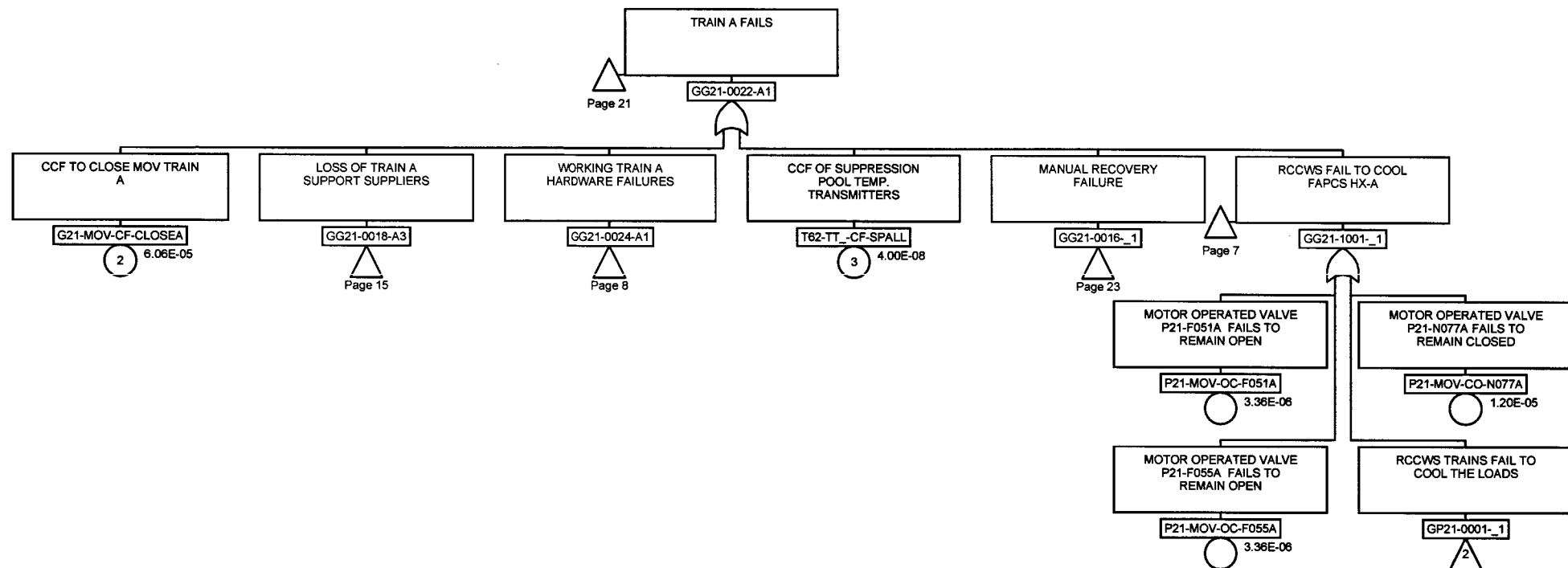








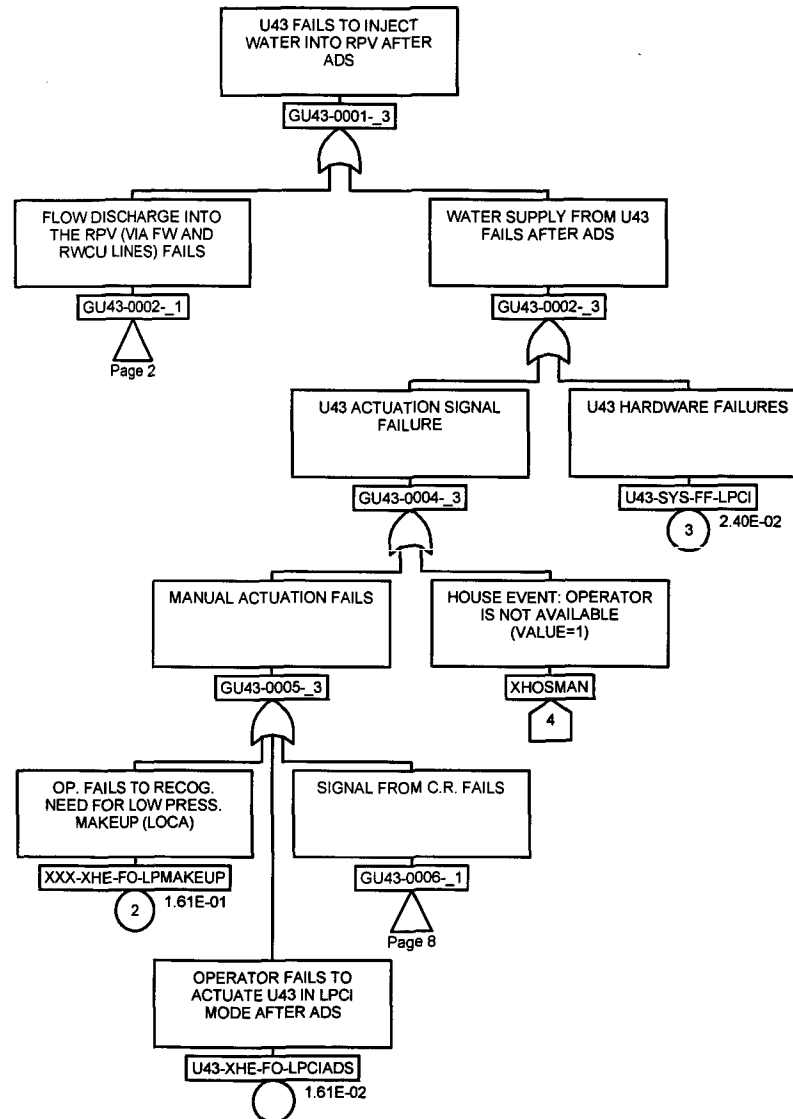
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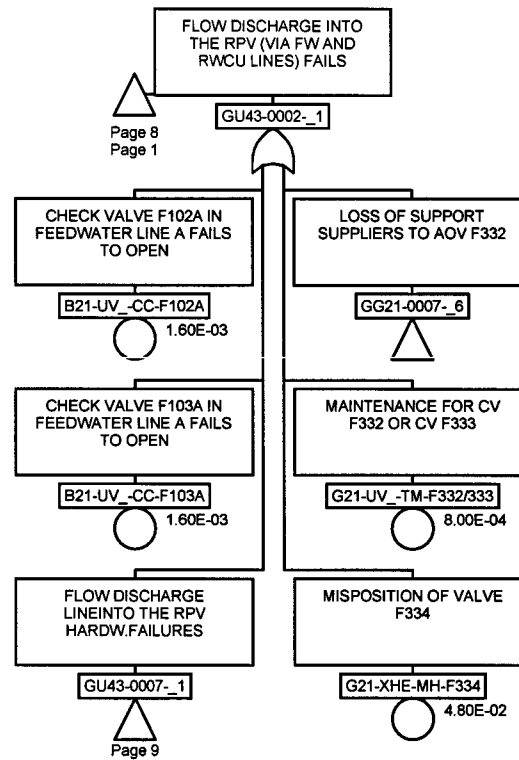


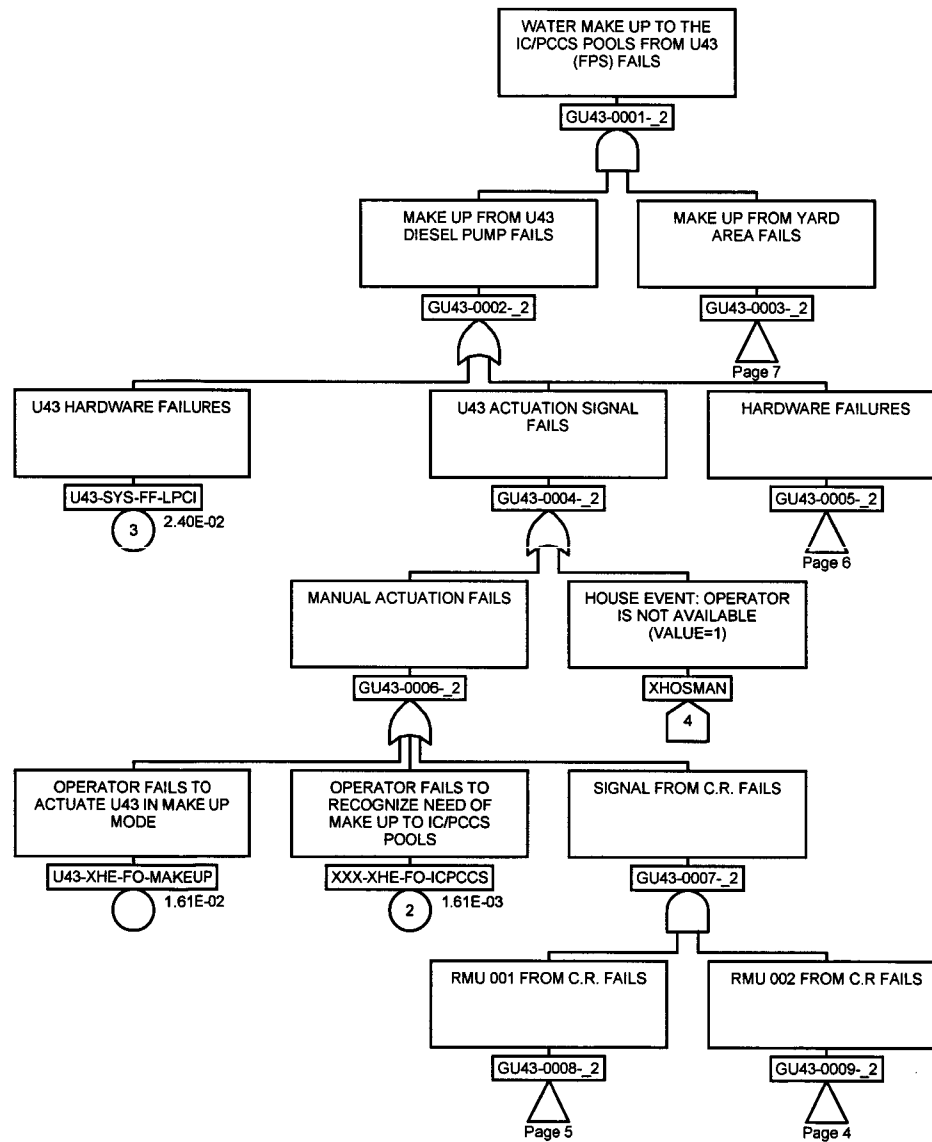
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| G21-ACV-TM-F321/F322 | 13 | 4 | GC62-0001- 2 | 22 | 3 | |
| G21-FT -CF-N014A/BLOW | 10 | 2 | GG21-0001- 1 | 21 | 3 | |
| G21-FT -LO-N014A | 8 | 2 | GG21-0001- 3 | 11 | 1 | |
| G21-FT -LO-N014B | 5 | 2 | GG21-0001- 3 | 13 | 5 | |
| G21-HX -CF-PLUG/LEAK | 10 | 1 | GG21-0001- 4 | 13 | 7 | |
| G21-HX -LK-B001A | 8 | 2 | GG21-0001- 5 | 11 | 2 | |
| G21-HX -LK-B001B | 5 | 2 | GG21-0001- 6 | 1 | 2 | |
| G21-MCB-CF-C001A/BCLOSE | 20 | 3 | GG21-0002- 1 | 21 | 3 | |
| G21-MCB-CO-C001A | 8 | 2 | GG21-0002- 3 | 13 | 5 | |
| G21-MCB-OO-C001A | 16 | 1 | GG21-0002- 5 | 11 | 2 | |
| G21-MCB-OO-C001B | 5 | 1 | GG21-0002- 5 | 12 | 4 | |
| G21-MOV-CC-F011A | 8 | 1 | GG21-0002- 6 | 1 | 2 | |
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| G21-MOV-CC-F306 | 9 | 1 | GG21-0005- 1 | 13 | 2 | |
| G21-MOV-CF-CLOSEA | 7 | 1 | GG21-0005- 3 | 13 | 3 | |
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| G21-MOV-CF-OPENA/B | 10 | 1 | GG21-0006- 6 | 13 | 6 | |
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| G21-MOV-TM-F306 | 9 | 3 | GG21-0007- 1 | 9 | 3 | |
| G21-MP -CR-C001A/B | 10 | 1 | GG21-0007- 1 | 21 | 4 | |
| G21-MP -CS-C001A/B | 20 | 2 | GG21-0007- 3 | 17 | 1 | |
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| G21-MP -FS-C001B | 5 | 1 | GG21-0008- 1 | 9 | 2 | |
| G21-PT -CF-N002A/BLOW | 10 | 2 | GG21-0009- 1 | 21 | 3 | |
| G21-PT -LO-N002A | 8 | 2 | GG21-0009- 3 | 13 | 5 | |
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| G21-SYS-TM-TRAINB | 2 | 5 | GG21-0010-B1 | 21 | 3 | |
| G21-SYS-TM-TRAINB | 13 | 6 | GG21-0010-B3 | 13 | 5 | |
| G21-SYS-TM-TRAINB | 21 | 5 | GG21-0010-B6 | 2 | 3 | |
| G21-UV -CC-F004B | 5 | 2 | GG21-0011-B1 | 21 | 2 | |
| G21-UV -CC-F307 | 9 | 2 | GG21-0011-B3 | 2 | 1 | |
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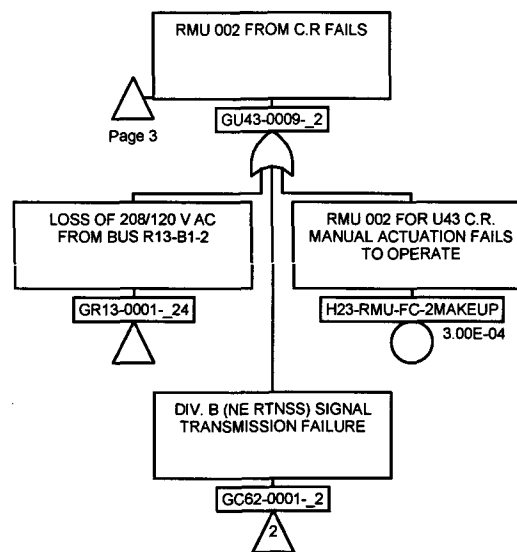
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| GG21-0013-B6 | 2 | 2 | GG21-0100- 3 | 13 | 8 | |
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| GG21-0014- 3 | 4 | 1 | GG21-0103- 3 | 20 | 2 | |
| GG21-0014- 3 | 12 | 3 | GG21-0104- 3 | 8 | 2 | |
| GG21-0014- 3 | 13 | 7 | GG21-0104- 3 | 16 | 2 | |
| GG21-0014- 3 | 23 | 1 | GG21-0105- 3 | 16 | 2 | |
| GG21-0015- 1 | 22 | 2 | GG21-0150- 1 | 22 | 2 | |
| GG21-0015- 3 | 12 | 2 | GG21-0150- 1 | 28 | 2 | |
| GG21-0016- 1 | 22 | 1 | GG21-1001- 1 | 7 | 2 | |
| GG21-0016- 1 | 23 | 2 | GG21-1001- 1 | 29 | 6 | |
| GG21-0016- 1 | 29 | 5 | GG21-1001- 2 | 2 | 4 | |
| GG21-0016- 3 | 12 | 4 | GG21-1001- 2 | 6 | 2 | |
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| GG21-0016- 6 | 7 | 2 | GP21-0001- 1 | 6 | 2 | |
| GG21-0017-B3 | 13 | 6 | GP21-0001- 1 | 29 | 7 | |
| GG21-0017-B3 | 15 | 2 | GR11-0001- 1 | 15 | 1 | |
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| GG21-0018-A3 | 7 | 1 | GR13-0001- 11 | 14 | 1 | |
| GG21-0018-A3 | 15 | 2 | GR13-0001- 11 | 19 | 1 | |
| GG21-0018-A3 | 29 | 2 | GR13-0001- 12 | 9 | 4 | |
| GG21-0021-B1 | 2 | 3 | GR13-0001- 13 | 14 | 2 | |
| GG21-0021-B1 | 5 | 2 | H23-EMS-CF-ALL | 28 | 2 | |
| GG21-0021-B1 | 13 | 5 | H23-EMS-FC-DIV1 | 24 | 1 | |
| GG21-0021-B1 | 21 | 3 | H23-EMS-FC-DIV2 | 25 | 3 | |
| GG21-0022-A1 | 21 | 4 | H23-EMS-FC-DIV3 | 26 | 4 | |
| GG21-0022-A1 | 29 | 4 | H23-EMS-FC-DIV4 | 27 | 3 | |
| GG21-0022-A6 | 2 | 4 | H23-RMU-CF-ALL | 28 | 1 | |
| GG21-0022-A6 | 7 | 2 | H23-RMU-FC-1G21 | 12 | 2 | |
| GG21-0022- 3 | 1 | 3 | H23-RMU-FC-2G21 | 12 | 4 | |
| GG21-0022- 3 | 10 | 2 | H23-RMU-FC-LMU9A | 24 | 2 | |
| GG21-0022- 3 | 13 | 6 | H23-RMU-FC-LMU9B | 25 | 2 | |
| GG21-0022- 3 | 21 | 4 | H23-RMU-FC-LMU9C | 26 | 1 | |
| GG21-0024-A1 | 7 | 1 | H23-RMU-FC-LMU9D | 27 | 2 | |
| GG21-0024-A1 | 8 | 2 | P21-MOV-CC-F051B | 6 | 1 | |
| GG21-0024-A1 | 15 | 3 | P21-MOV-CC-F055B | 6 | 1 | |
| GG21-0024-A1 | 29 | 3 | P21-MOV-CO-N077A | 29 | 7 | |
| GG21-0027- 1 | 22 | 2 | P21-MOV-CO-N077B | 6 | 2 | |
| GG21-0030- 1 | 22 | 1 | P21-MOV-OC-F051A | 29 | 6 | |
| GG21-0030- 1 | 24 | 4 | P21-MOV-OC-F055A | 29 | 6 | |
| GG21-0031- 1 | 24 | 2 | T23-POL-RP-SP | 13 | 4 | |
| GG21-0032- 1 | 24 | 2 | T62-TT -CF-SPALL | 7 | 2 | |
| GG21-0033- 1 | 24 | 3 | T62-TT -CF-SPALL | 22 | 3 | |
| GG21-0033- 1 | 25 | 2 | T62-TT -CF-SPALL | 29 | 4 | |
| GG21-0034- 1 | 25 | 2 | T62-TT -NO-001 | 24 | 3 | |
| GG21-0035- 1 | 24 | 4 | T62-TT -NO-002 | 25 | 3 | |
| GG21-0035- 1 | 26 | 3 | T62-TT -NO-003 | 26 | 2 | |
| GG21-0036- 1 | 26 | 2 | T62-TT -NO-004 | 27 | 3 | |
| GG21-0037- 1 | 24 | 5 | XHOSLOSP | 16 | 3 | |
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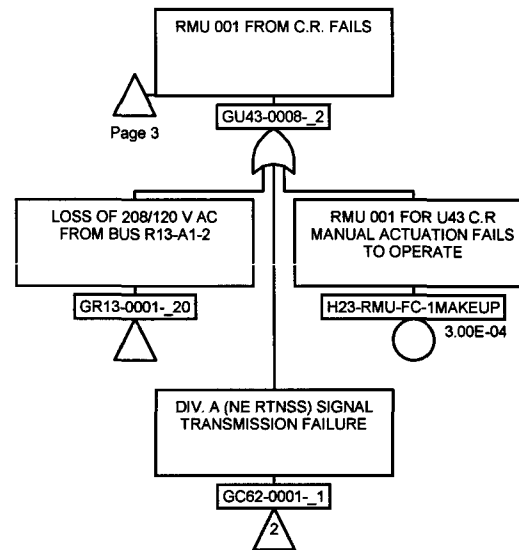
Appendix B.4.7-2
Fire Protection Protection System U43
Fault Tree

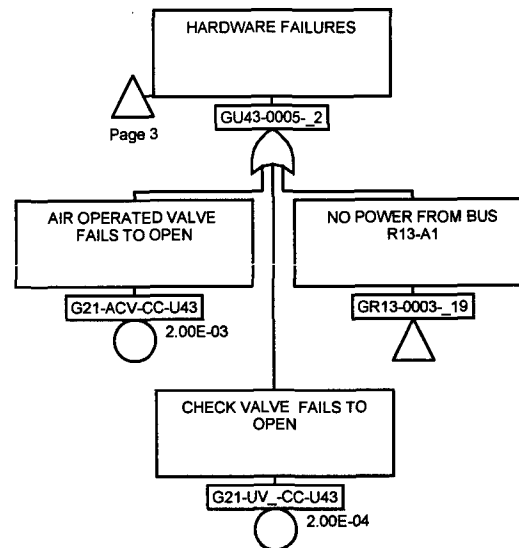


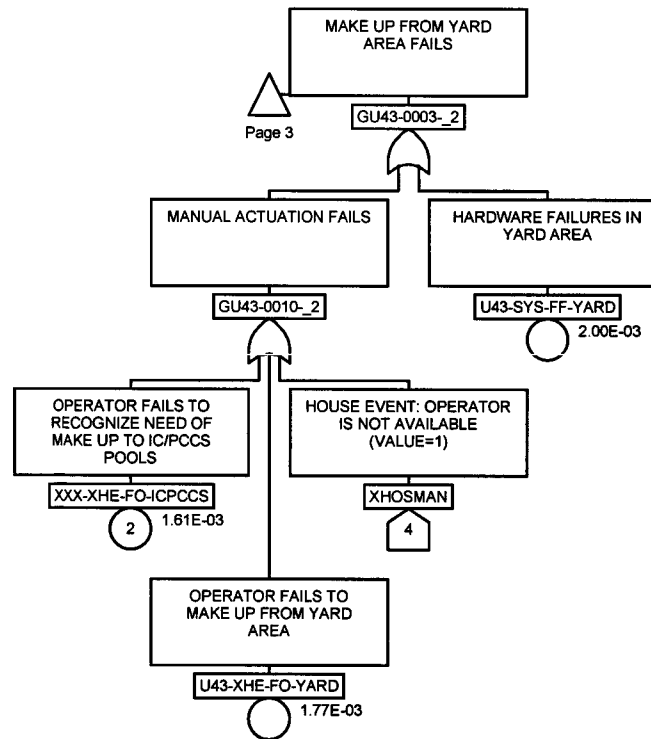




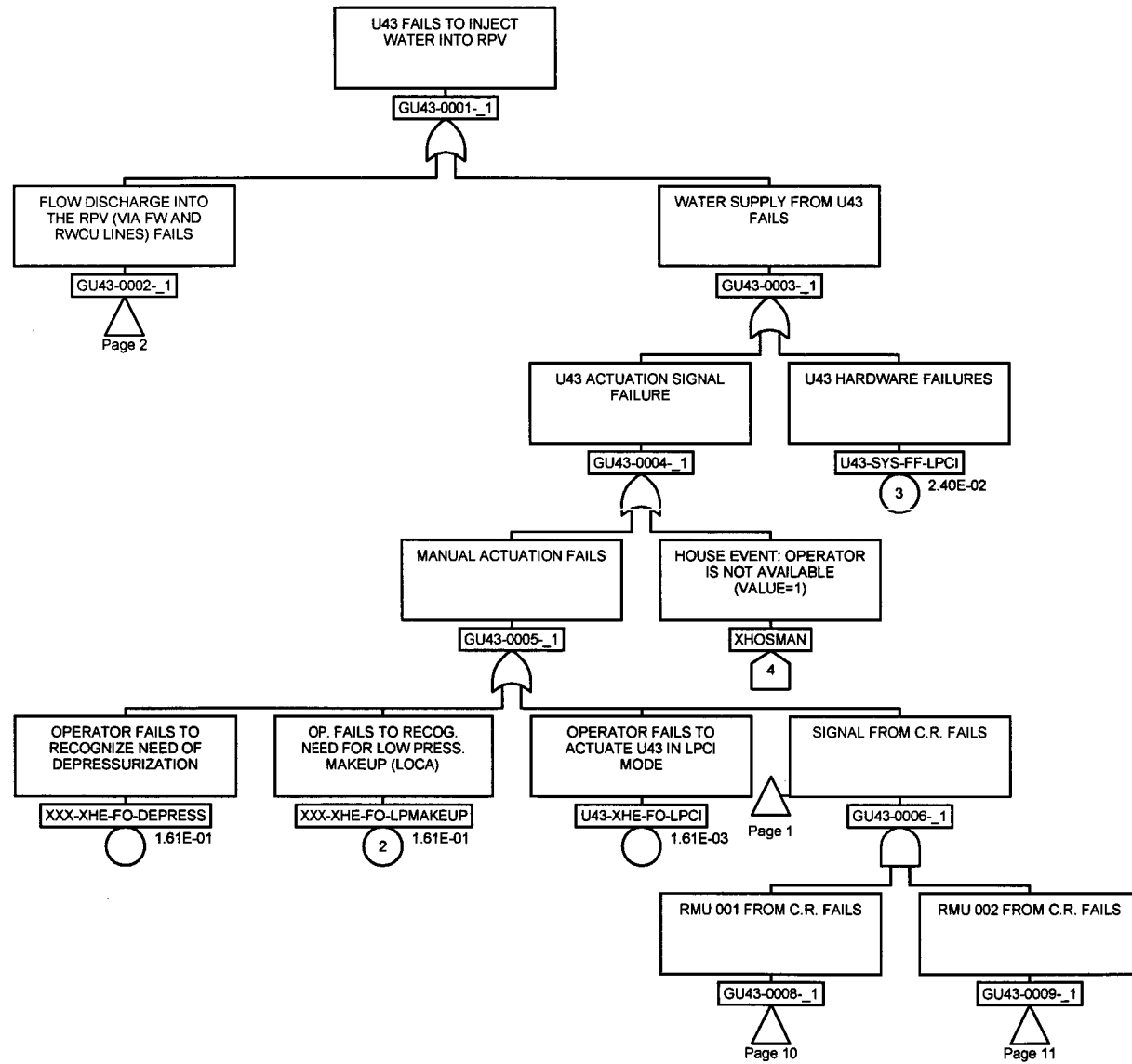


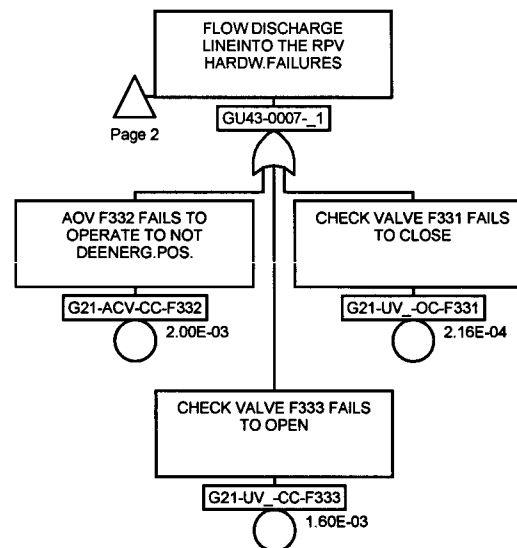


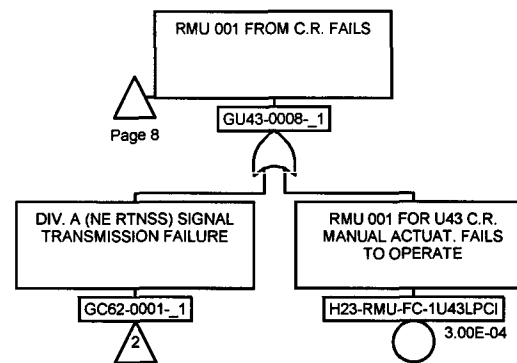


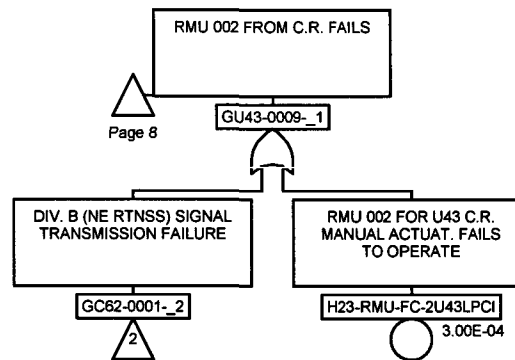


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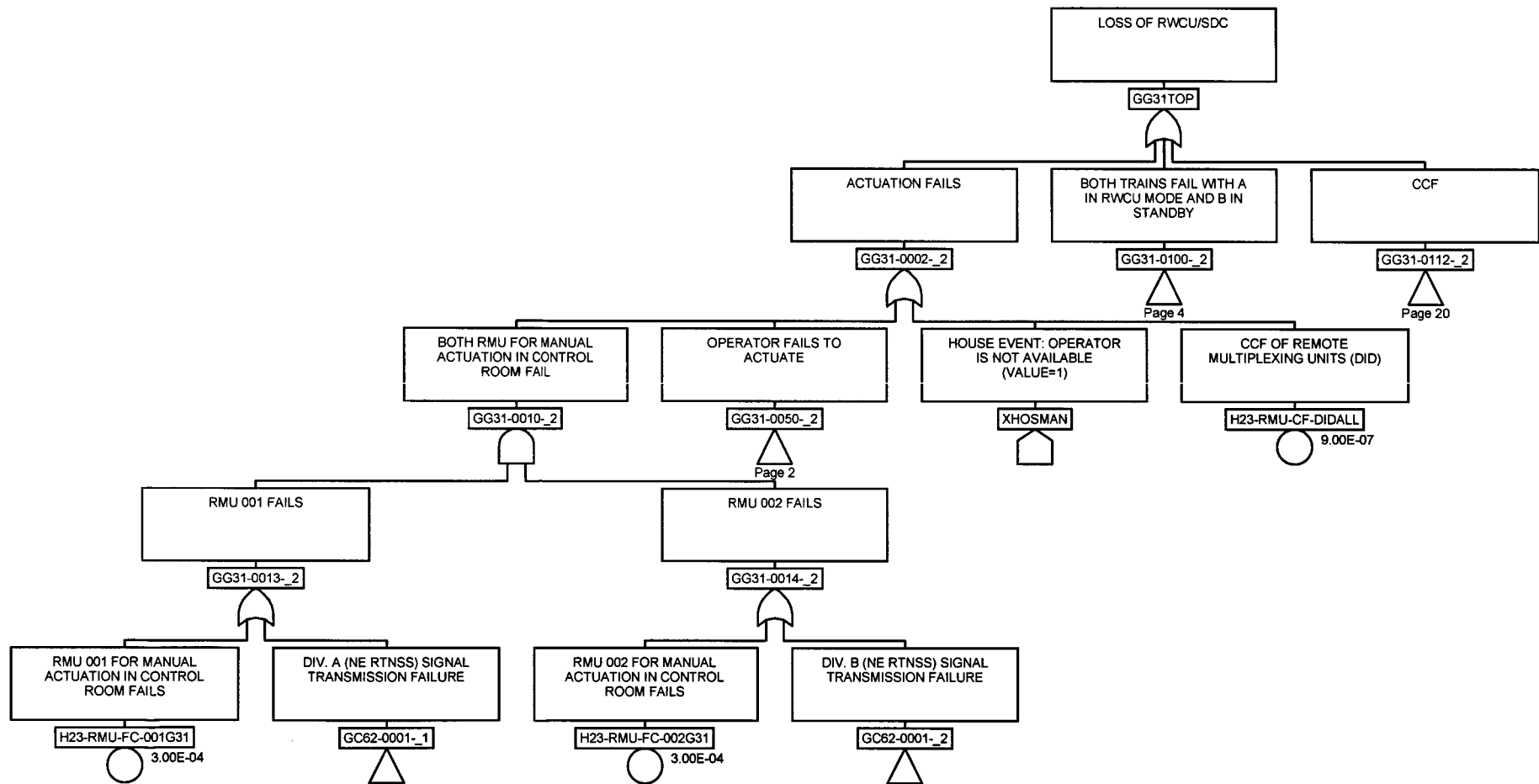


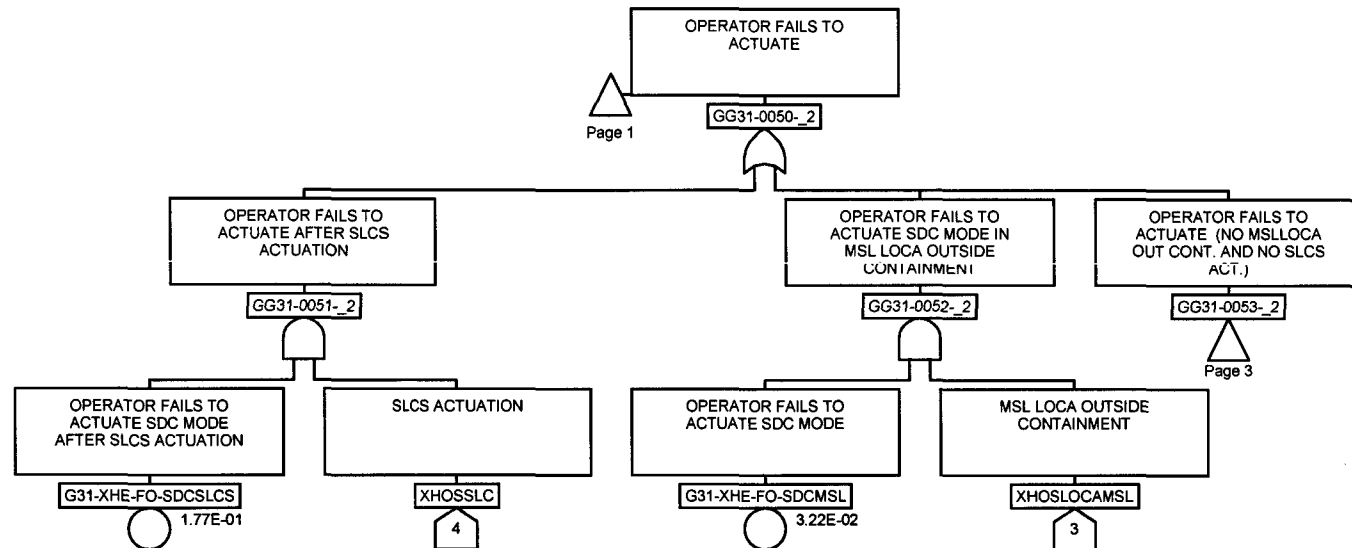


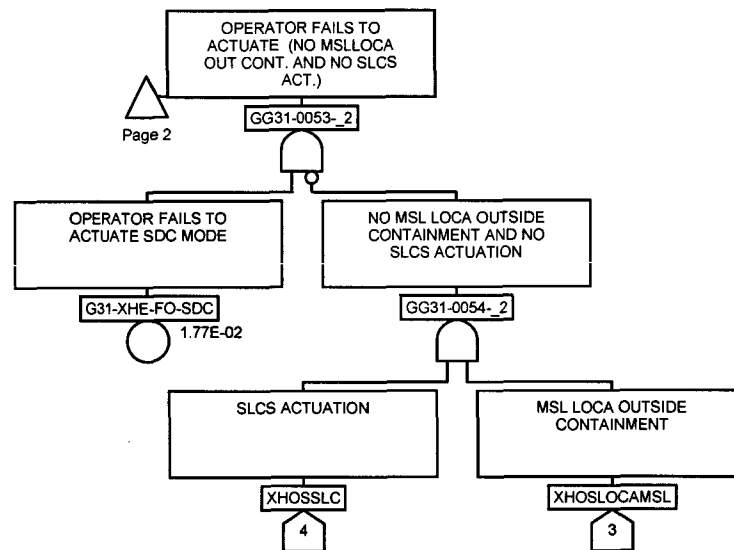


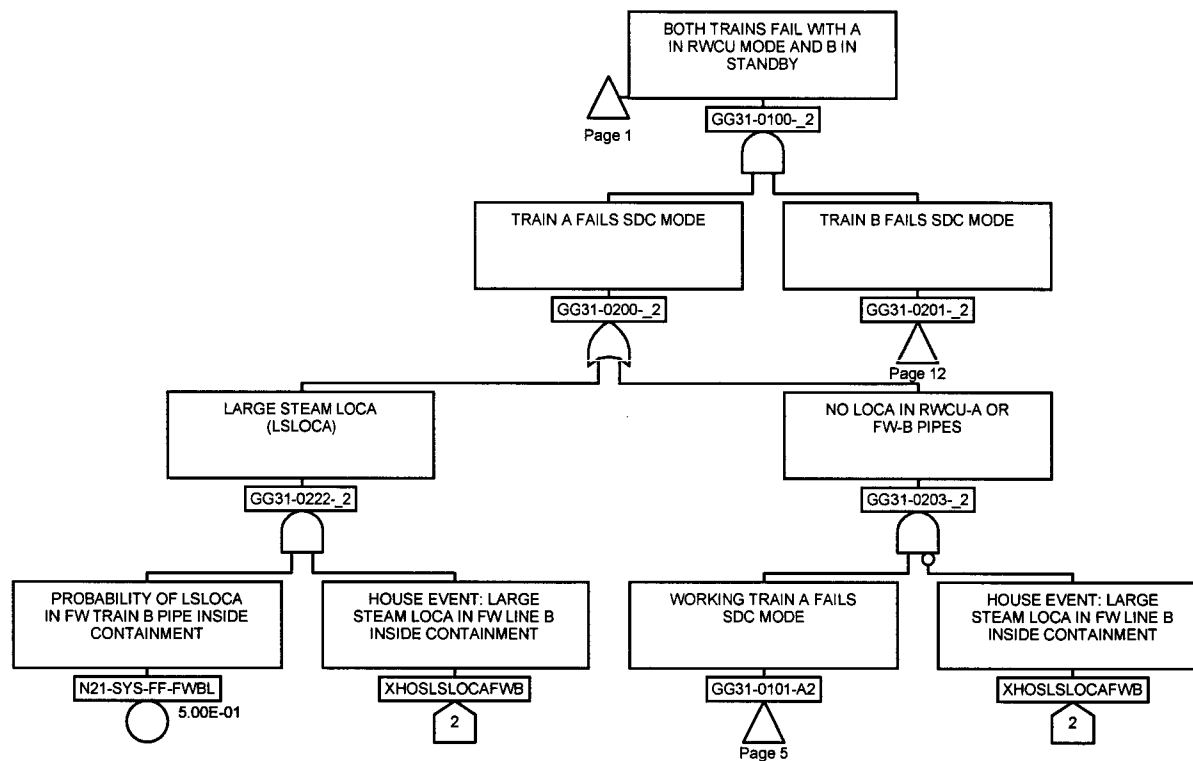
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| B21-UV -CC-F103A | 2 | 1 | U43-SYS-FF-LPCI | 3 | 1 | |
| G21-ACV-CC-F332 | 9 | 1 | U43-SYS-FF-LPCI | 8 | 4 | |
| G21-ACV-CC-U43 | 6 | 1 | U43-SYS-FF-YARD | 7 | 3 | |
| G21-UV -CC-F333 | 9 | 2 | U43-XHE-FO-LPCI | 8 | 3 | |
| G21-UV -CC-U43 | 6 | 2 | U43-XHE-FO-LPCIADS | 1 | 2 | |
| G21-UV -OC-F331 | 9 | 2 | U43-XHE-FO-MAKEUP | 3 | 1 | |
| G21-UV -TM-F332/333 | 2 | 2 | U43-XHE-FO-YARD | 7 | 2 | |
| G21-XHE-MH-F334 | 2 | 2 | XHOSMAN | 1 | 3 | |
| GC62-0001- 1 | 5 | 2 | XHOSMAN | 3 | 3 | |
| GC62-0001- 1 | 10 | 1 | XHOSMAN | 7 | 2 | |
| GC62-0001- 2 | 4 | 2 | XHOSMAN | 8 | 4 | |
| GC62-0001- 2 | 11 | 1 | XXX-XHE-FO-DEPRESS | 8 | 1 | |
| GG21-0007- 6 | 2 | 2 | XXX-XHE-FO-ICPCCS | 3 | 2 | |
| GR13-0001- 20 | 5 | 1 | XXX-XHE-FO-ICPCCS | 7 | 1 | |
| GR13-0001- 24 | 4 | 1 | XXX-XHE-FO-LPMAKEUP | 1 | 1 | |
| GR13-0003- 19 | 6 | 2 | XXX-XHE-FO-LPMAKEUP | 8 | 2 | |
| GU43-0001- 1 | 8 | 2 | | | | |
| GU43-0001- 2 | 3 | 3 | | | | |
| GU43-0001- 3 | 1 | 2 | | | | |
| GU43-0002- 1 | 1 | 1 | | | | |
| GU43-0002- 1 | 2 | 2 | | | | |
| GU43-0002- 1 | 8 | 1 | | | | |
| GU43-0002- 2 | 3 | 2 | | | | |
| GU43-0002- 3 | 1 | 2 | | | | |
| GU43-0003- 1 | 8 | 3 | | | | |
| GU43-0003- 2 | 3 | 3 | | | | |
| GU43-0003- 2 | 7 | 2 | | | | |
| GU43-0004- 1 | 8 | 3 | | | | |
| GU43-0004- 2 | 3 | 3 | | | | |
| GU43-0004- 3 | 1 | 2 | | | | |
| GU43-0005- 1 | 8 | 3 | | | | |
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| GU43-0009- 1 | 11 | 2 | | | | |
| GU43-0009- 2 | 3 | 4 | | | | |
| GU43-0009- 2 | 4 | 2 | | | | |
| GU43-0010- 2 | 7 | 2 | | | | |
| H23-RMU-FC-1MAKEUP | 5 | 2 | | | | |
| H23-RMU-FC-1U43LPCI | 10 | 2 | | | | |
| H23-RMU-FC-2MAKEUP | 4 | 2 | | | | |
| H23-RMU-FC-2U43LPCI | 11 | 2 | | | | |
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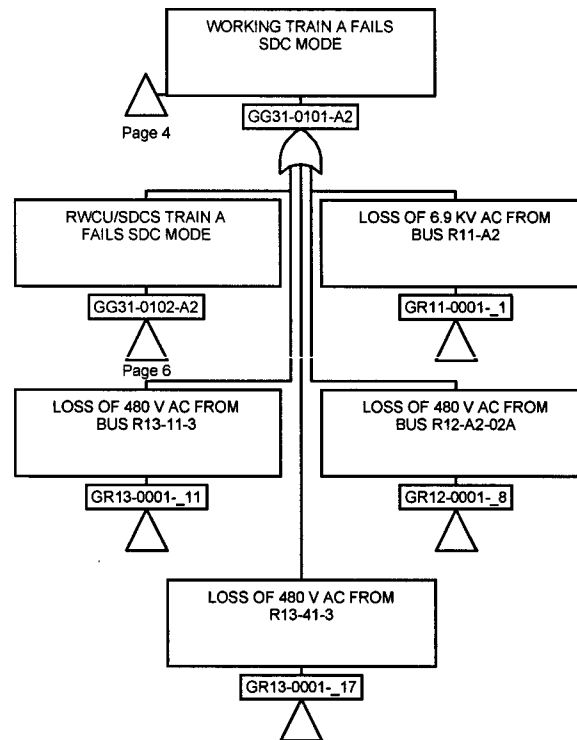
Appendix B.4.8
Reactor Water Cleanup/Shutdown Cooling System
Fault Tree

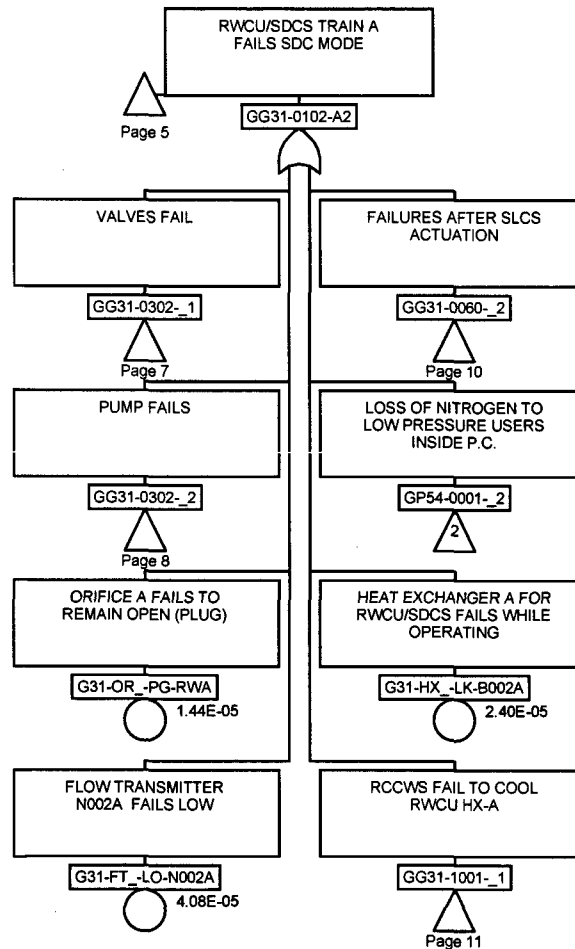




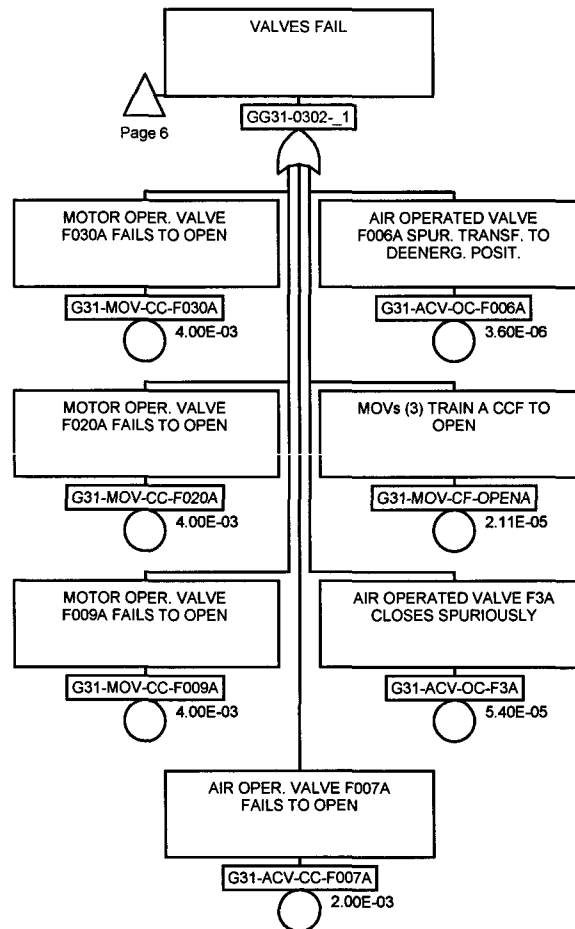


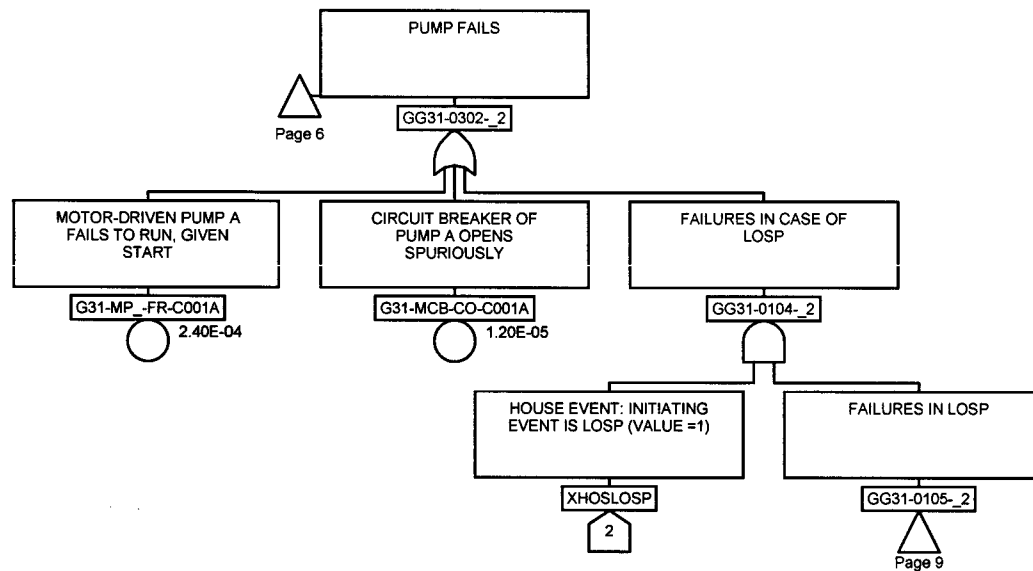


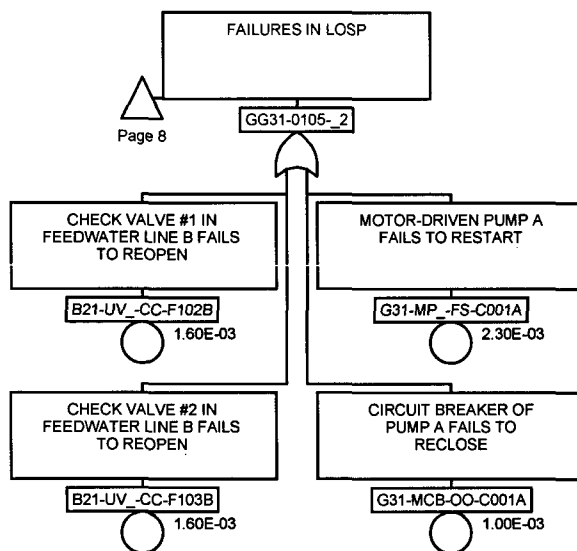


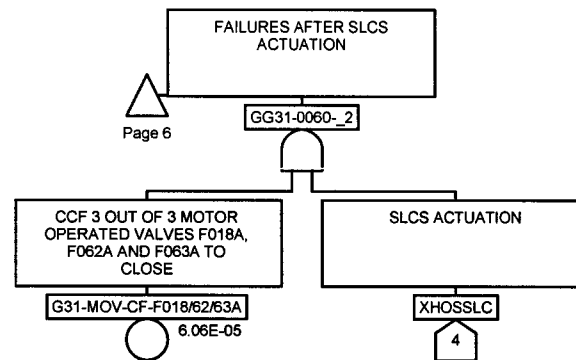


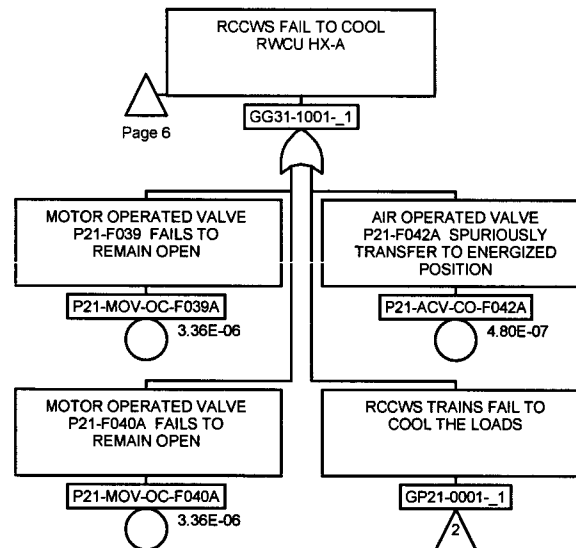
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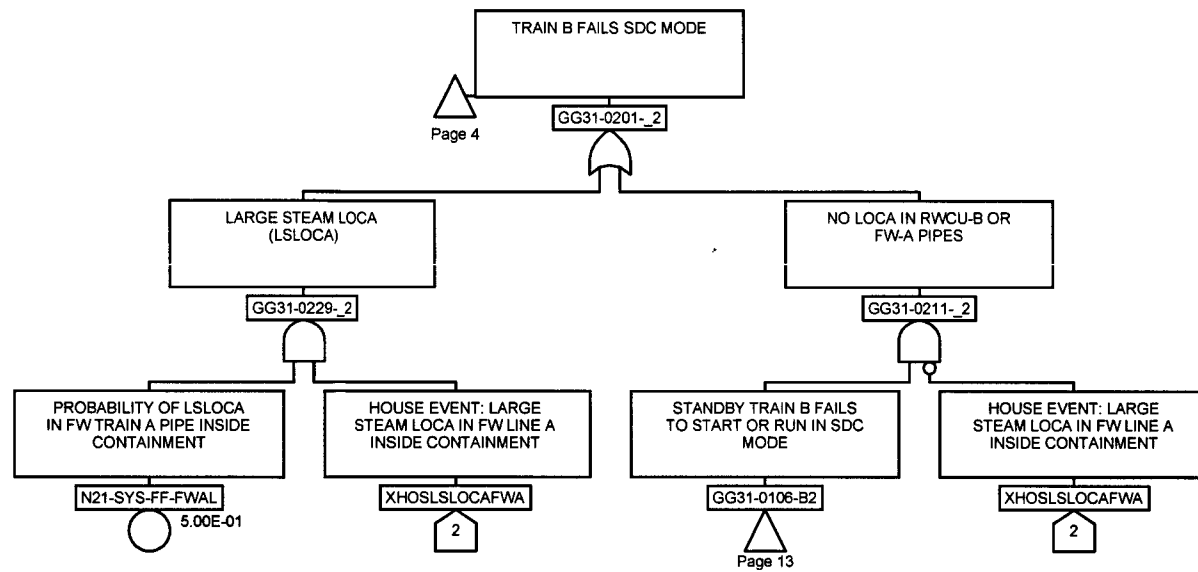


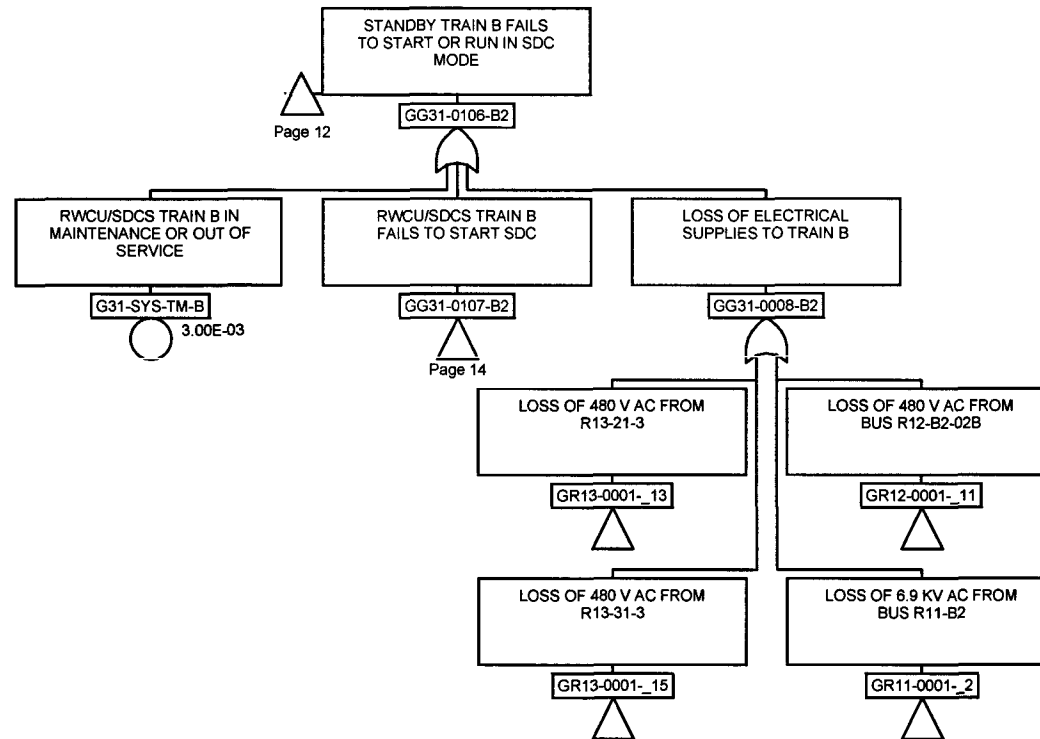




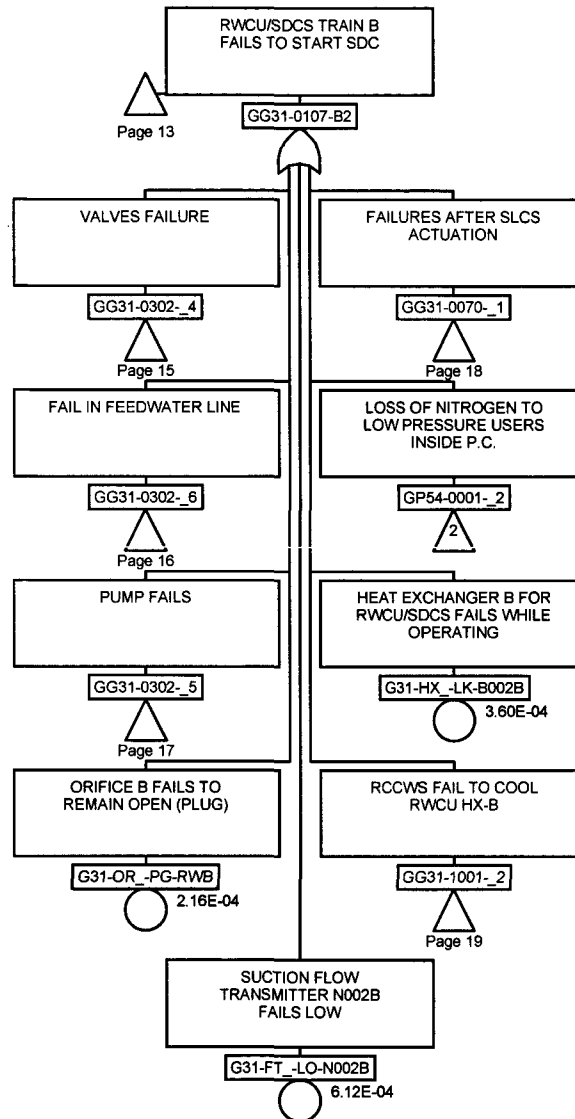


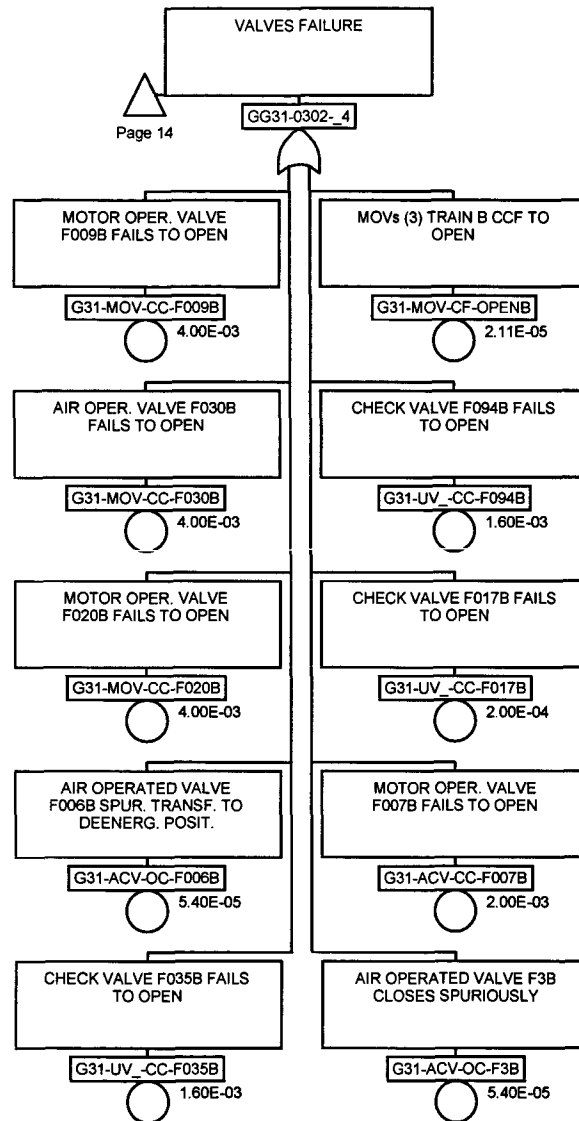


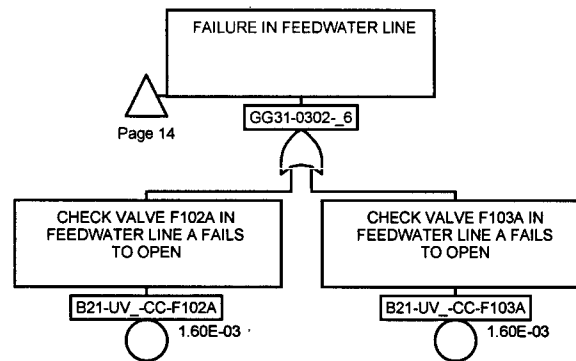


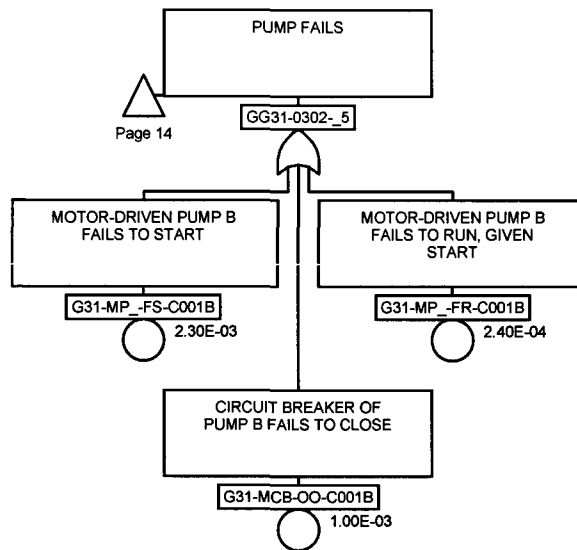


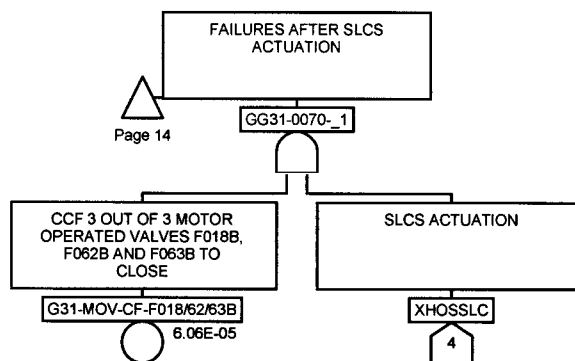
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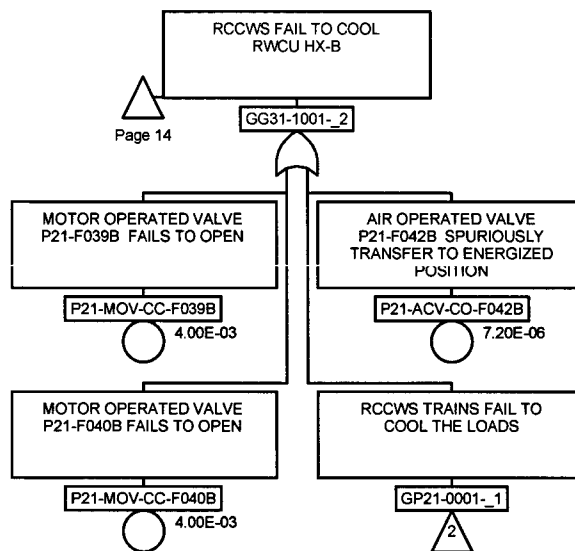


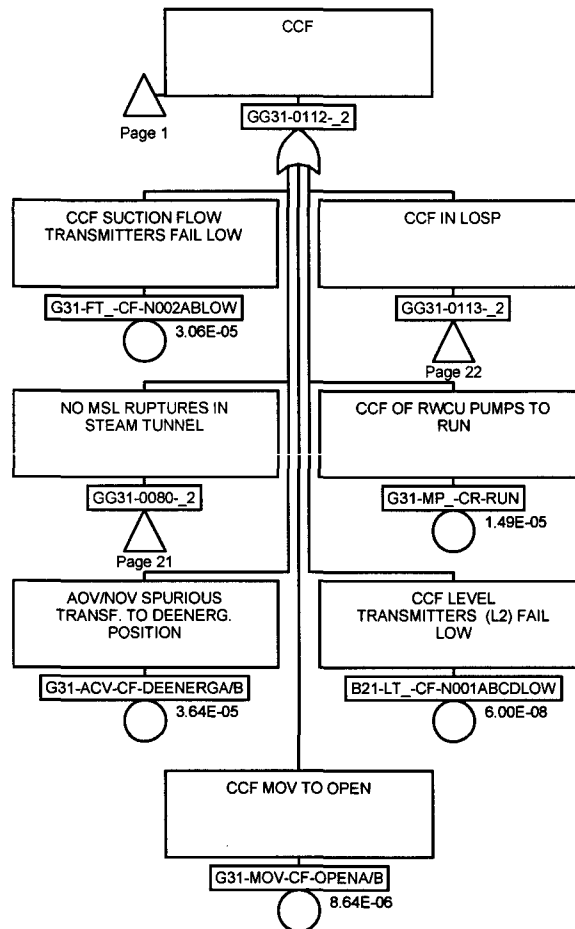


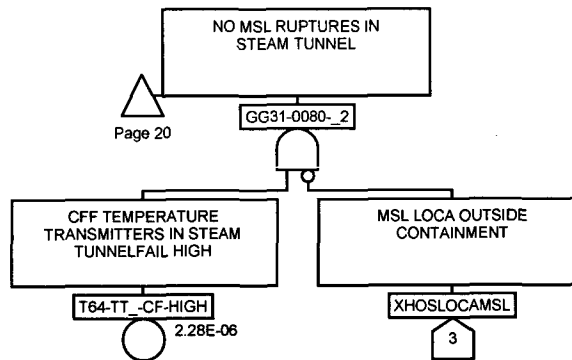


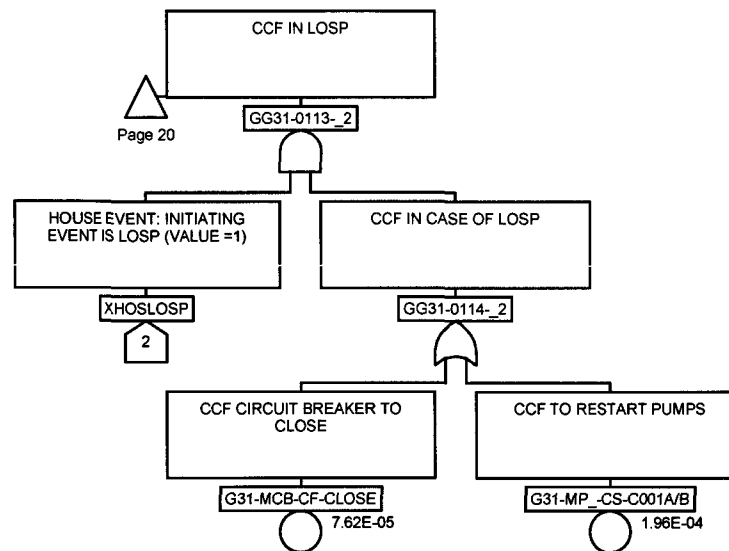








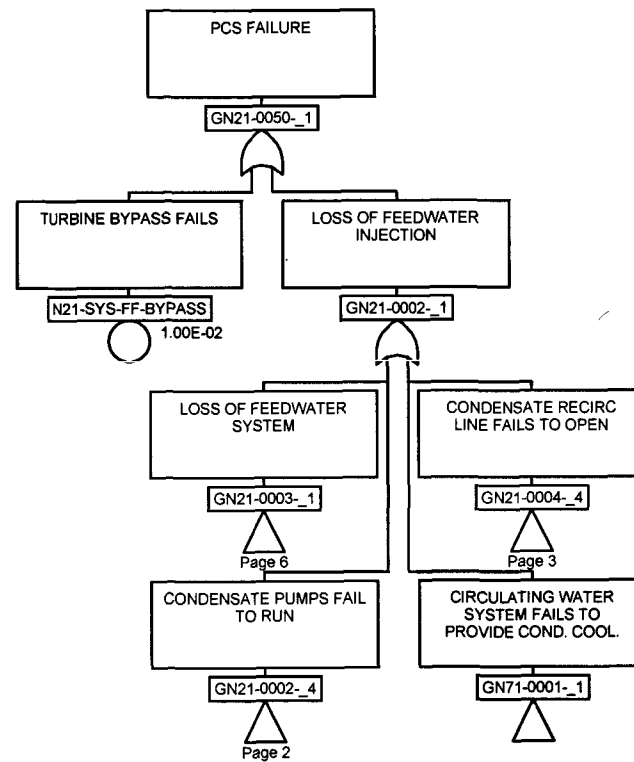




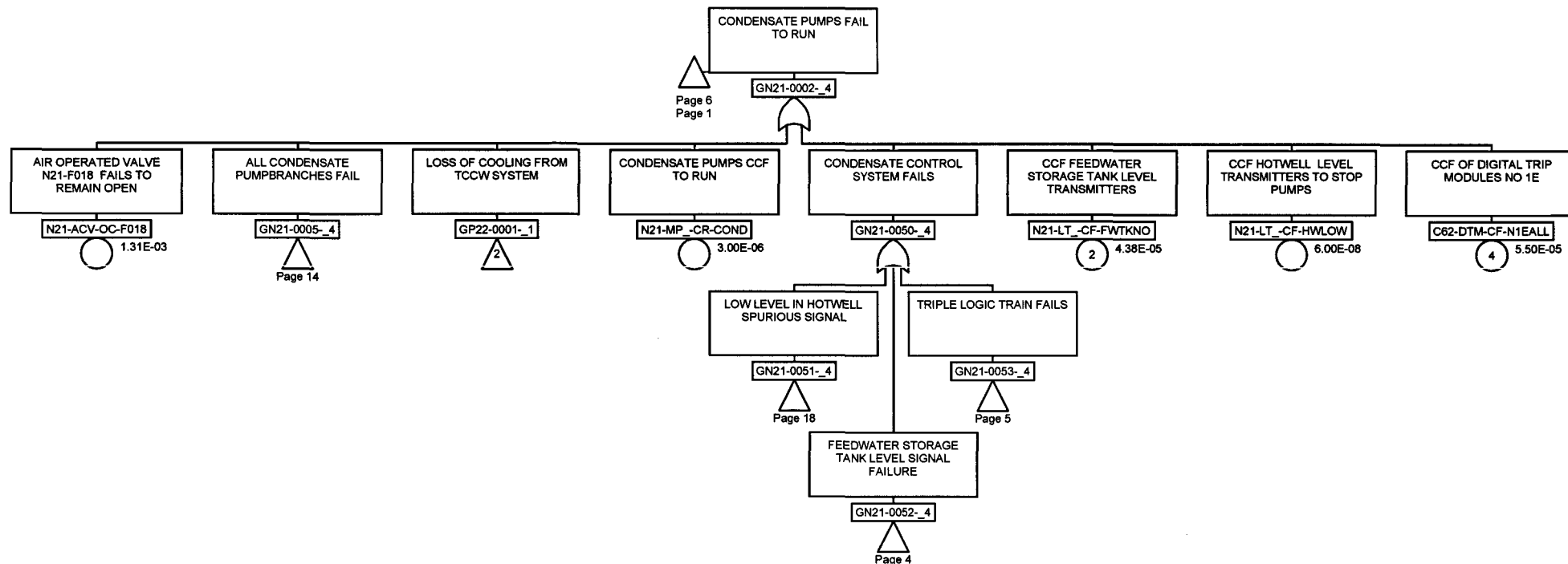
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| G31-ACV-CC-F007A | 7 | 2 | GG31-0053- 2 | | 3 | 2 | GR12-0001- 11 | 13 | 4 |
| G31-ACV-CC-F007B | 15 | 2 | GG31-0054- 2 | | 3 | 2 | GR12-0001- 8 | 5 | 2 |
| G31-ACV-CF-DEENERGA/B | 20 | 1 | GG31-0060- 2 | | 6 | 2 | GR13-0001- 11 | 5 | 1 |
| G31-ACV-OC-F006A | 7 | 2 | GG31-0060- 2 | | 10 | 2 | GR13-0001- 13 | 13 | 3 |
| G31-ACV-OC-F006B | 15 | 1 | GG31-0070- 1 | | 14 | 2 | GR13-0001- 15 | 13 | 3 |
| G31-ACV-OC-F3A | 7 | 2 | GG31-0070- 1 | | 18 | 2 | GR13-0001- 17 | 5 | 2 |
| G31-ACV-OC-F3B | 15 | 2 | GG31-0080- 2 | | 20 | 1 | H23-RMU-CF-DIDALL | 1 | 5 |
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| G31-MOV-CC-F009A | 7 | 1 | GG31-0105- 2 | | 9 | 2 | P21-MOV-OC-F040A | 11 | 1 |
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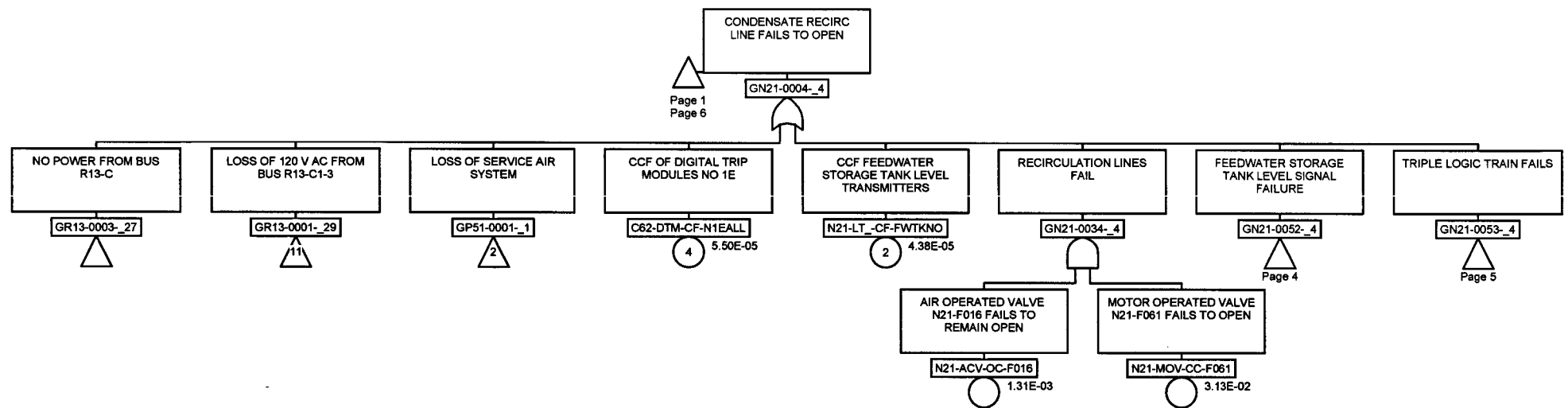
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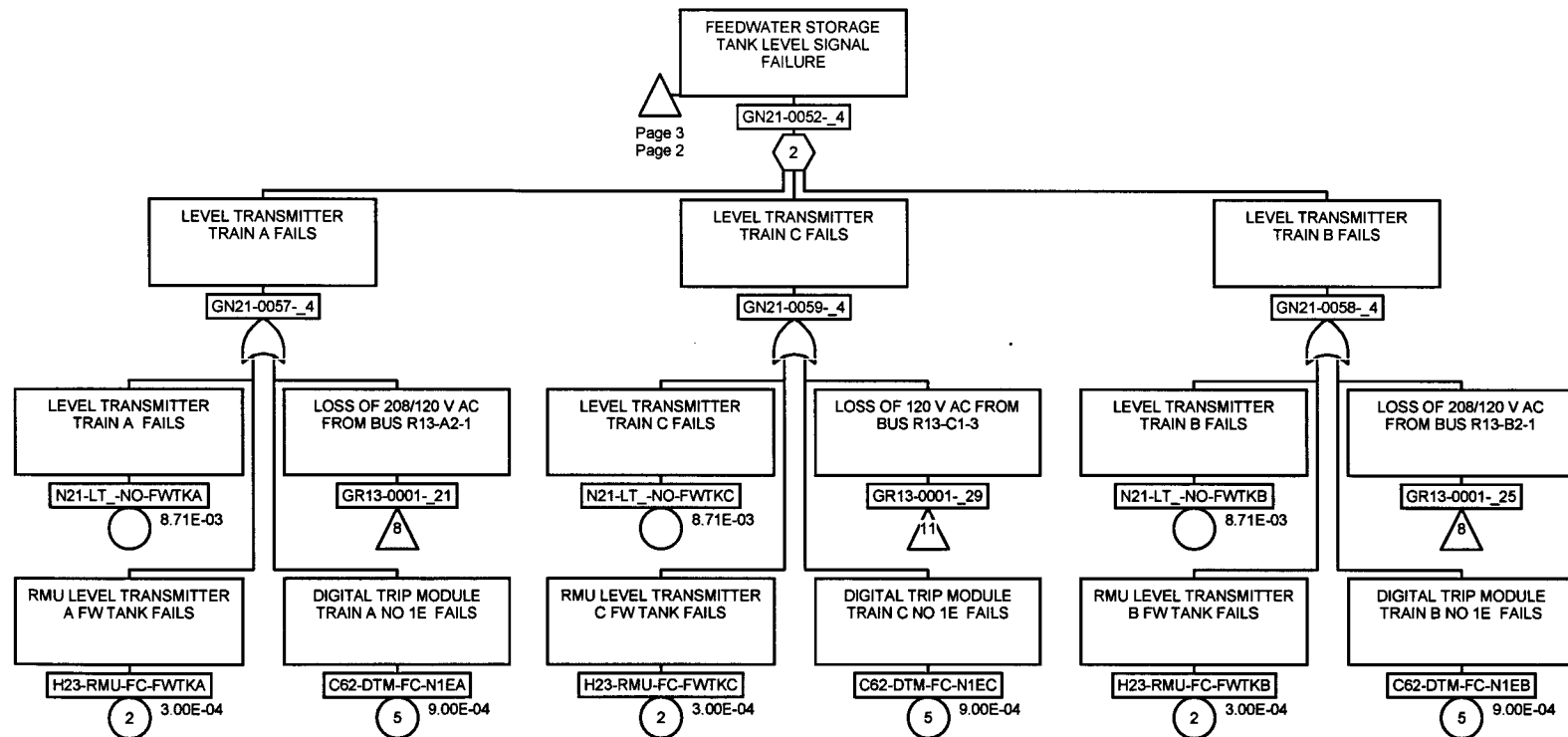


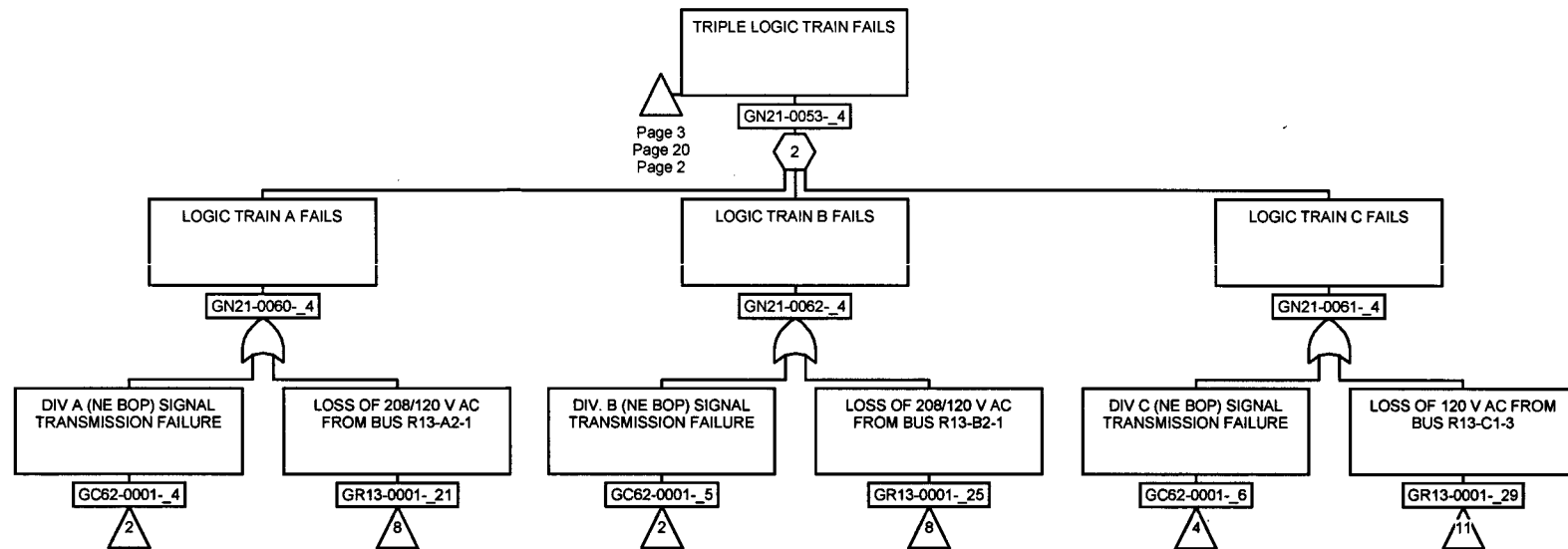
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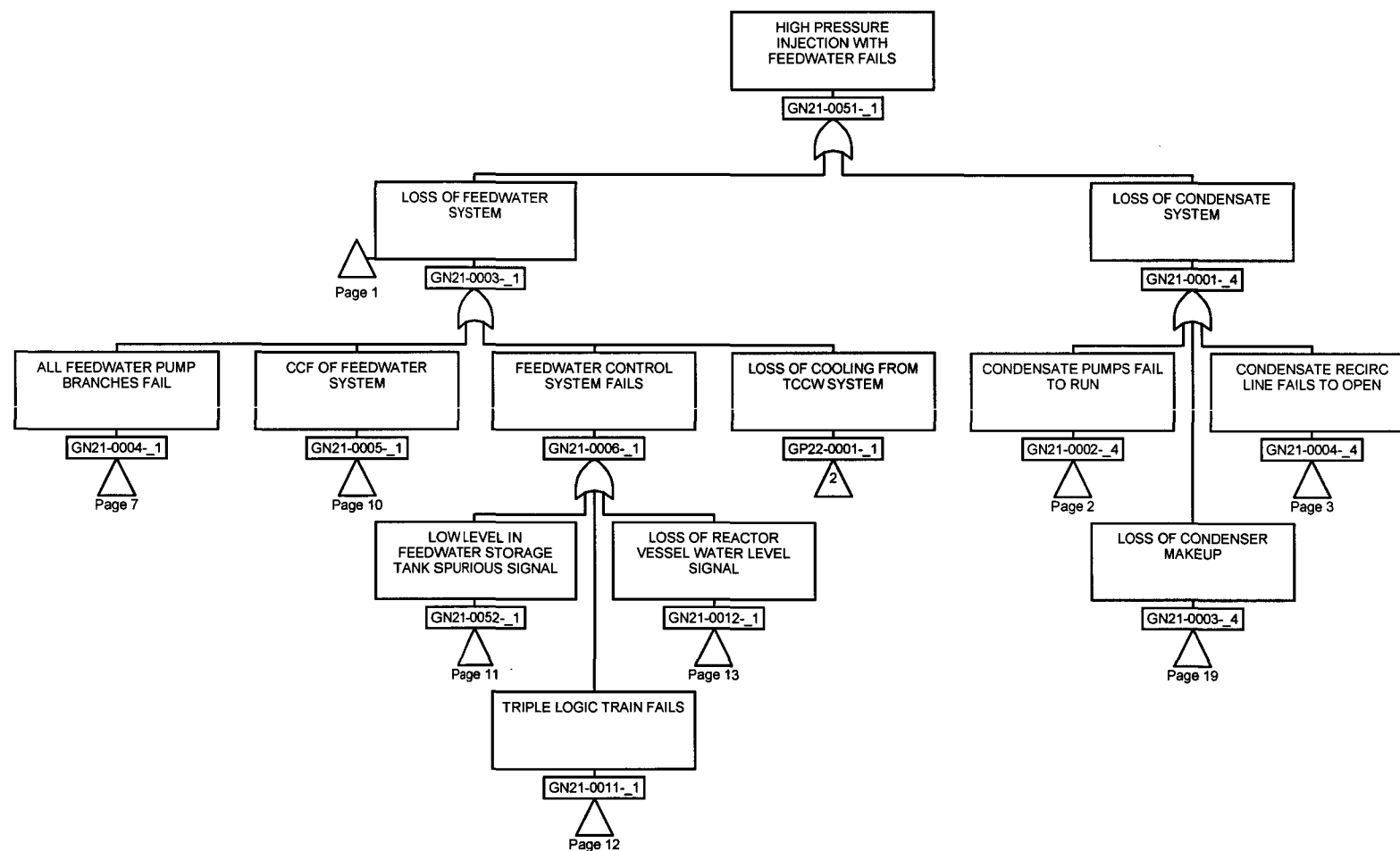


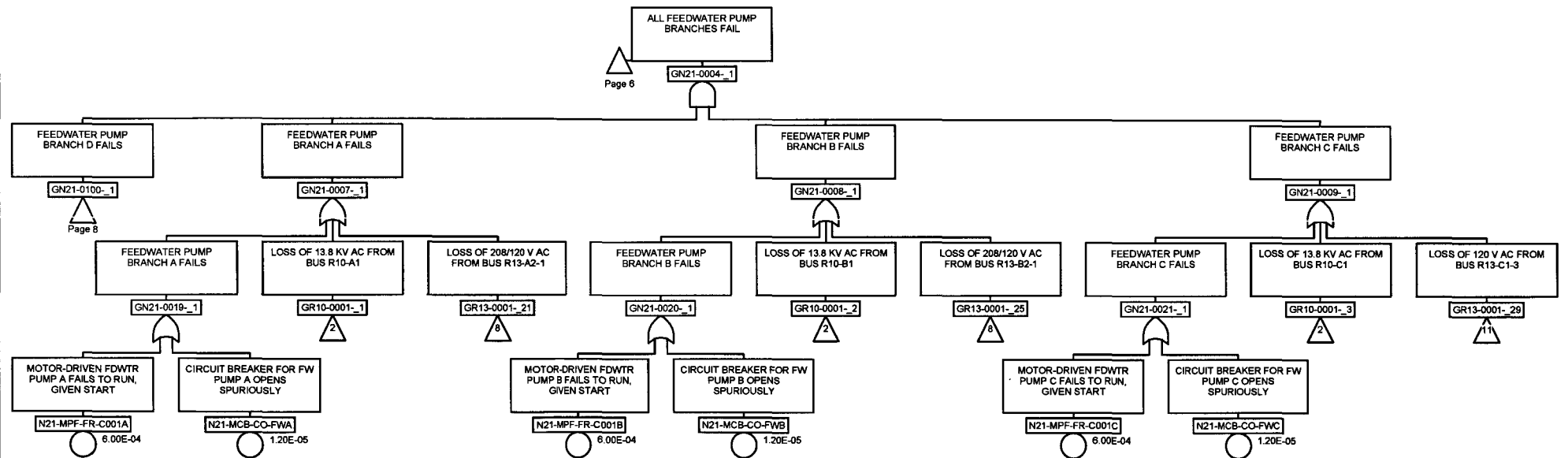
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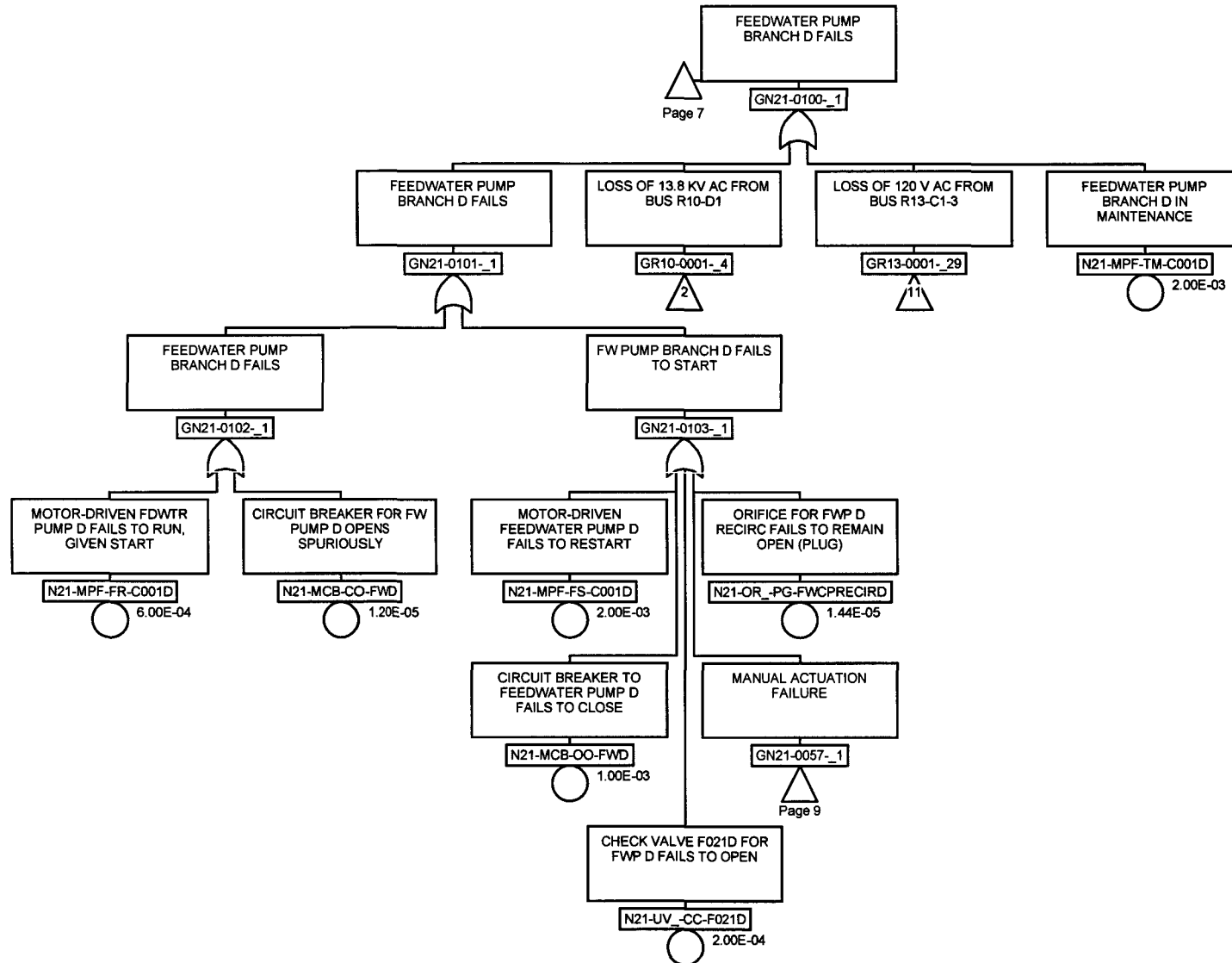


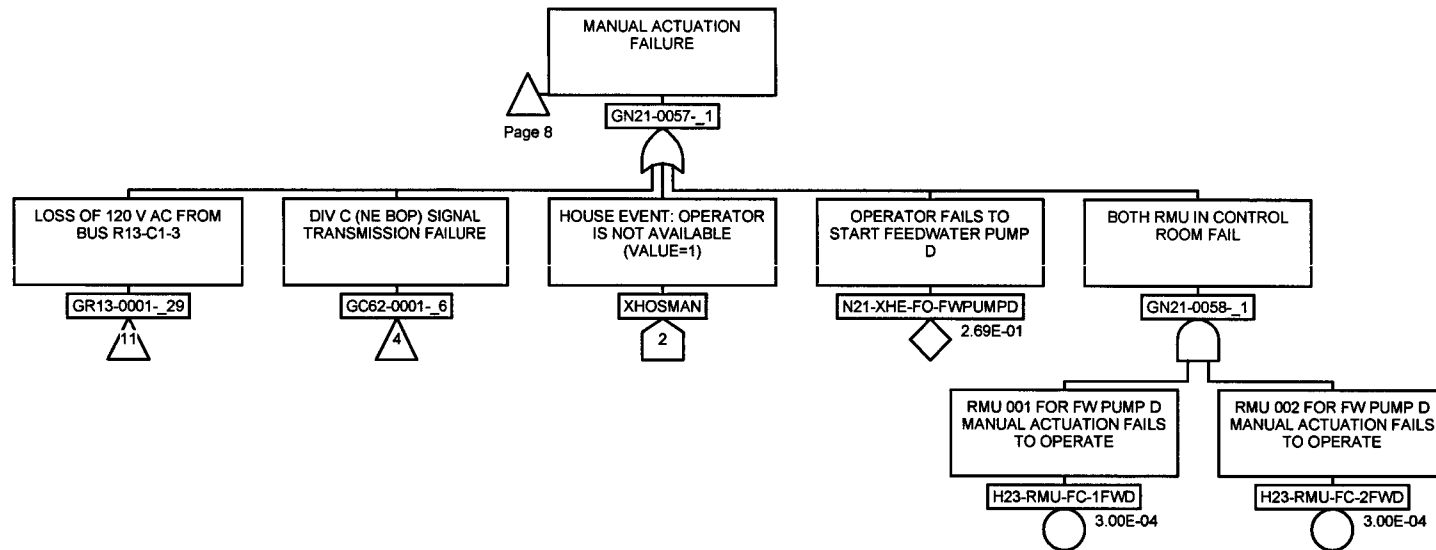


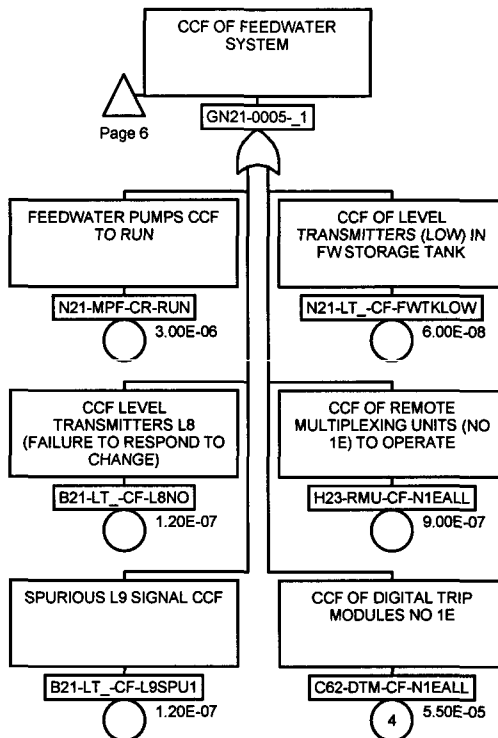


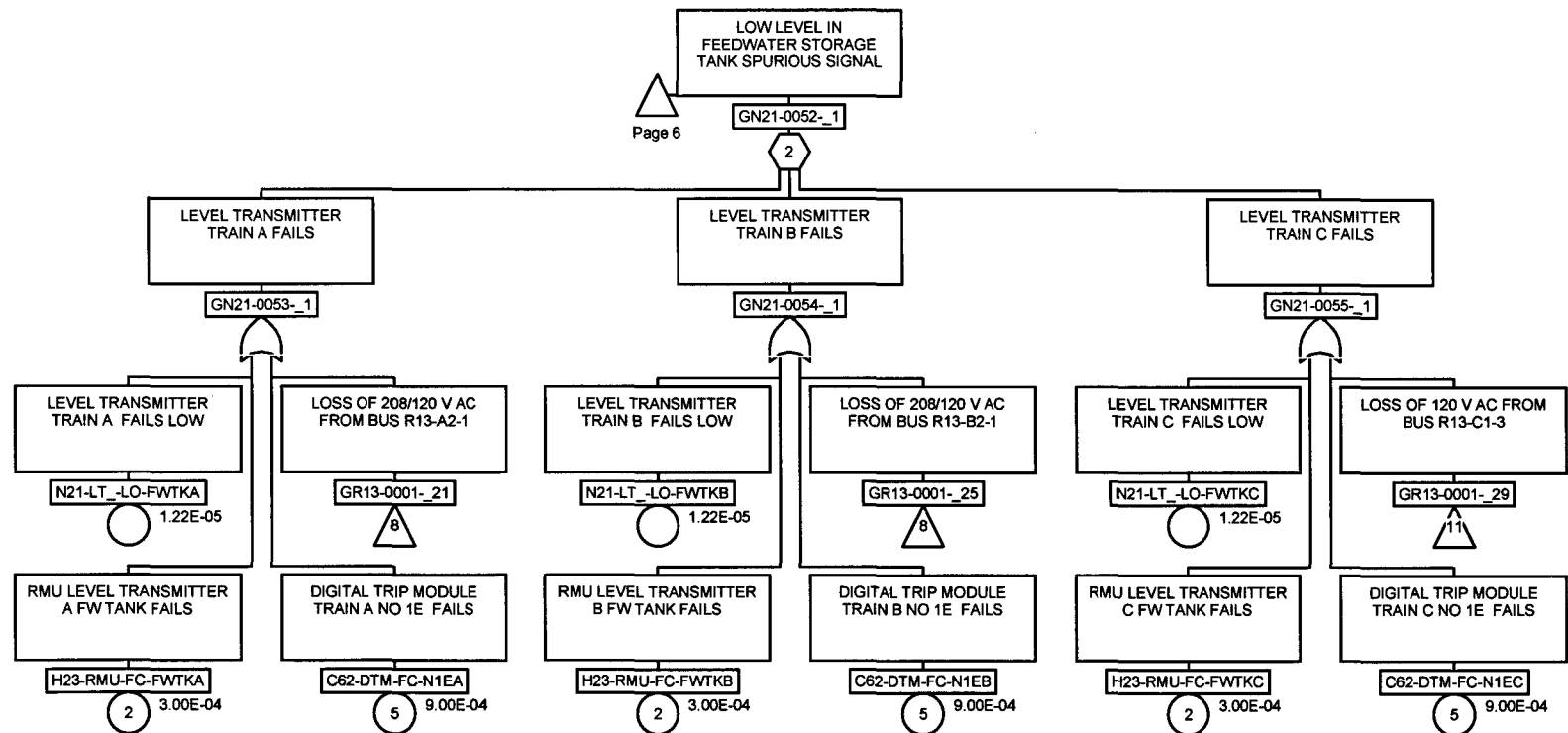


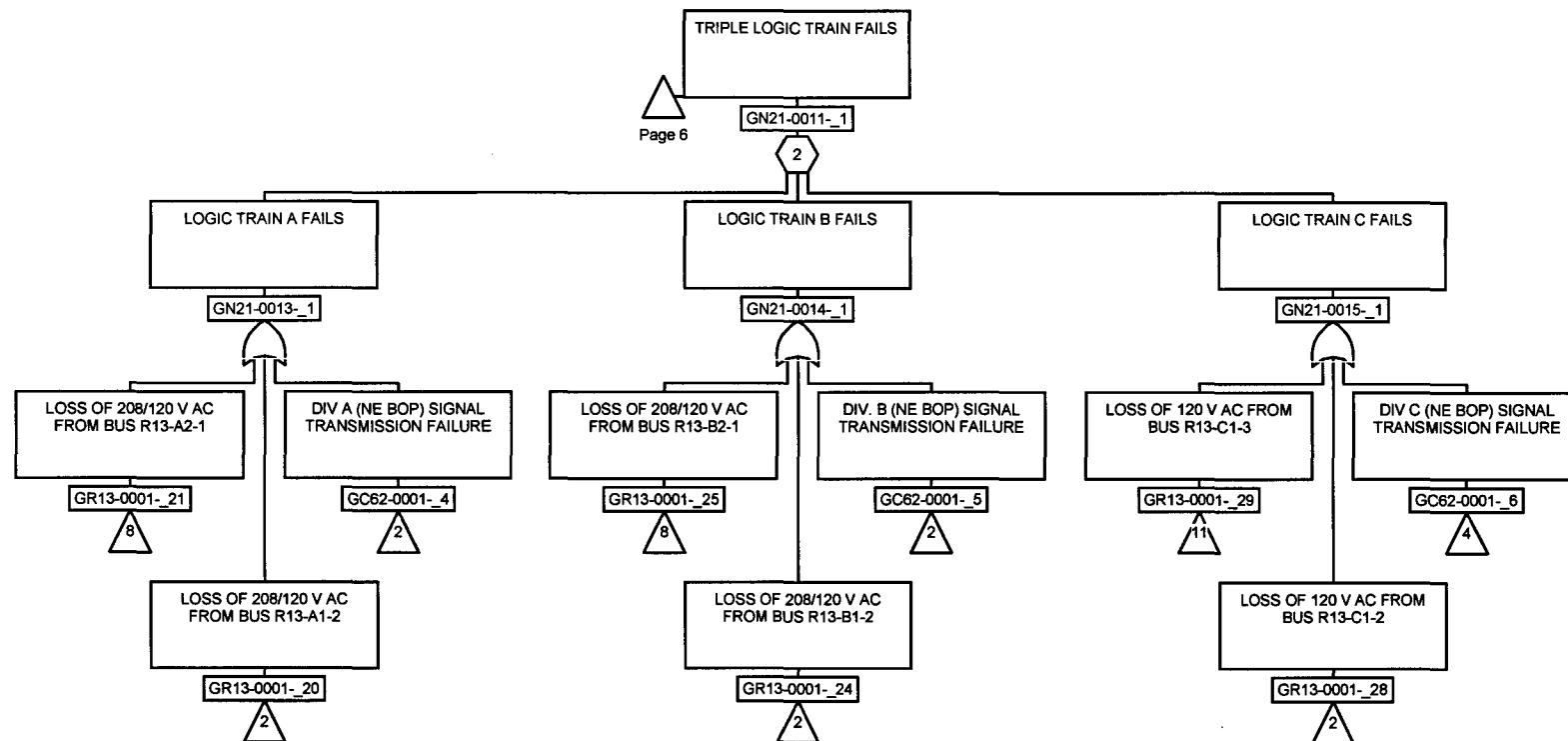


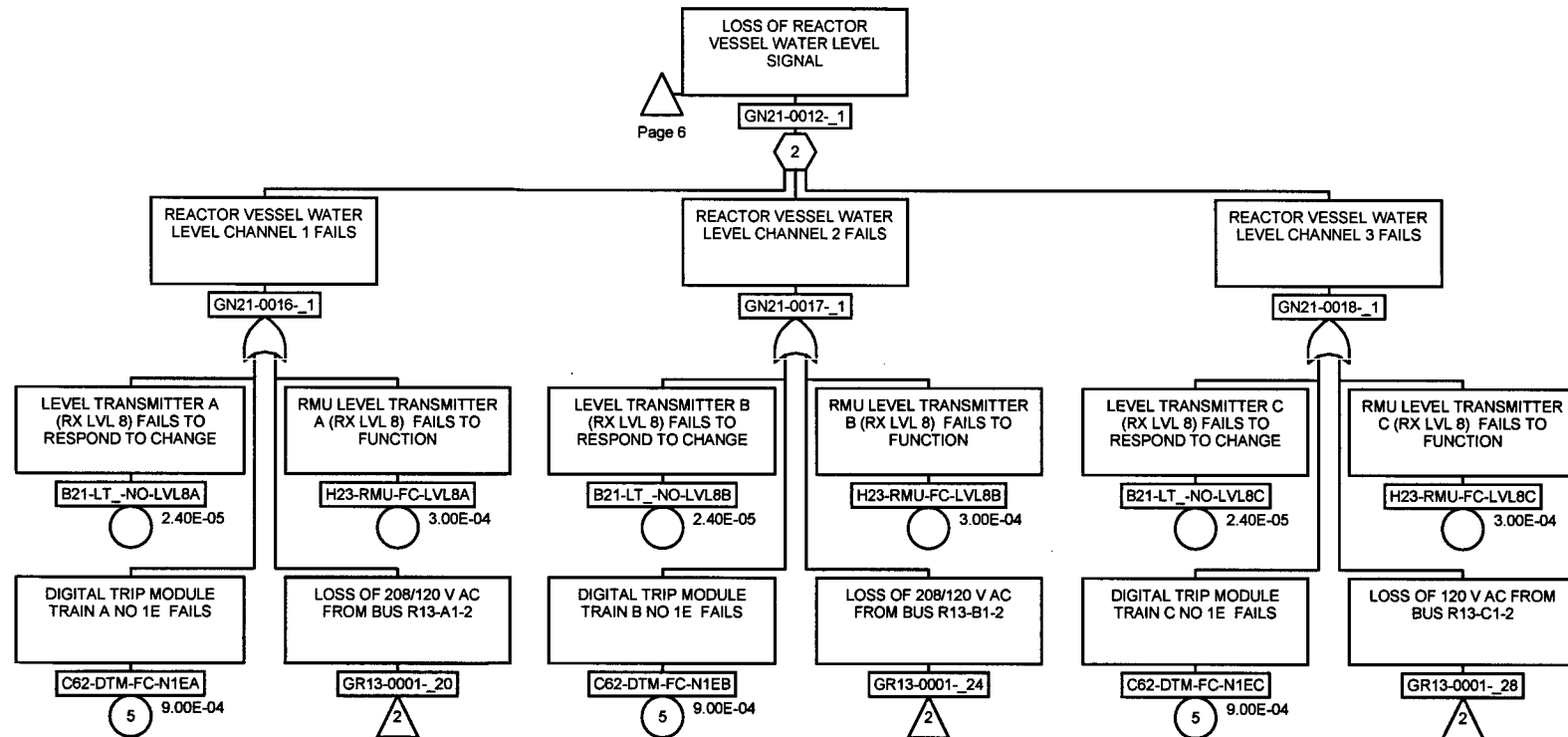


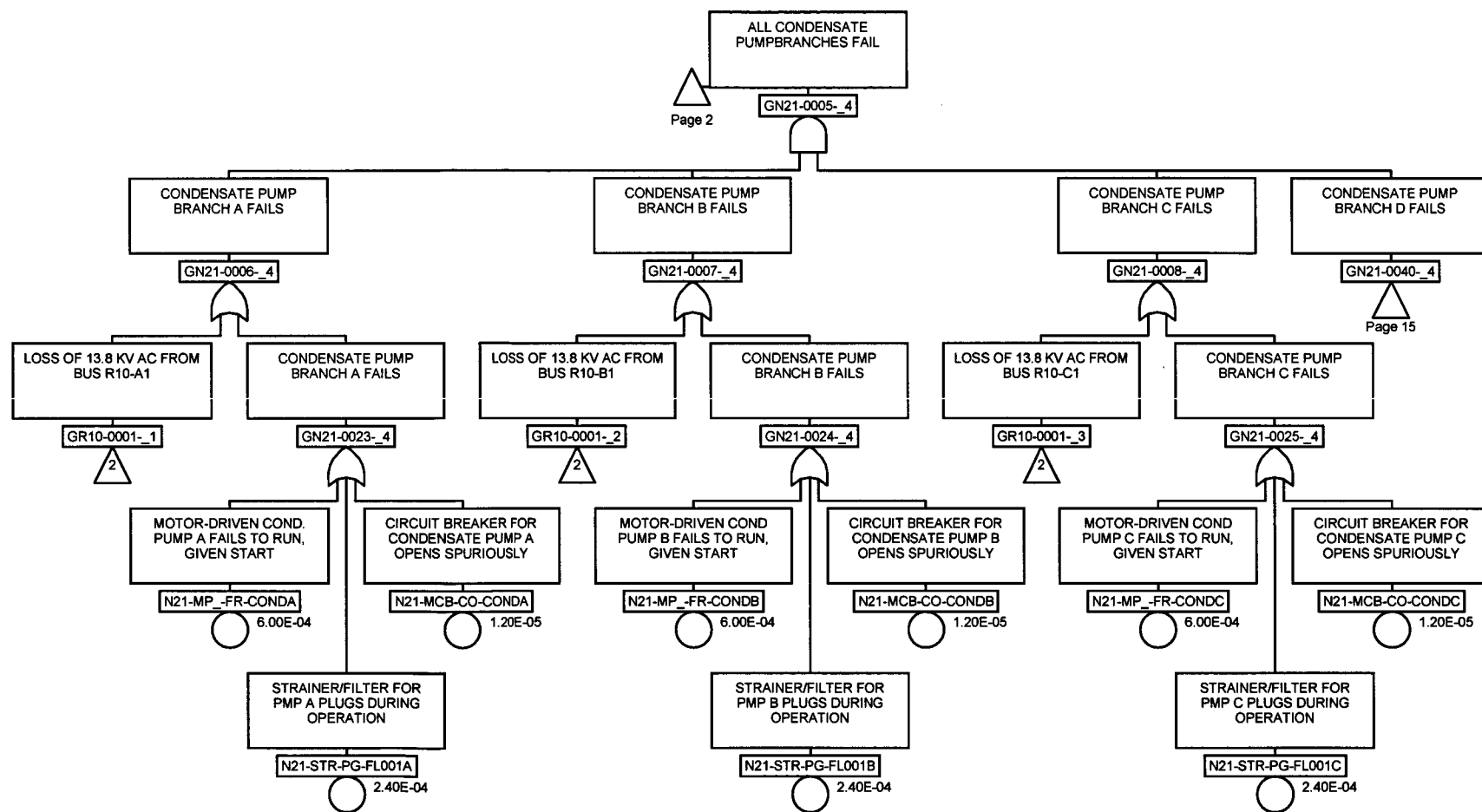






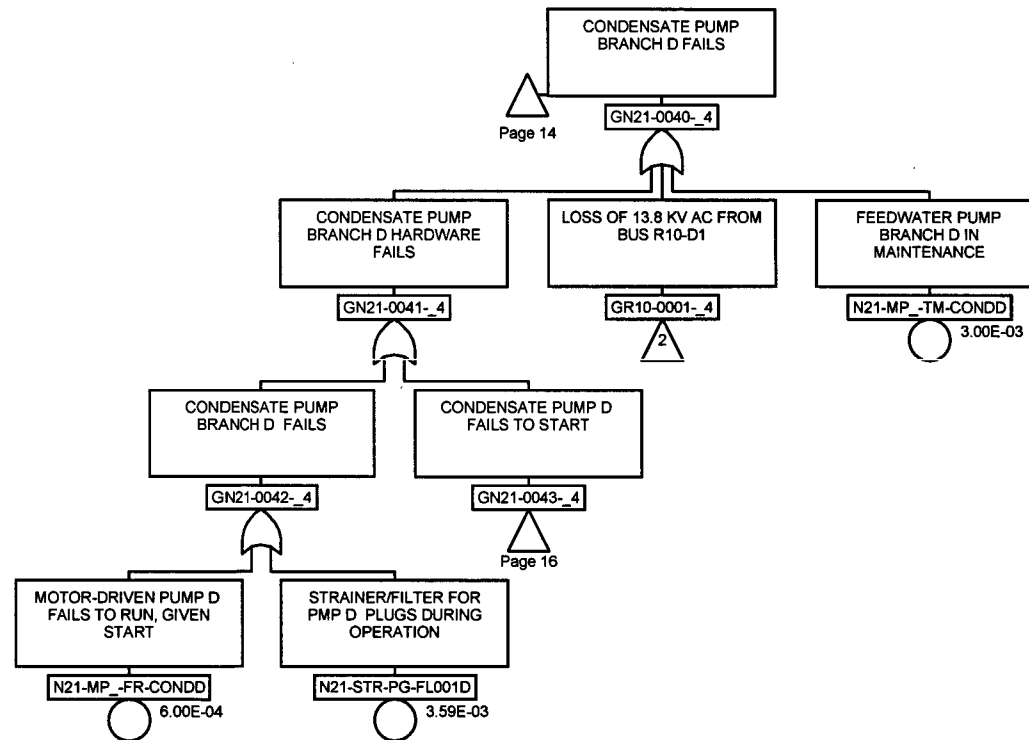


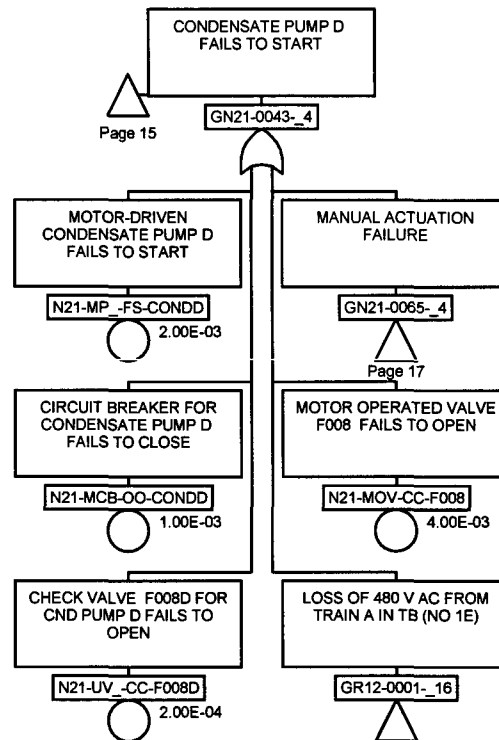


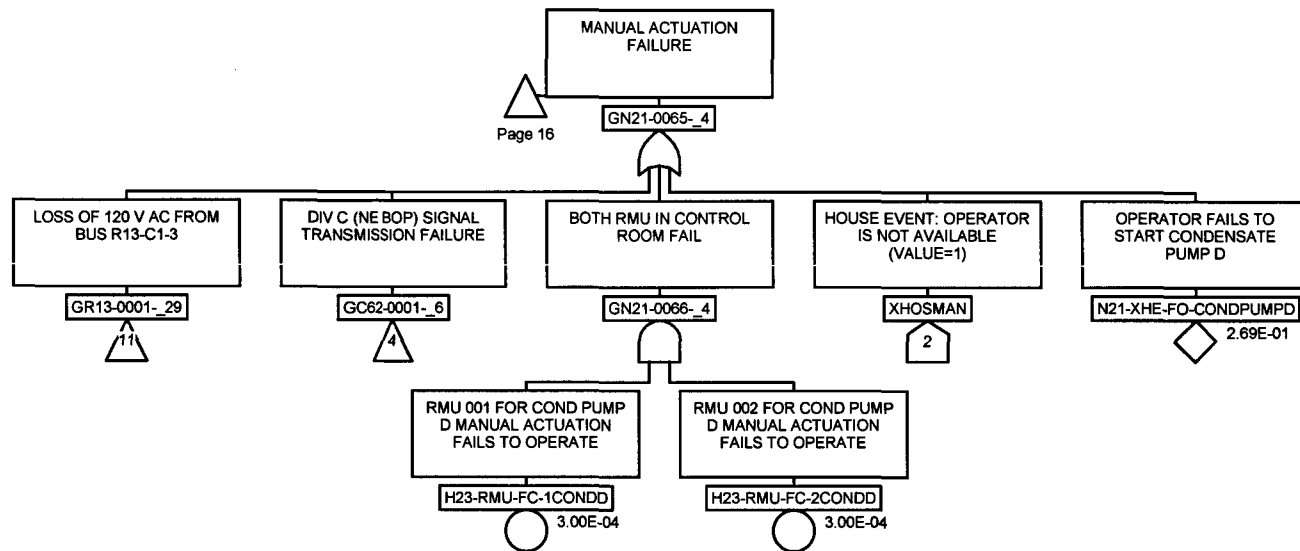


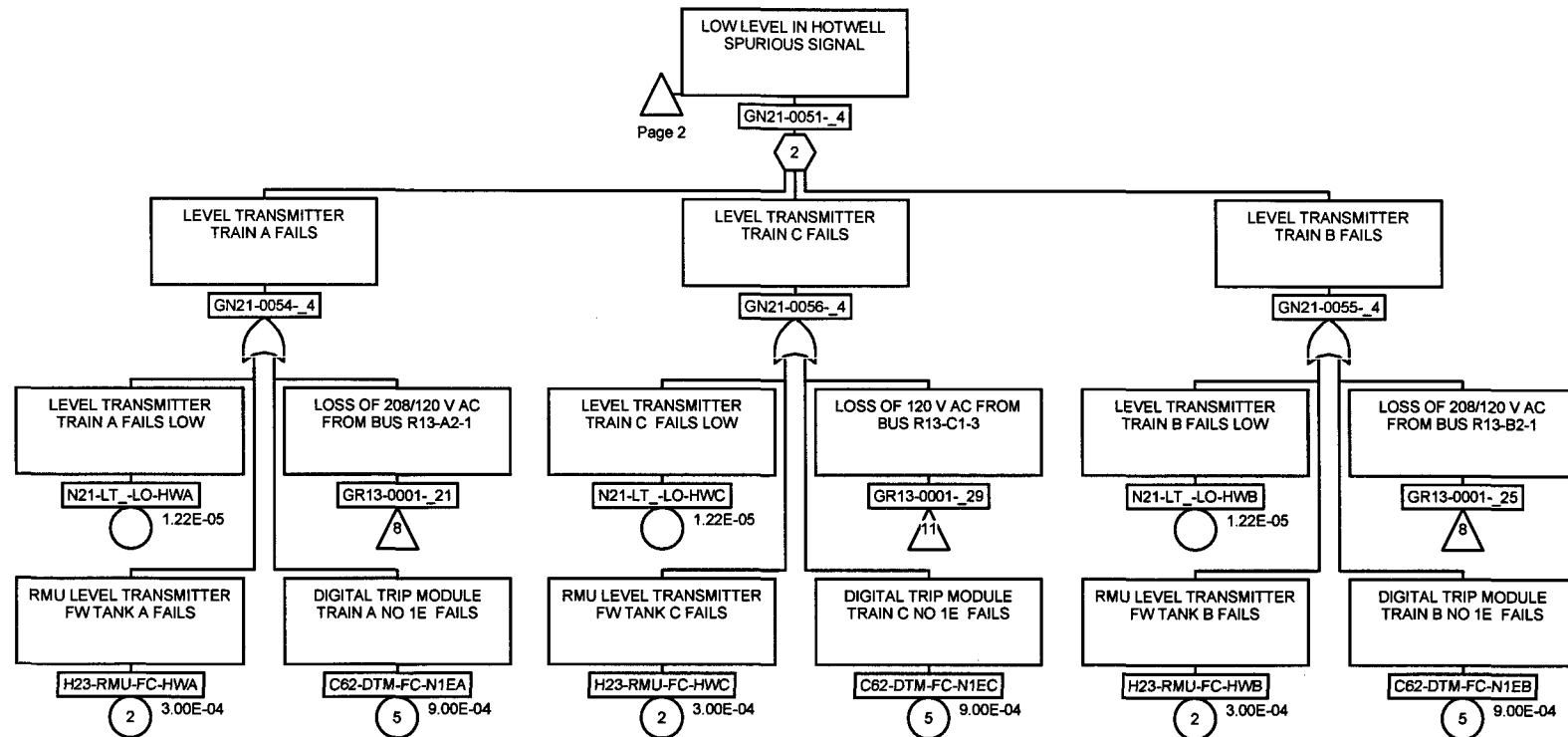
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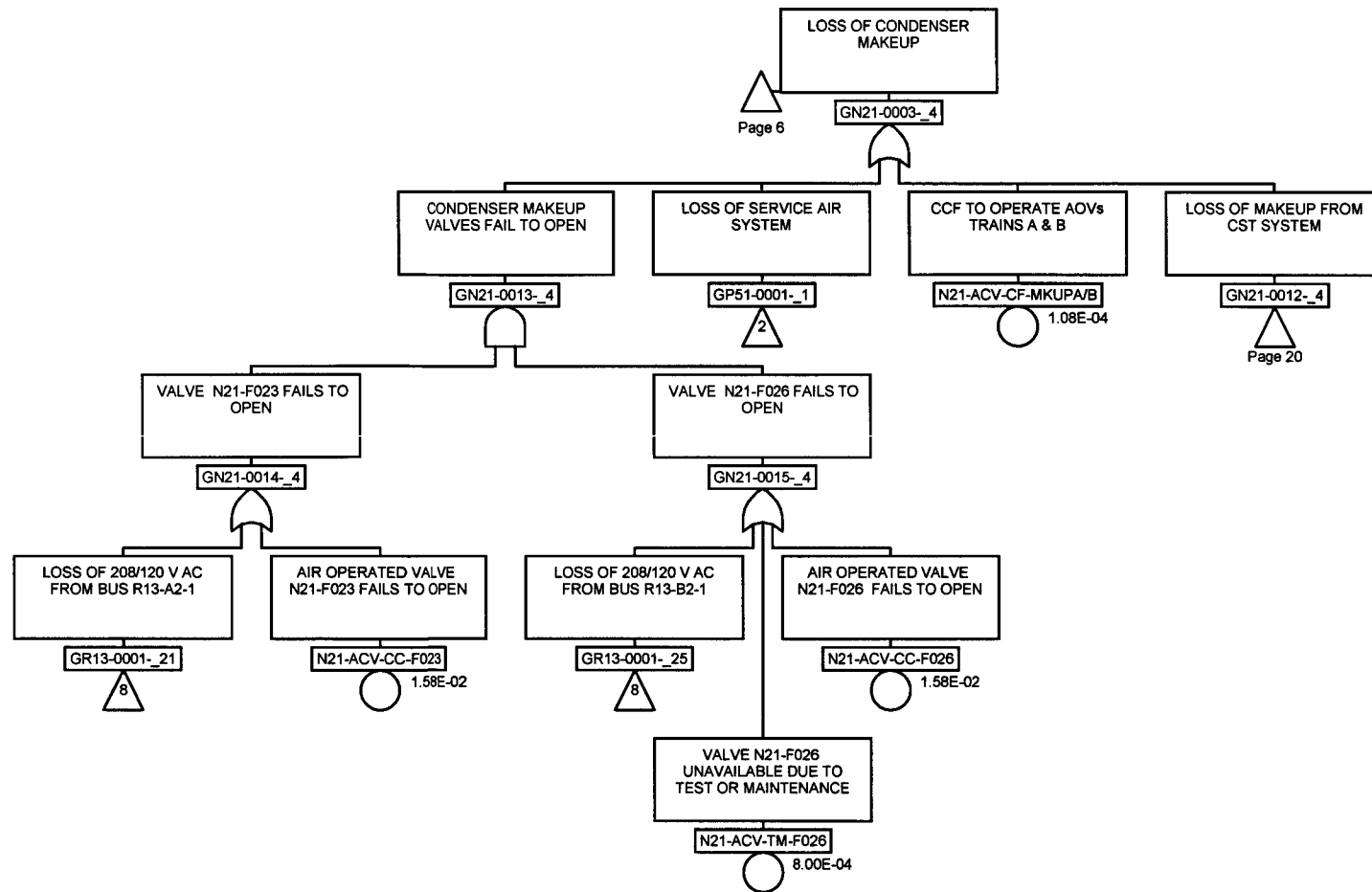
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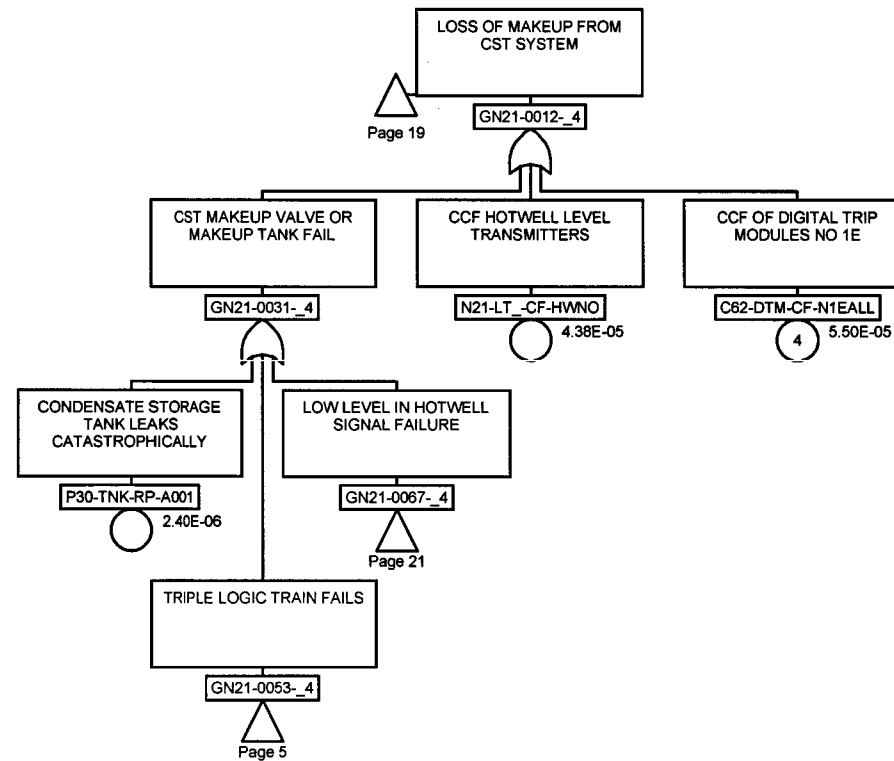


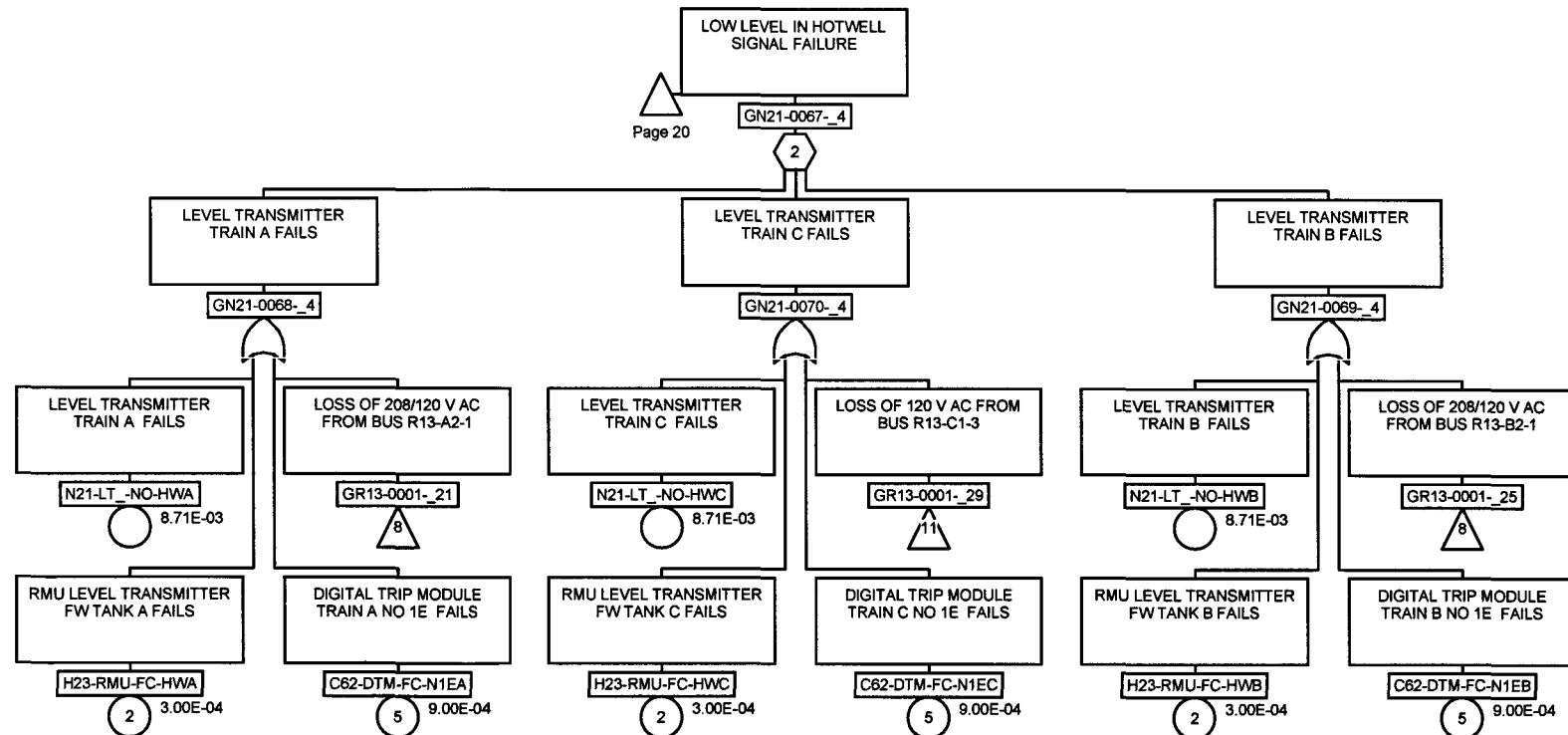








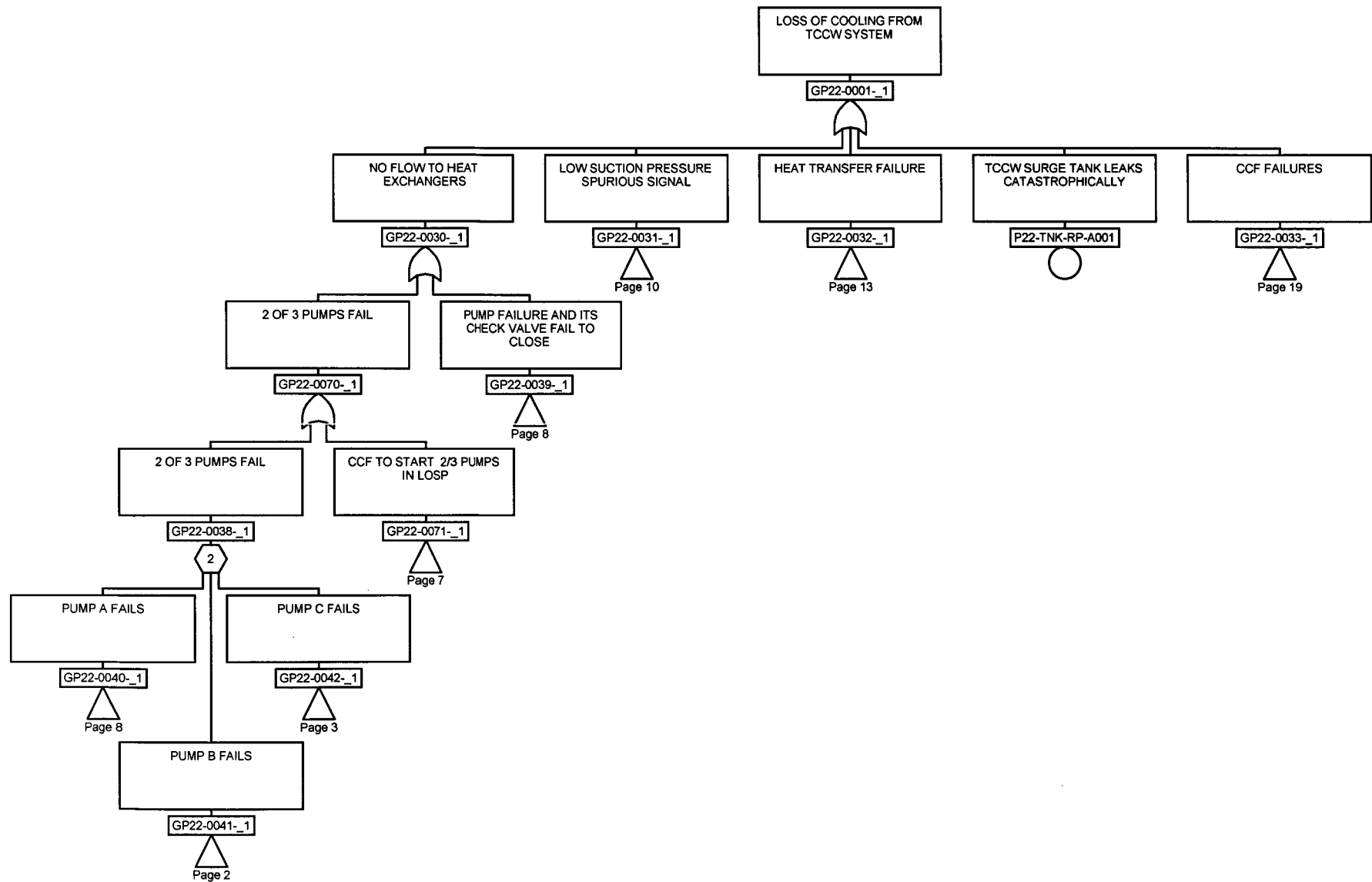


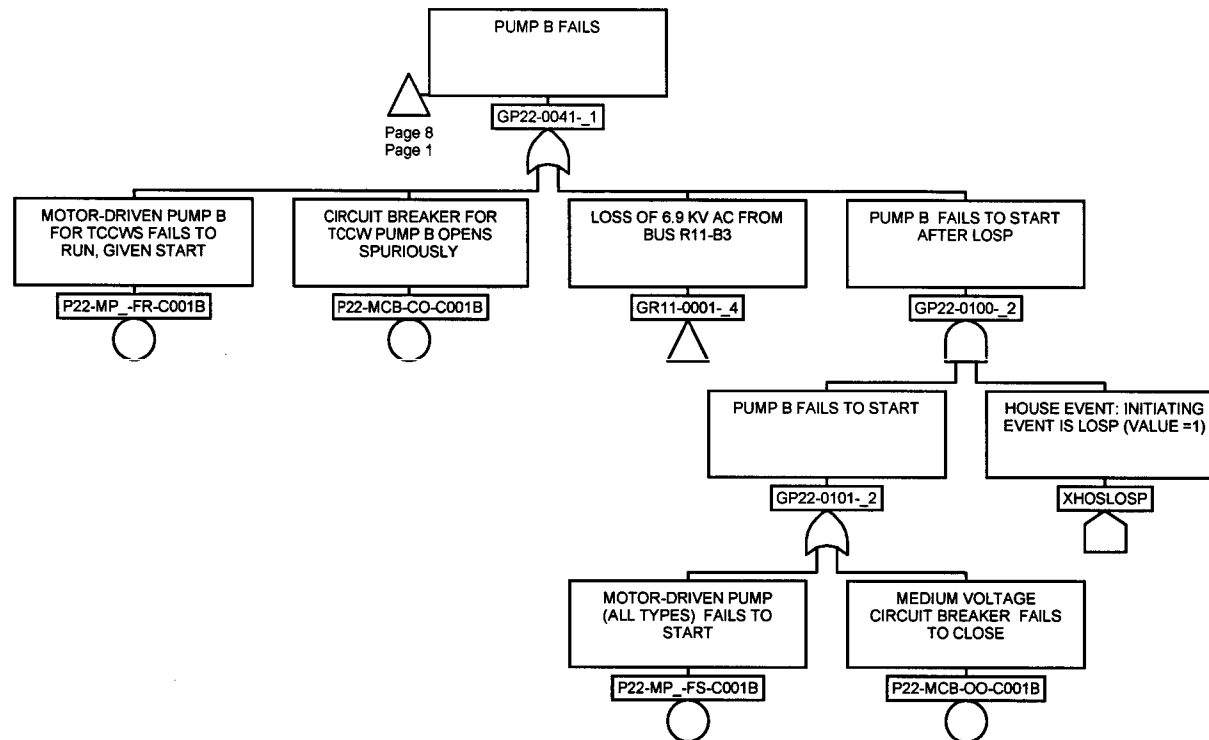


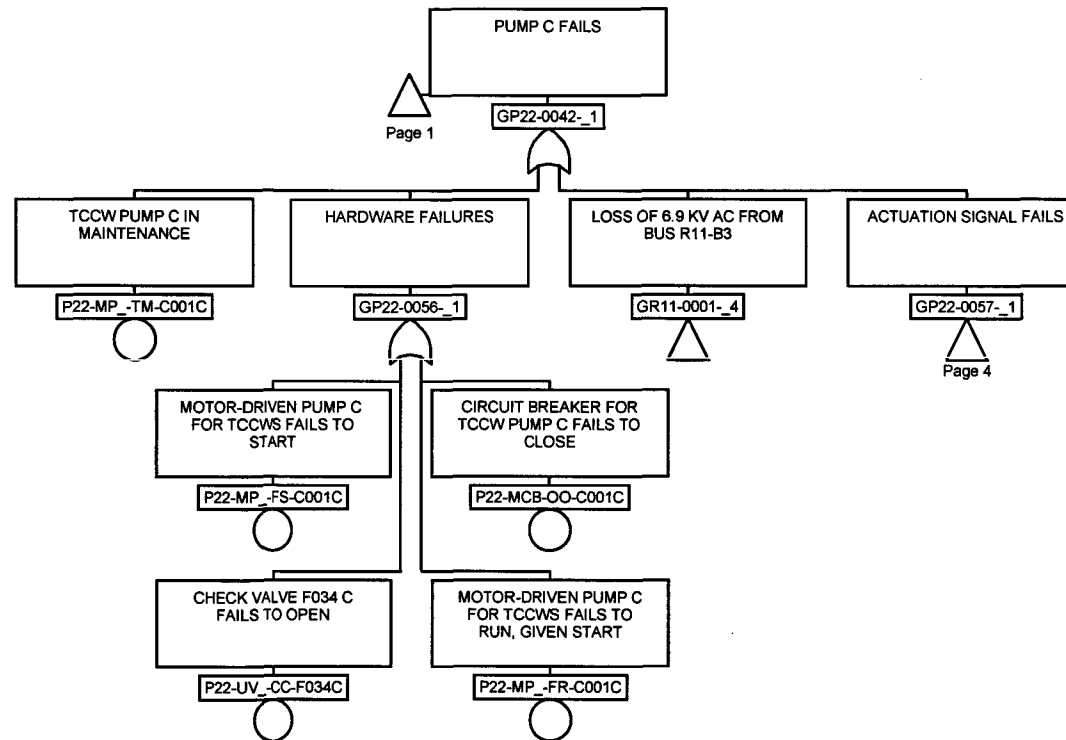
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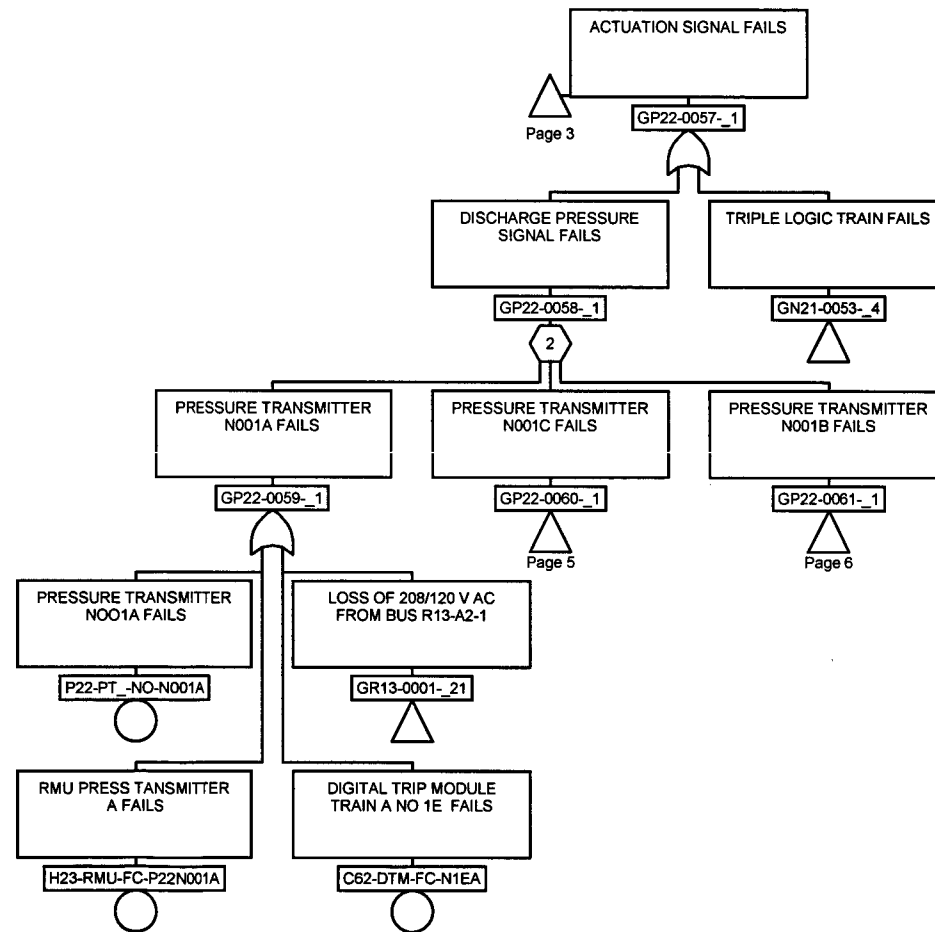
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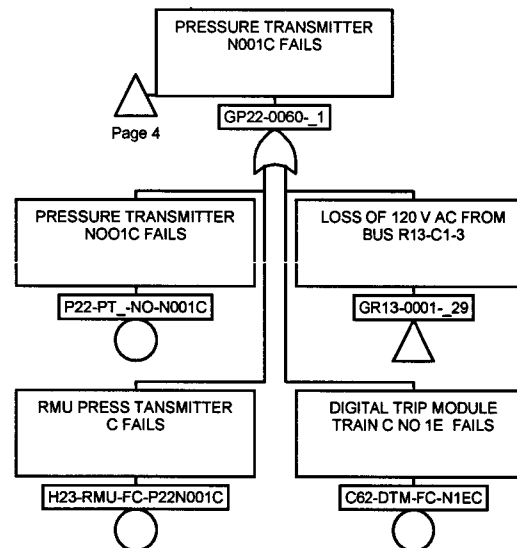
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Feedwater/Condensate and TCCW
Fault Tree



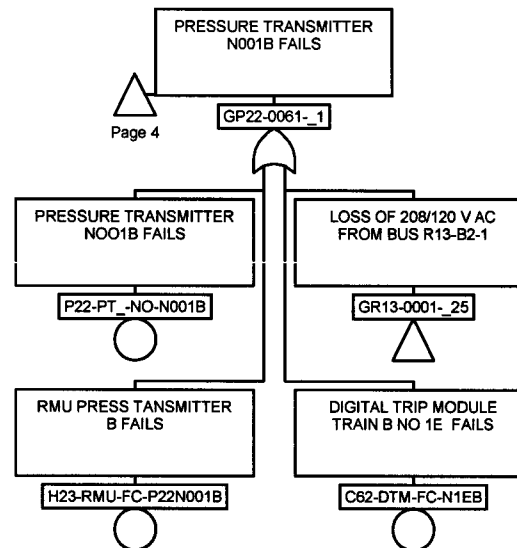


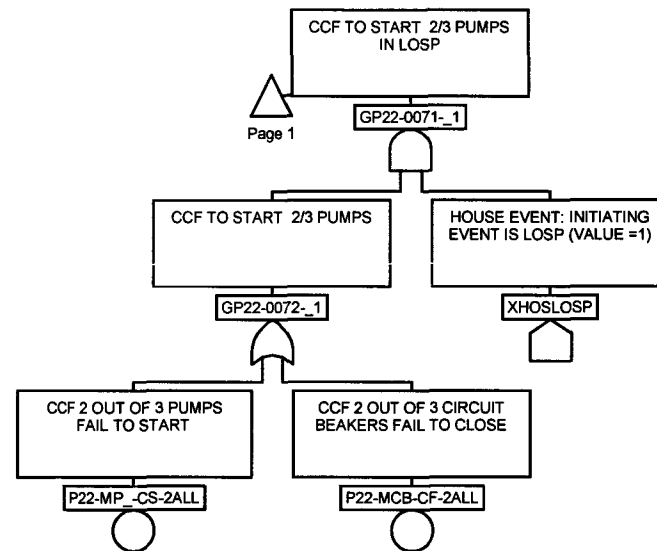




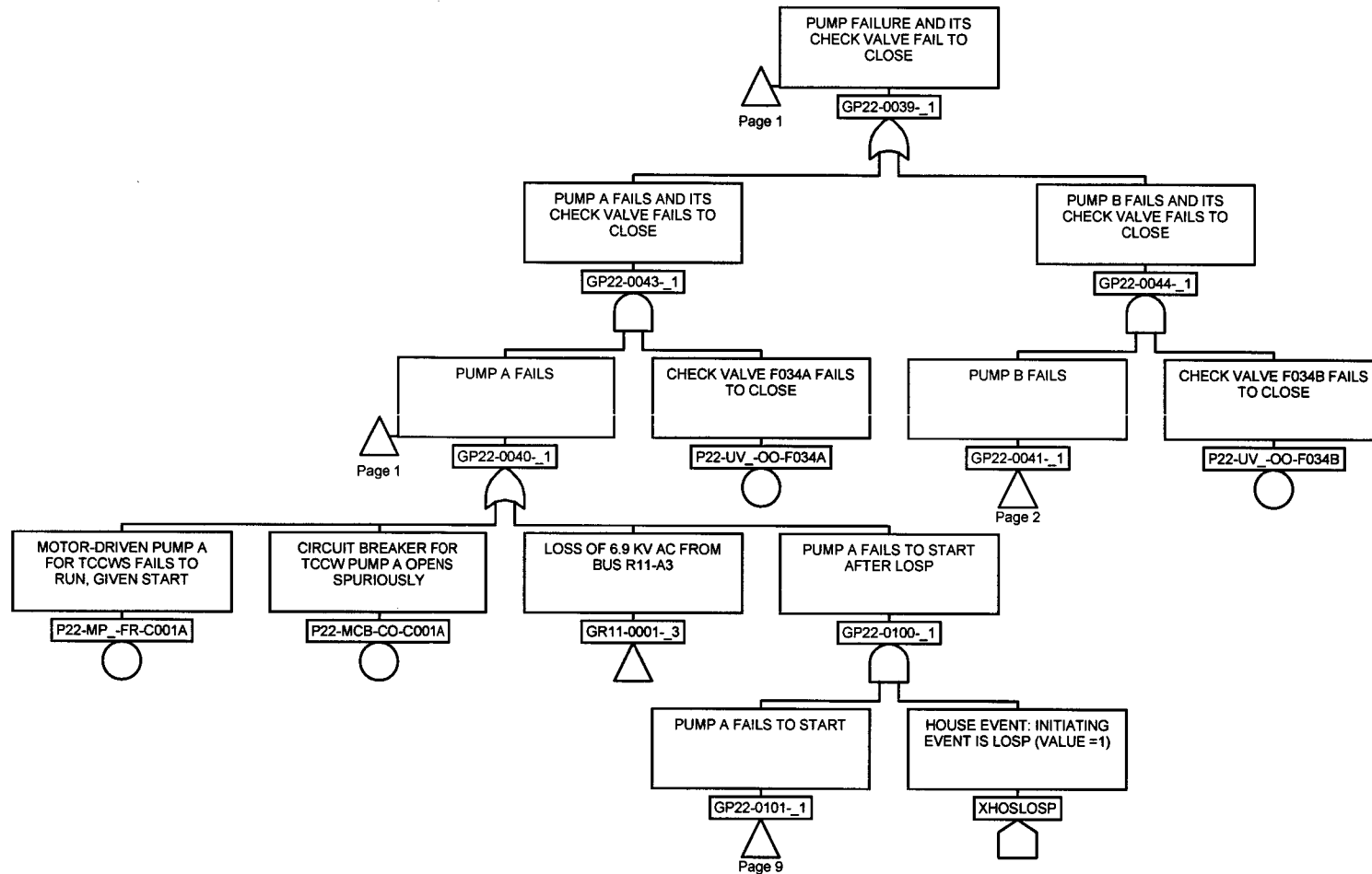


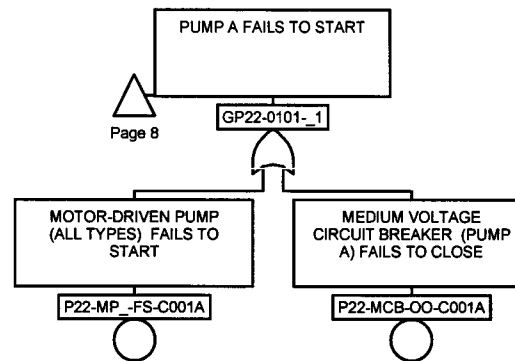
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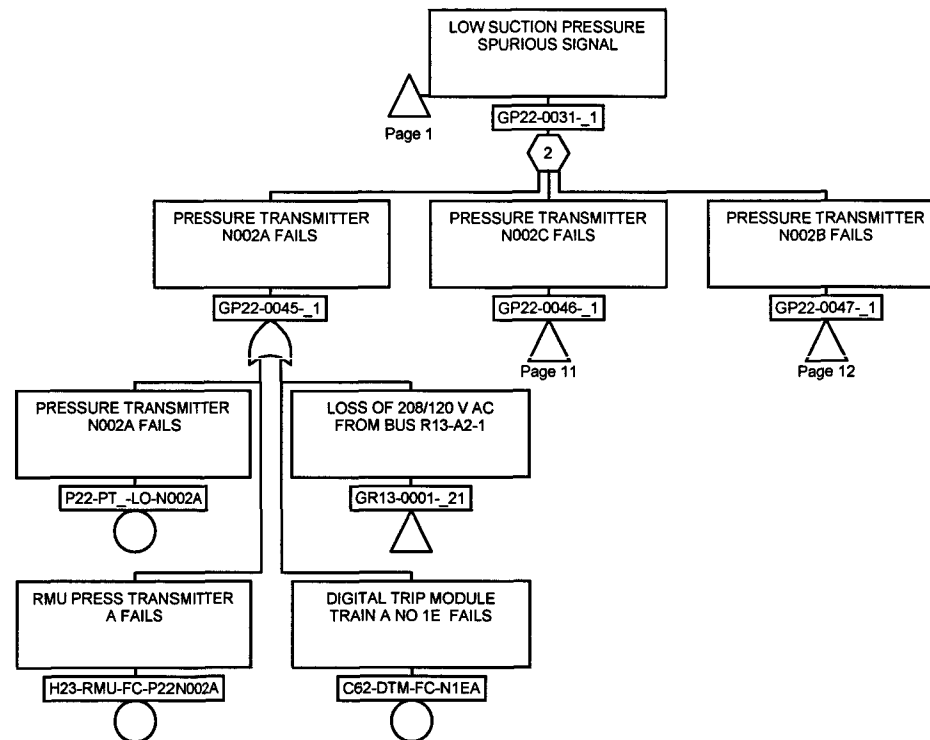


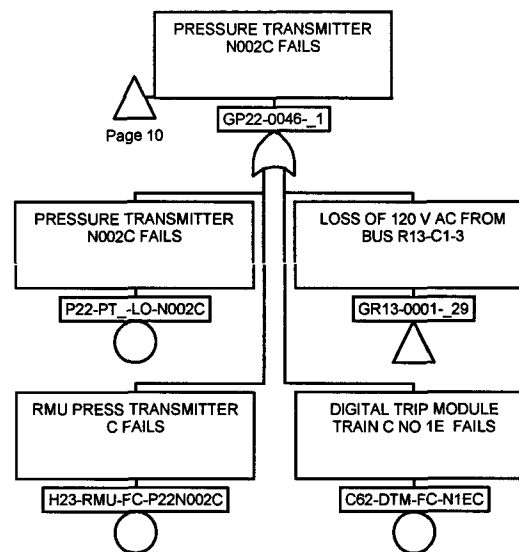


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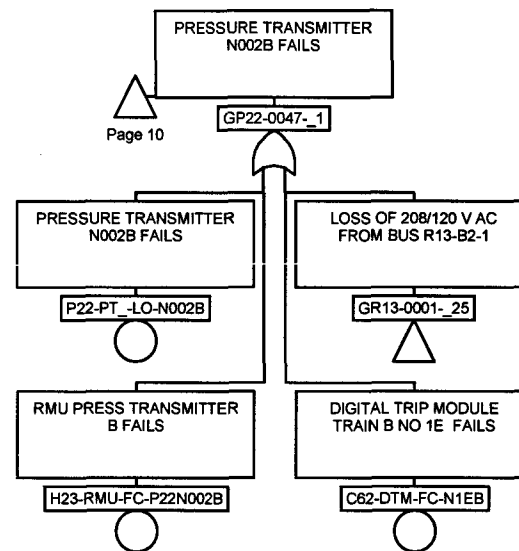


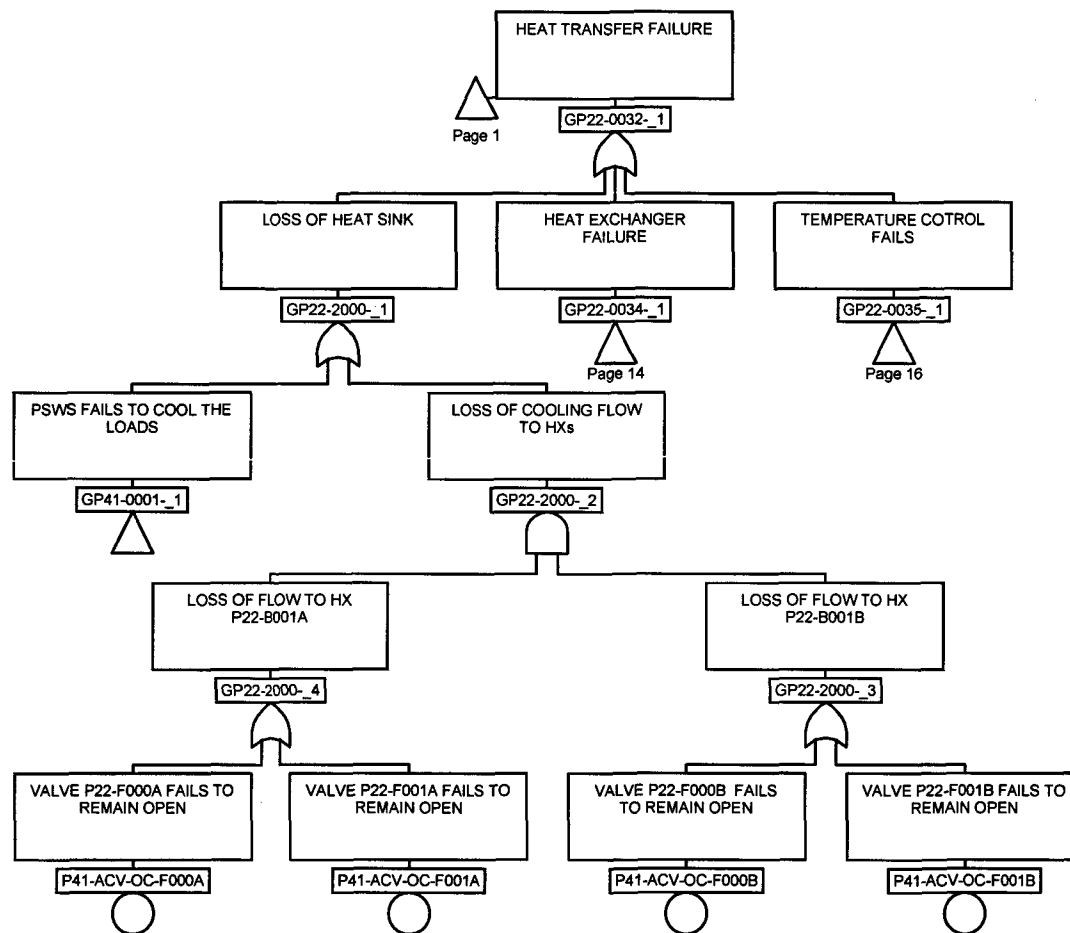


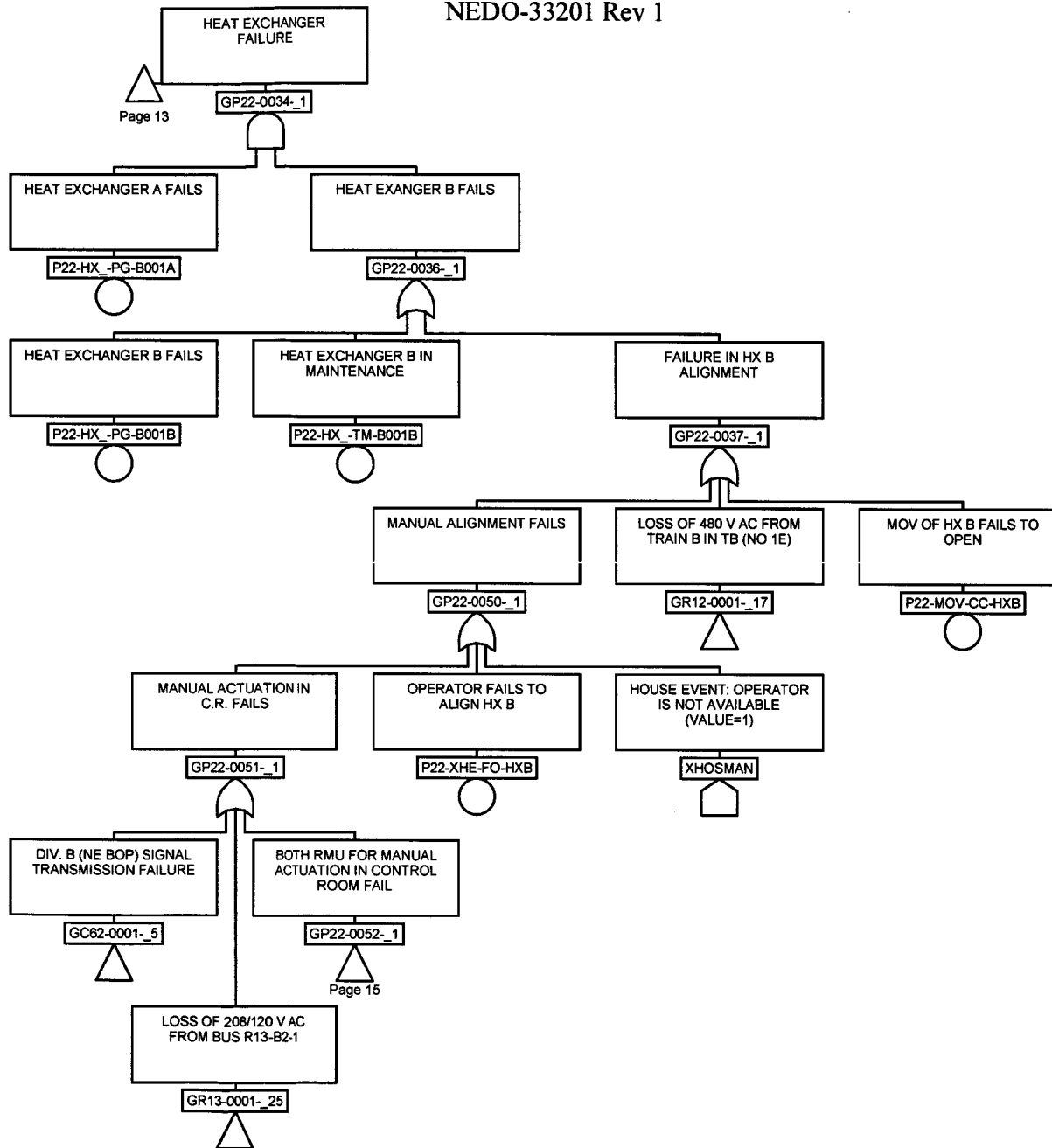


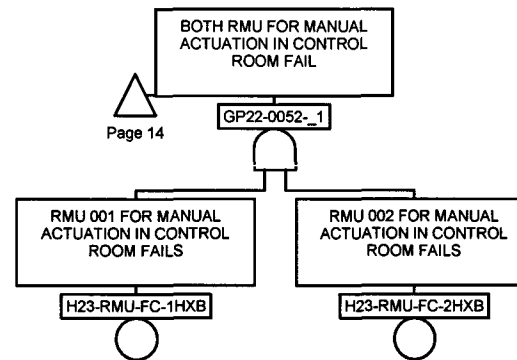


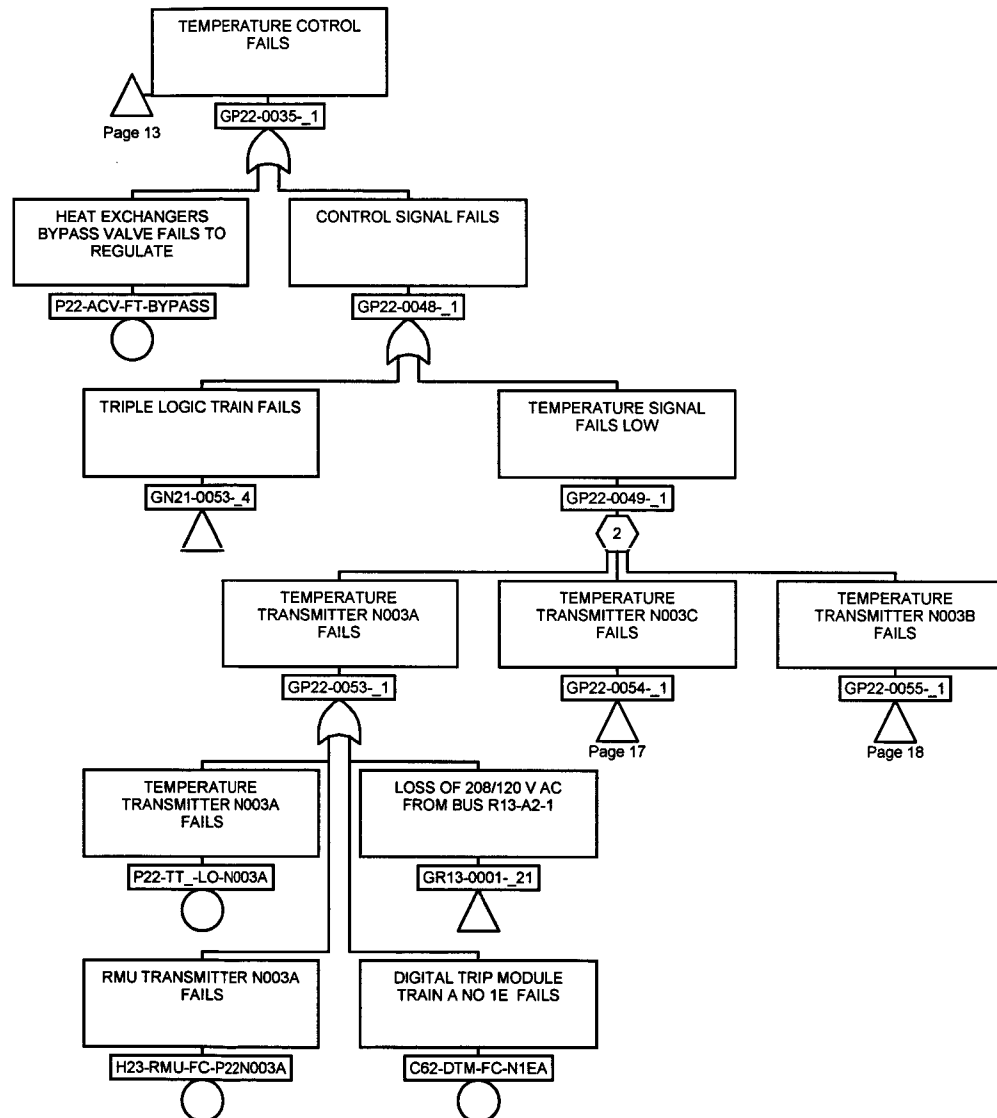
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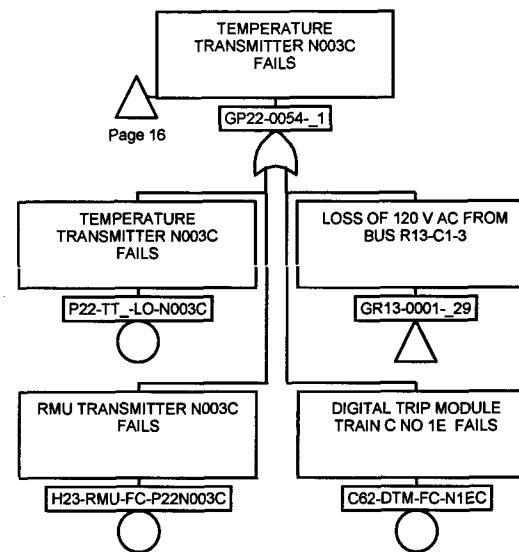


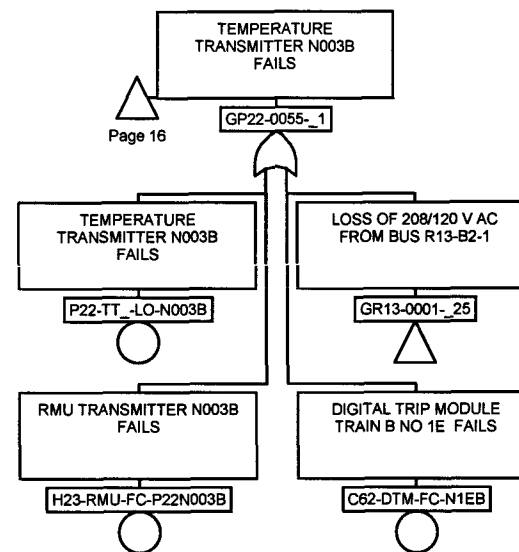


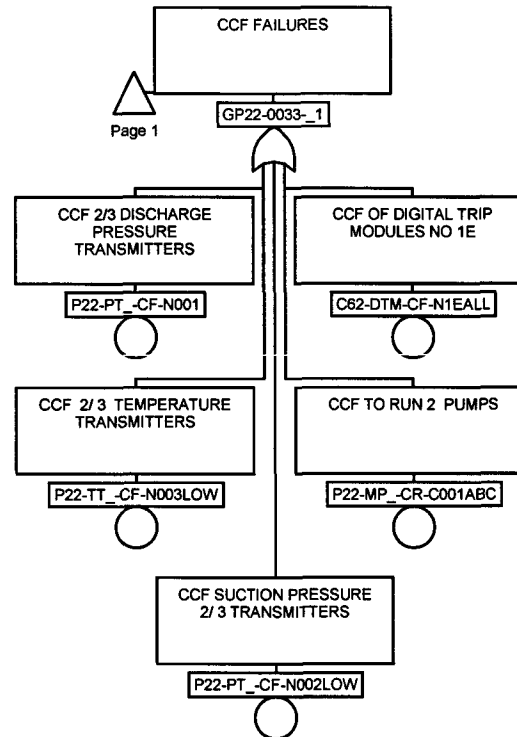












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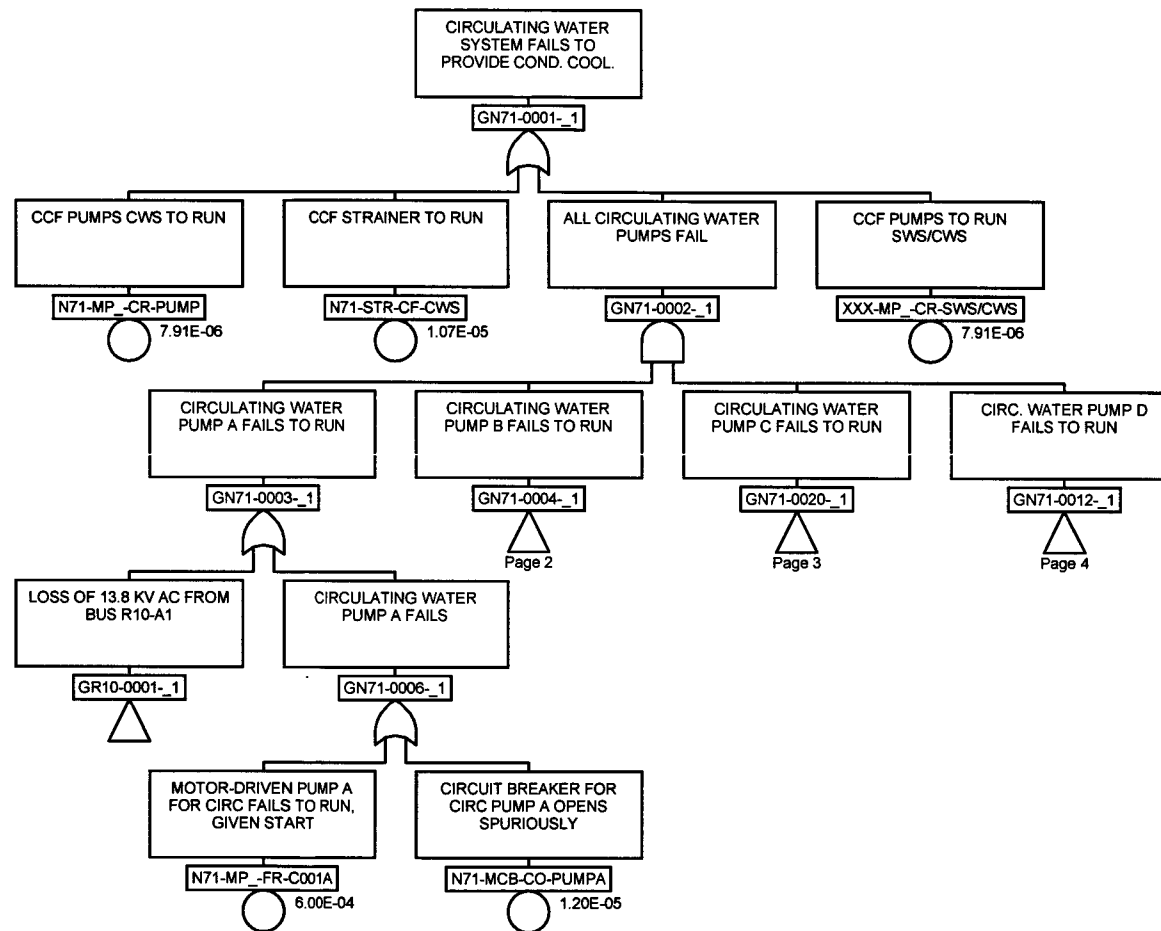
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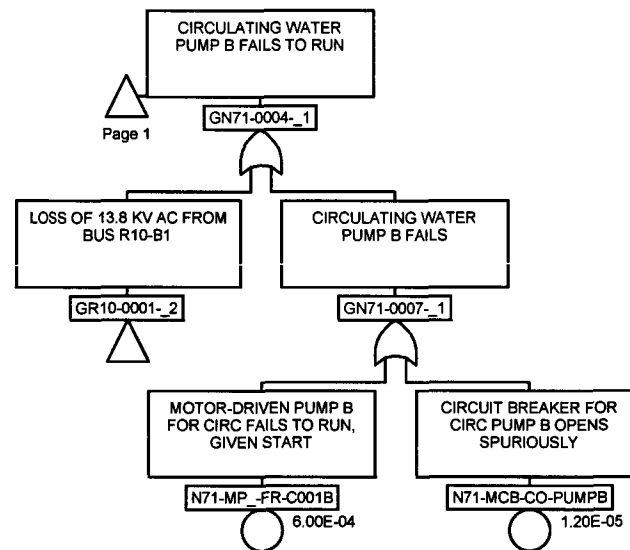
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| XHOSLOSP | 8 | 5 | | | | | |
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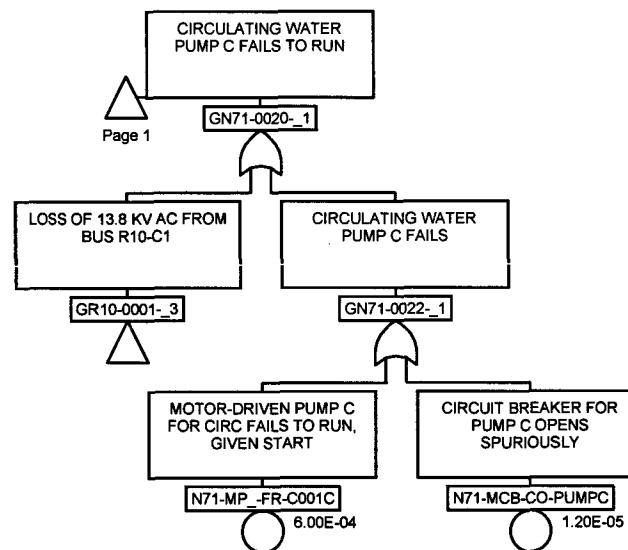
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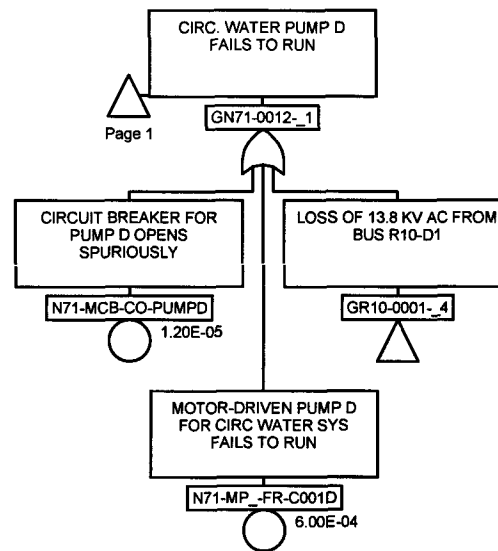
Feedwater/Condensate and TCCW

Fault Tree

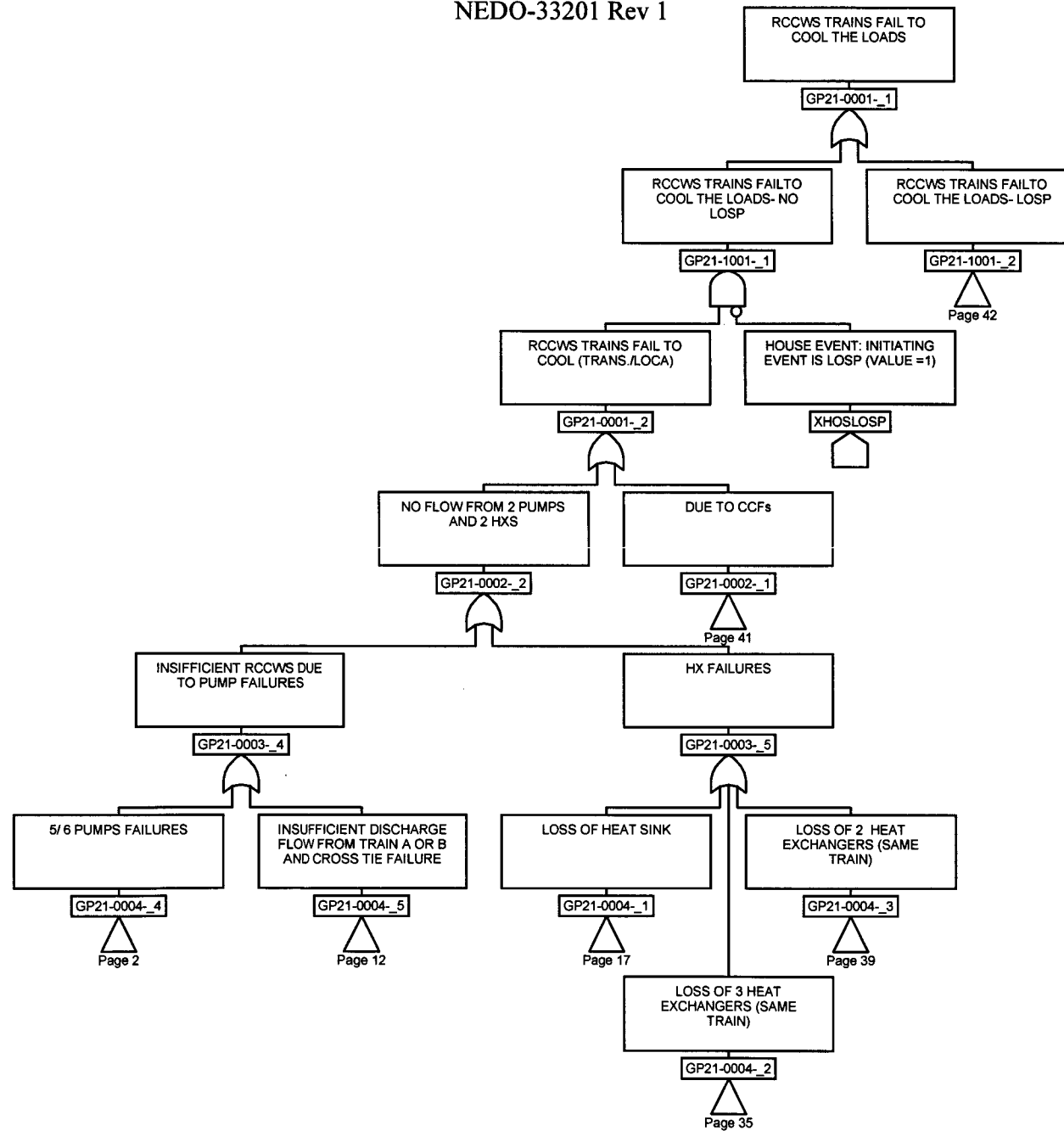


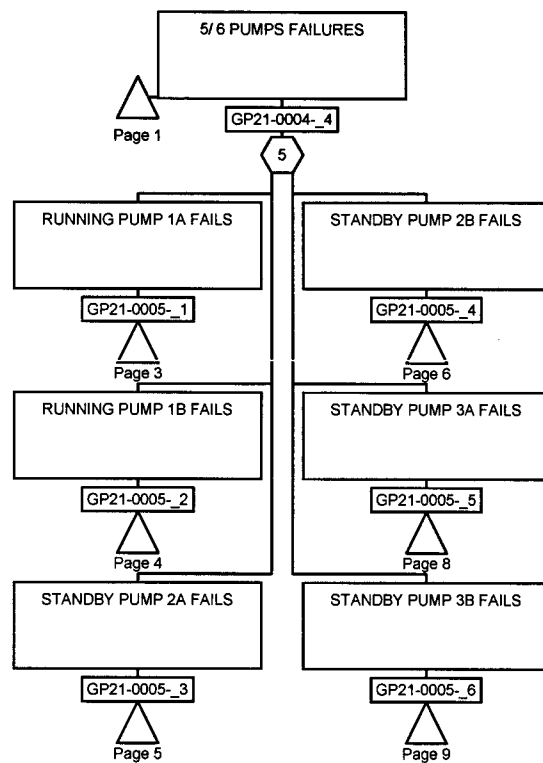


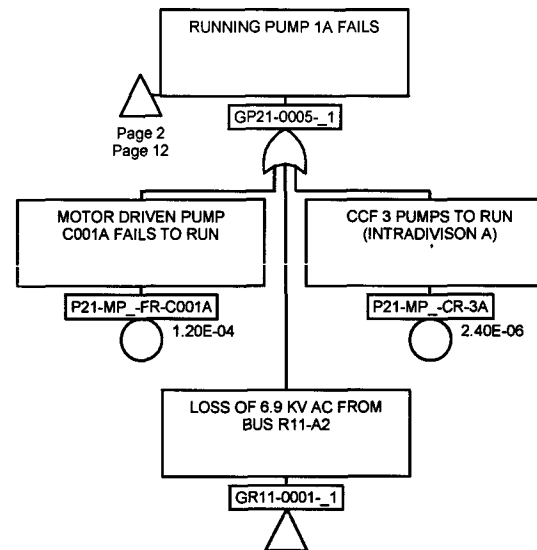


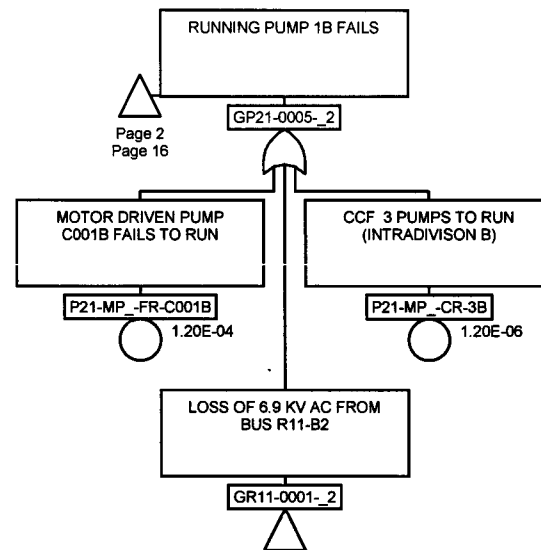


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| GN71-0006- 1 | 1 | 2 | | | | | | |
| GN71-0007- 1 | 2 | 2 | | | | | | |
| GN71-0012- 1 | 1 | 5 | | | | | | |
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| GN71-0020- 1 | 1 | 4 | | | | | | |
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| GN71-0022- 1 | 3 | 2 | | | | | | |
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| N71-MCB-CO-PUMPD | 4 | 1 | | | | | | |
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| N71-MP -FR-C001B | 2 | 2 | | | | | | |
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| Circulating Water System | | | | | | Circ Water.caf | Appendix B.4.9-3 | Page 5 |



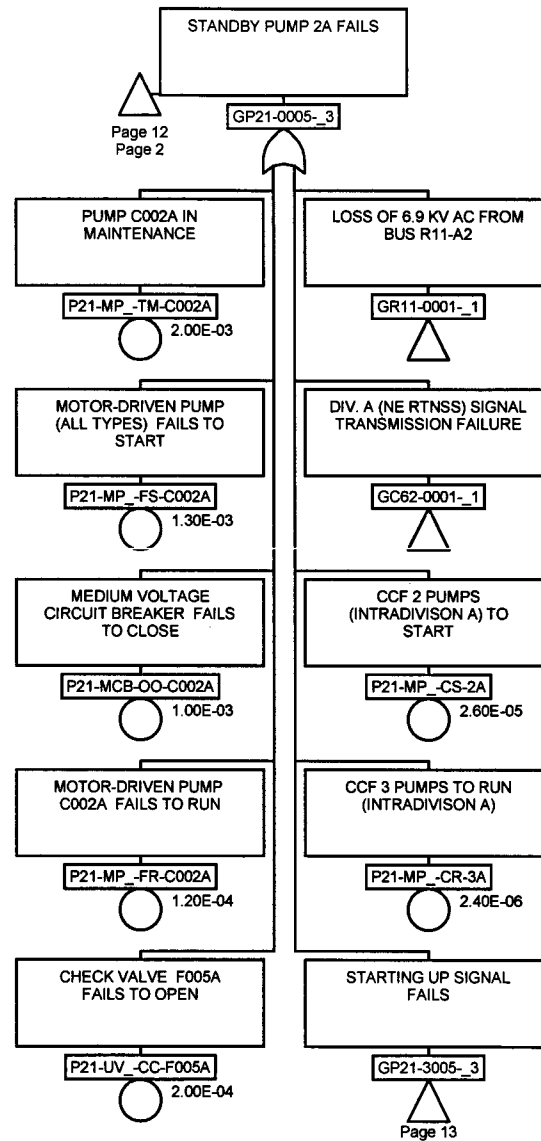


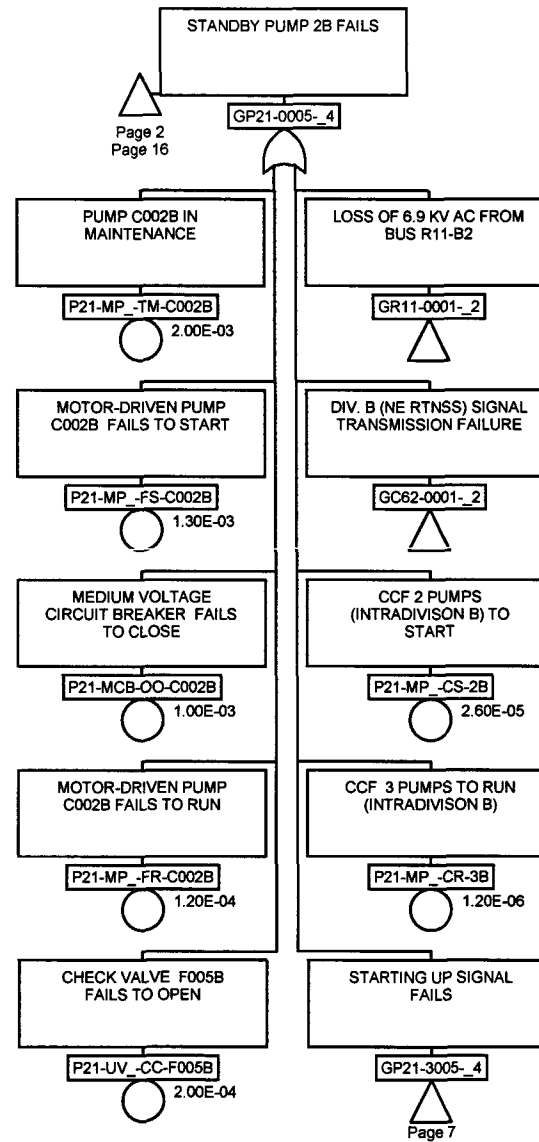


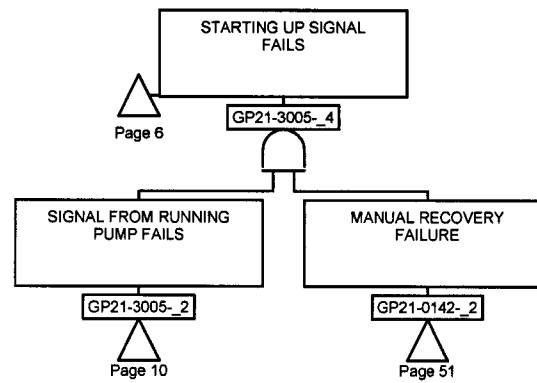


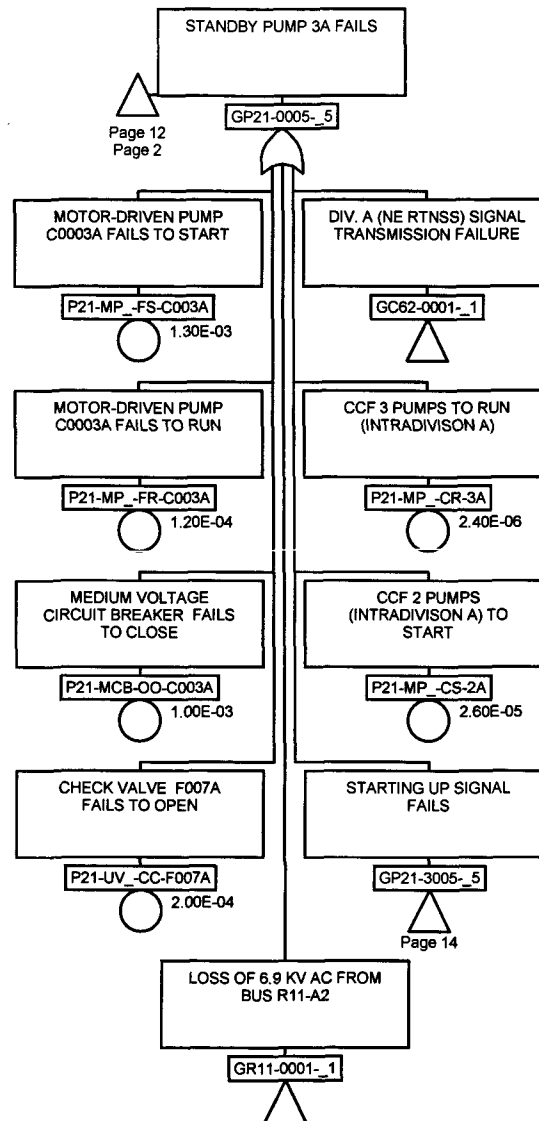
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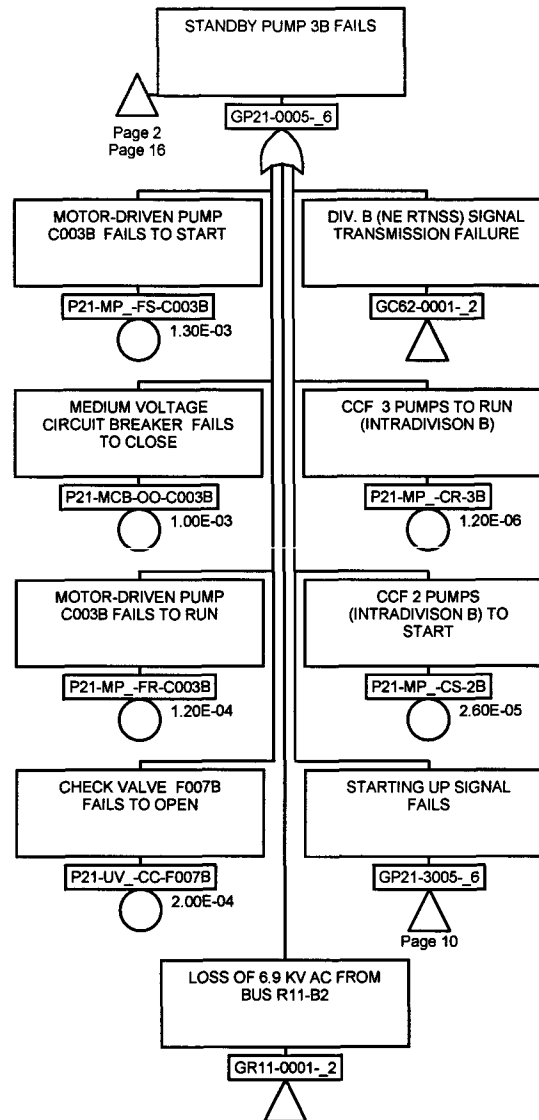
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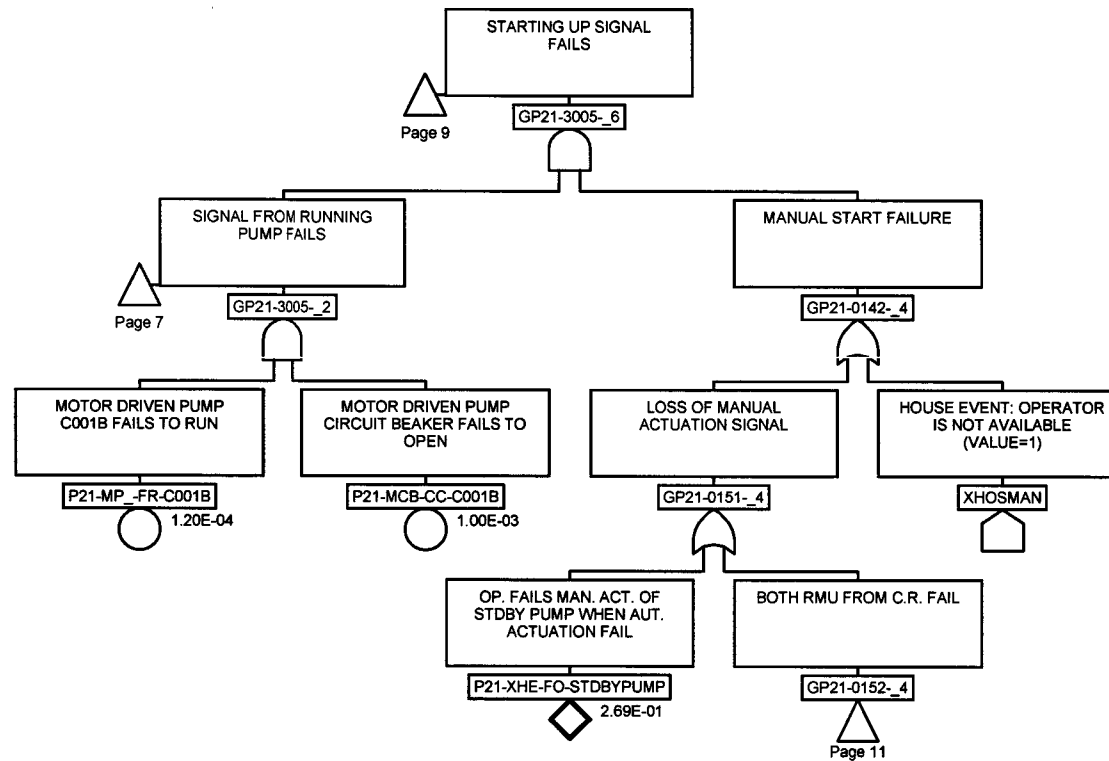


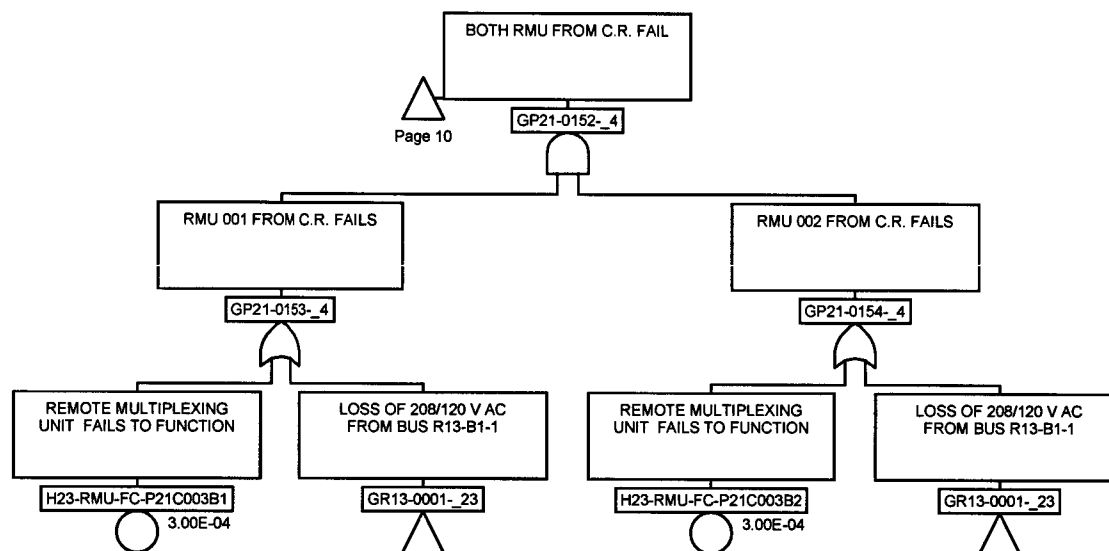


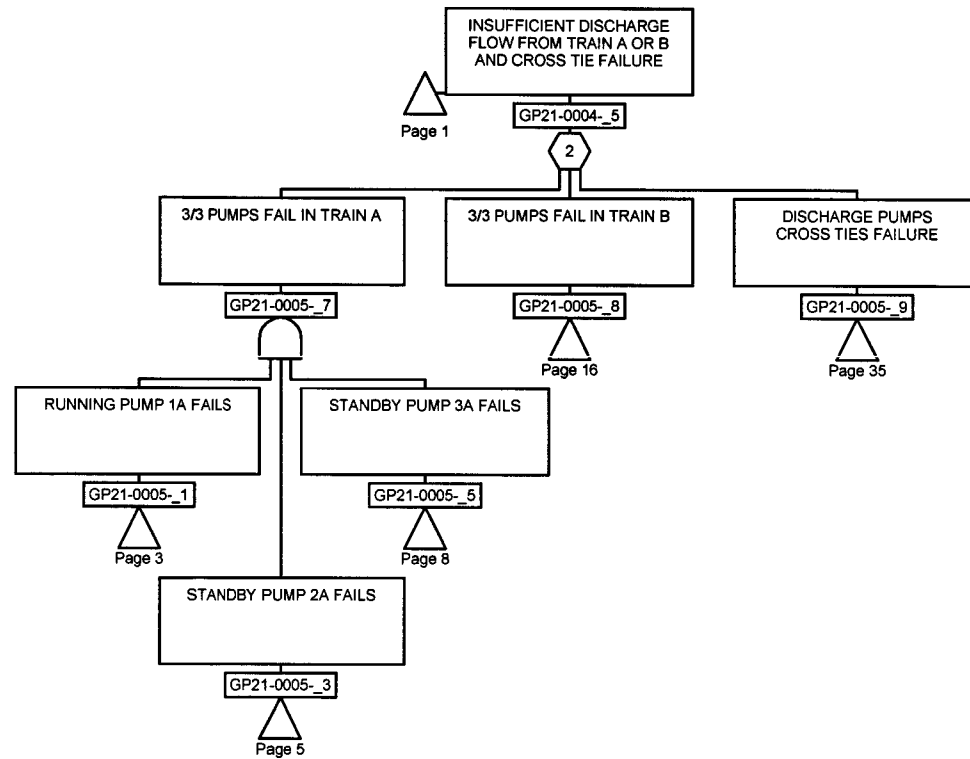


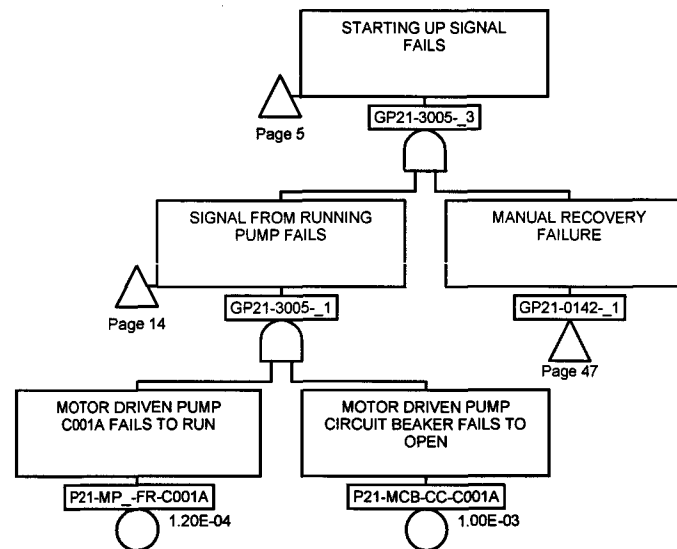


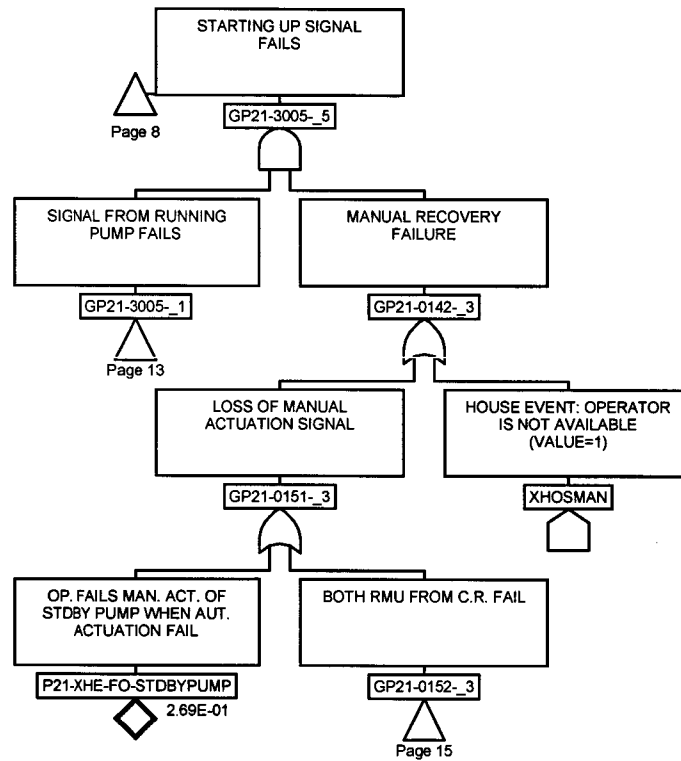


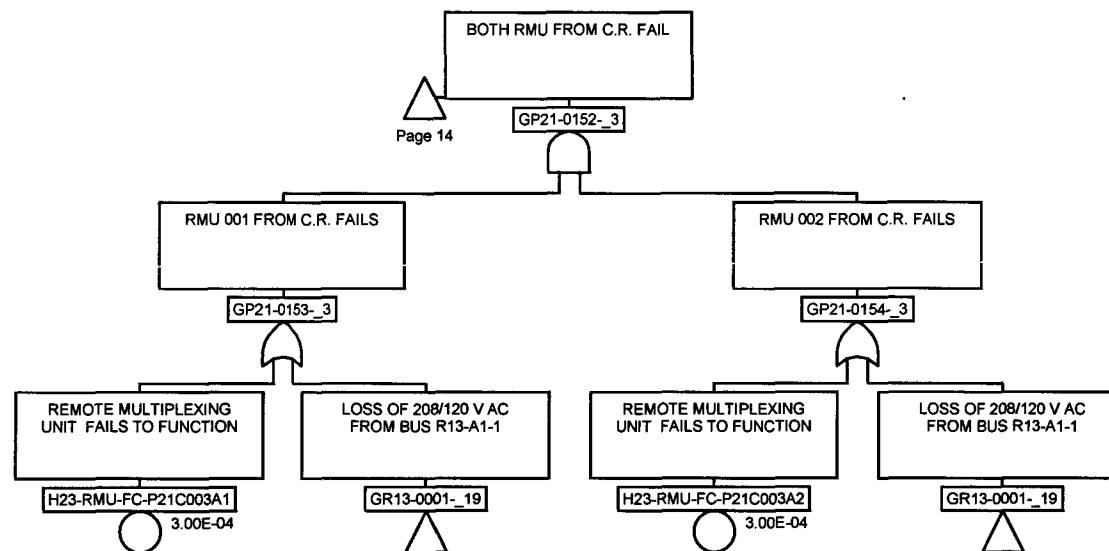


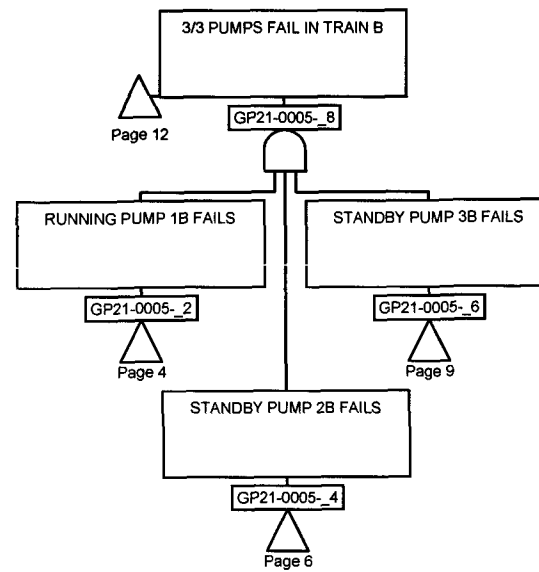


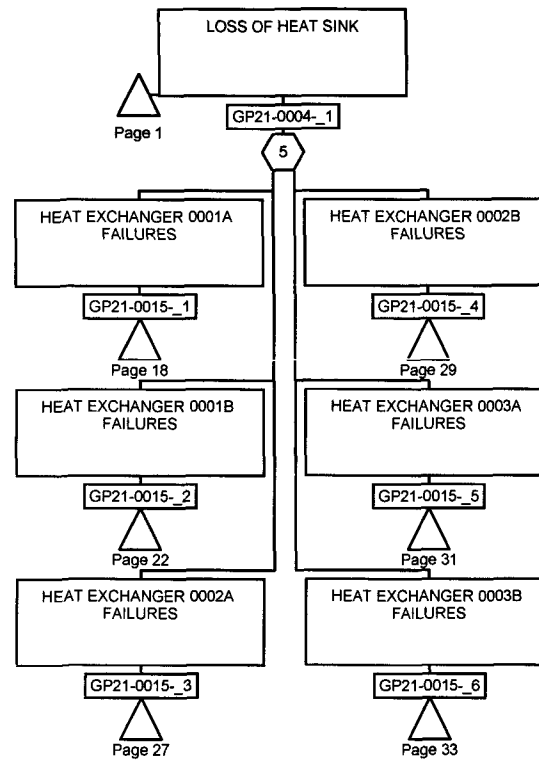


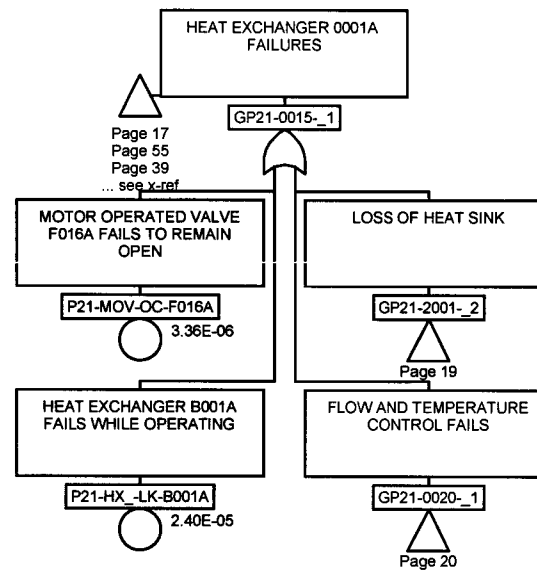


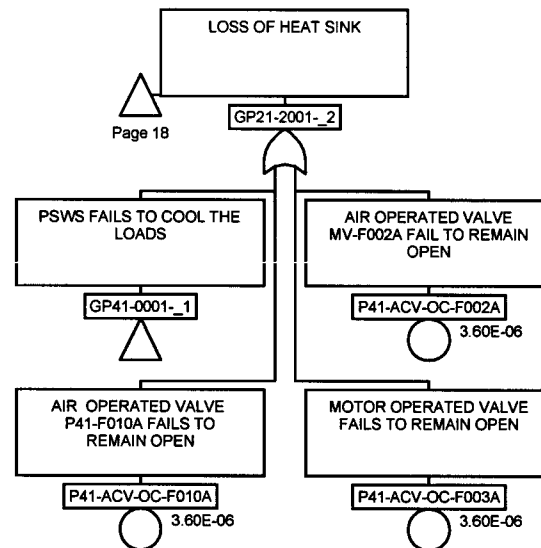


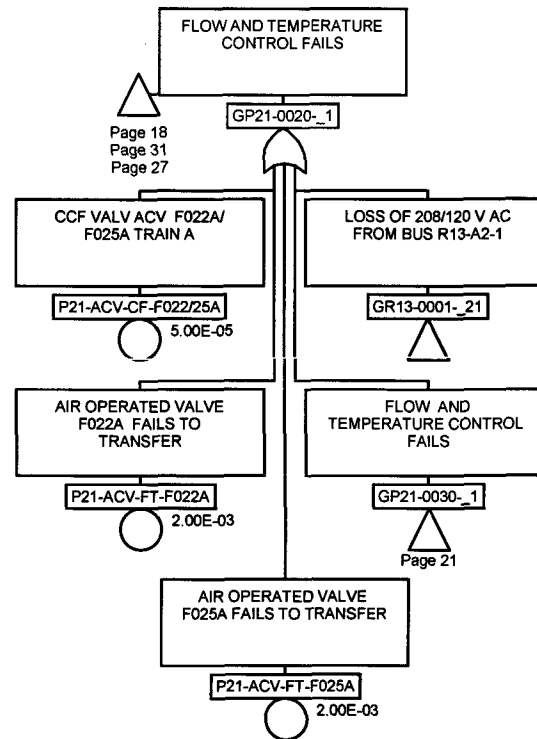


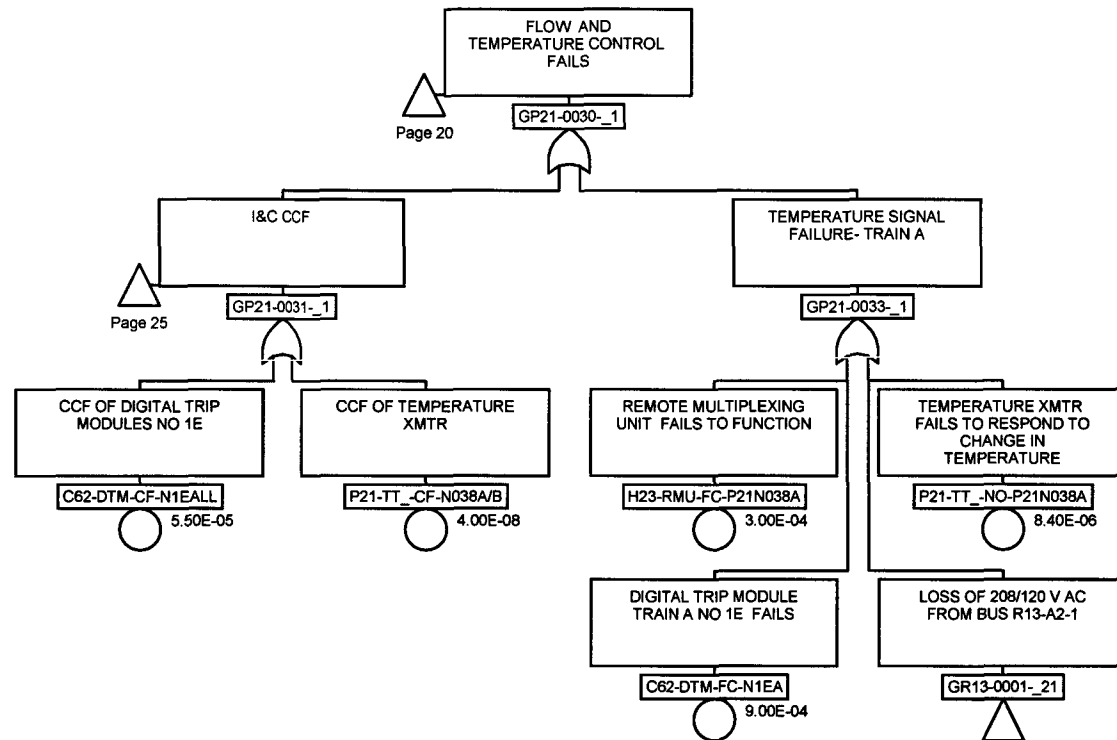


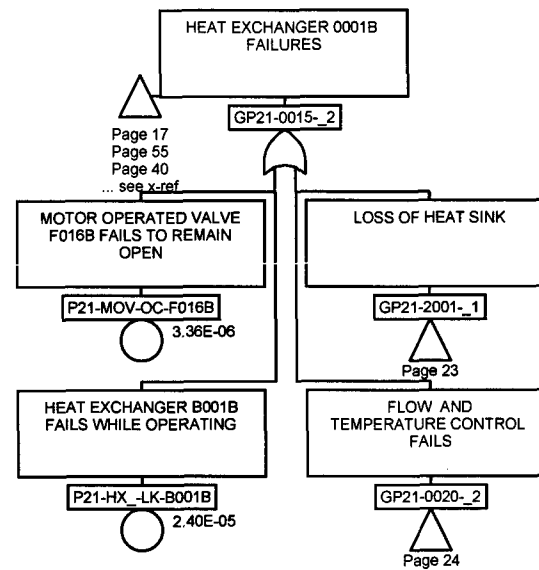


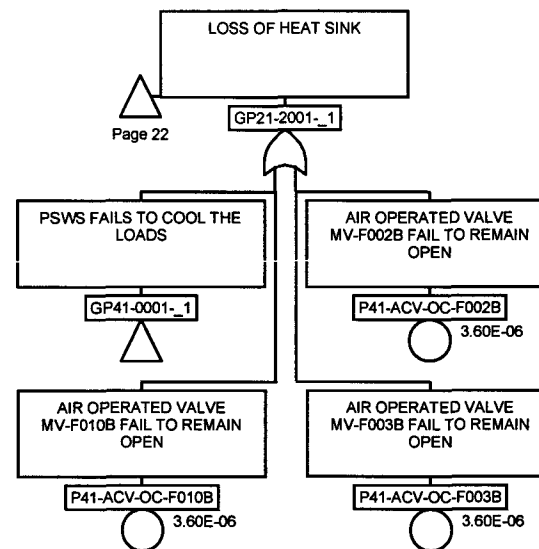


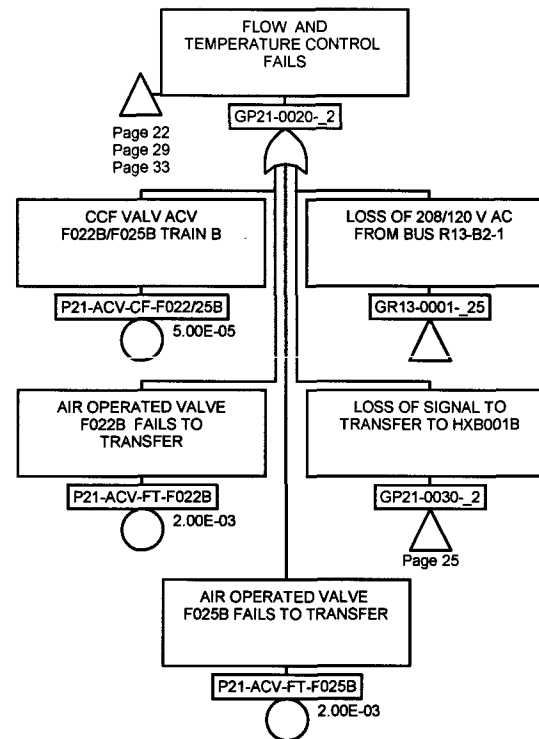


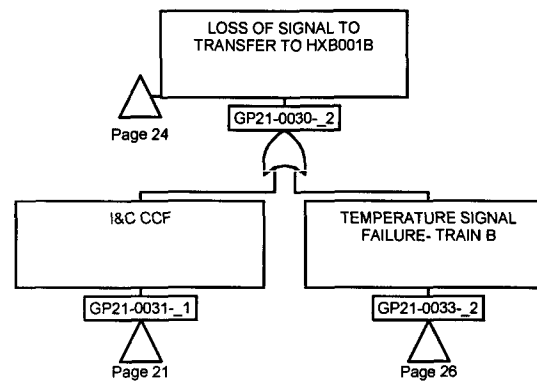


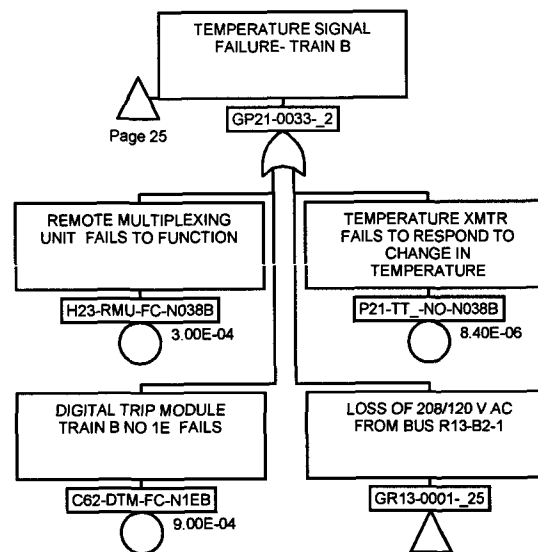


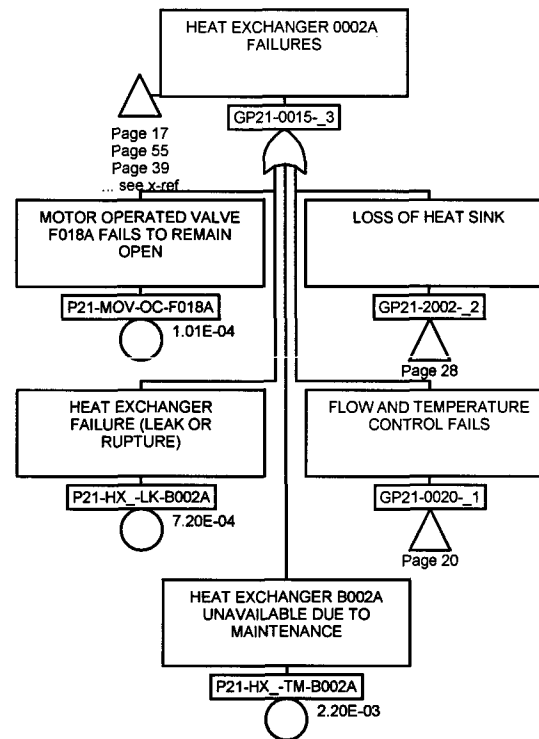


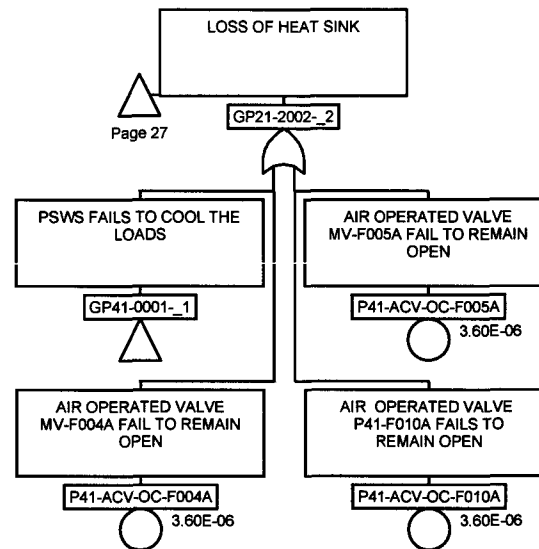




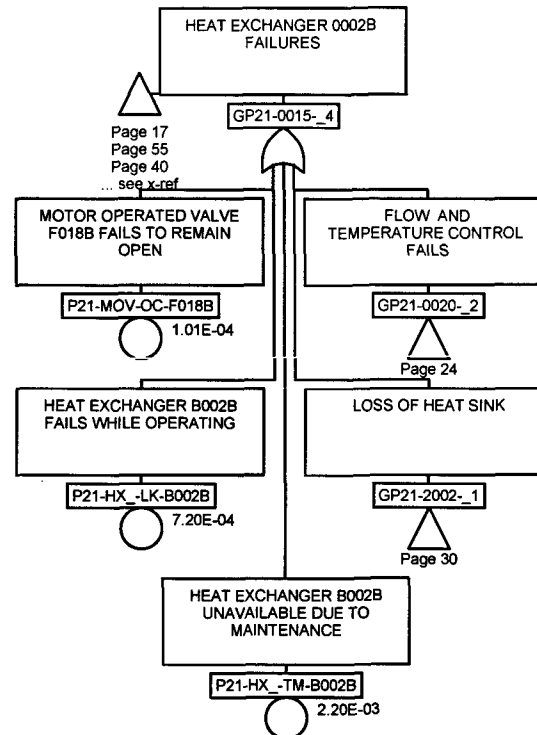


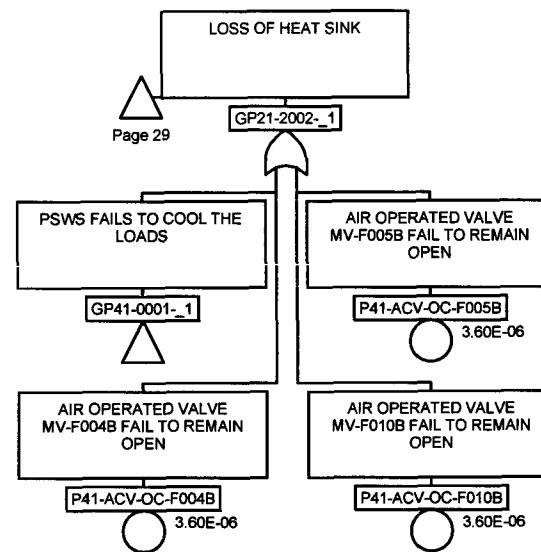


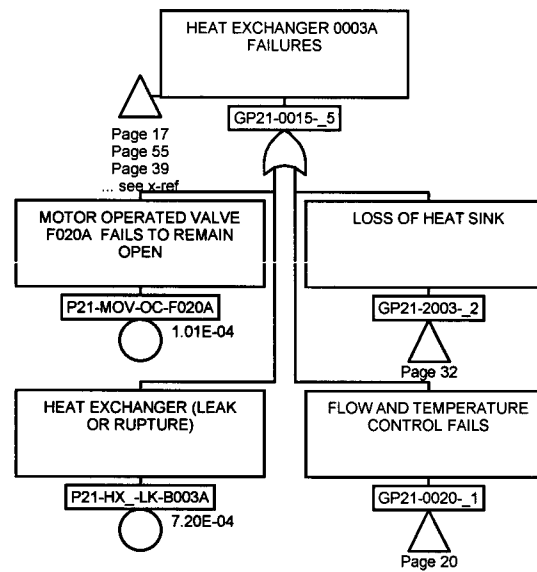


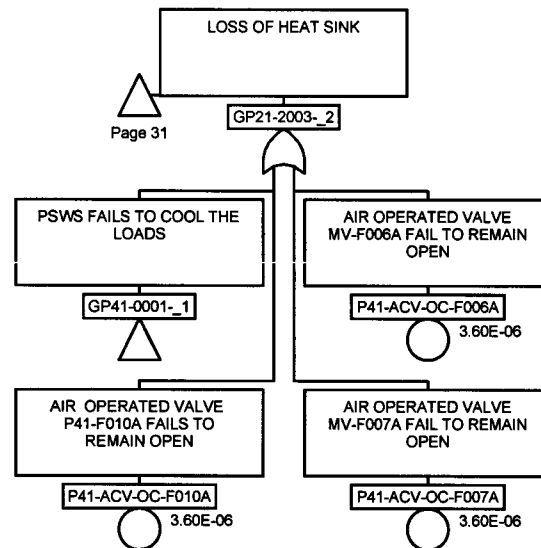


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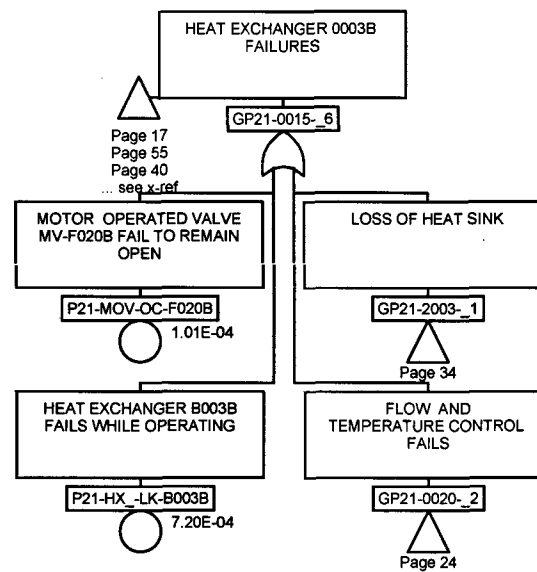


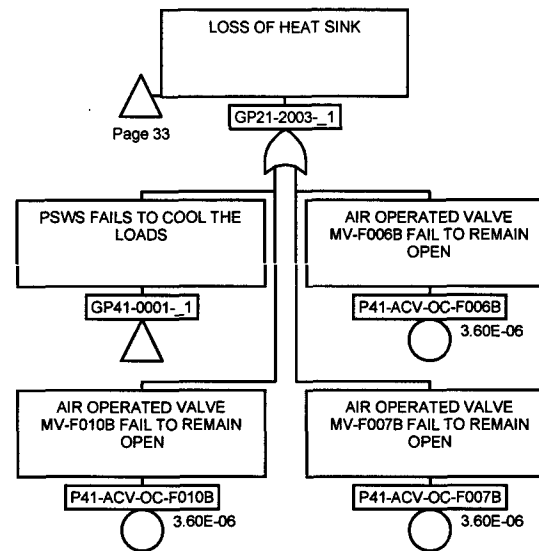






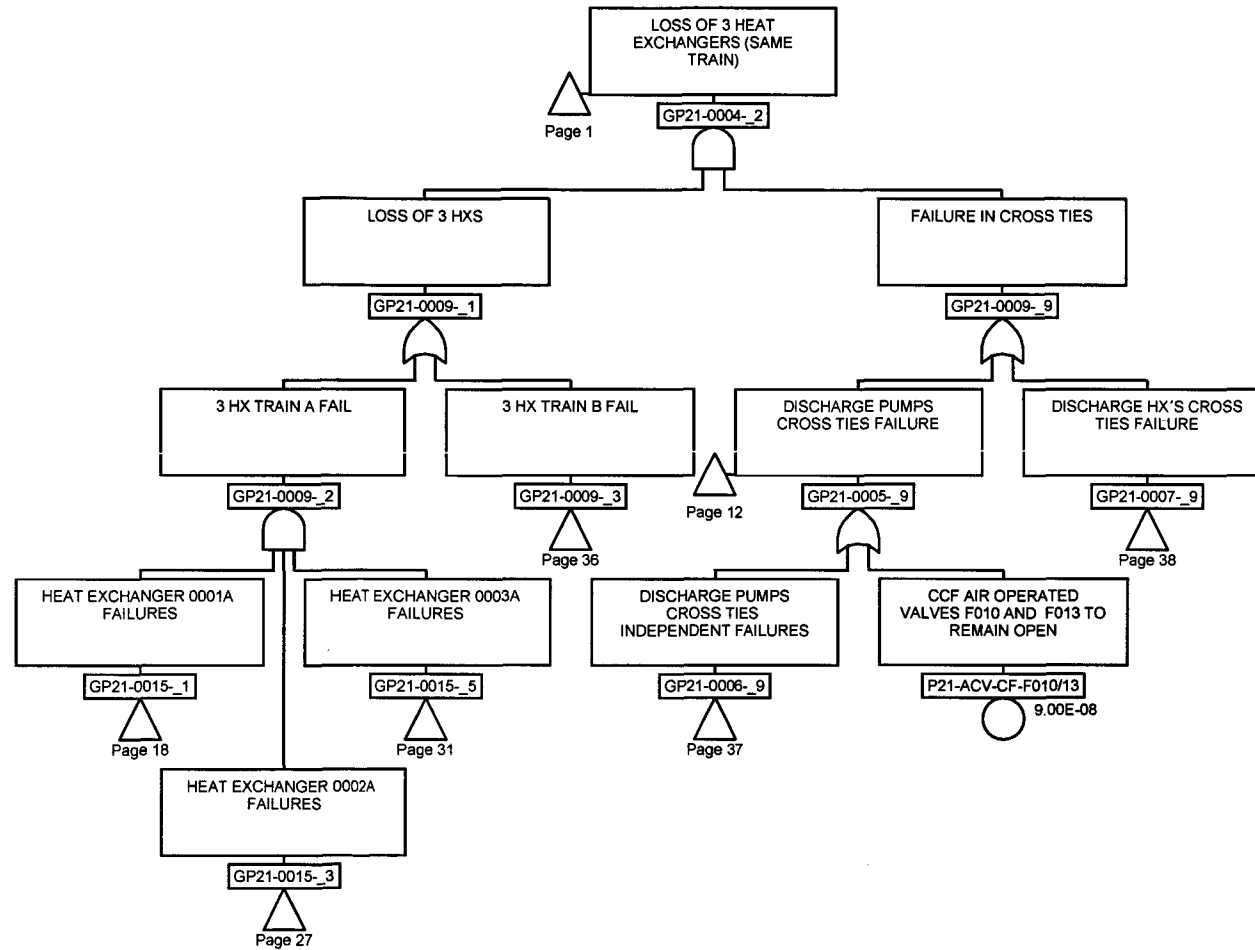
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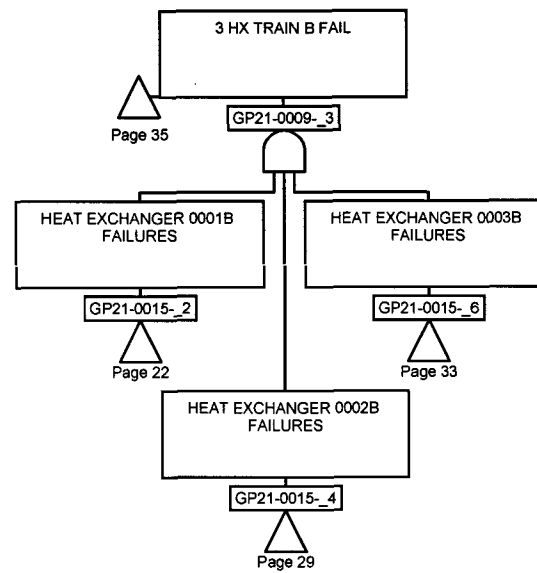


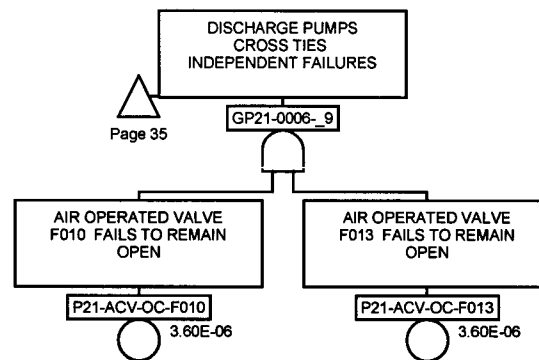


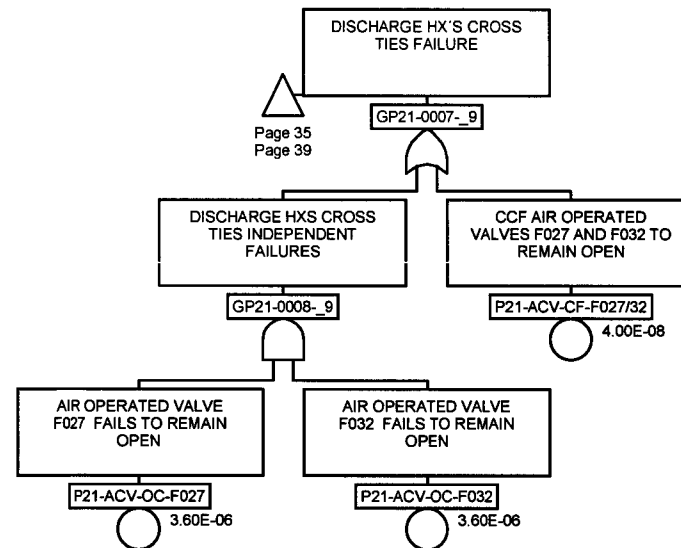
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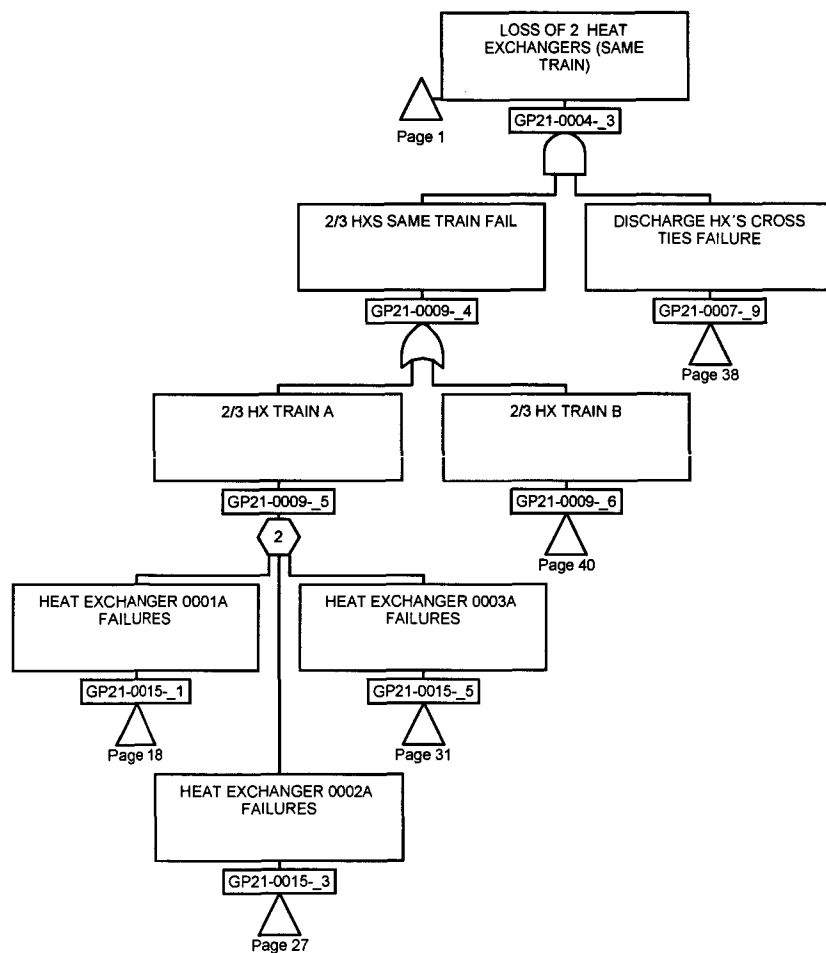
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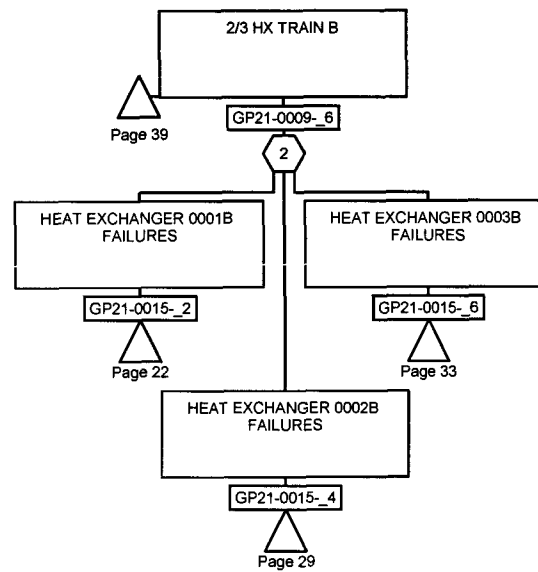


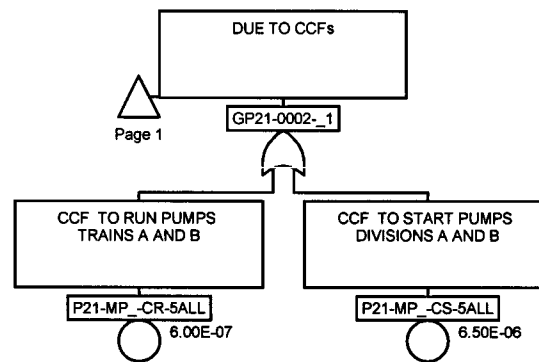




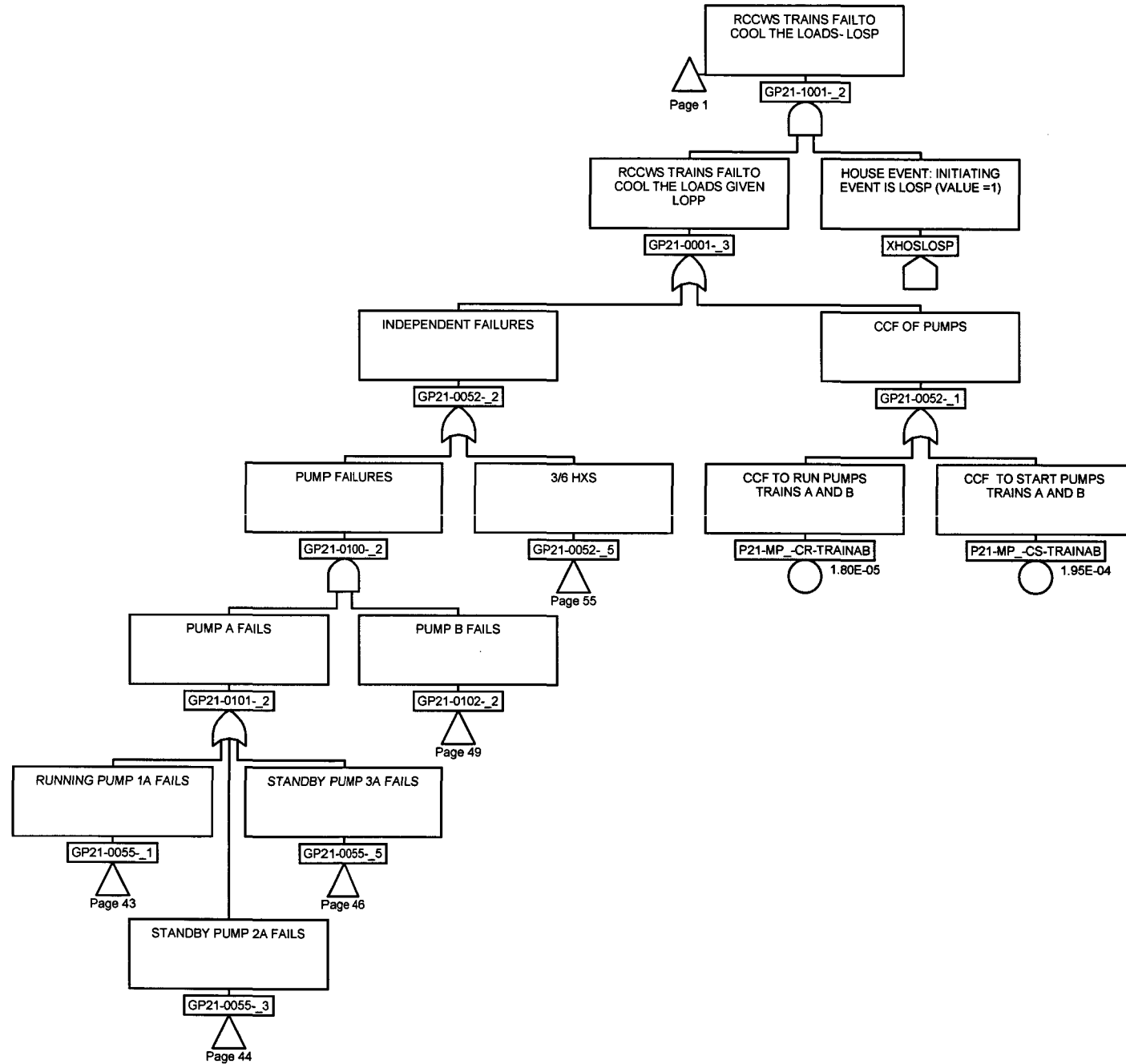


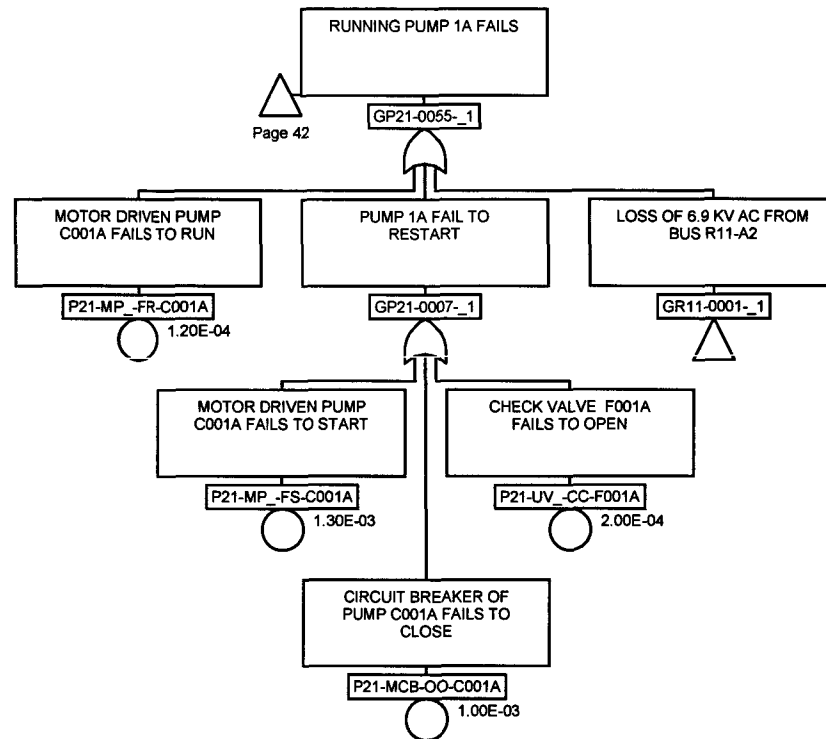


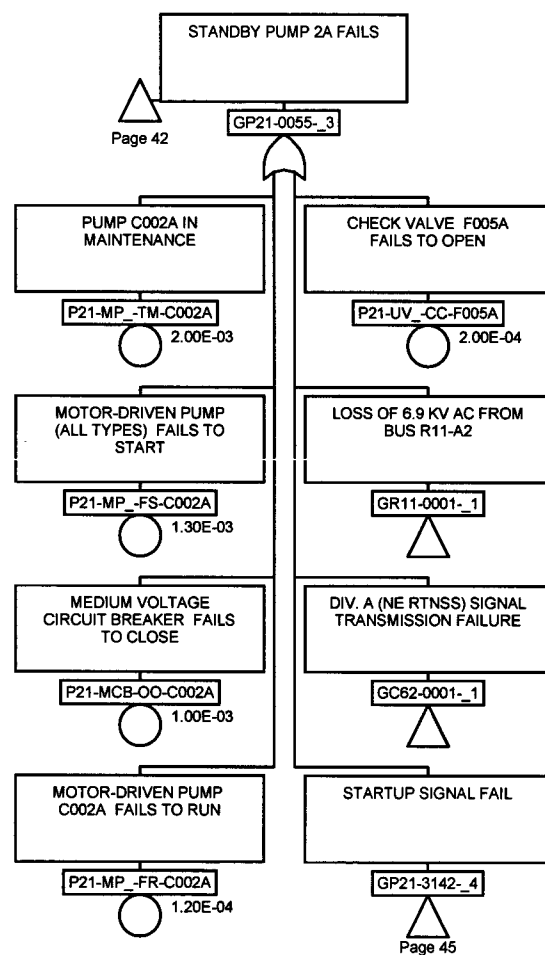


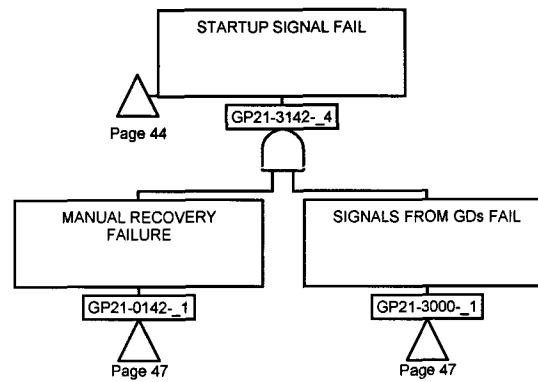


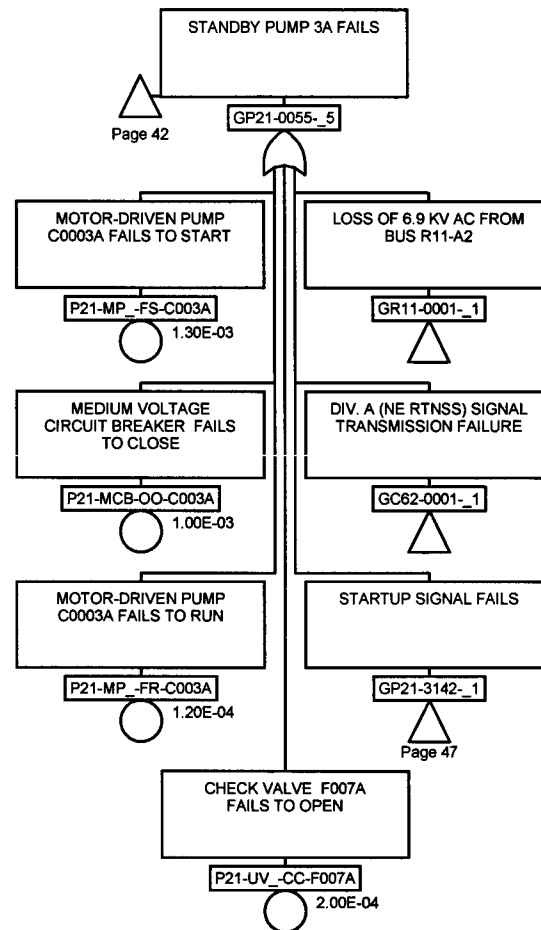
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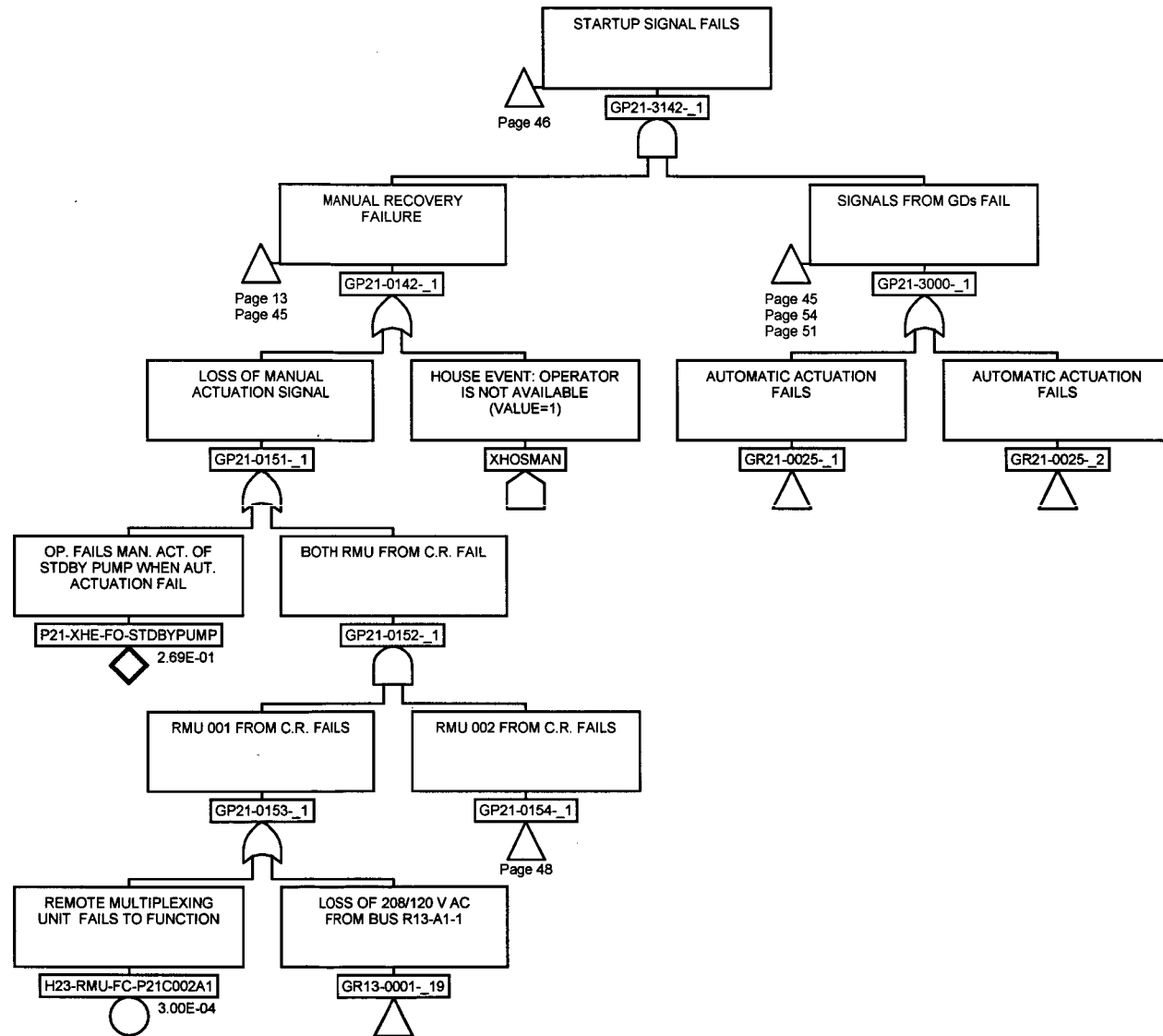


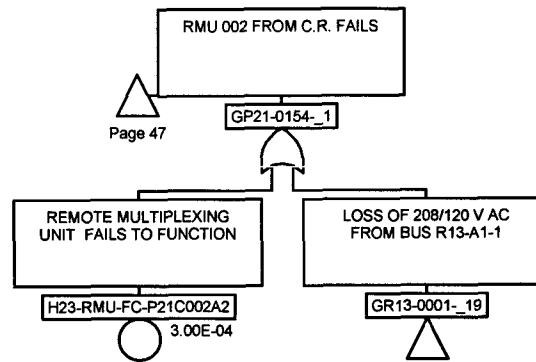


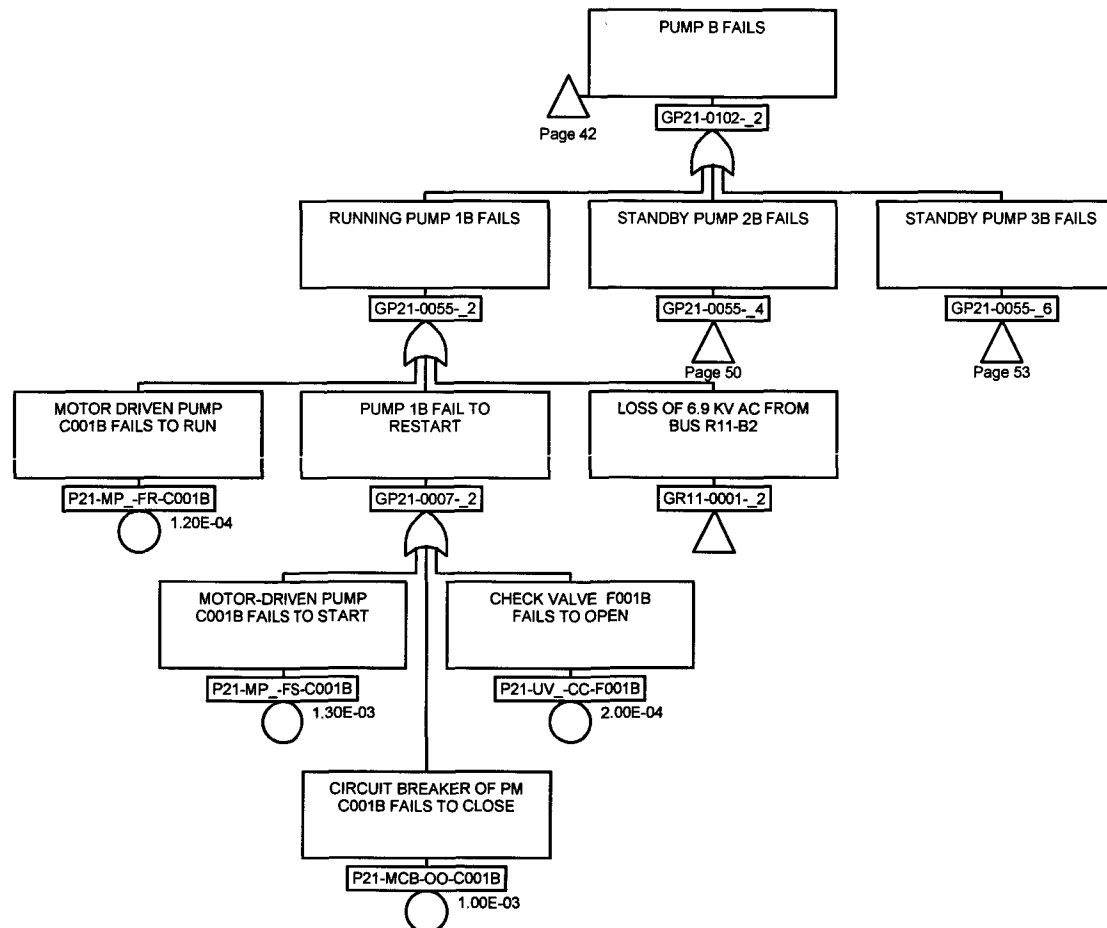


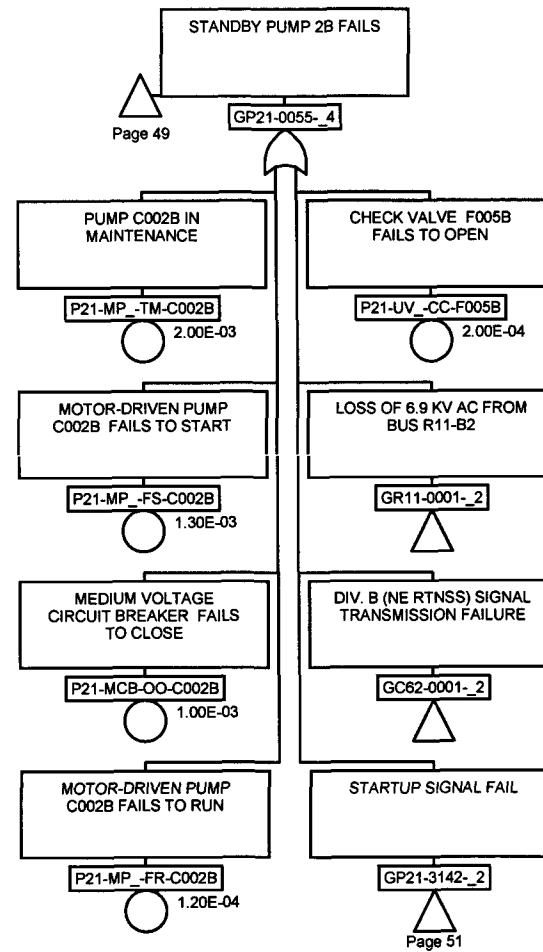




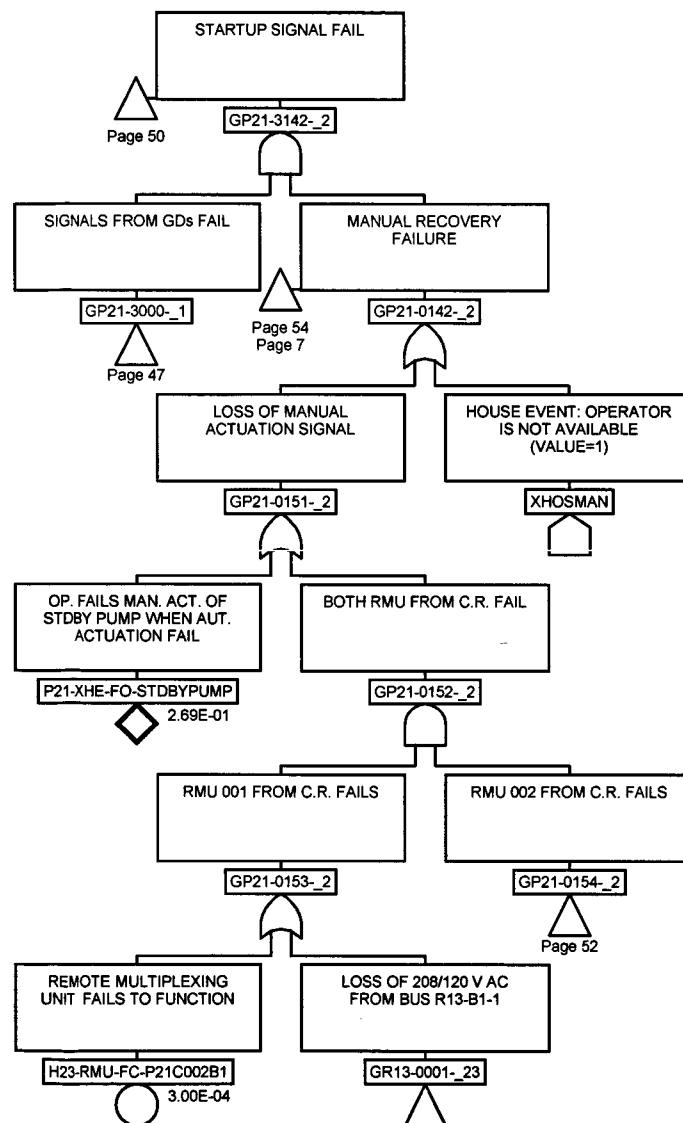


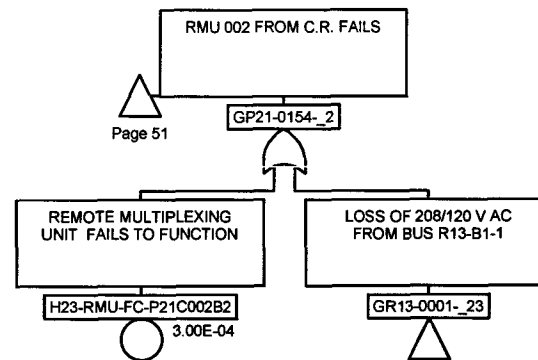


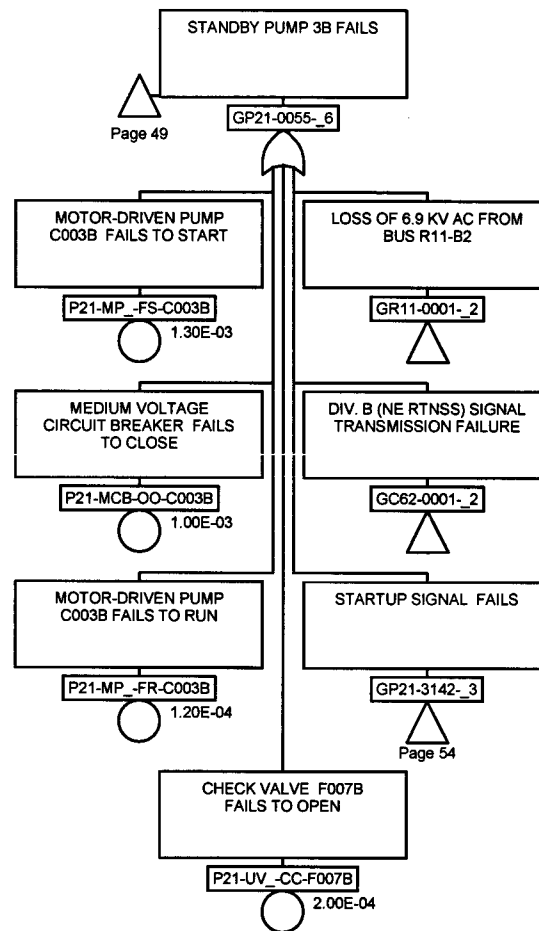


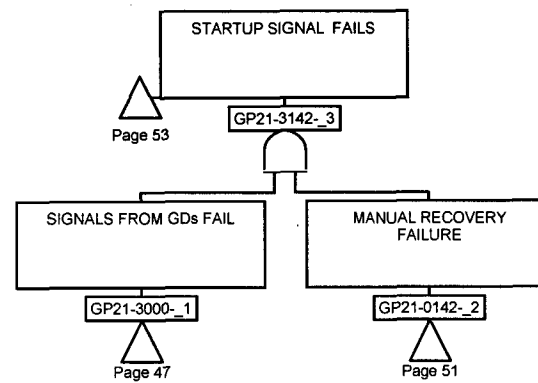


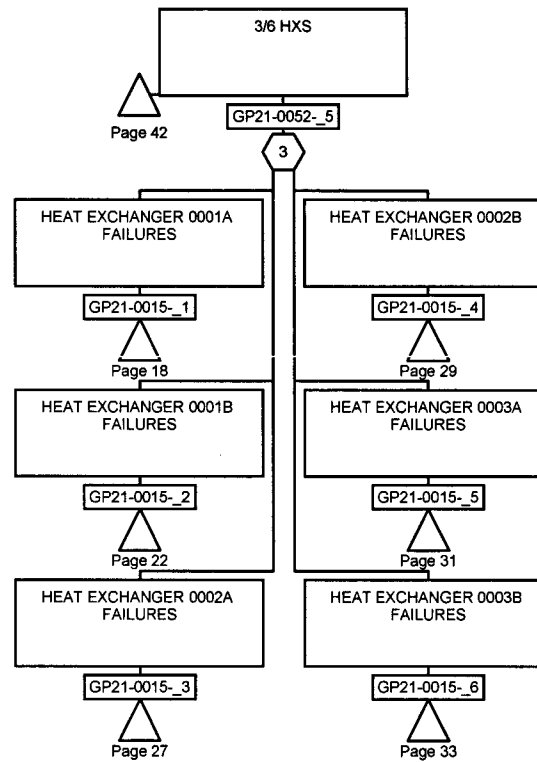
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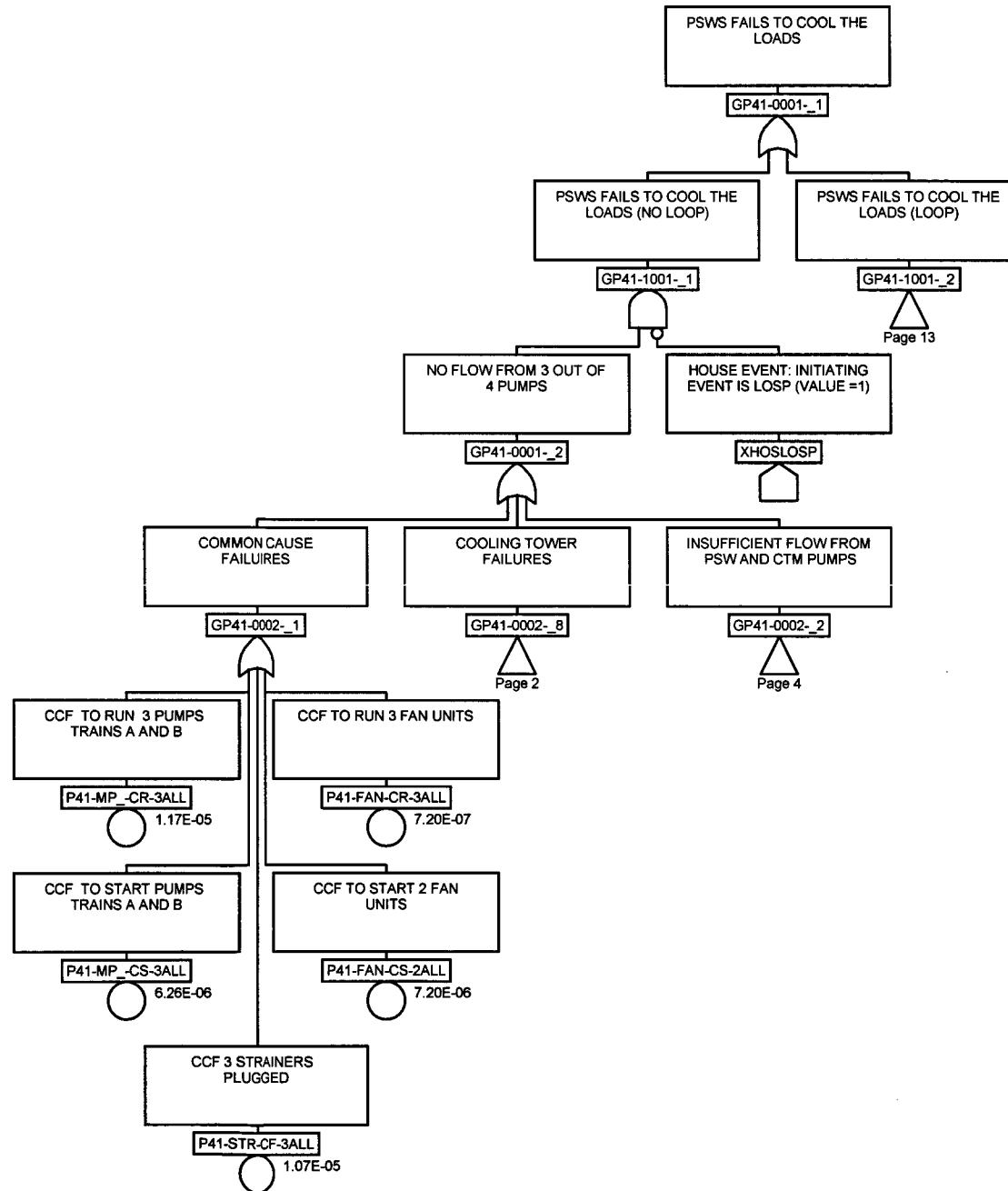


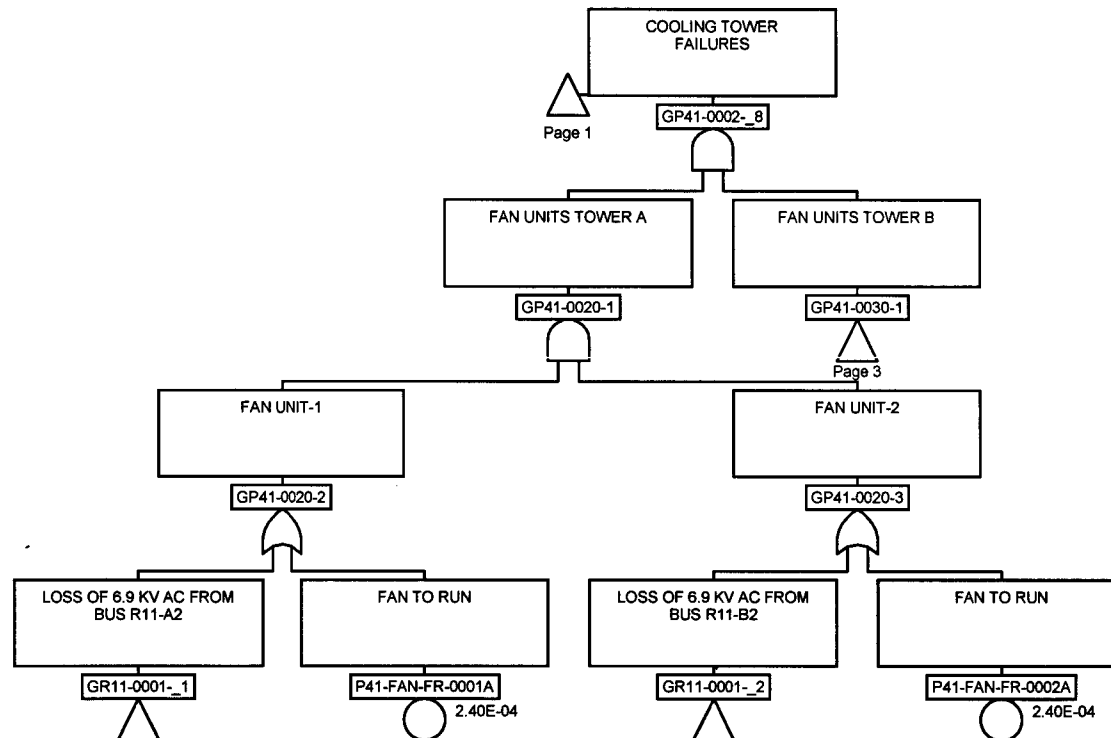
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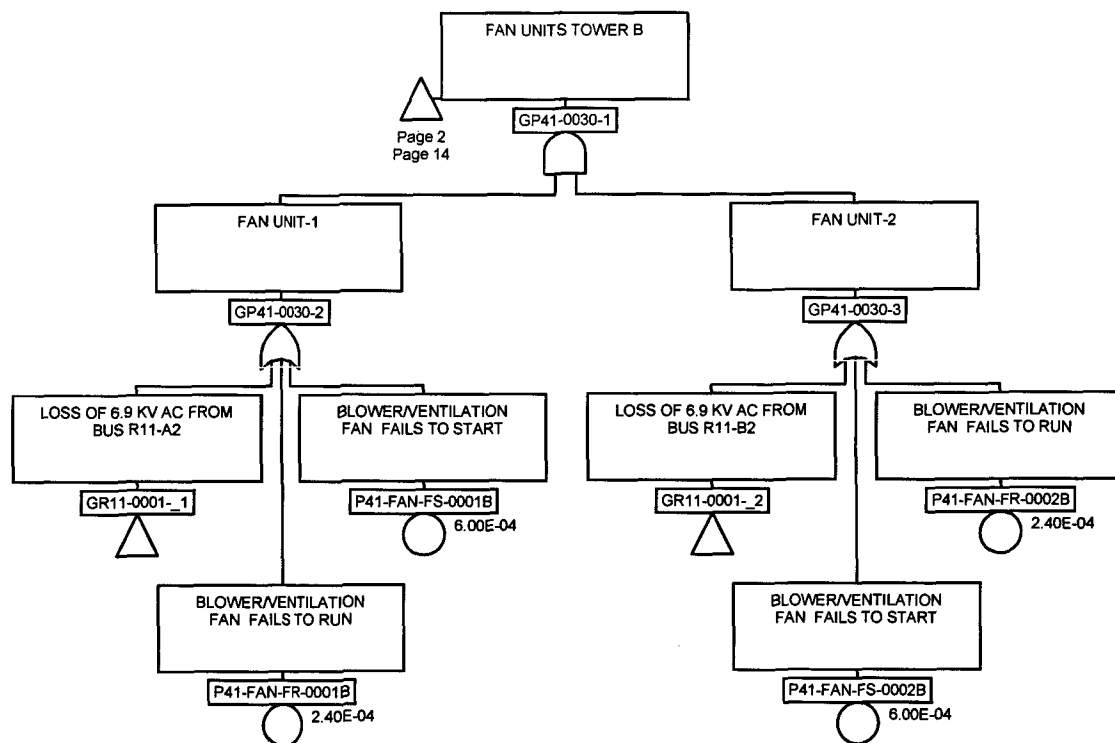
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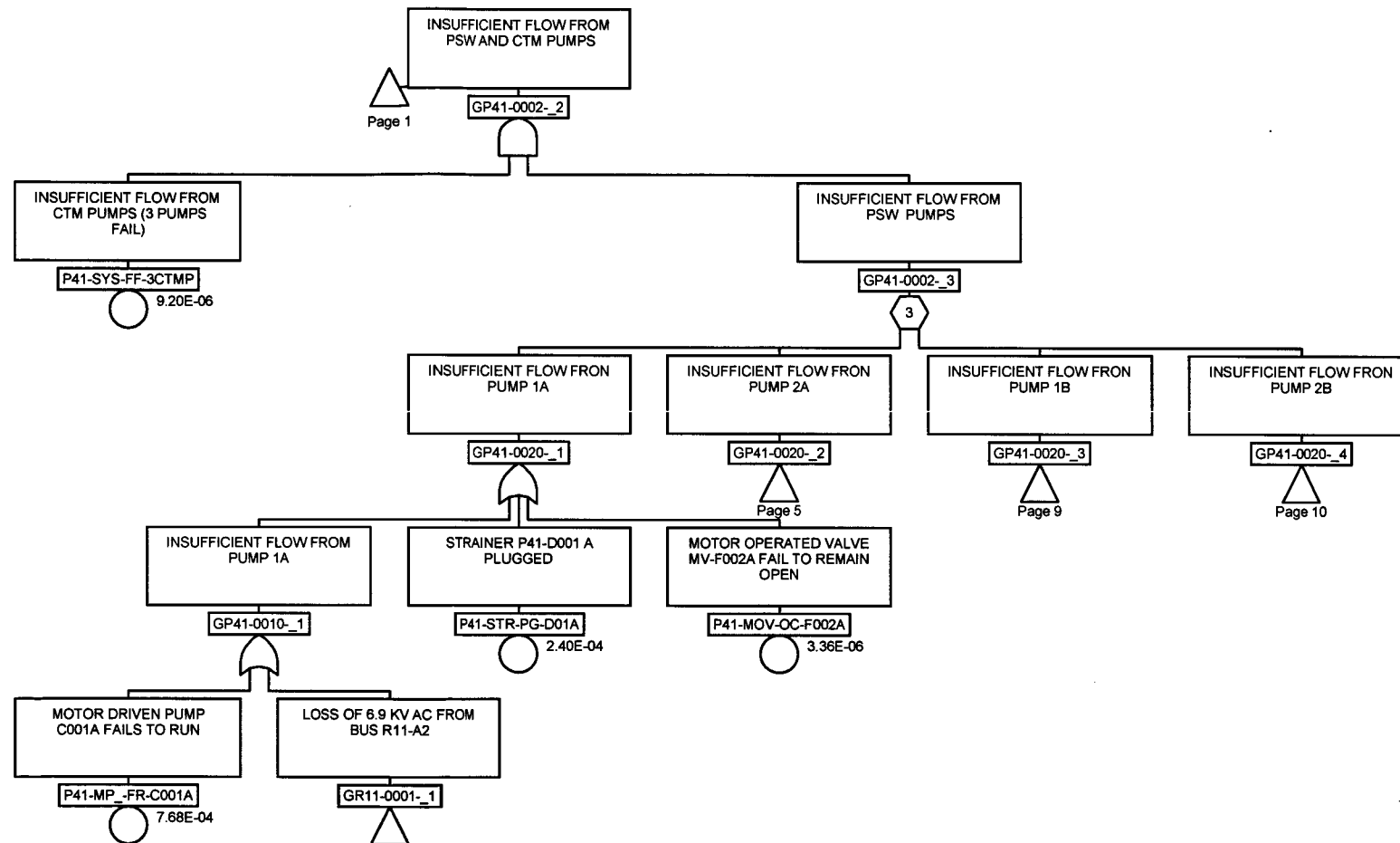
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| XHOSMAN | 51 | 3 | | | | | | |
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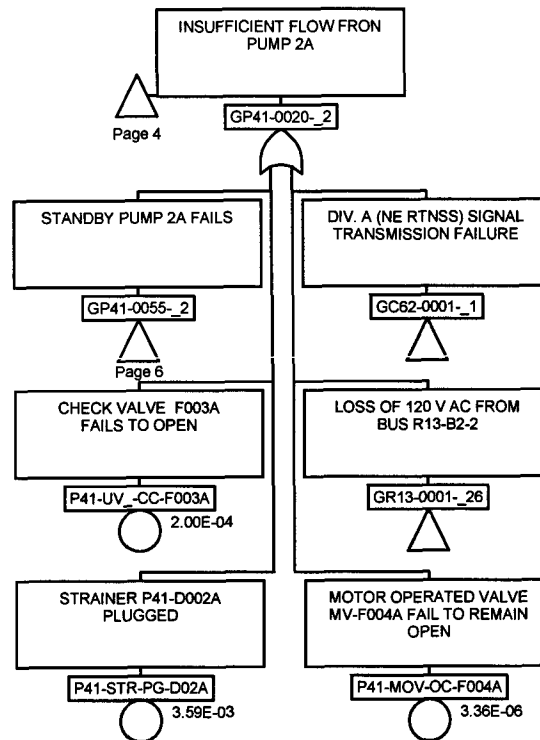
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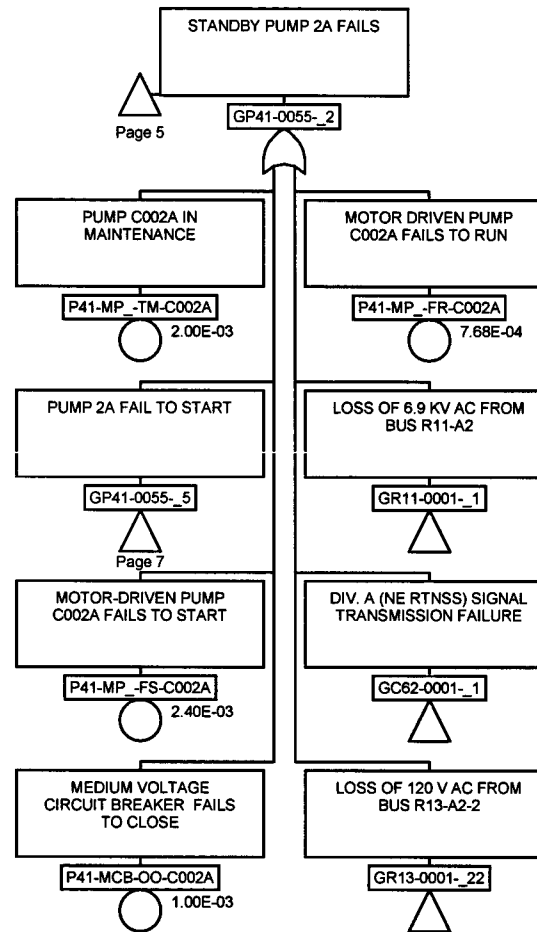


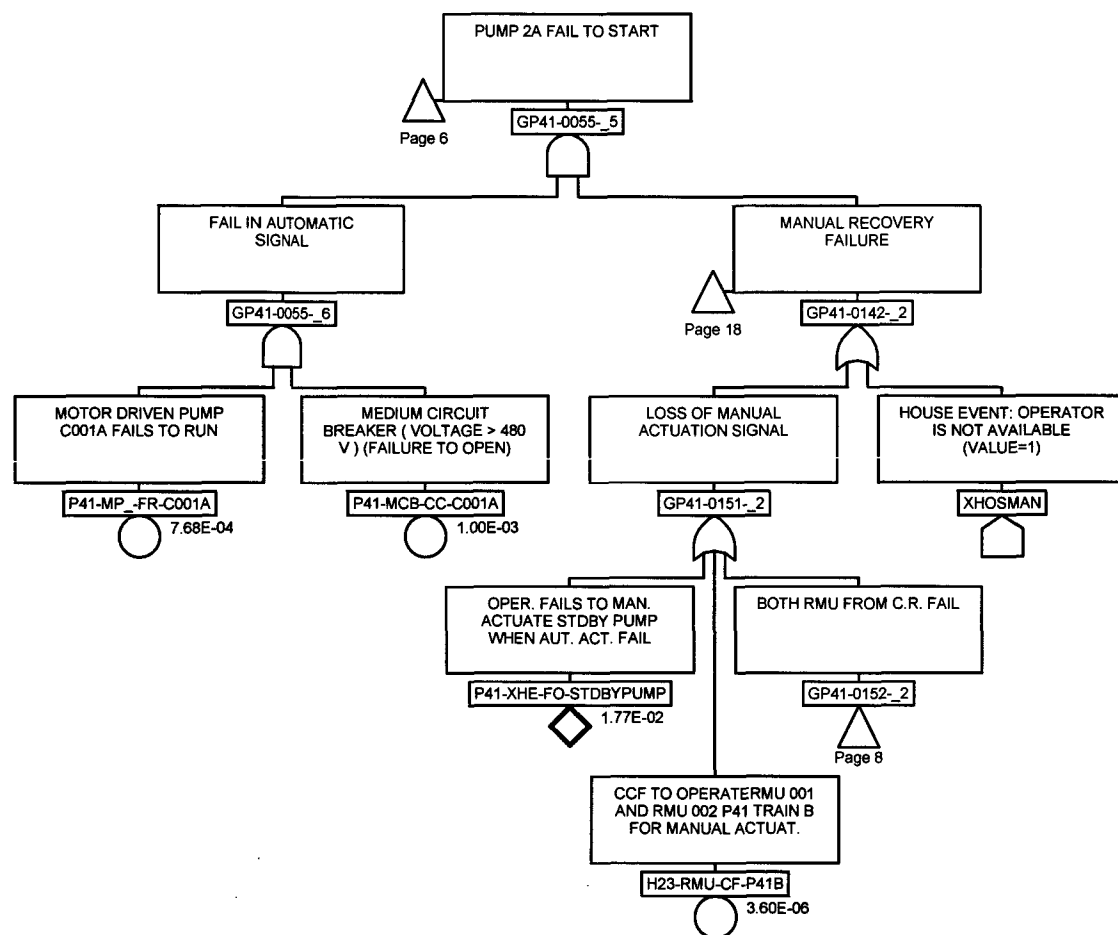


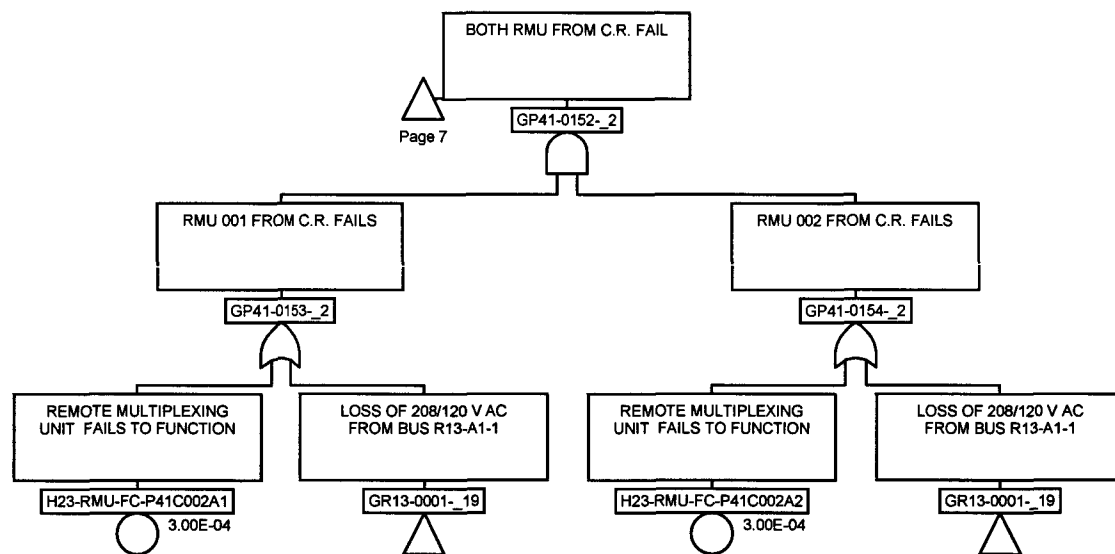


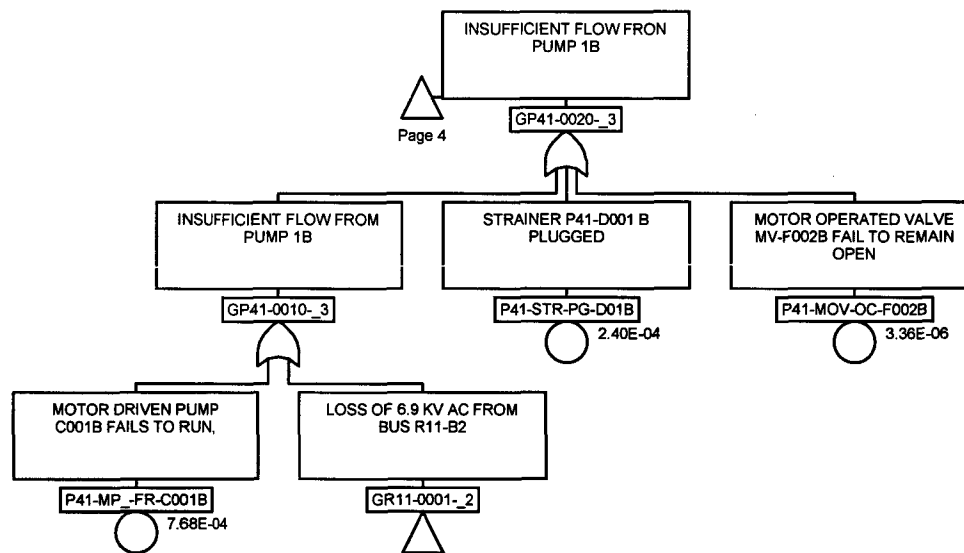


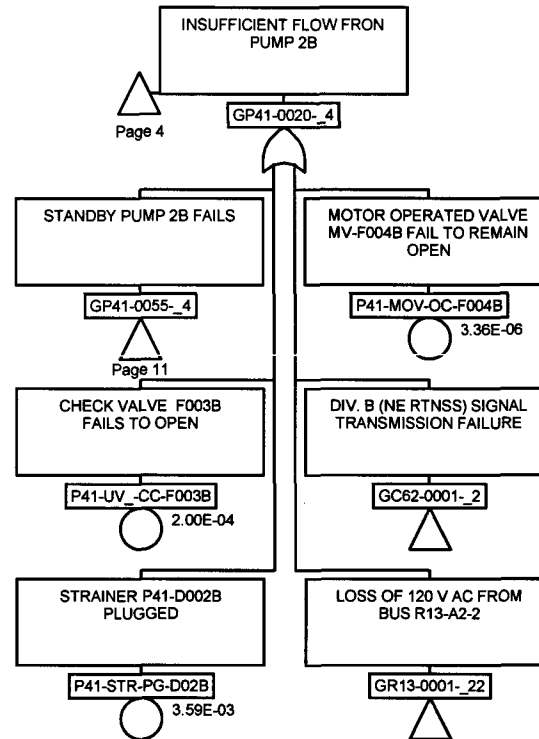


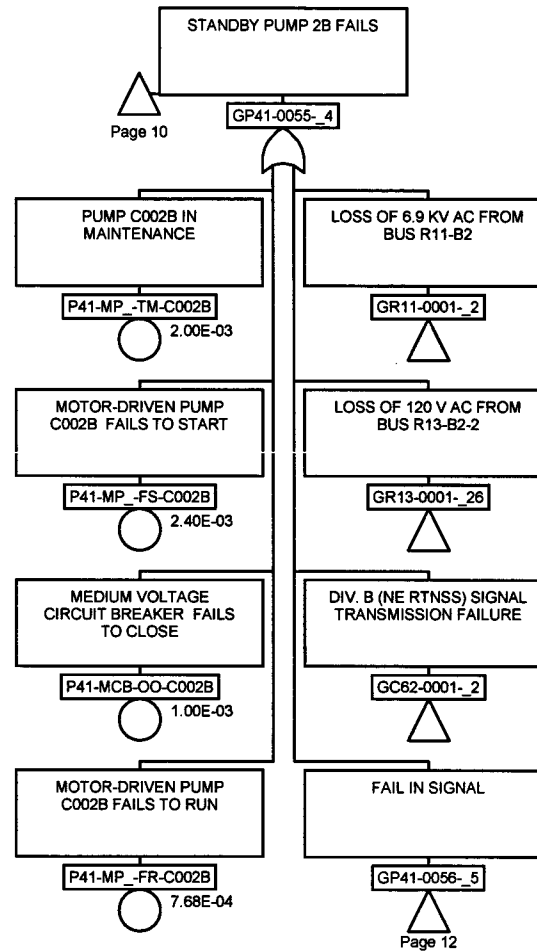


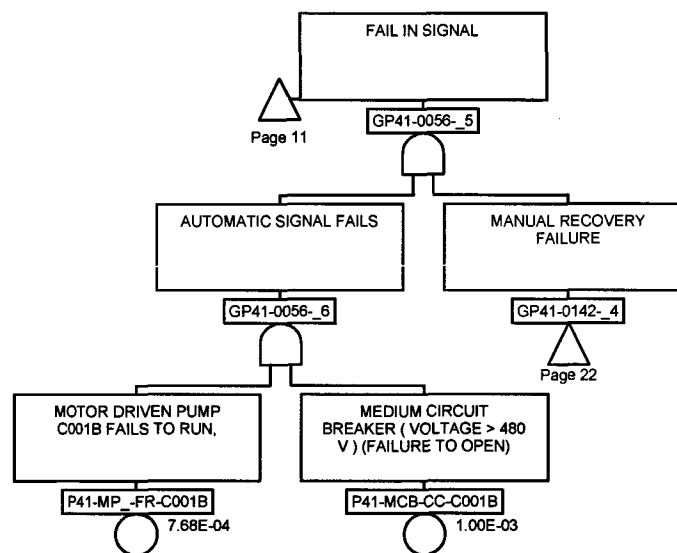


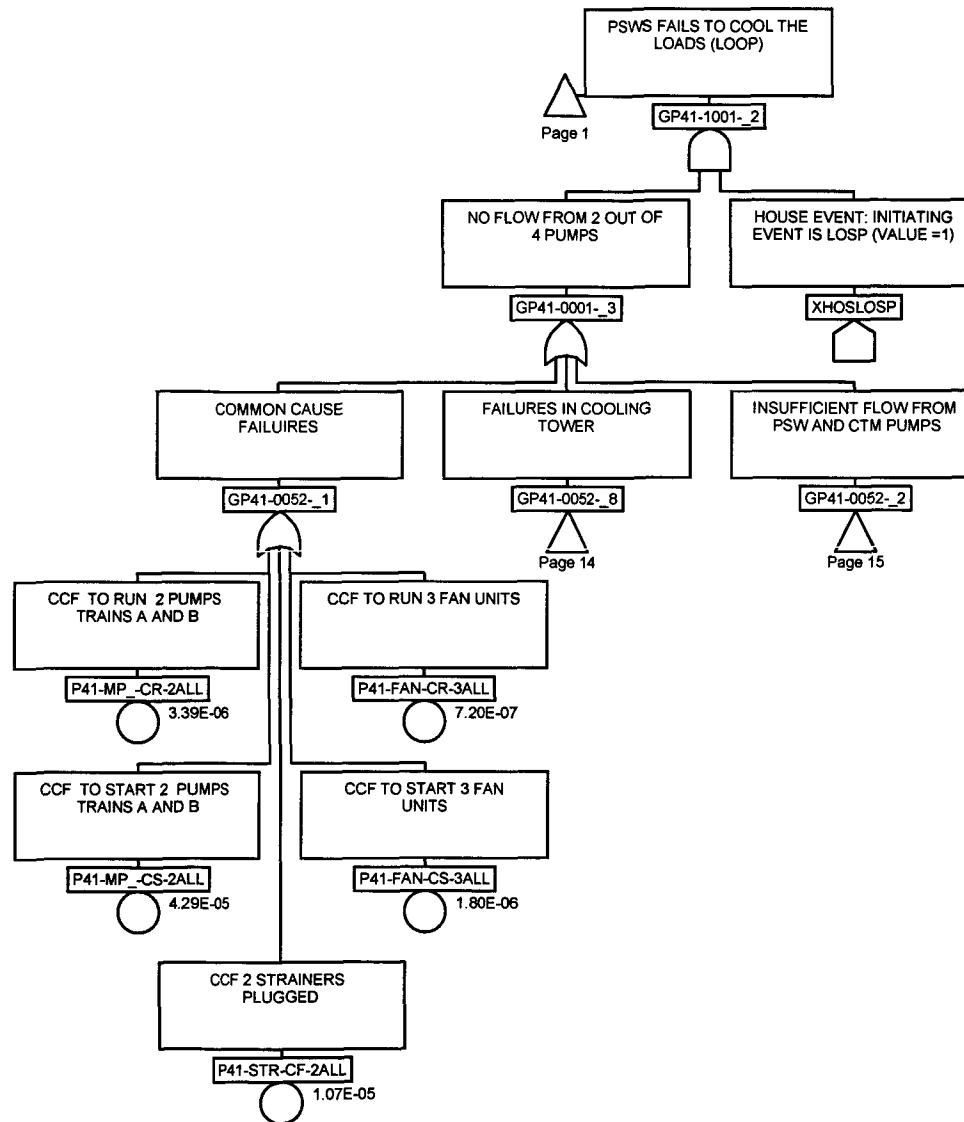


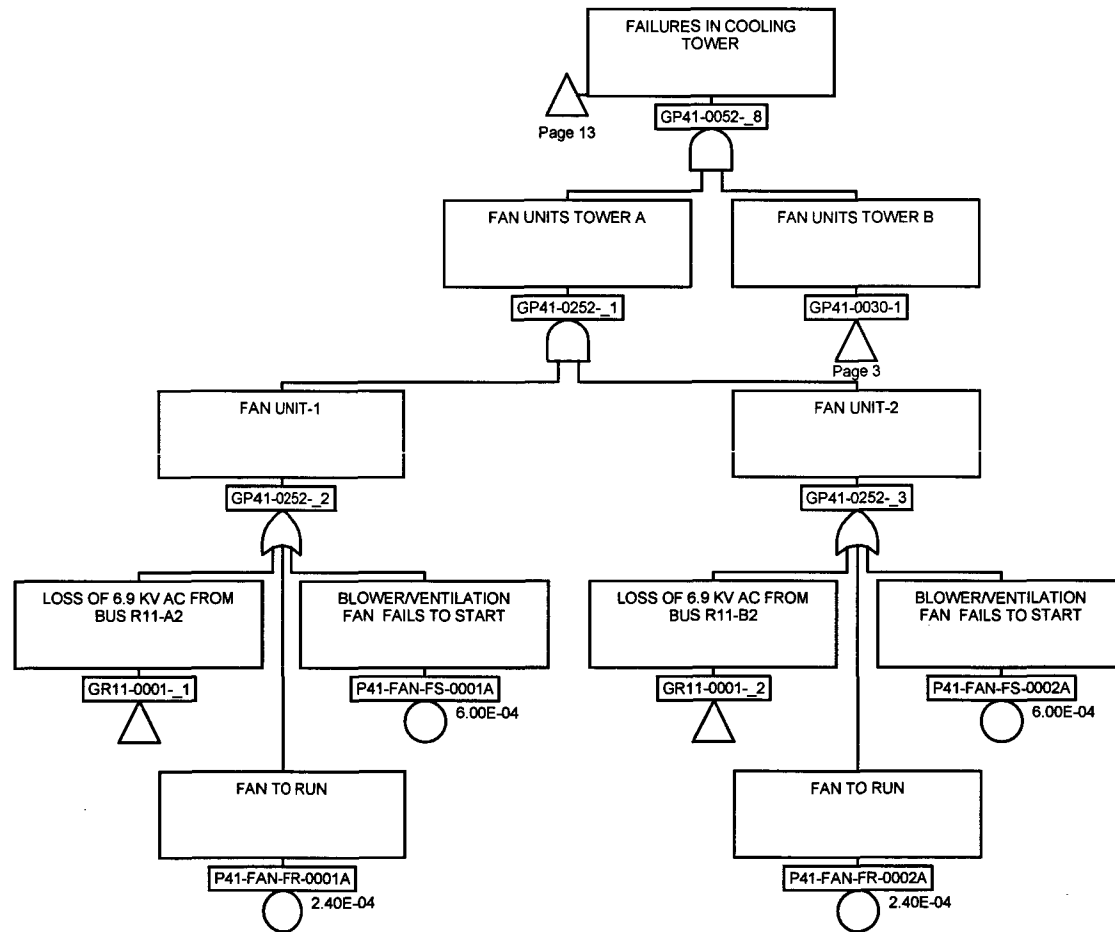


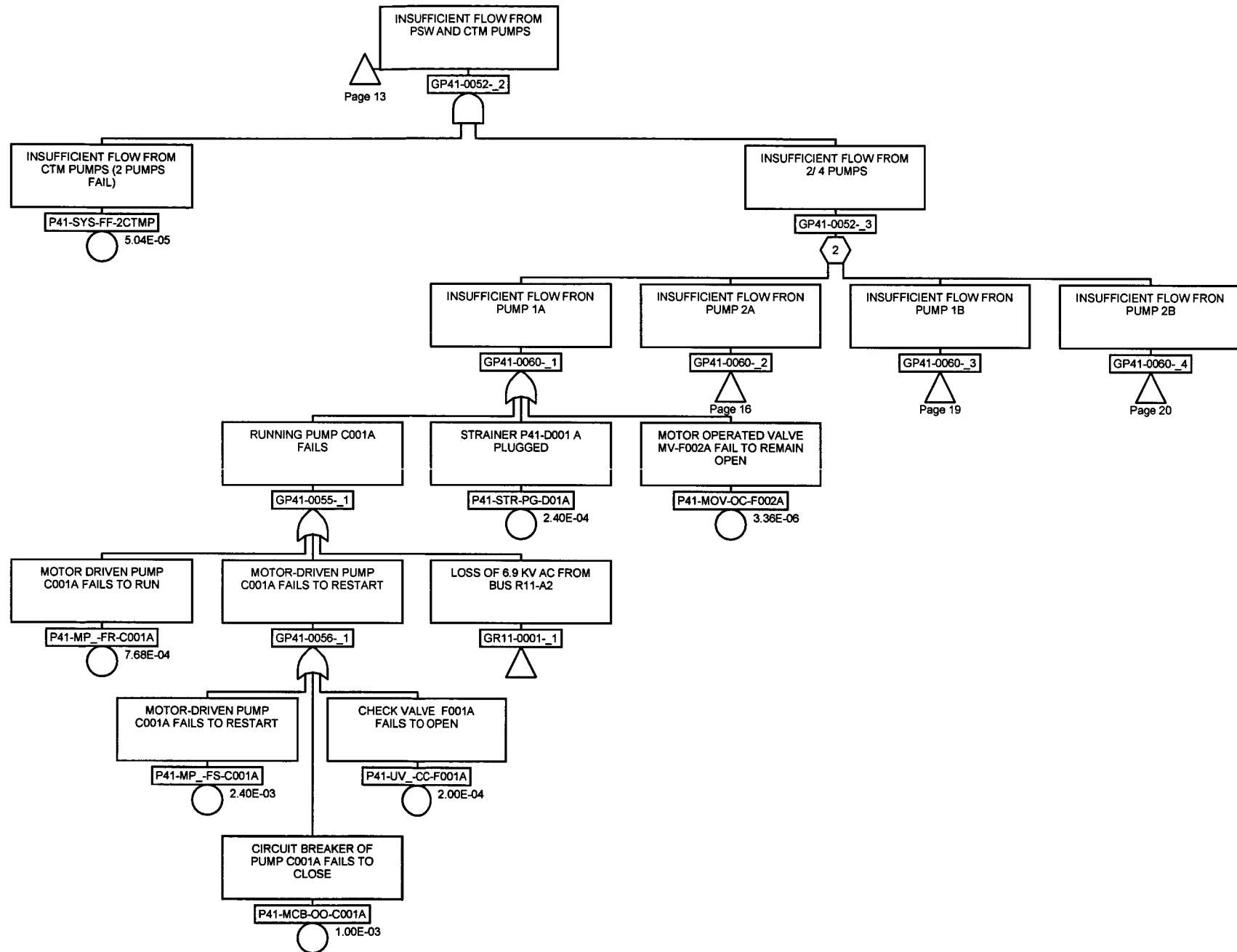


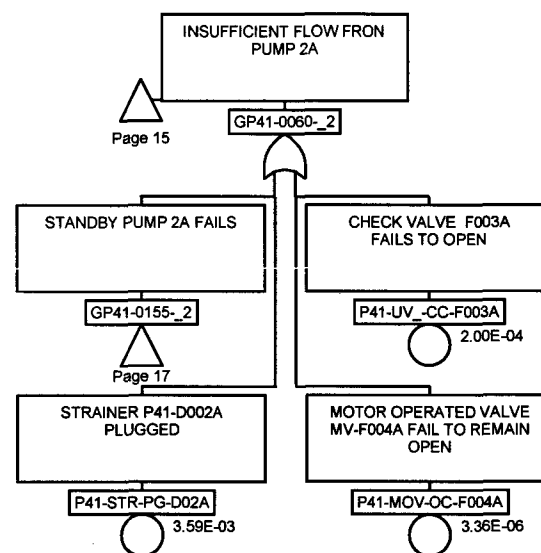


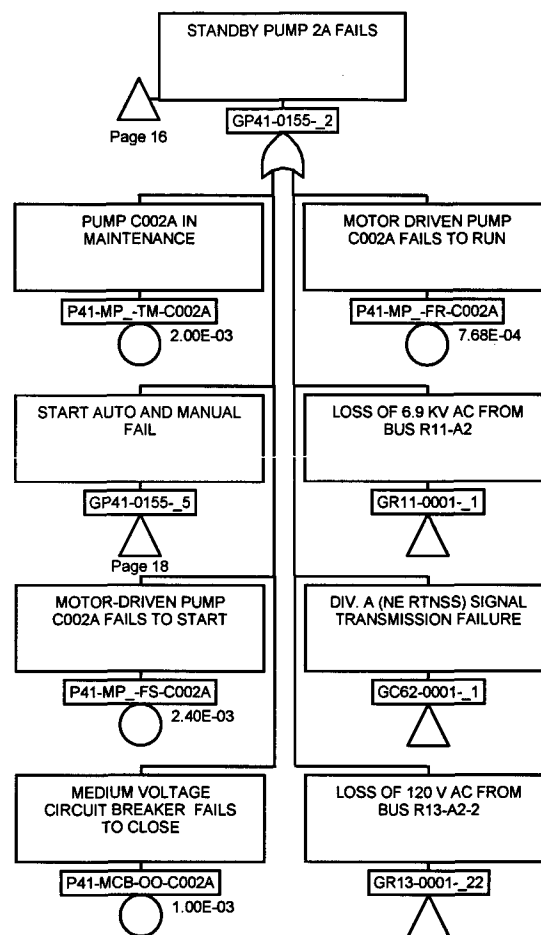


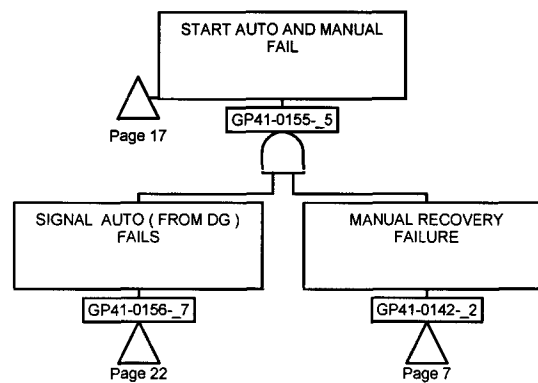


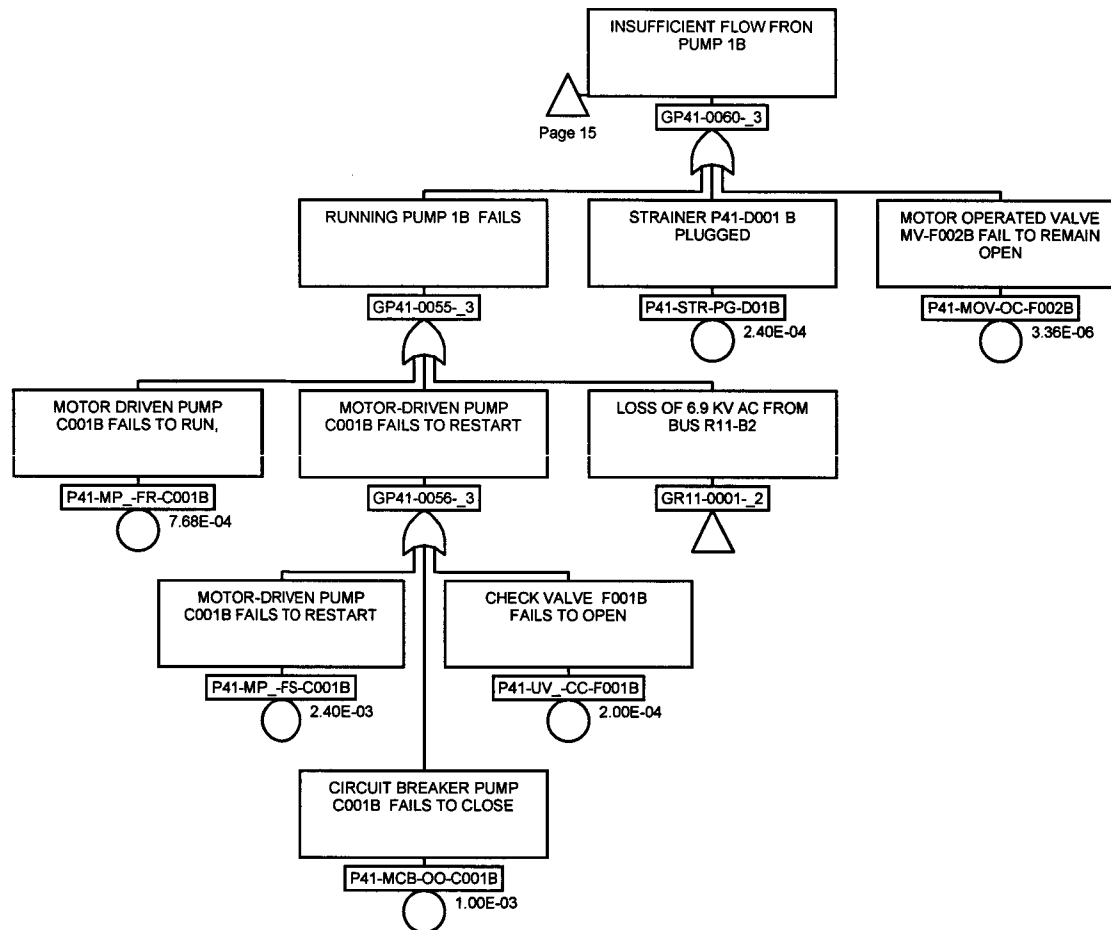


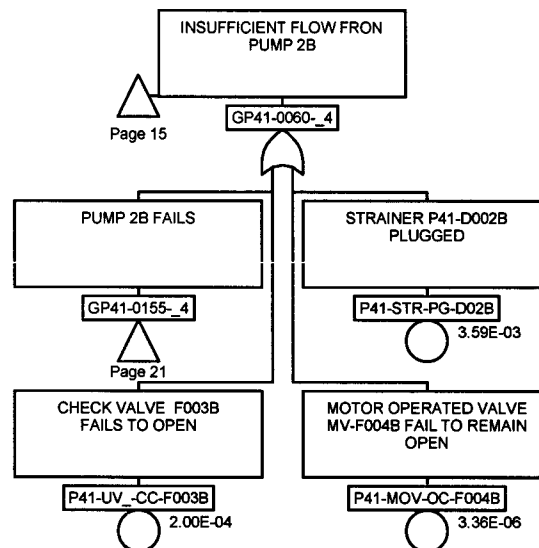


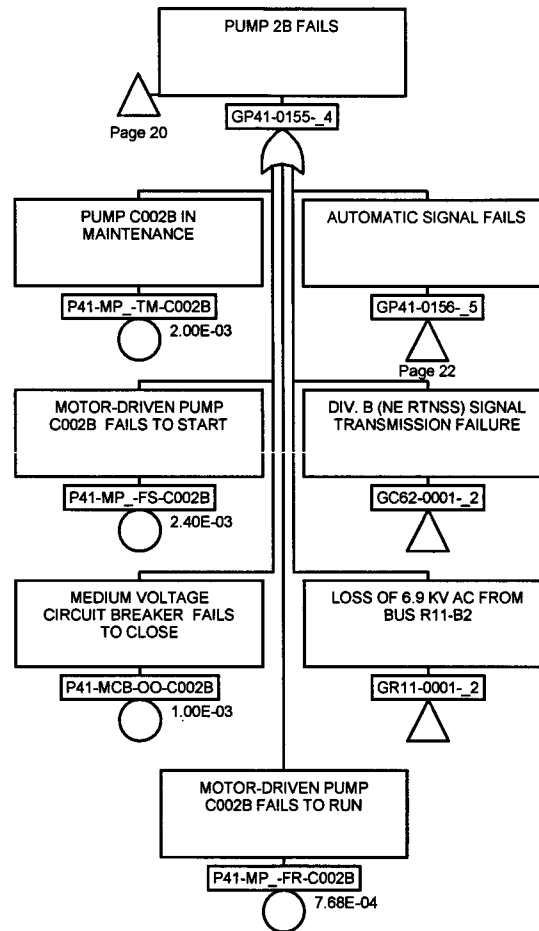


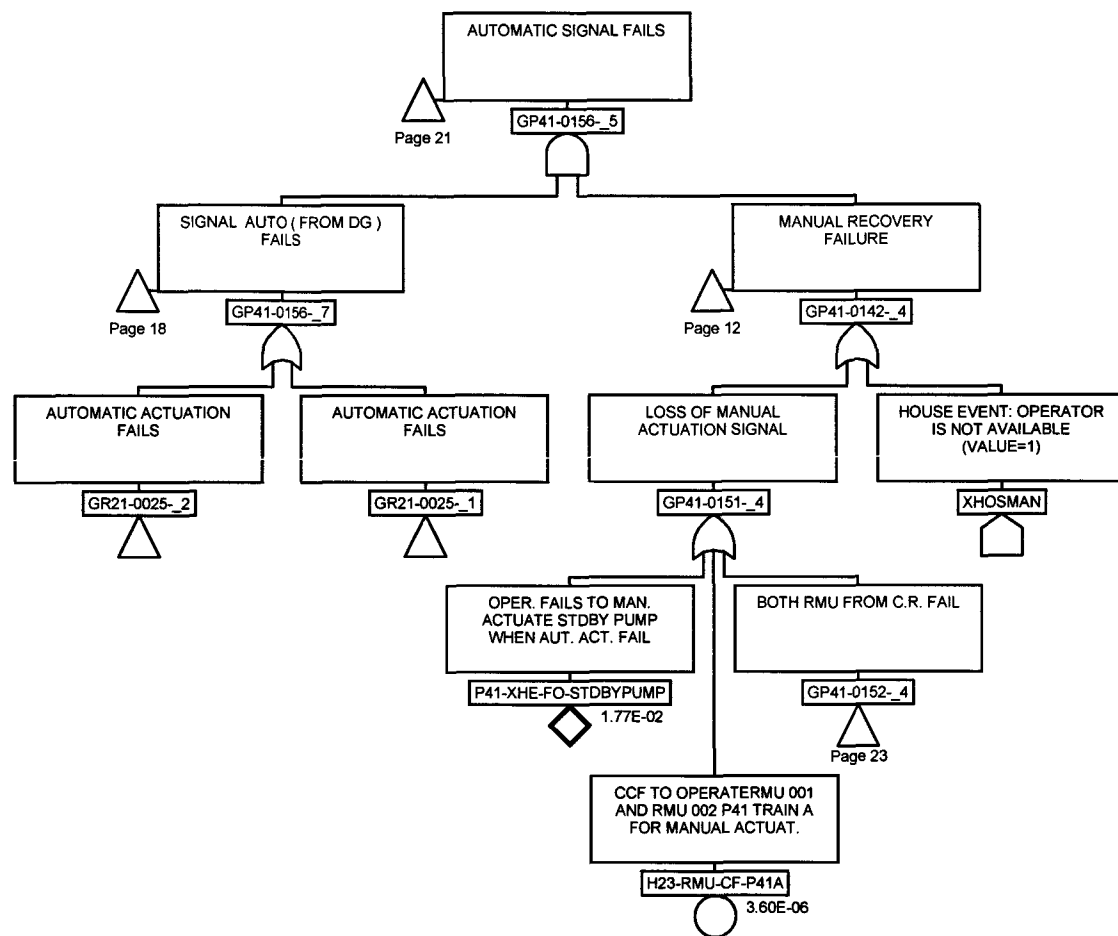








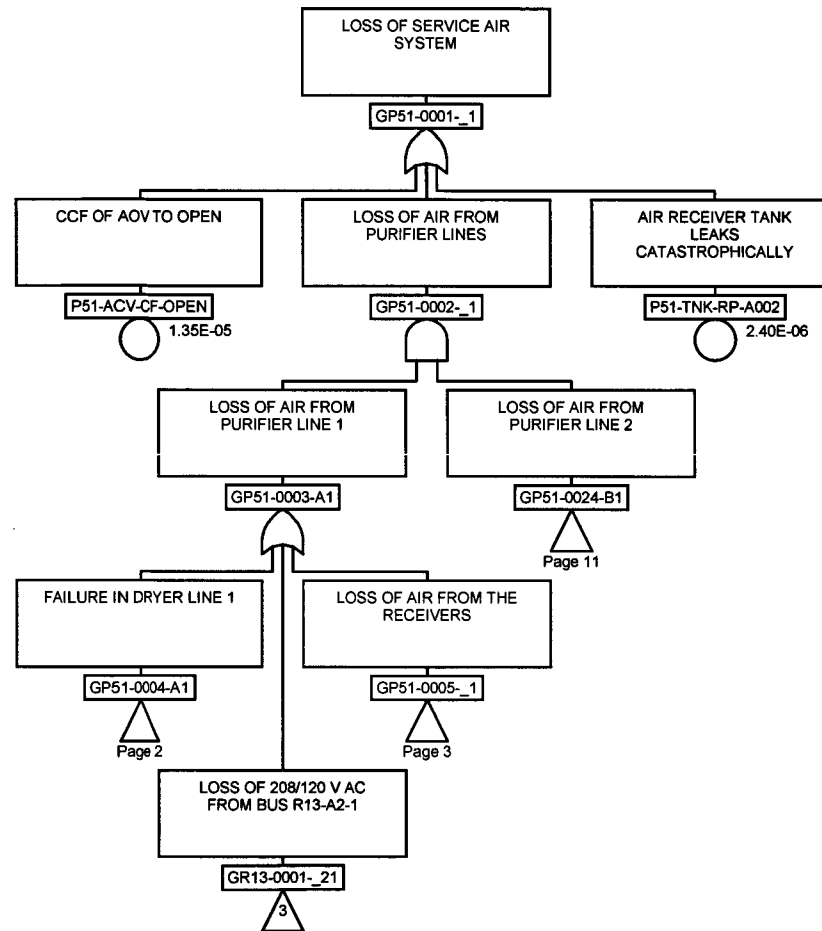




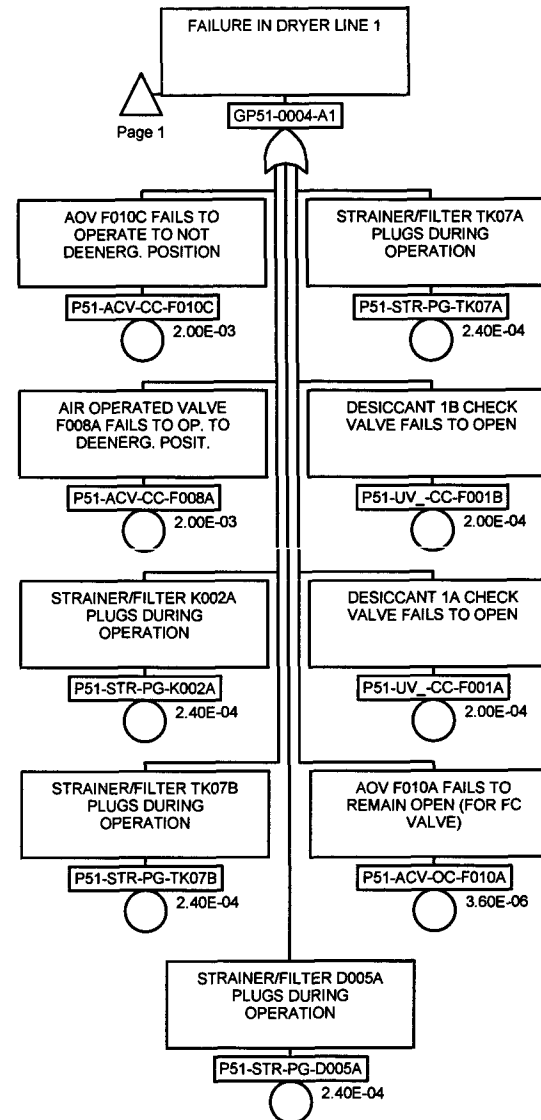
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| P41 PLANT SERVICE WATER SYSTEM (PSWS) | | | | | | X-PSW.caf | Appendix B.4.11 | Page 24 |

| Name | Page | Zone | Name | Page | Zone | Name | Page | Zone |
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| P41-MP -FS-C002A | 17 | 1 | | | | | | |
| P41-MP -FS-C002B | 11 | 1 | | | | | | |
| P41-MP -FS-C002B | 21 | 1 | | | | | | |
| P41-MP -TM-C002A | 6 | 1 | | | | | | |
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| P41-MP -TM-C002B | 11 | 1 | | | | | | |
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| P41-XHE-FO-STDBYPUMP | 22 | 3 | | | | | | |
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| XHOSLOSP | 13 | 4 | | | | | | |
| XHOSMAN | 7 | 4 | | | | | | |
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| P41 PLANT SERVICE WATER SYSTEM (PSWS) | | | | | | X-PSW.caf | Appendix B.4.11 | Page 25 |

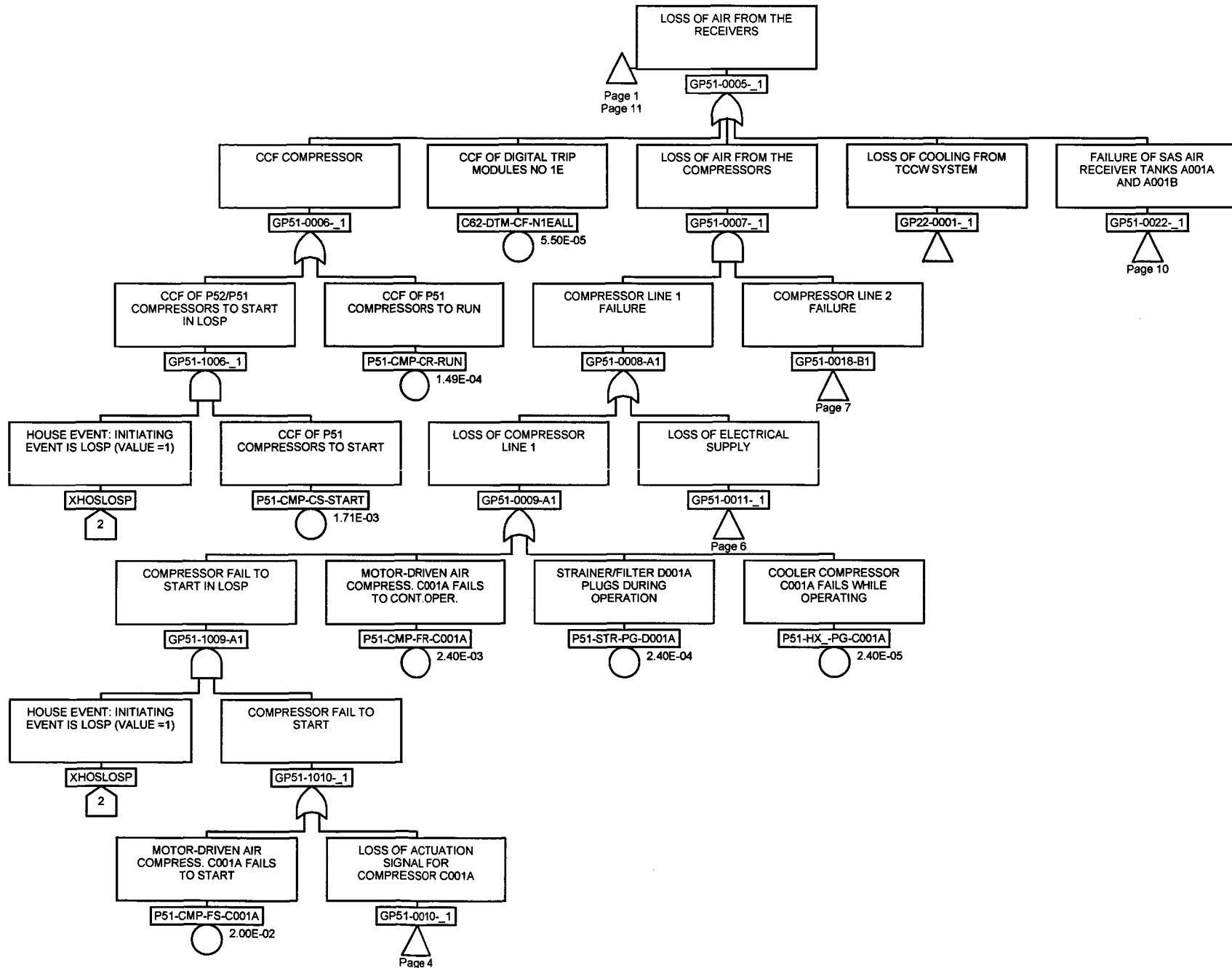
Appendix B.4.12
Instrument Air System and Service Air System Fault Tree

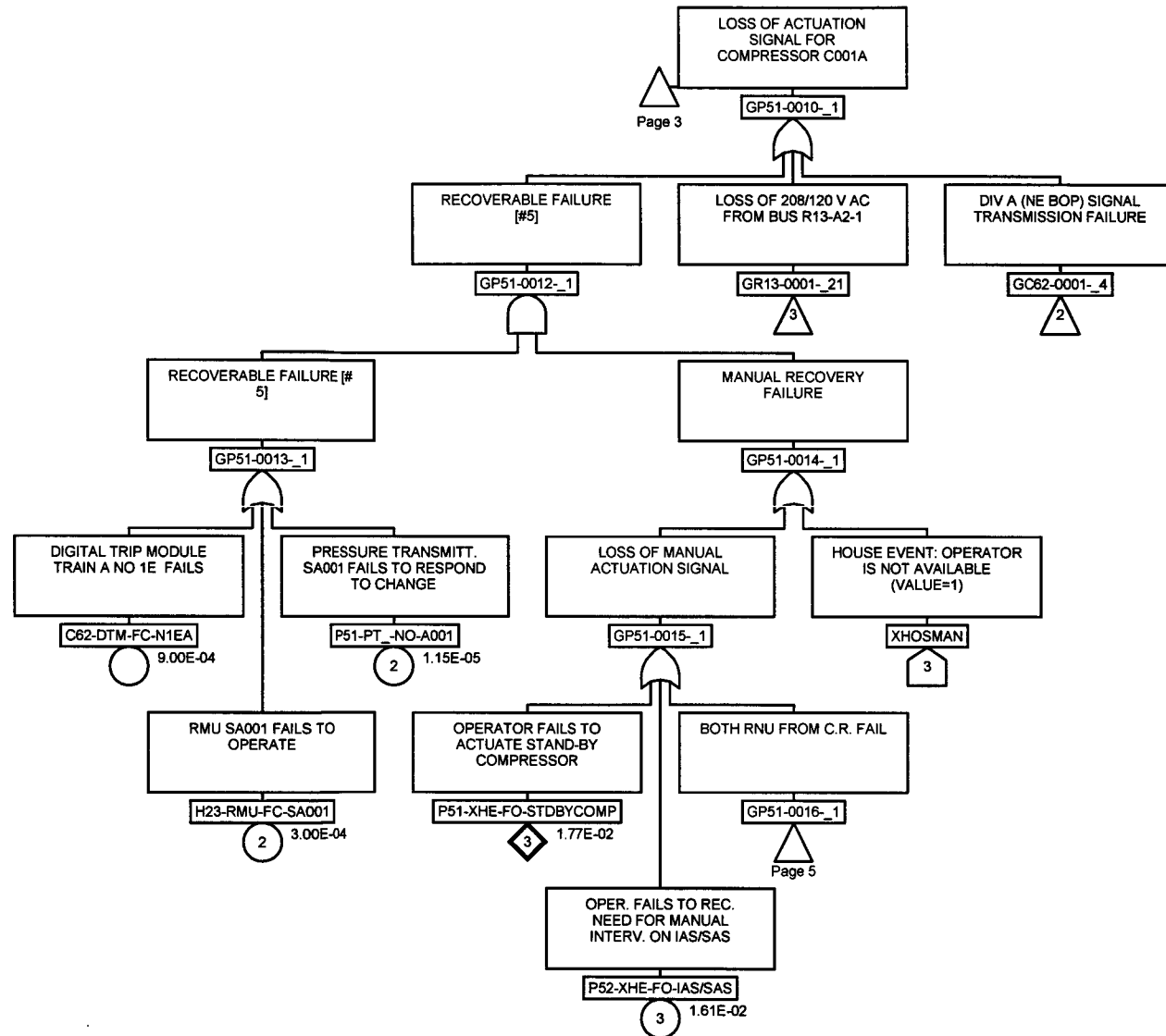


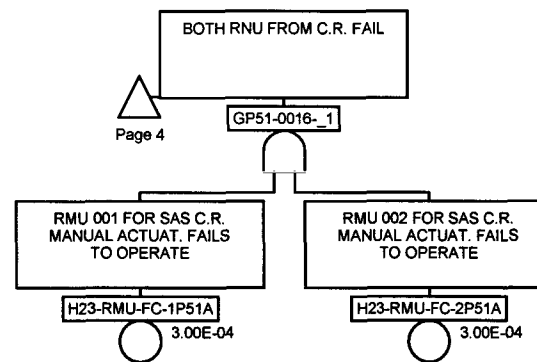
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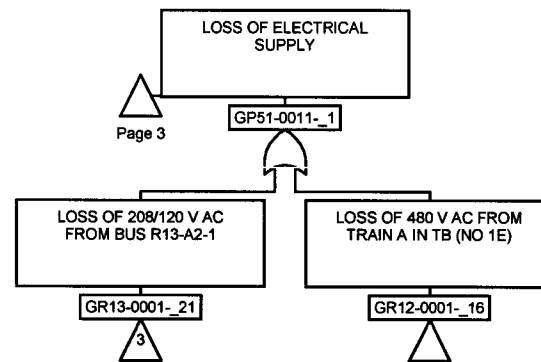
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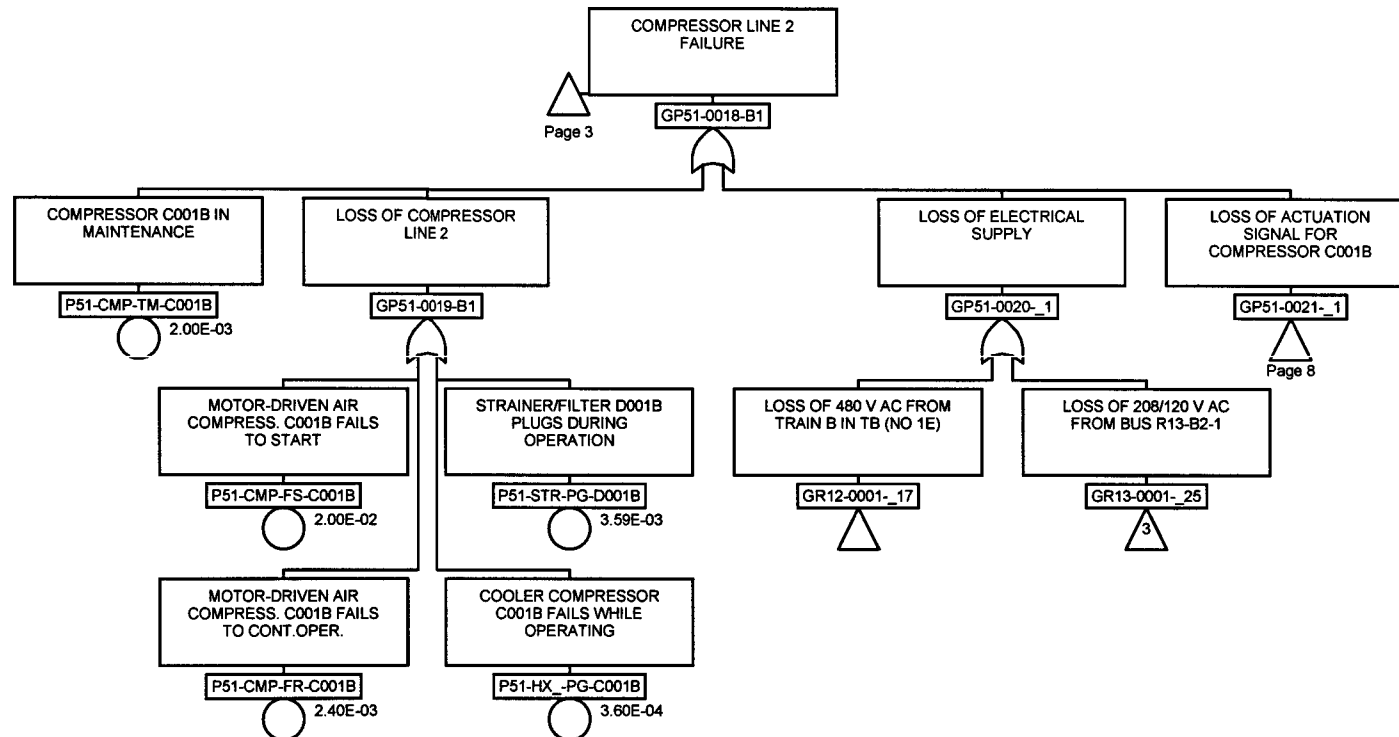


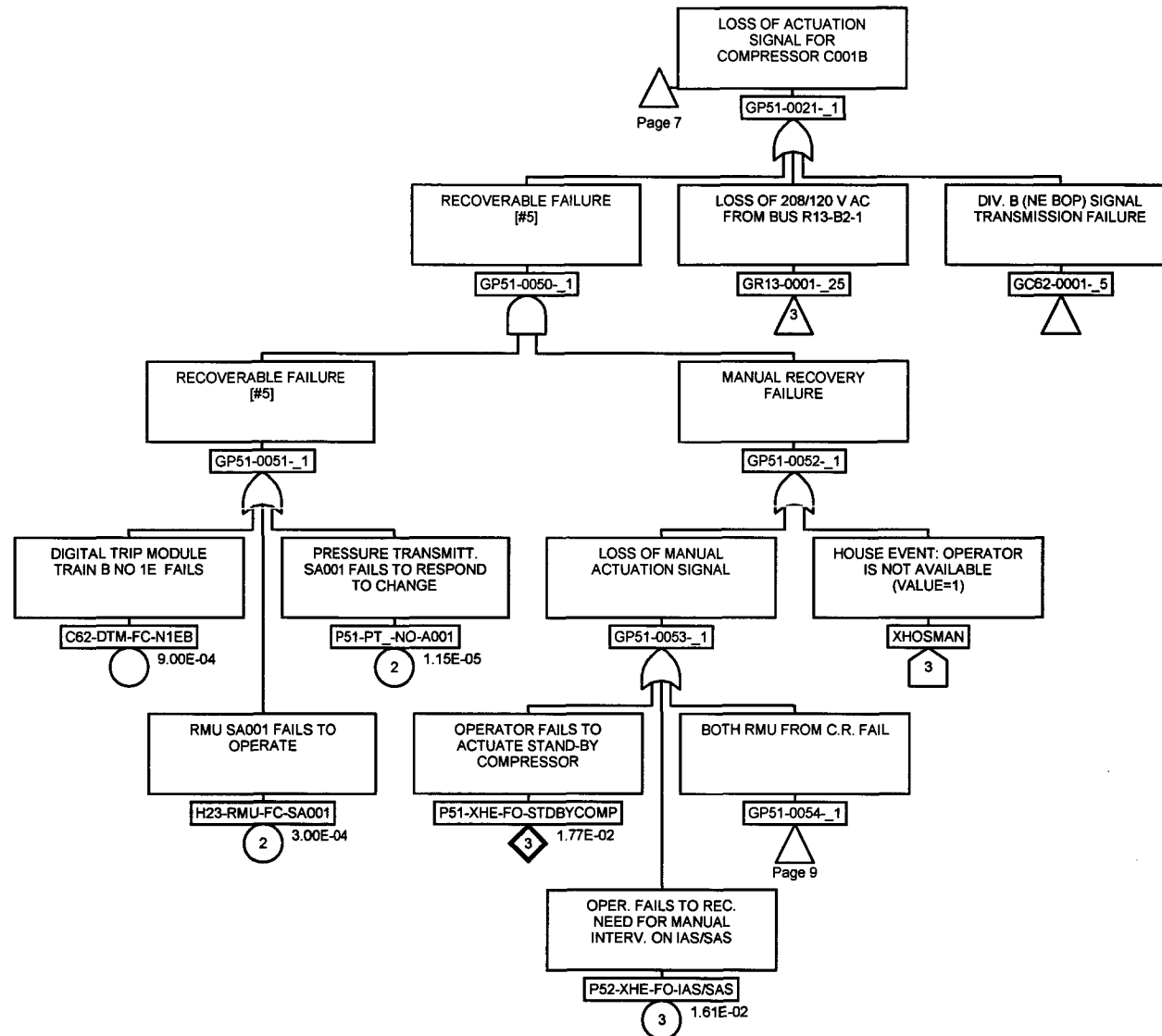


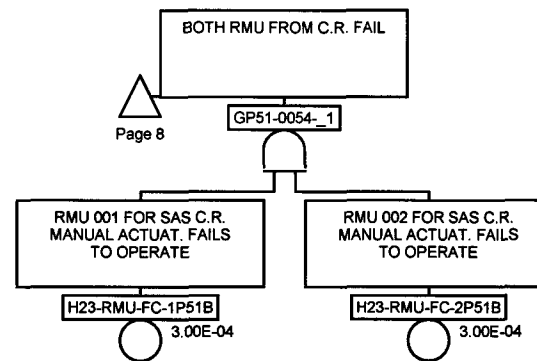


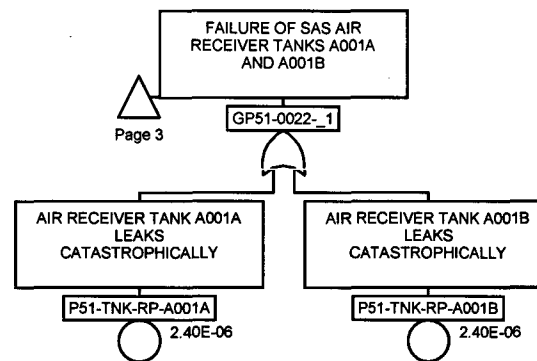
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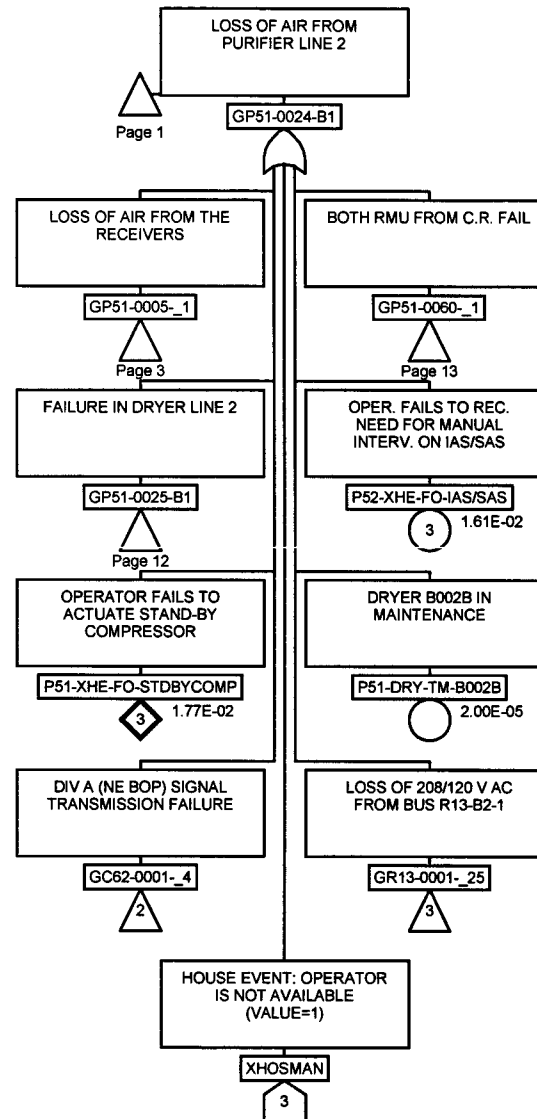


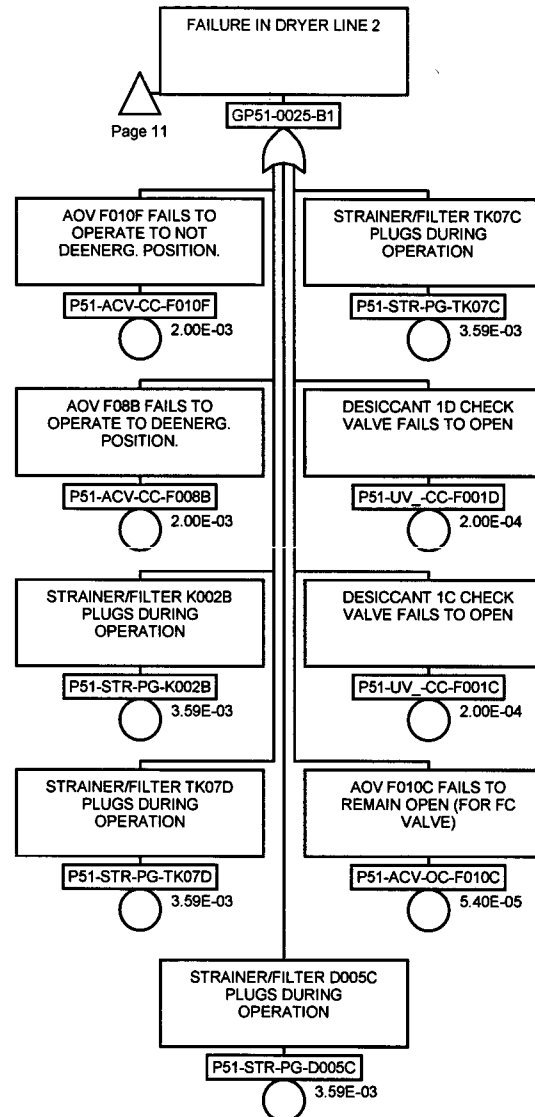


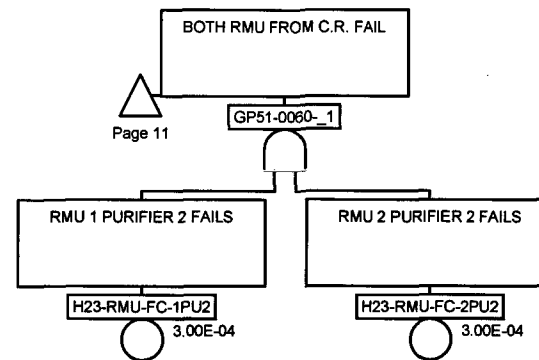




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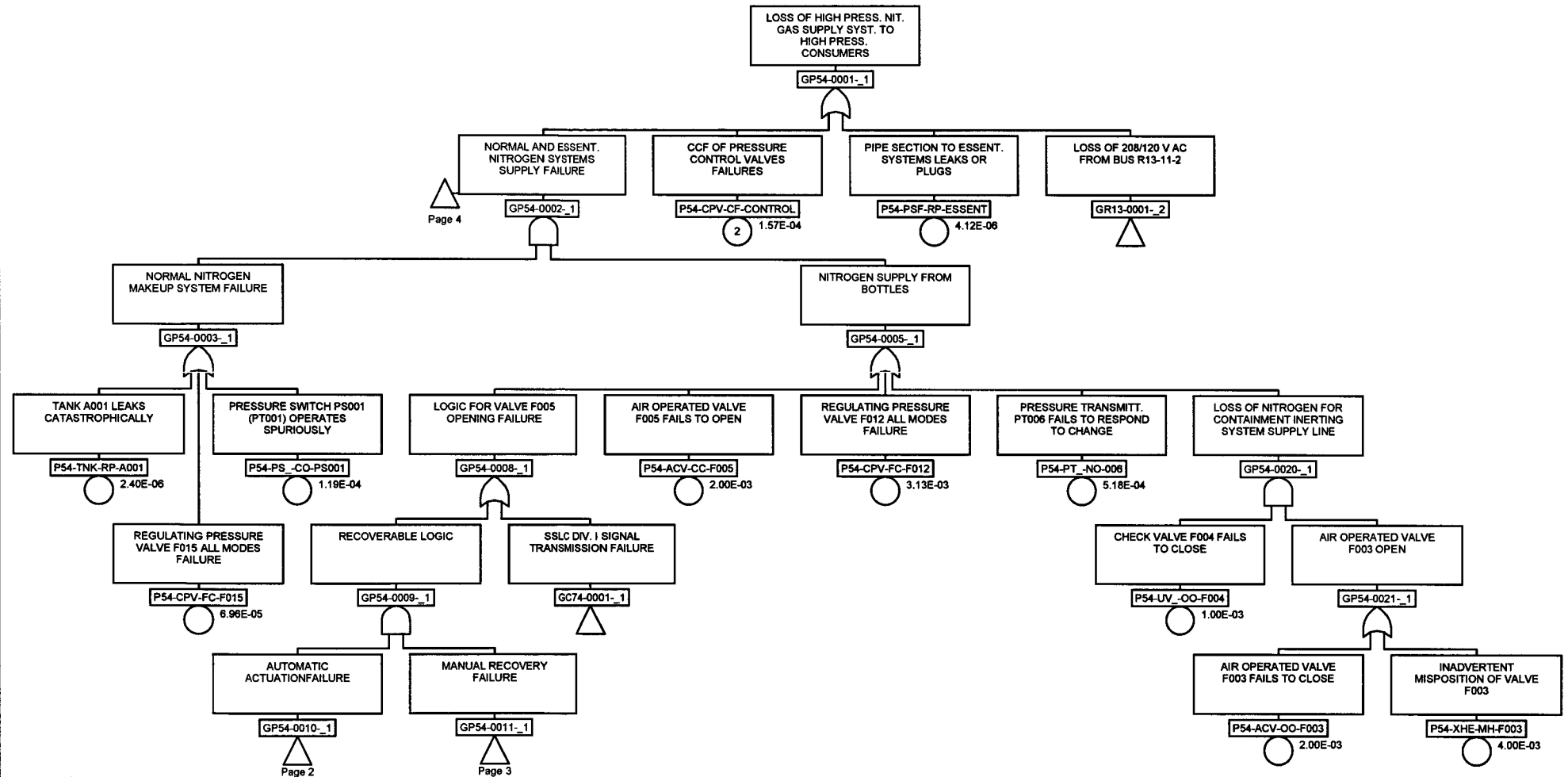


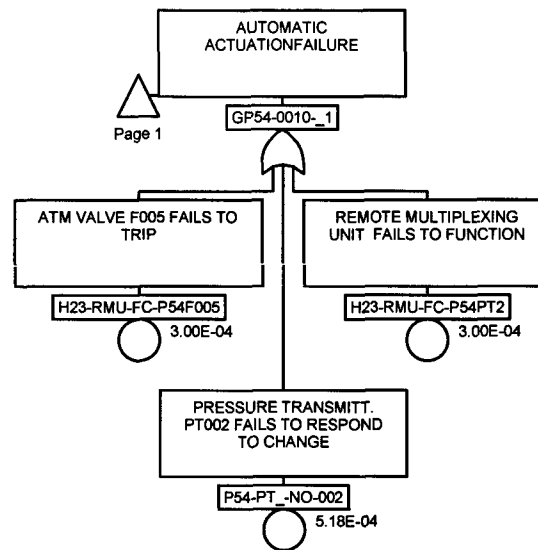


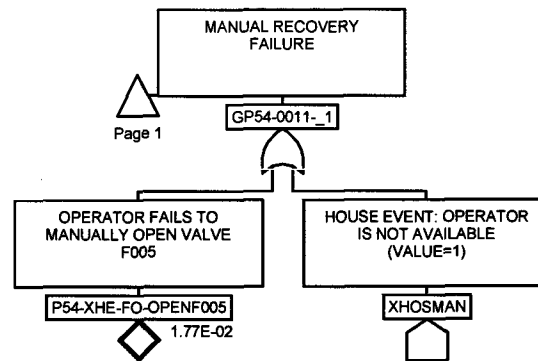


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| C62-DTM-FC-N1EA | 4 | 1 | GR13-0001- 21 | 4 | 4 | |
| C62-DTM-FC-N1EB | 8 | 1 | GR13-0001- 21 | 6 | 1 | |
| GC62-0001- 4 | 4 | 5 | GR13-0001- 25 | 7 | 5 | |
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| GC62-0001- 5 | 8 | 5 | GR13-0001- 25 | 11 | 2 | |
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| GP51-0003-A1 | 1 | 2 | H23-RMU-FC-2P51A | 5 | 2 | |
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| GP51-0005- 1 | 1 | 2 | H23-RMU-FC-SA001 | 4 | 2 | |
| GP51-0005- 1 | 3 | 4 | H23-RMU-FC-SA001 | 8 | 2 | |
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| GP51-0007- 1 | 3 | 4 | P51-ACV-CC-F010C | 2 | 1 | |
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| GP51-0010- 1 | 3 | 3 | P51-ACV-OC-F010A | 2 | 2 | |
| GP51-0010- 1 | 4 | 4 | P51-ACV-OC-F010C | 12 | 2 | |
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| GP51-0012- 1 | 4 | 3 | P51-CMP-FR-C001A | 3 | 3 | |
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| GP51-0024-B1 | 1 | 3 | P51-STR-PG-K002A | 2 | 1 | |
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| GP51-1010- 1 | 3 | 2 | P51-XHE-FO-STDBYCOMP | 8 | 3 | |
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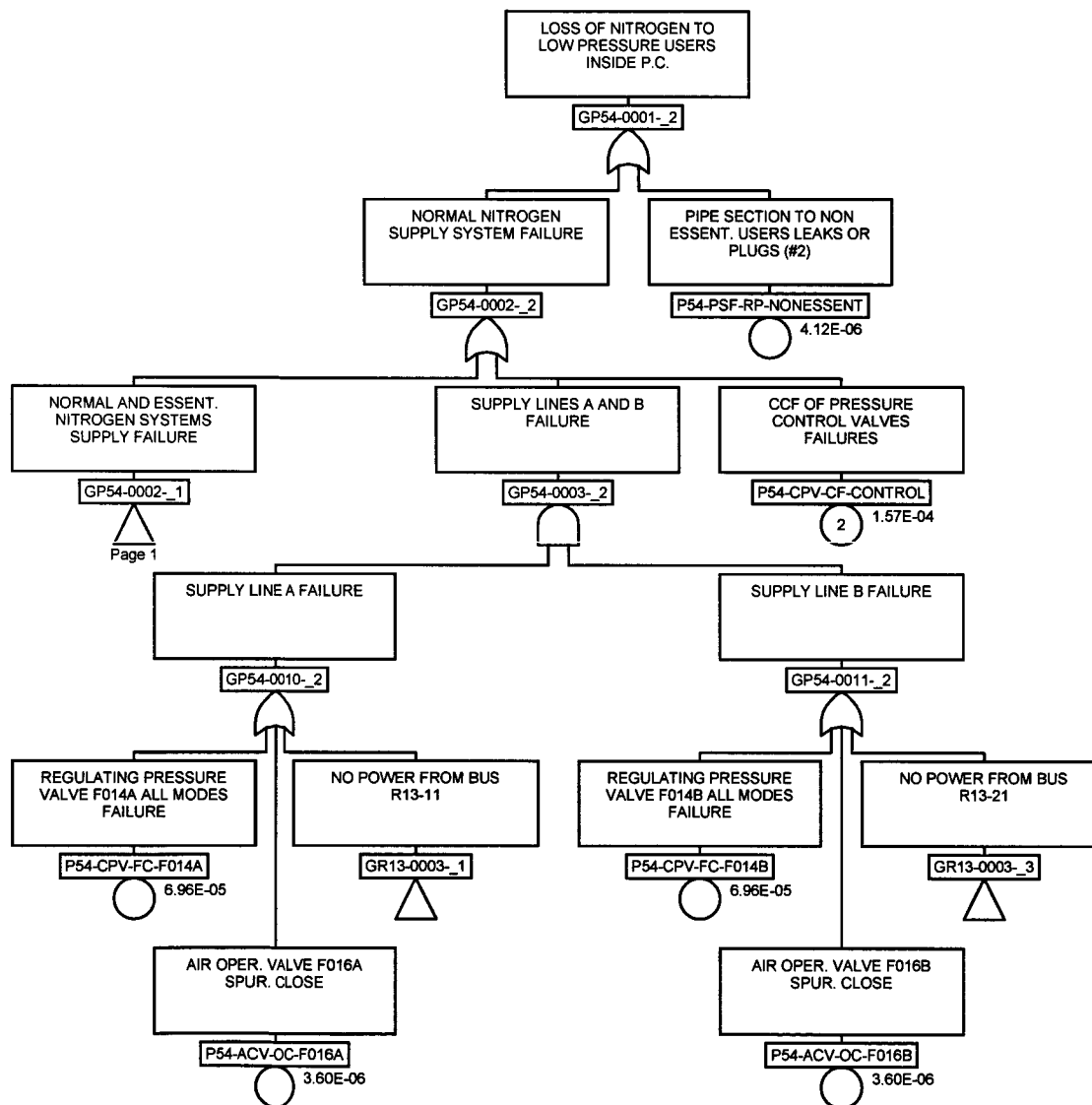
Appendix B.4.13 High Pressure Nitrogen Supply System Fault Tree





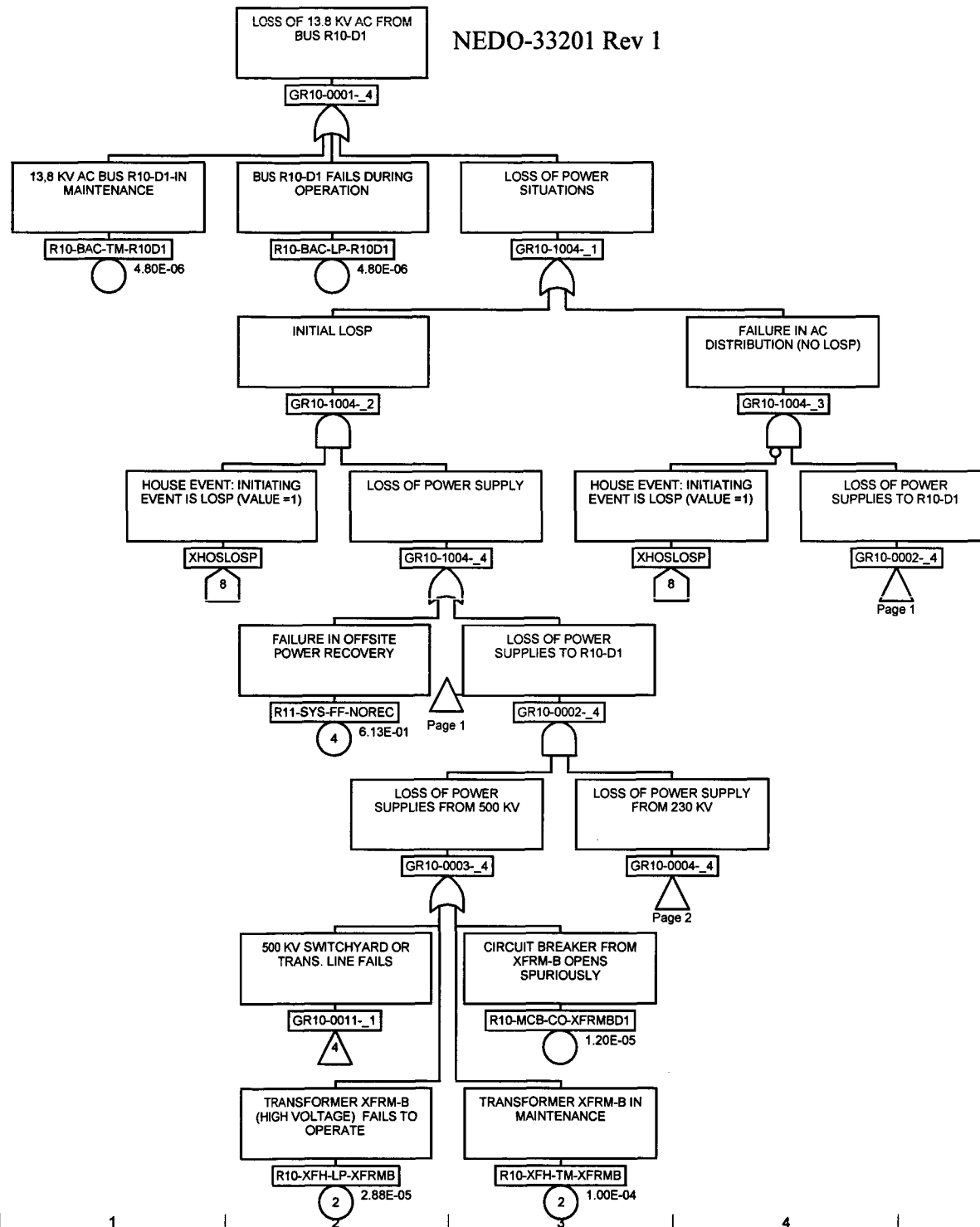


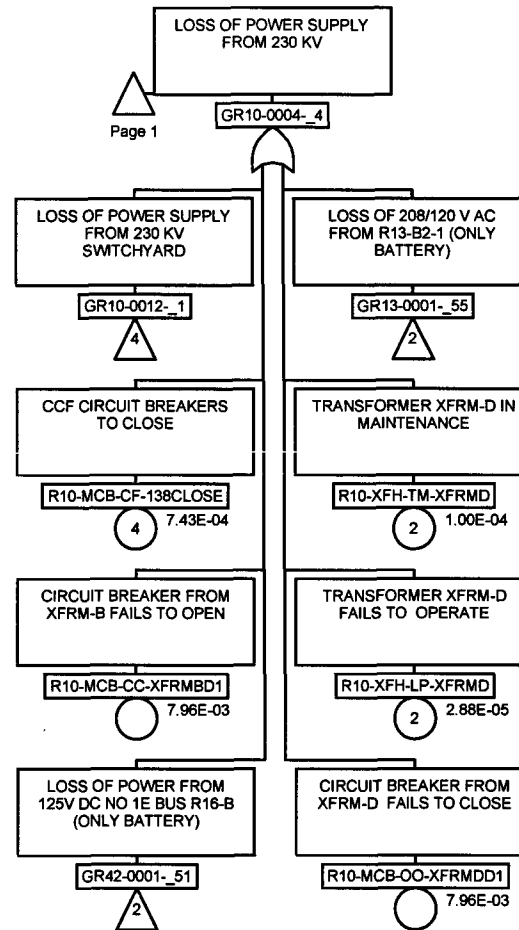
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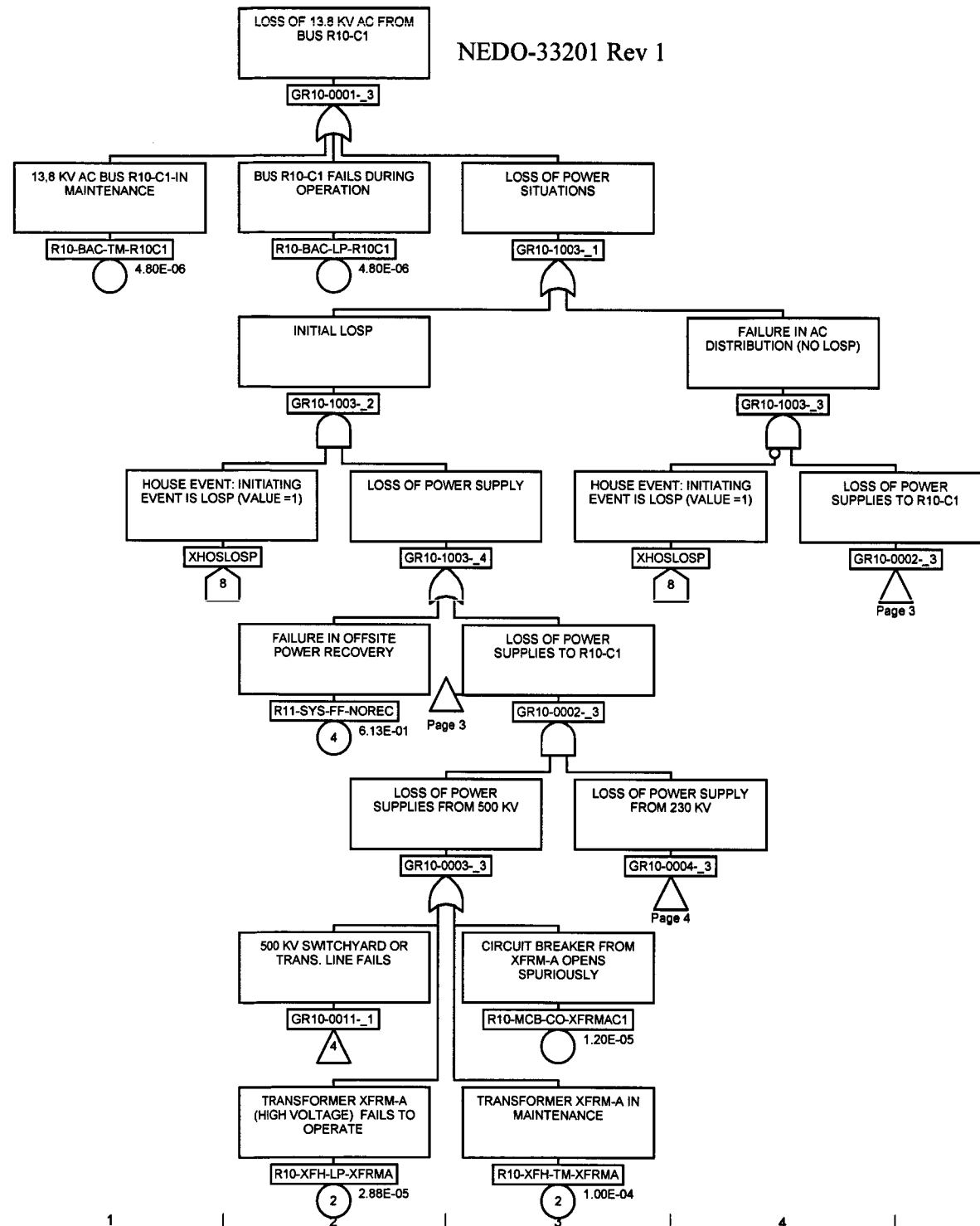


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| GP54-0003- 1 | 1 | 2 | | | | | |
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| GP54-0005- 1 | 1 | 5 | | | | | |
| GP54-0008- 1 | 1 | 3 | | | | | |
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| GP54-0010- 1 | 1 | 2 | | | | | |
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| GP54-0011- 1 | 1 | 3 | | | | | |
| GP54-0011- 1 | 3 | 2 | | | | | |
| GP54-0011- 2 | 4 | 4 | | | | | |
| GP54-0020- 1 | 1 | 7 | | | | | |
| GP54-0021- 1 | 1 | 8 | | | | | |
| GR13-0001- 2 | 1 | 6 | | | | | |
| GR13-0003- 1 | 4 | 2 | | | | | |
| GR13-0003- 3 | 4 | 4 | | | | | |
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| H23-RMU-FC-P54PT2 | 2 | 2 | | | | | |
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| P54-ACV-OC-F016A | 4 | 2 | | | | | |
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| P54-ACV-OO-F003 | 1 | 7 | | | | | |
| P54-CPV-CF-CONTROL | 1 | 4 | | | | | |
| P54-CPV-CF-CONTROL | 4 | 4 | | | | | |
| P54-CPV-FC-F012 | 1 | 5 | | | | | |
| P54-CPV-FC-F014A | 4 | 1 | | | | | |
| P54-CPV-FC-F014B | 4 | 3 | | | | | |
| P54-CPV-FC-F015 | 1 | 2 | | | | | |
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| P54-PSF-RP-NONESSENT | 4 | 3 | | | | | |
| P54-PS -CO-PS001 | 1 | 2 | | | | | |
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| P54-PT -NO-006 | 1 | 6 | | | | | |
| P54-TNK-RP-A001 | 1 | 1 | | | | | |
| P54-UV -OO-F004 | 1 | 7 | | | | | |
| P54-XHE-FO-OPENF005 | 3 | 1 | | | | | |
| P54-XHE-MH-F003 | 1 | 8 | | | | | |
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| P54 - High Pressure Nitrogen Supply System | | | | | HPNSS.caf | Appendix B.4.13 | Page 5 |

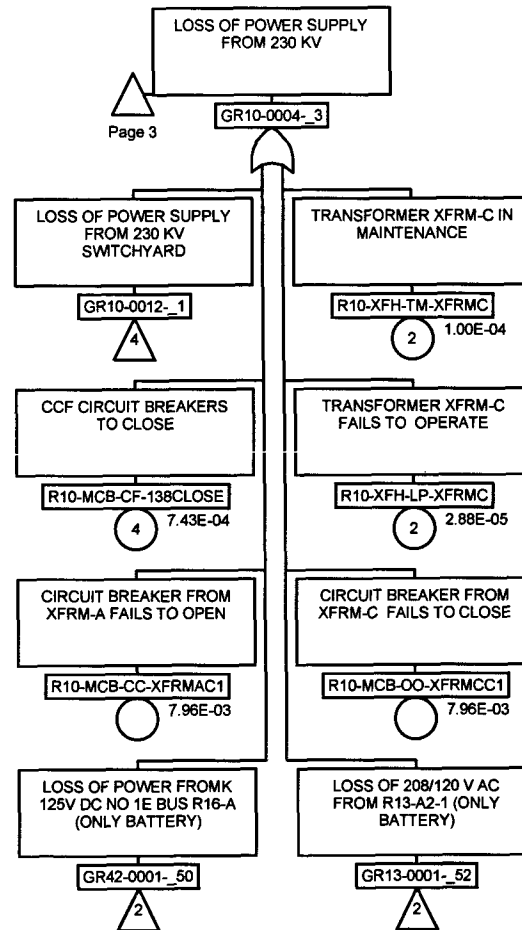
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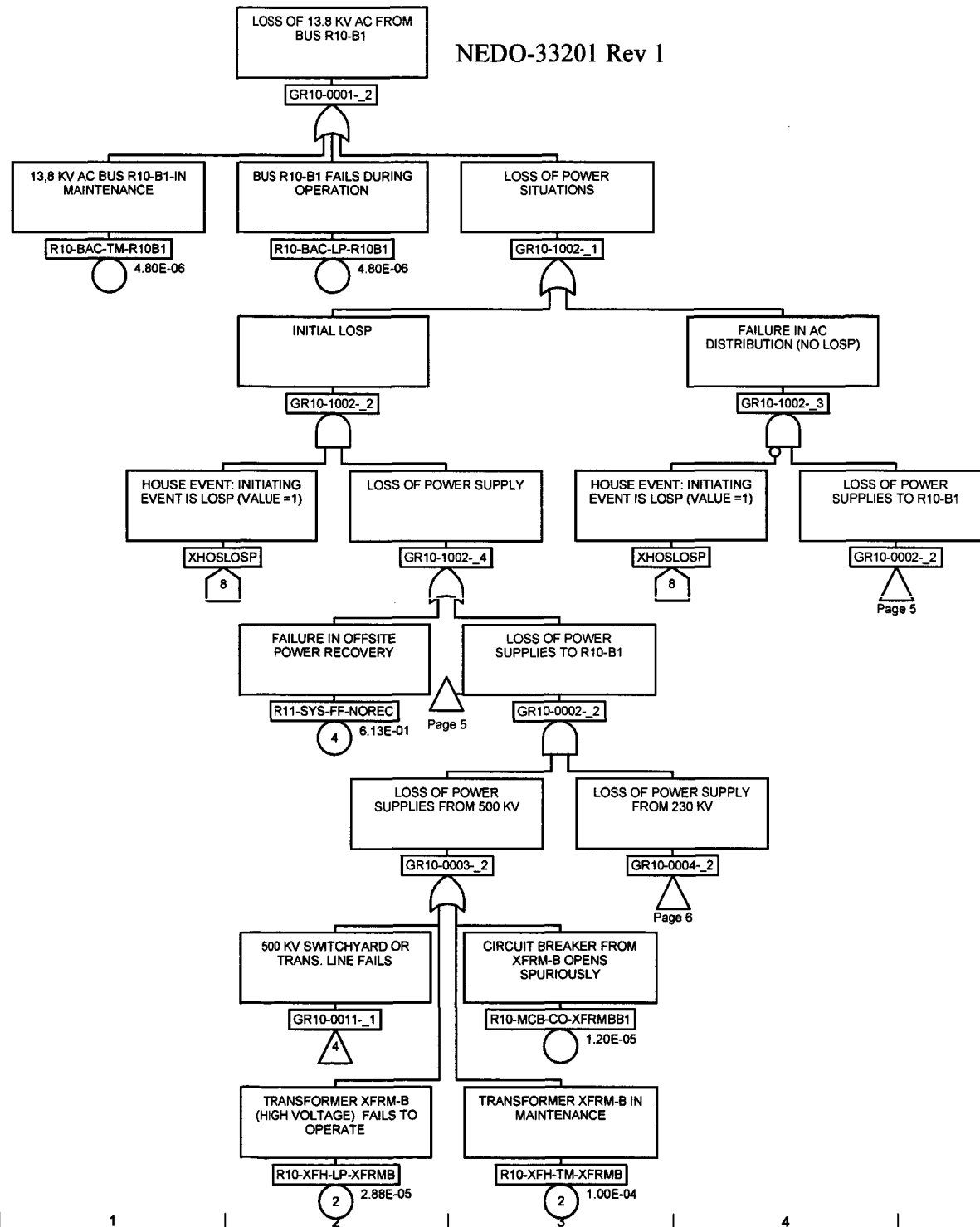


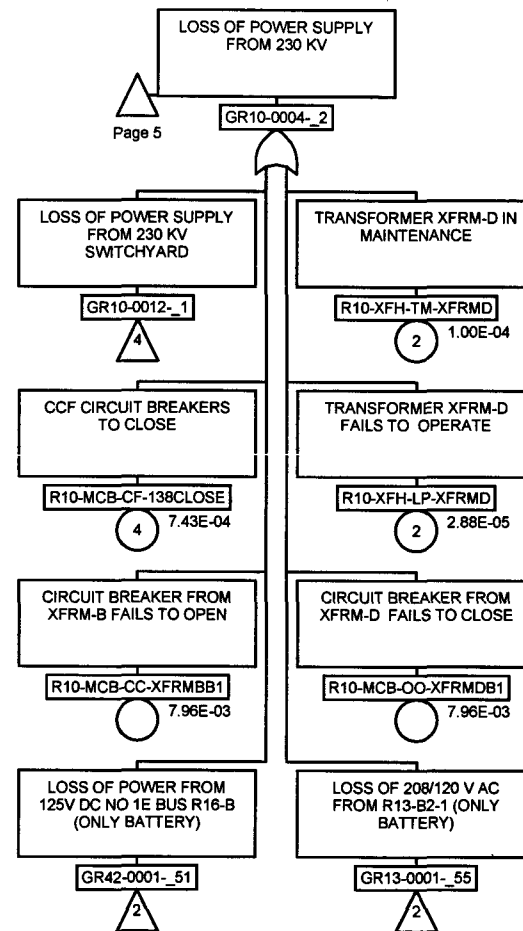


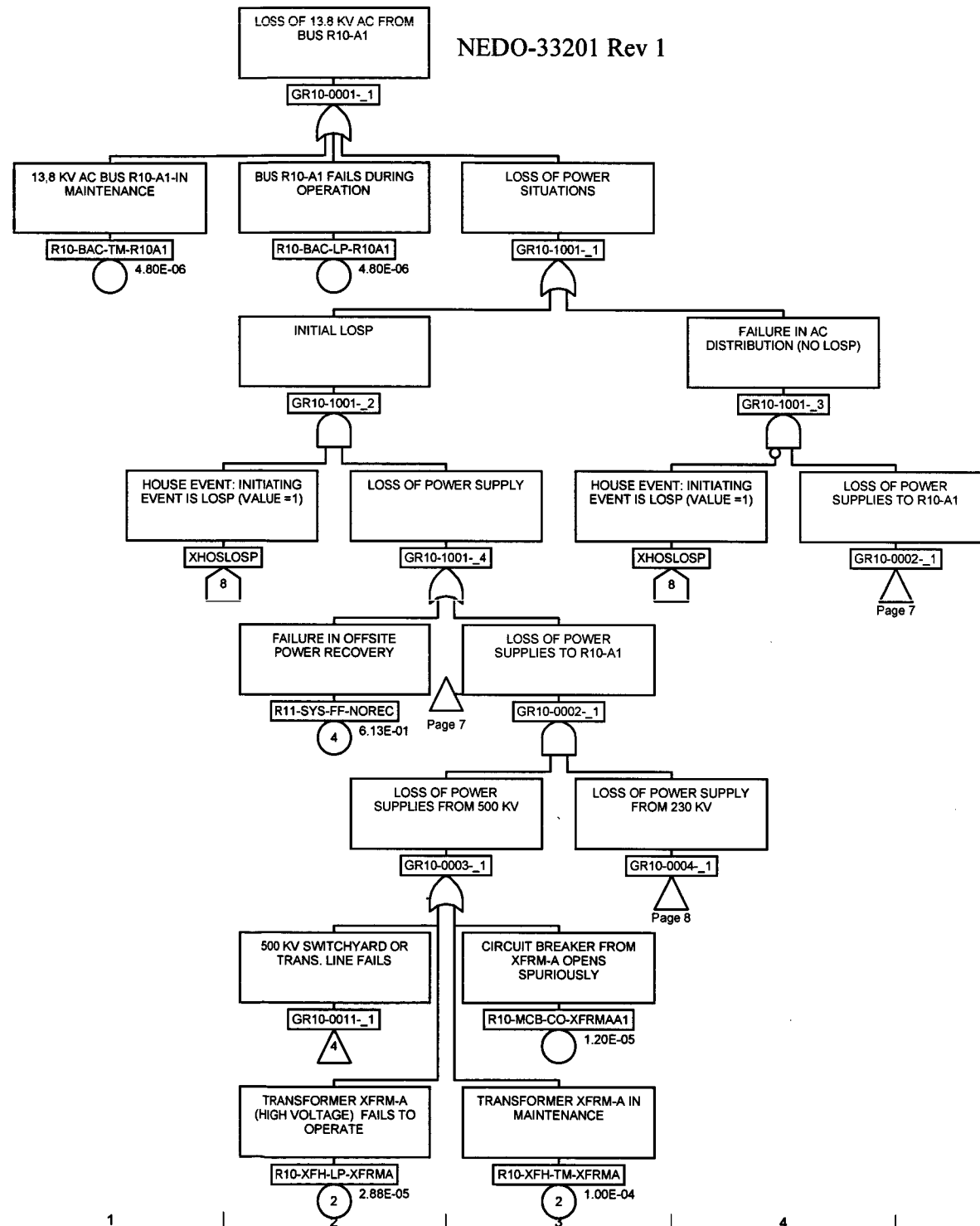


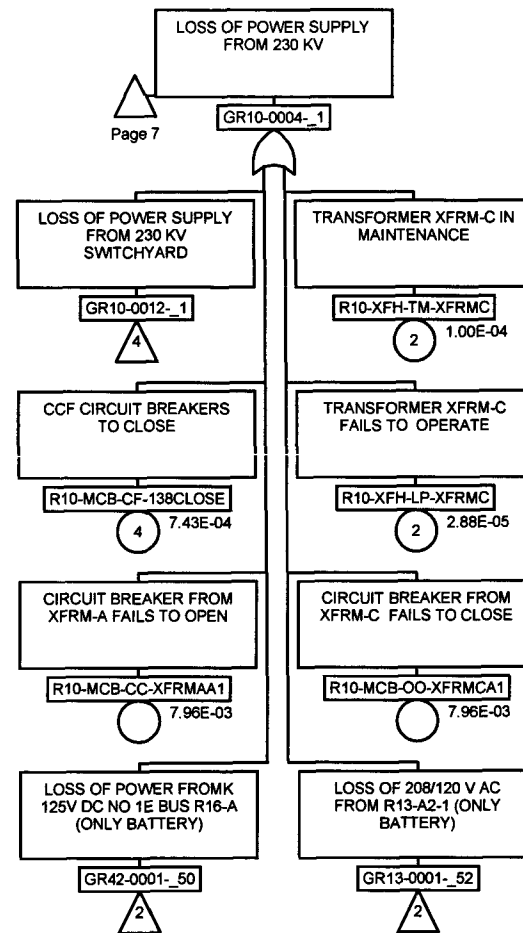
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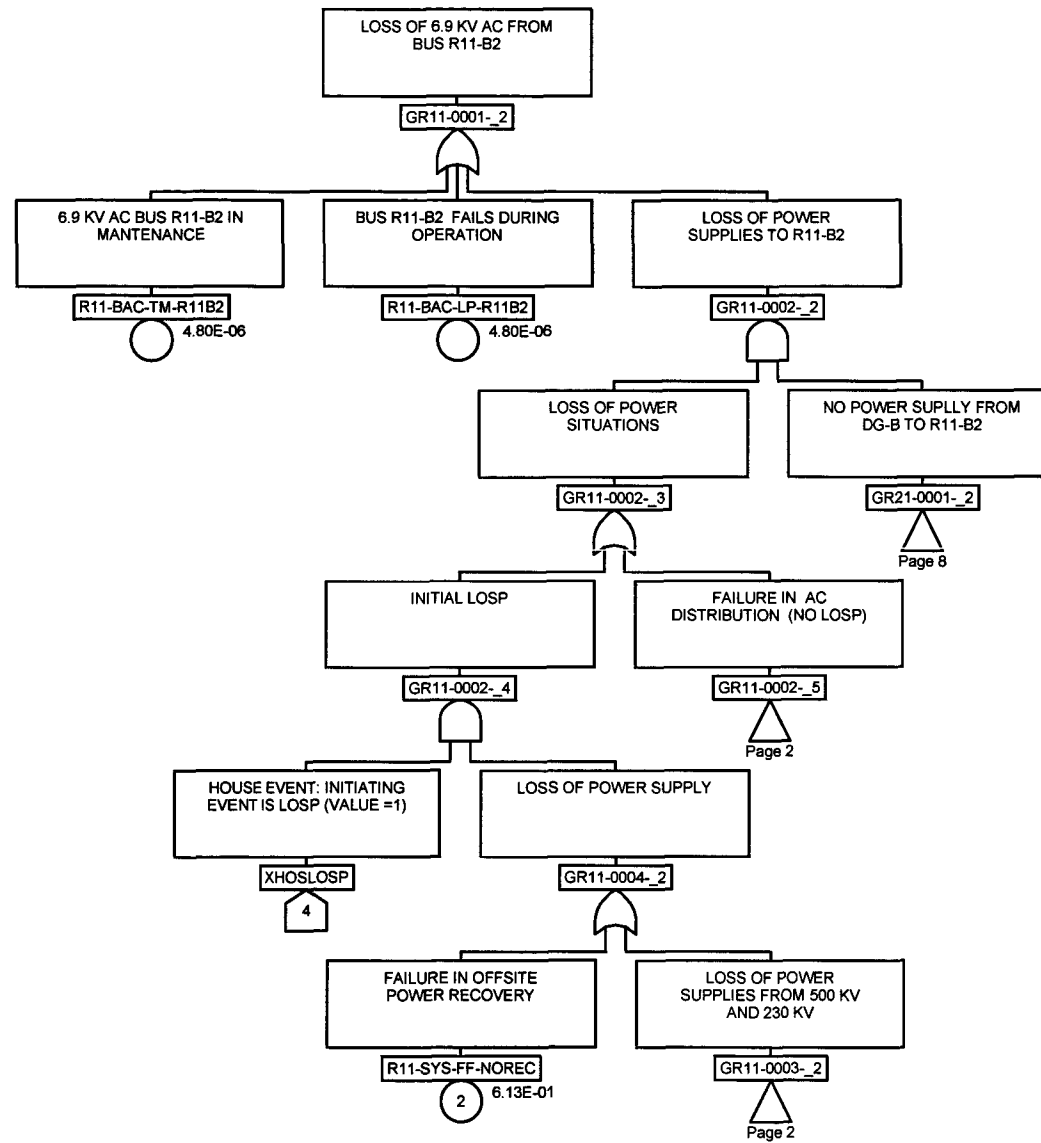
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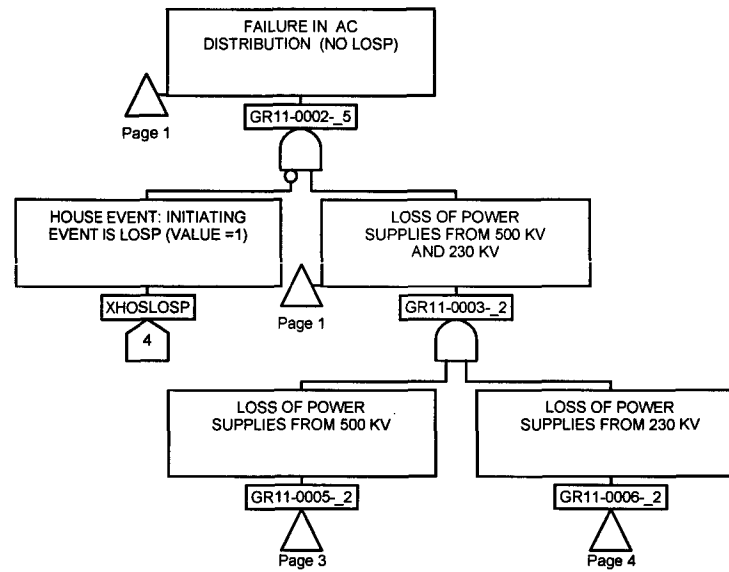
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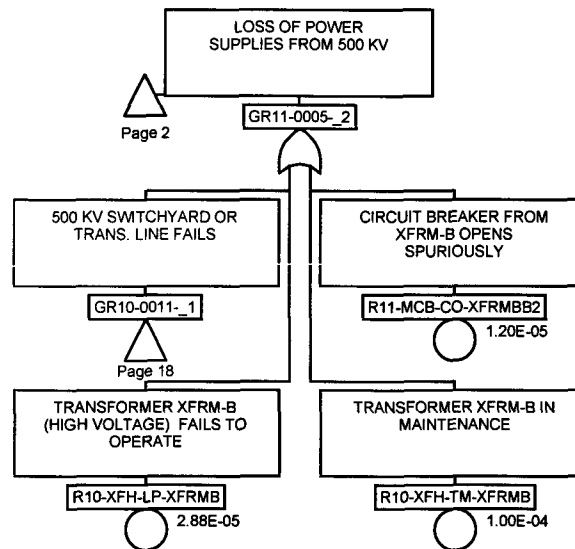
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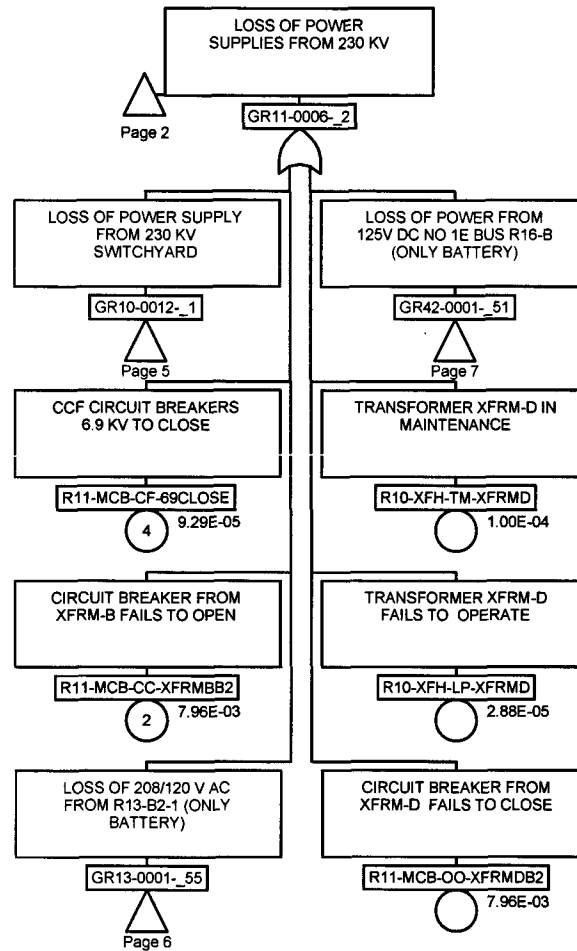
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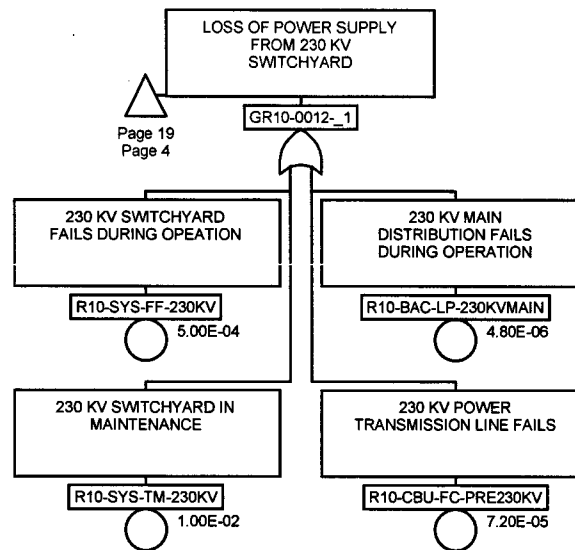
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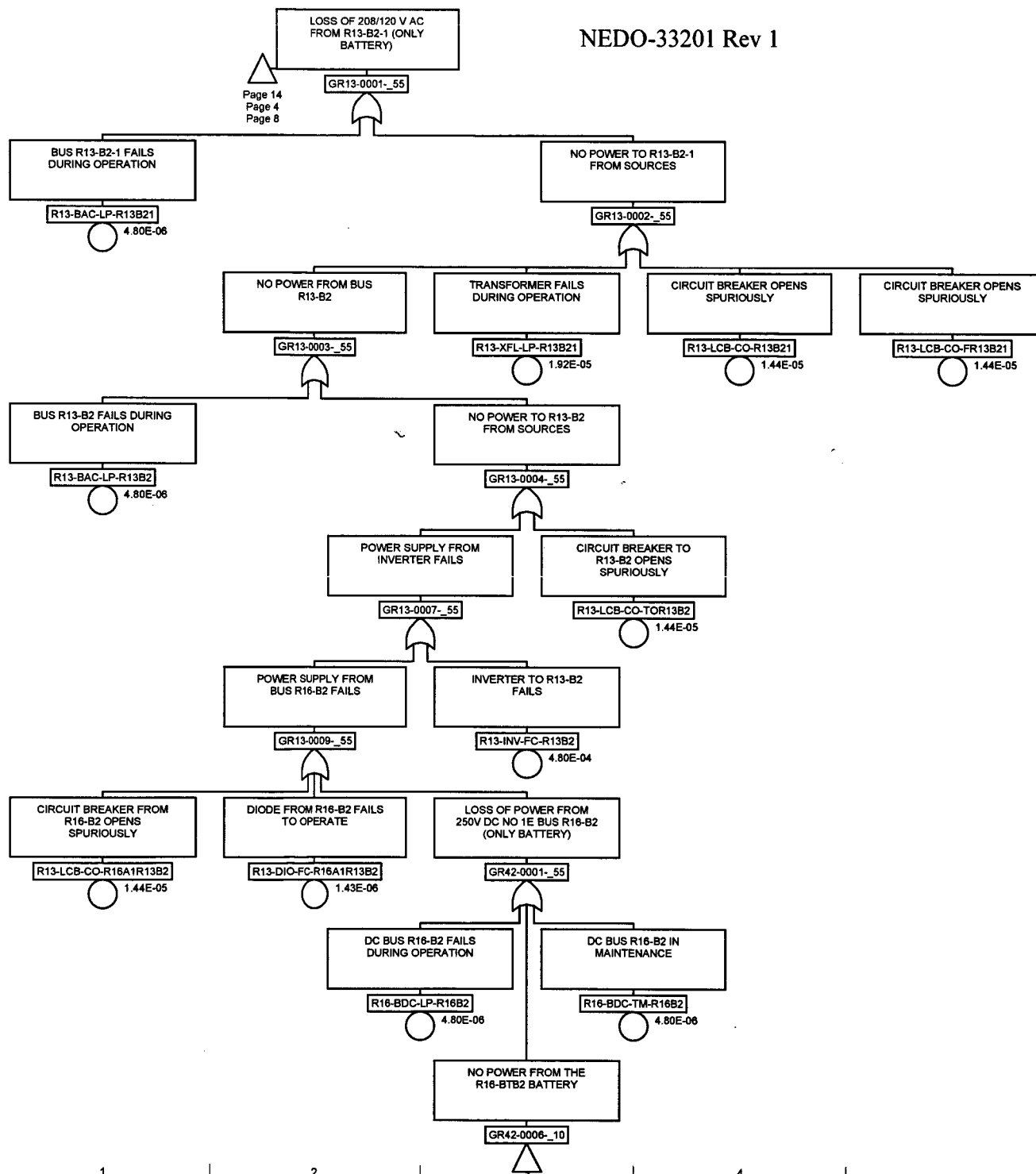








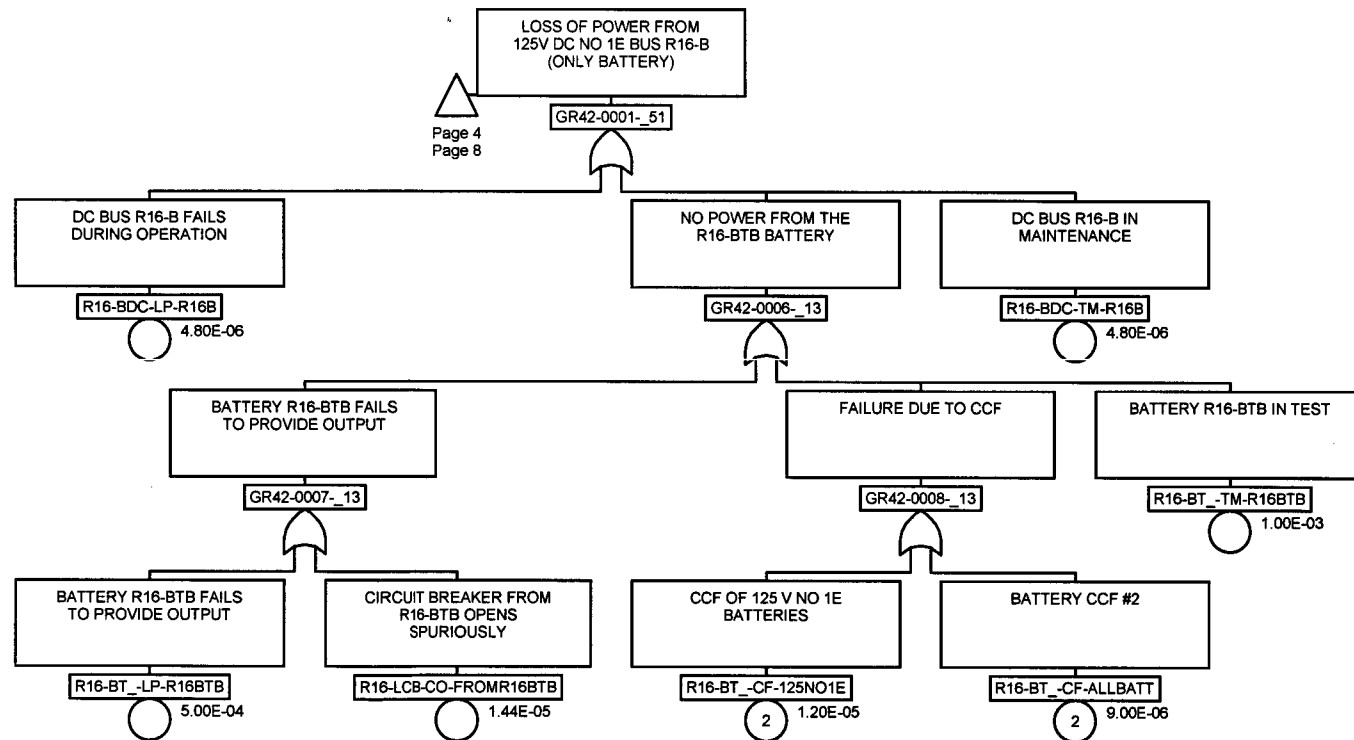


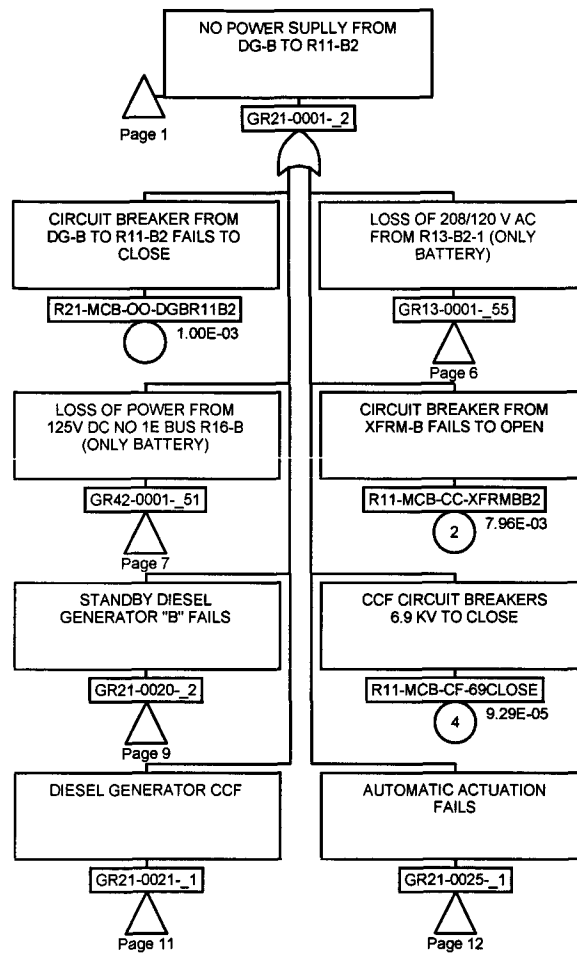


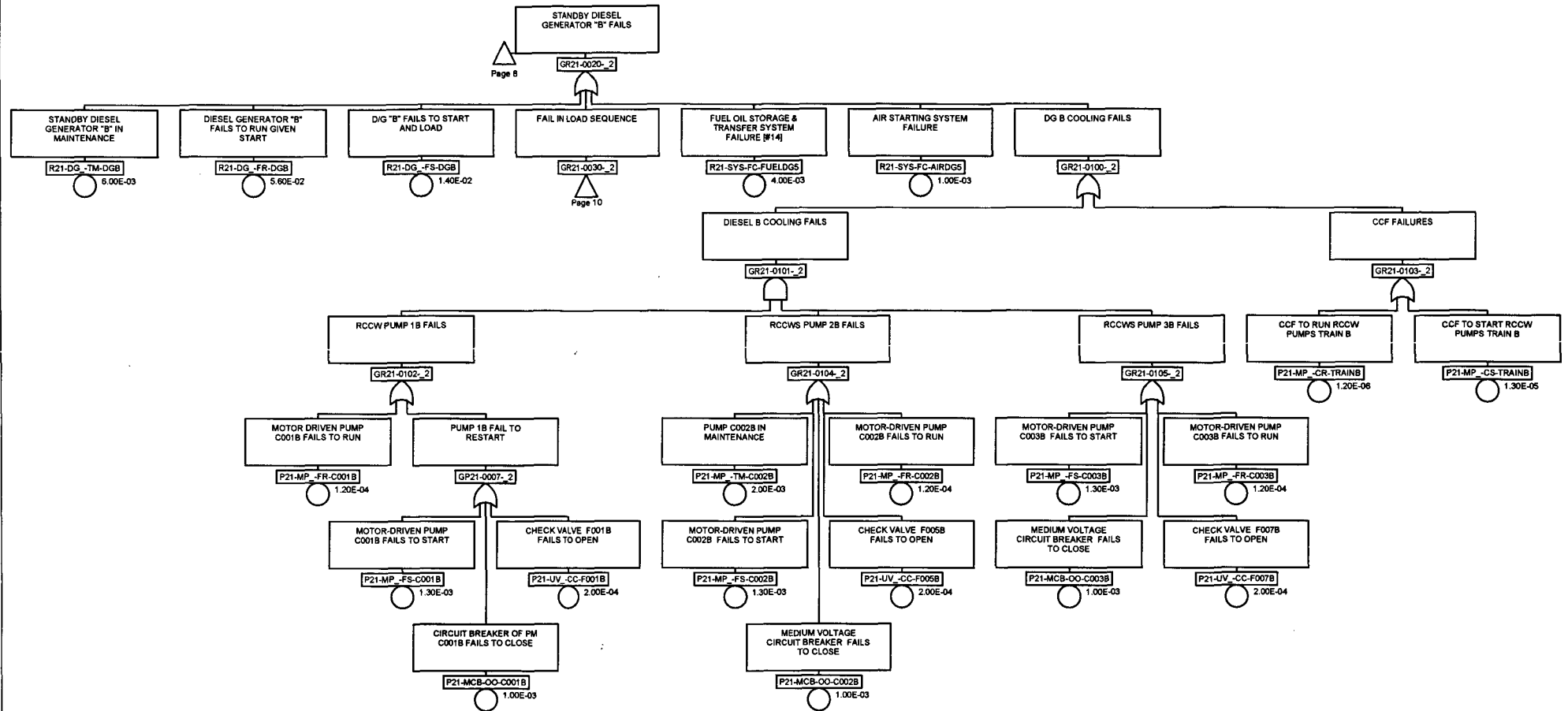
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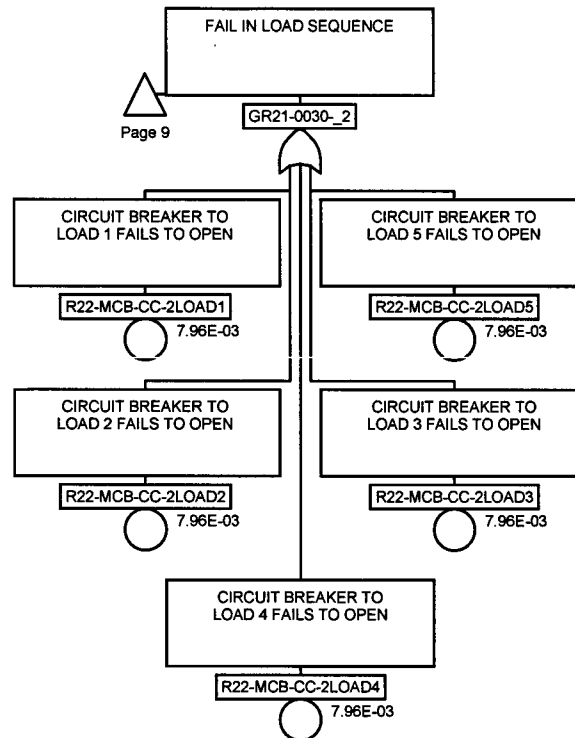
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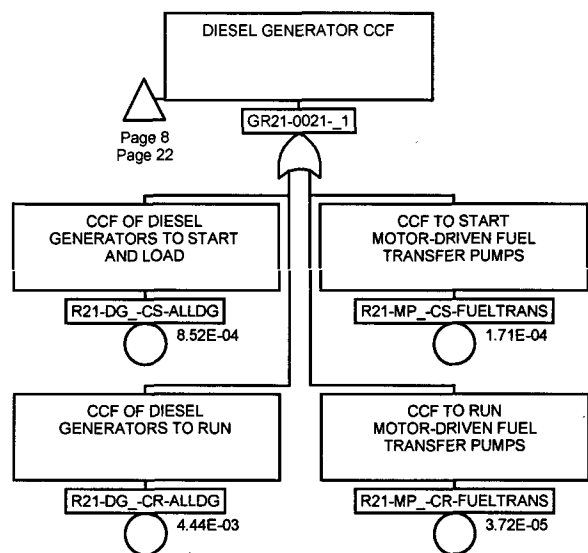


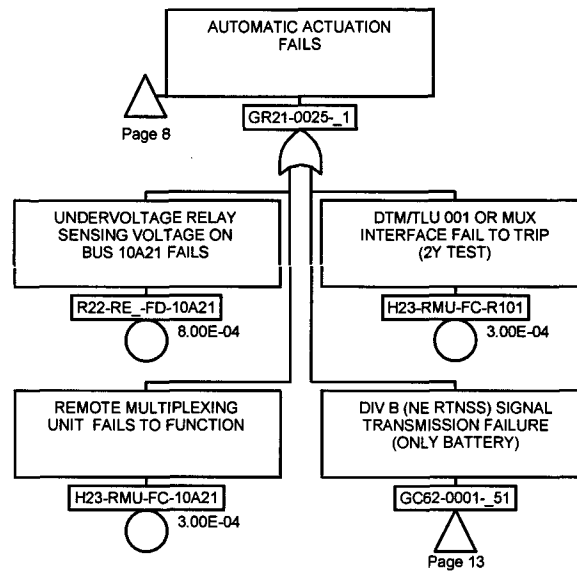


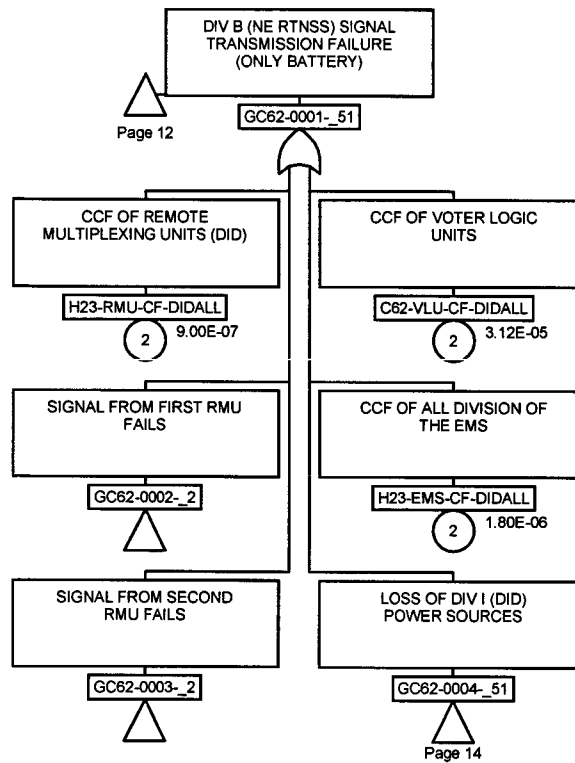


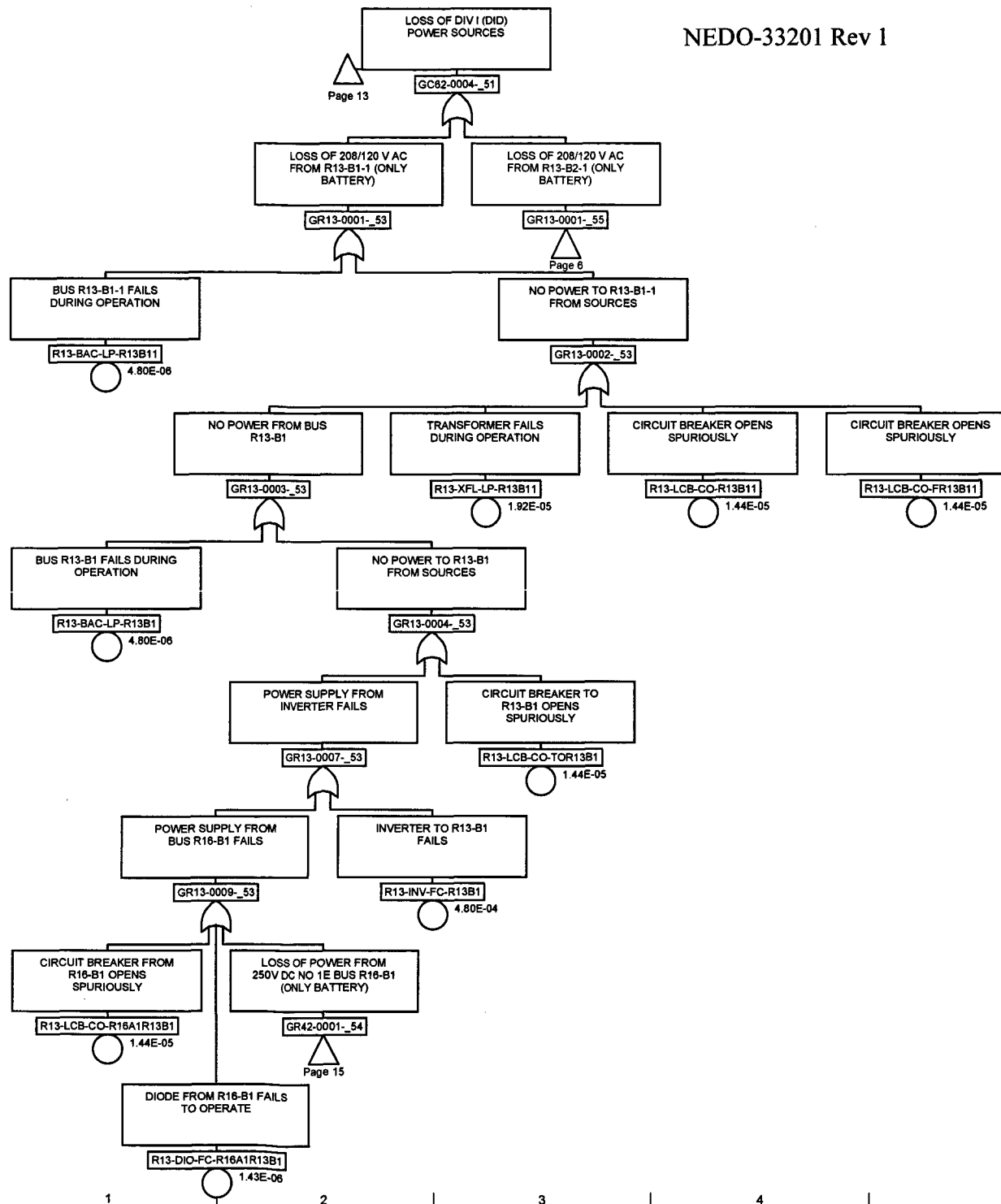


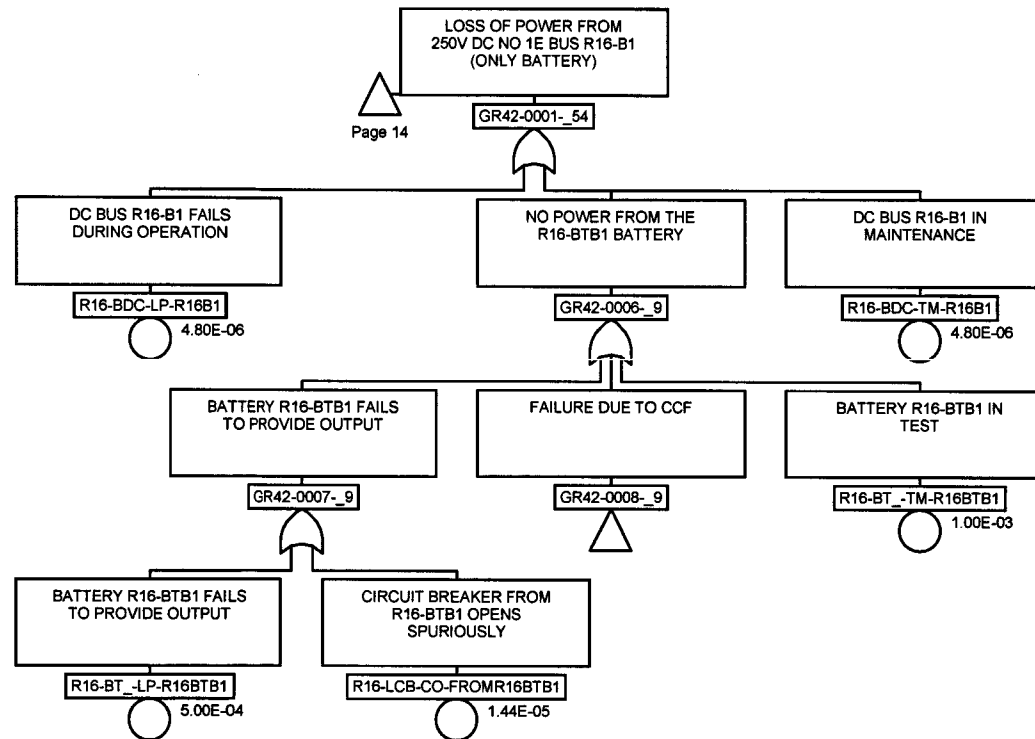
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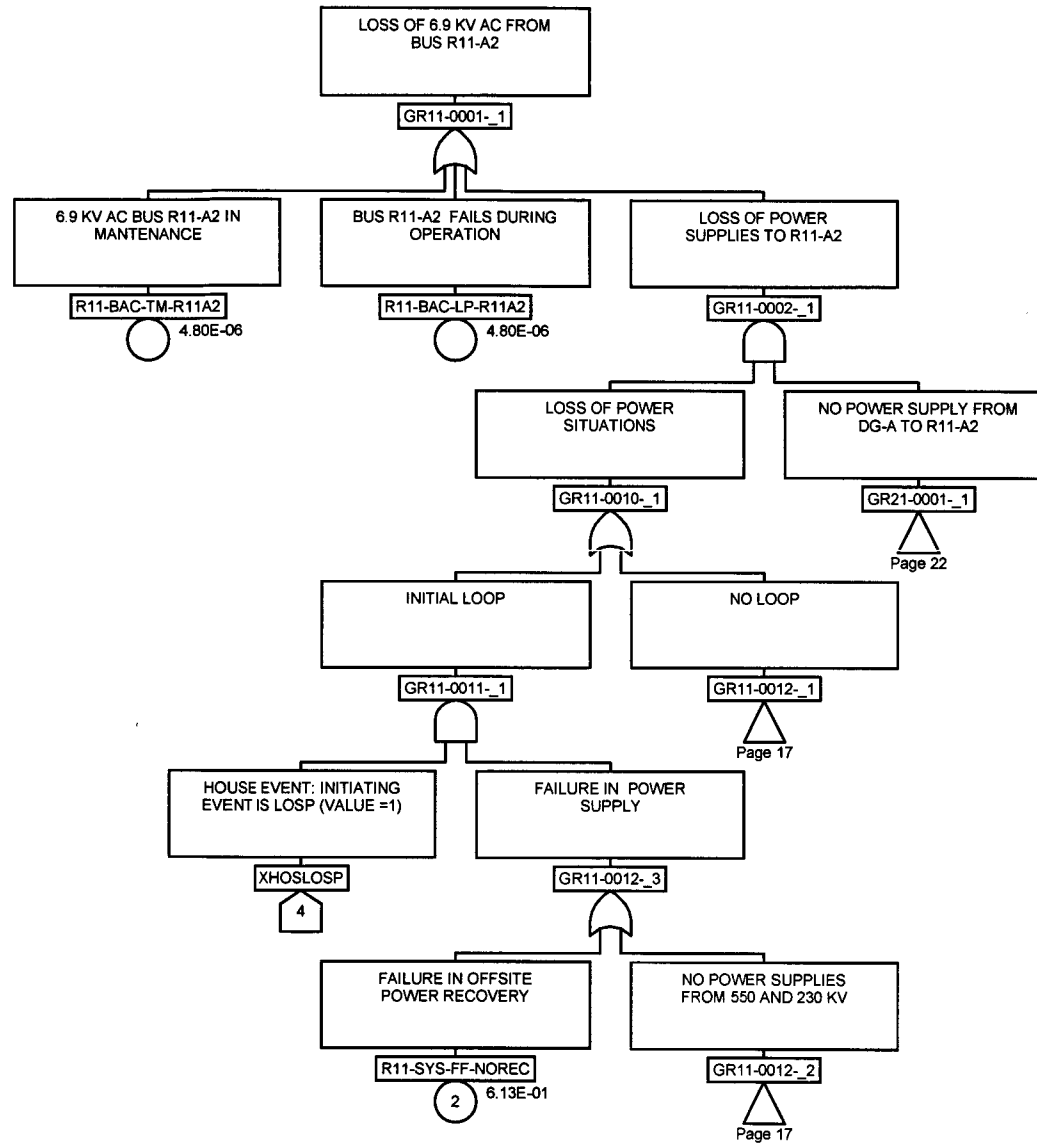


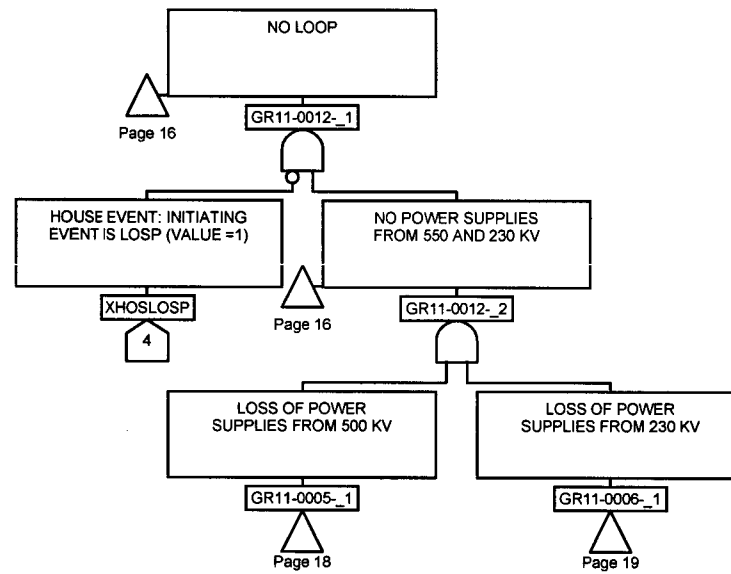


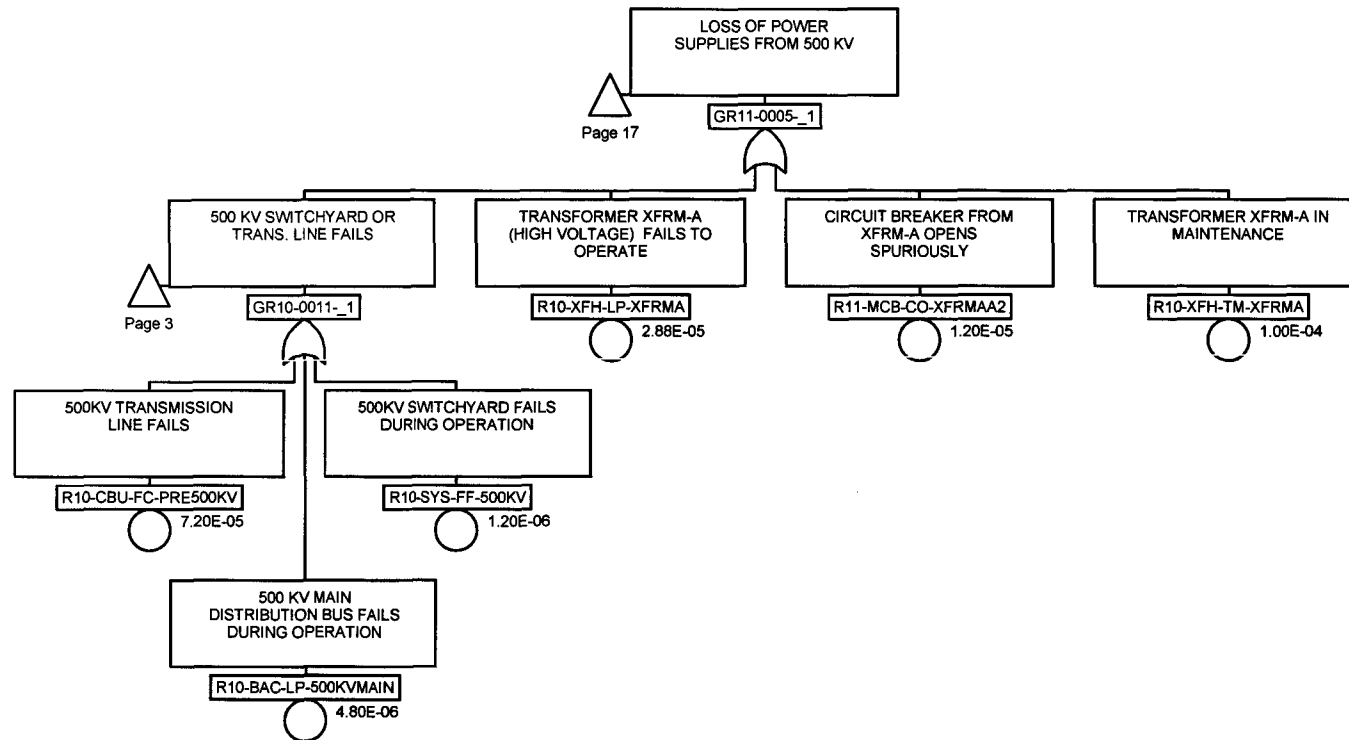


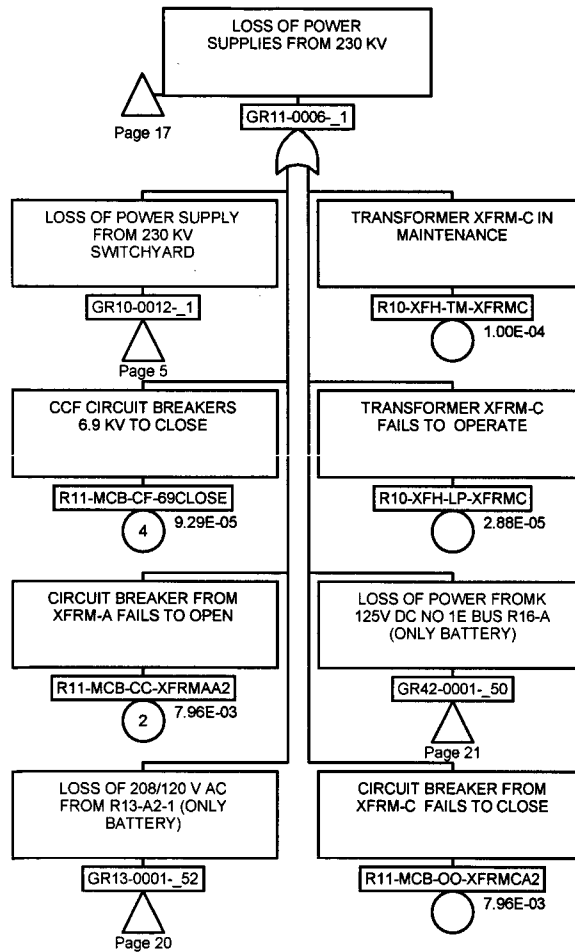


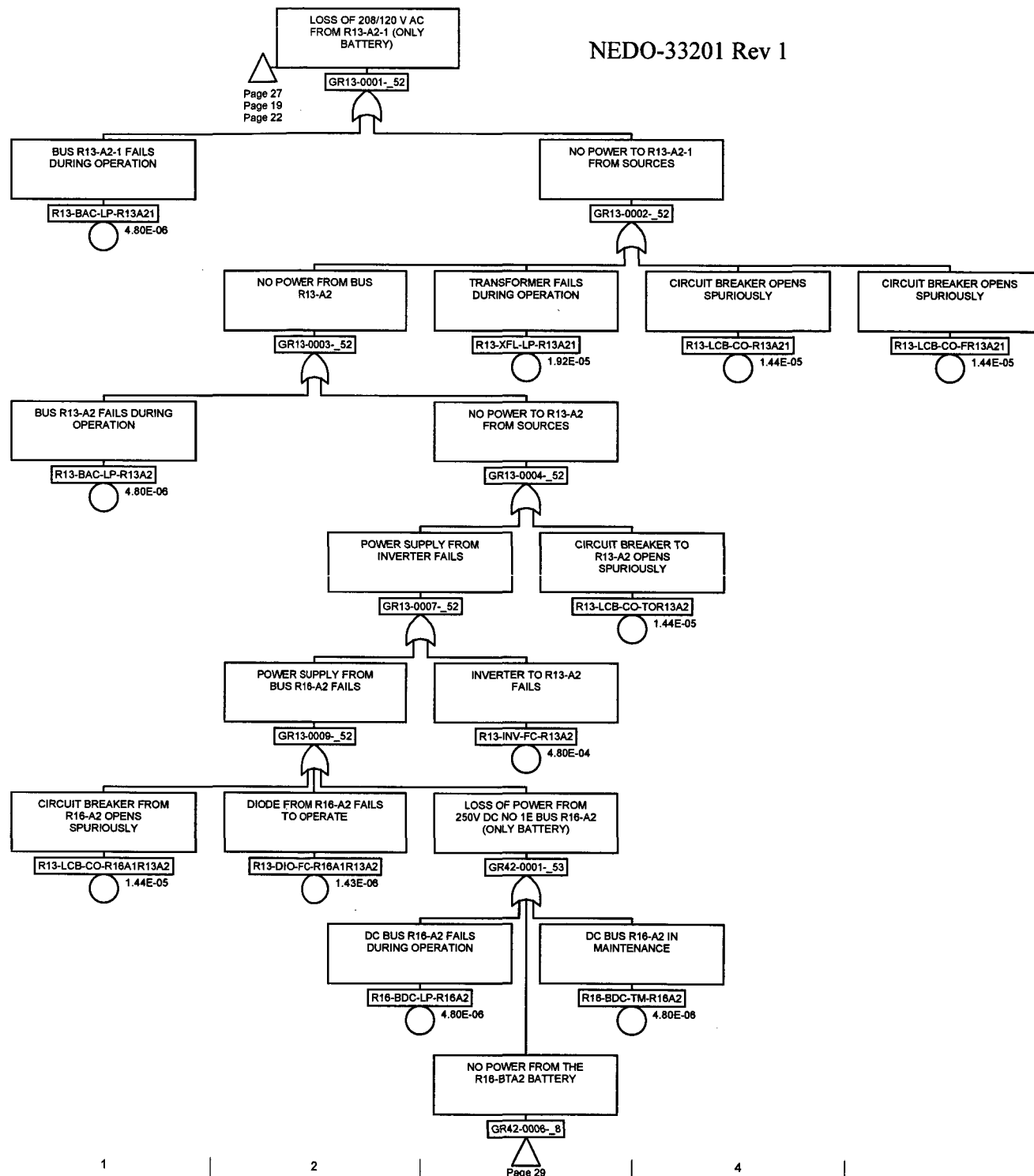
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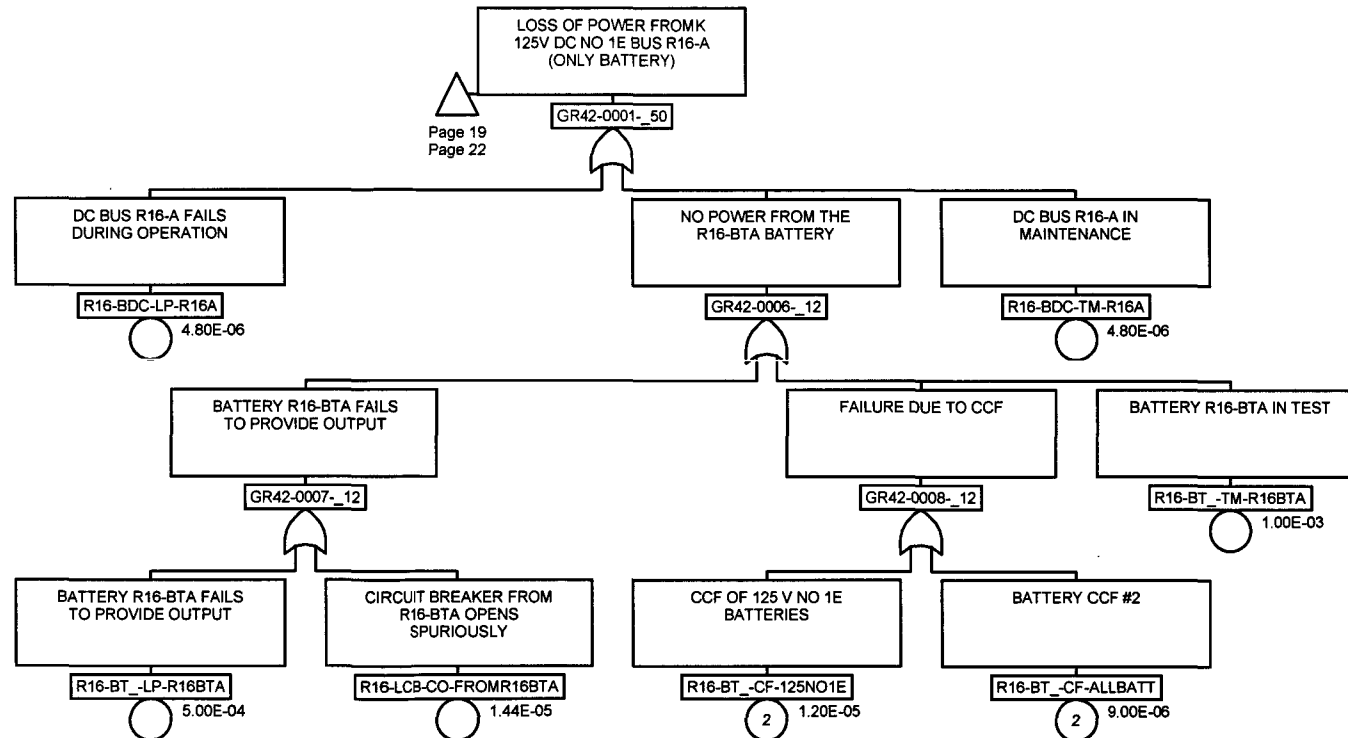




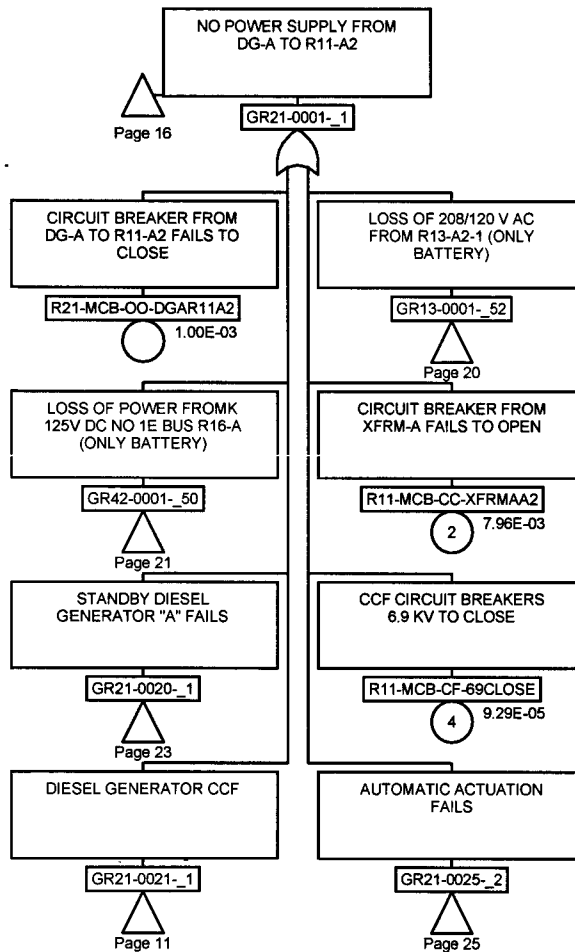


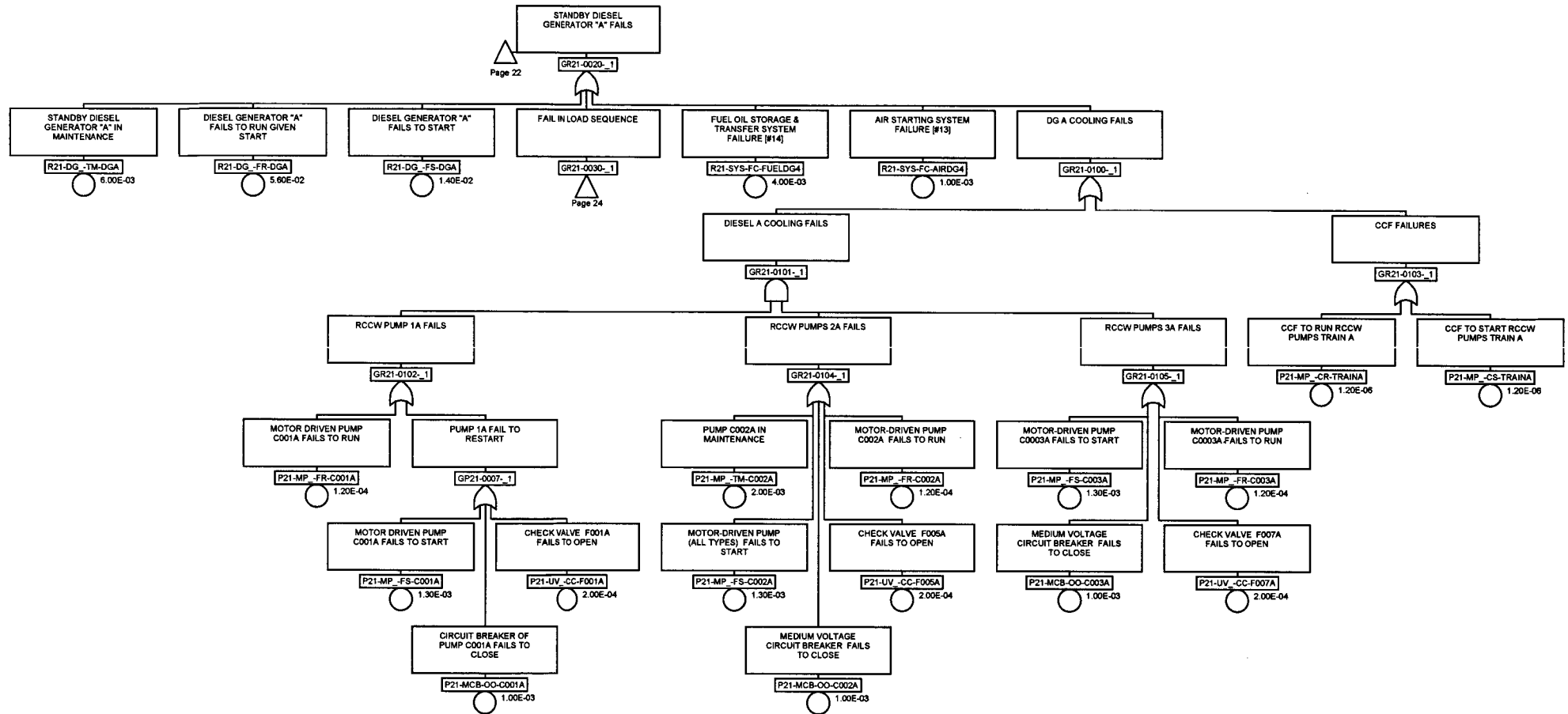


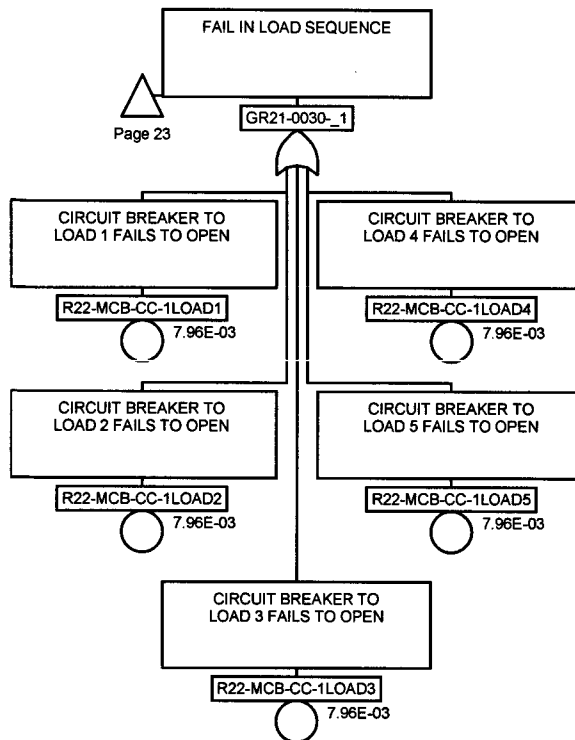




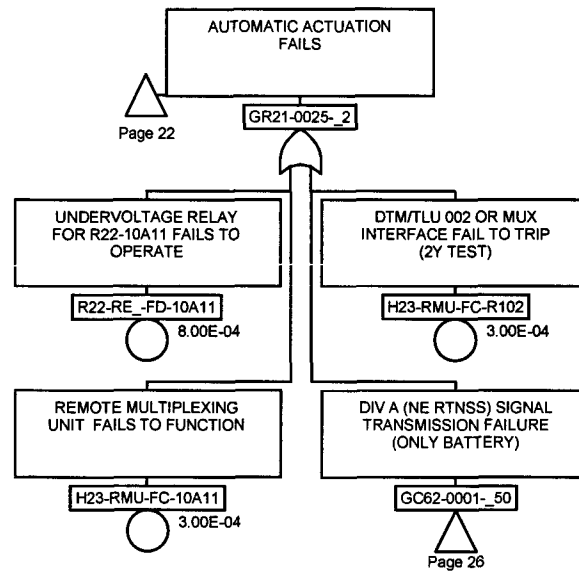
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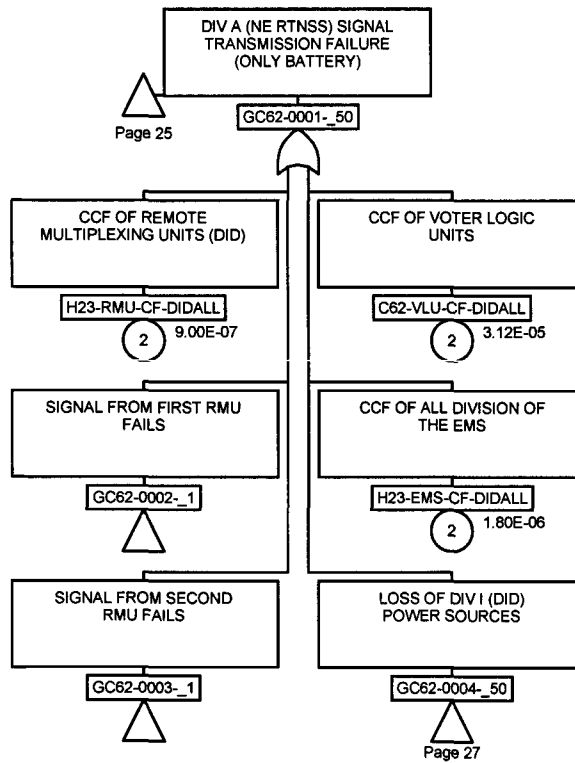


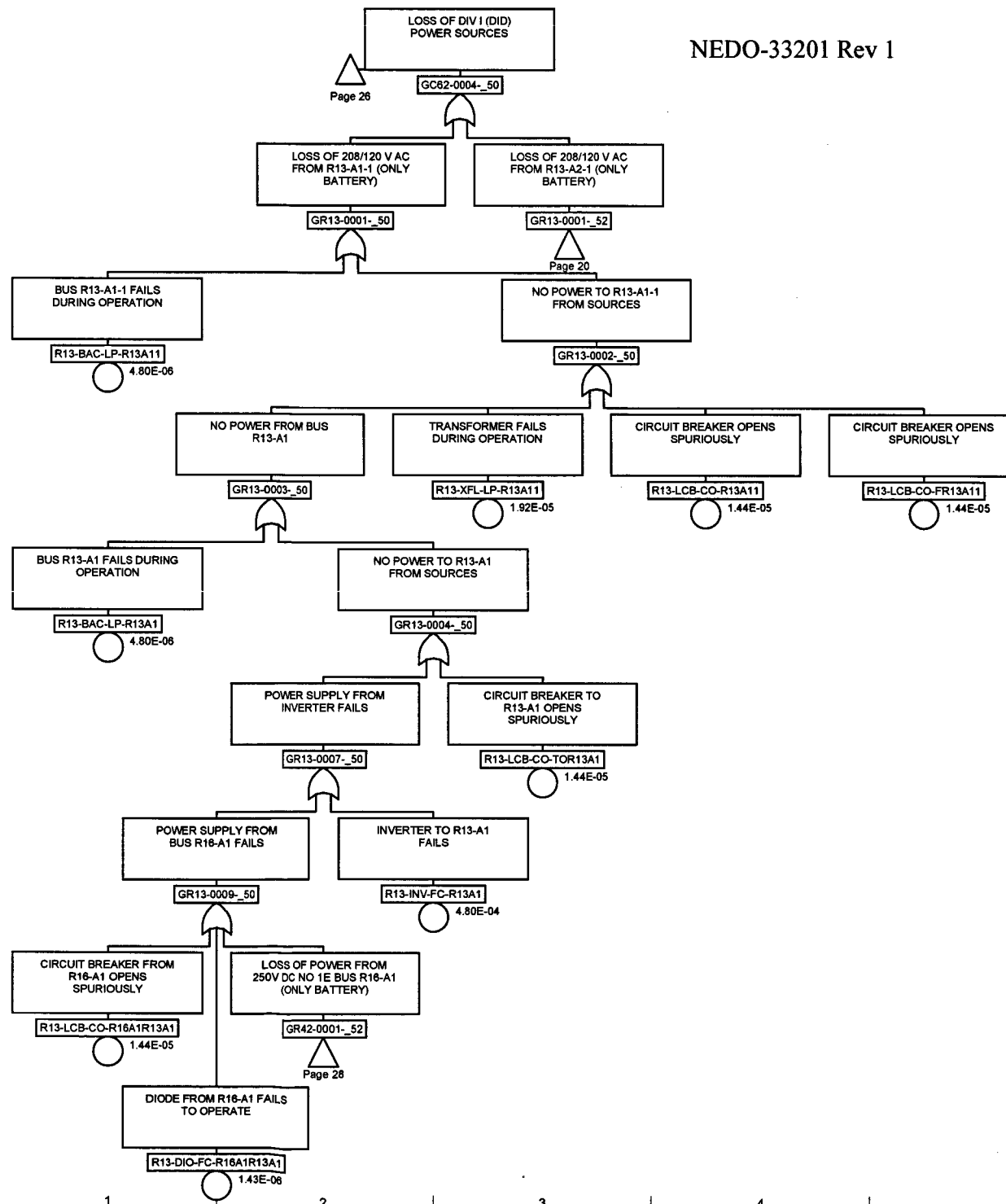


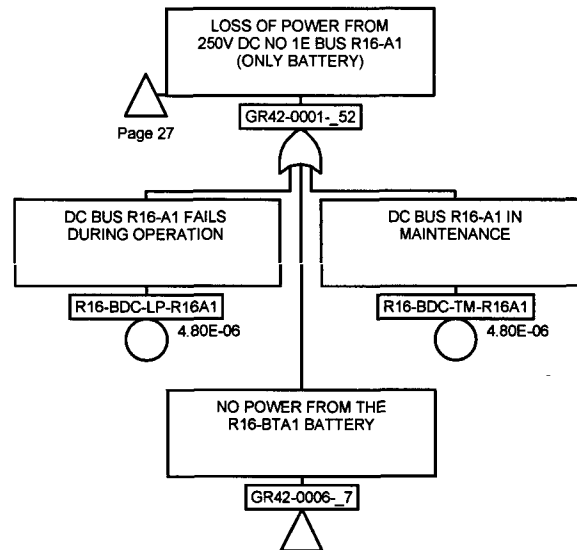


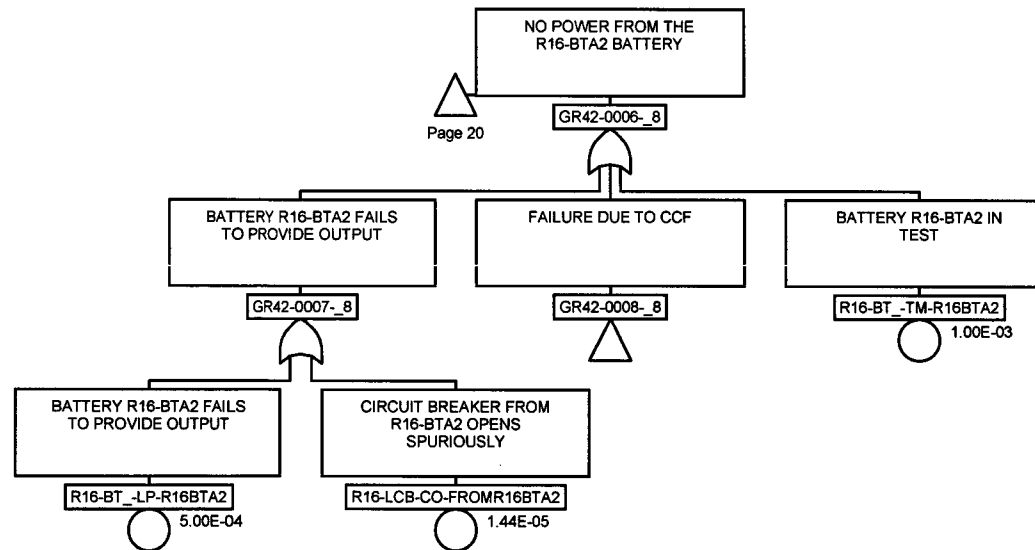
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| Name | Page | Zone | Name | NEDO-33201 Rev 1 | Page | Zone | |
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| GC62-0001- 50 | 26 | 2 | GR13-0002- | 50 | 27 | 3 | |
| GC62-0001- 51 | 12 | 2 | GR13-0002- | 52 | 20 | 3 | |
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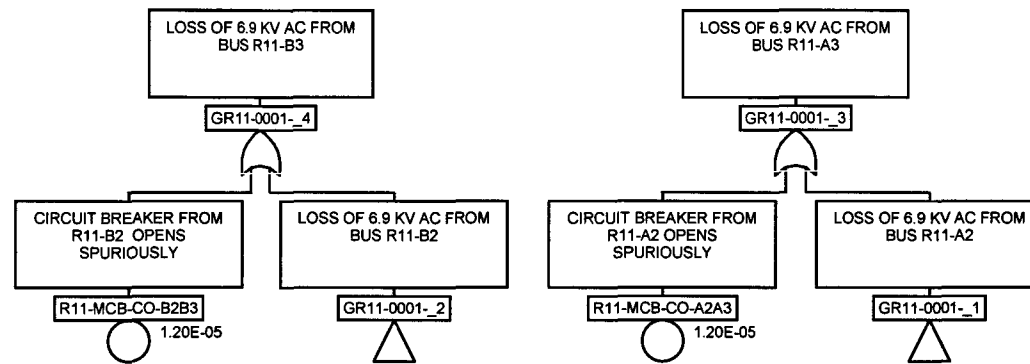
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| | | | | | | Page 31 |

| Name | Page | Zone | Name | Page | Zone | |
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| R13-LCB-CO-R16A1R13A2 | 20 | 1 | R21-SYS-FC-FUELDG5 | 9 | 5 | |
| R13-LCB-CO-R16A1R13B1 | 14 | 1 | R22-MCB-CC-1LOAD1 | 24 | 1 | |
| R13-LCB-CO-R16A1R13B2 | 6 | 1 | R22-MCB-CC-1LOAD2 | 24 | 1 | |
| R13-LCB-CO-TOR13A1 | 27 | 3 | R22-MCB-CC-1LOAD3 | 24 | 2 | |
| R13-LCB-CO-TOR13A2 | 20 | 4 | R22-MCB-CC-1LOAD4 | 24 | 2 | |
| R13-LCB-CO-TOR13B1 | 14 | 3 | R22-MCB-CC-1LOAD5 | 24 | 2 | |
| R13-LCB-CO-TOR13B2 | 6 | 4 | R22-MCB-CC-2LOAD1 | 10 | 1 | |
| R13-XFL-LP-R13A11 | 27 | 3 | R22-MCB-CC-2LOAD2 | 10 | 1 | |
| R13-XFL-LP-R13A21 | 20 | 3 | R22-MCB-CC-2LOAD3 | 10 | 2 | |
| R13-XFL-LP-R13B11 | 14 | 3 | R22-MCB-CC-2LOAD4 | 10 | 2 | |
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| R16-BDC-LP-R16A | 21 | 1 | R22-RE -FD-10A11 | 25 | 1 | |
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| R16-BDC-LP-R16B | 7 | 1 | XHOSLOSP | 2 | 1 | |
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| R16-BDC-LP-R16B2 | 6 | 3 | XHOSLOSP | 17 | 1 | |
| R16-BDC-TM-R16A | 21 | 4 | | | | |
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| R16-BDC-TM-R16B2 | 6 | 4 | | | | |
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| R16-BT -CF-125NO1E | 21 | 3 | | | | |
| R16-BT -CF-ALLBATT | 7 | 4 | | | | |
| R16-BT -CF-ALLBATT | 21 | 4 | | | | |
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| R16-BT -LP-R16BTA2 | 29 | 1 | | | | |
| R16-BT -LP-R16BTB | 7 | 1 | | | | |
| R16-BT -LP-R16BTB1 | 15 | 1 | | | | |
| R16-BT -TM-R16BTA | 21 | 5 | | | | |
| R16-BT -TM-R16BTA2 | 29 | 4 | | | | |
| R16-BT -TM-R16BTB | 7 | 5 | | | | |
| R16-BT -TM-R16BTB1 | 15 | 4 | | | | |
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| R16-LCB-CO-FROMR16BTA2 | 29 | 2 | | | | |
| R16-LCB-CO-FROMR16BTB | 7 | 2 | | | | |
| R16-LCB-CO-FROMR16BTB1 | 15 | 2 | | | | |
| R11A2B2 | | | R11A2B2-NEW-A.caf | | | Appendix B.4.14-2 |
| | | | | | | Page 32 |

Appendix B.4.14-3

A.C. Electric Power System

Fault Tree

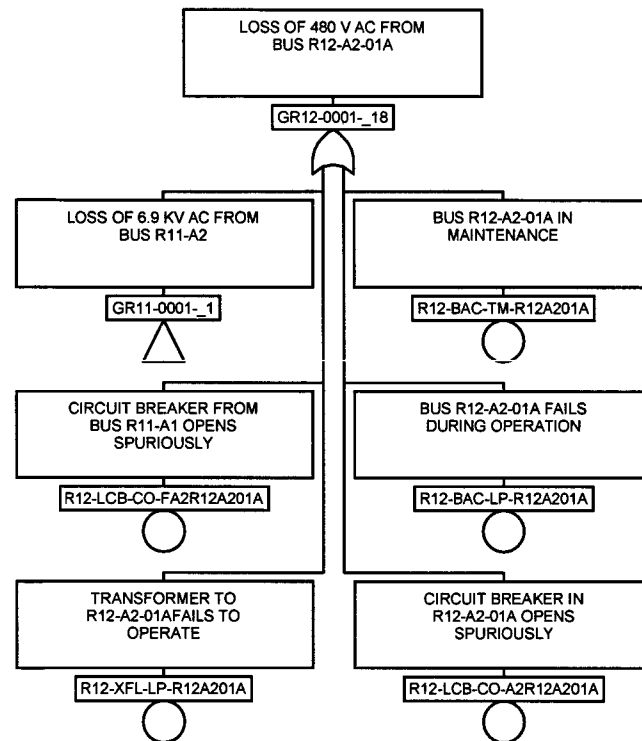


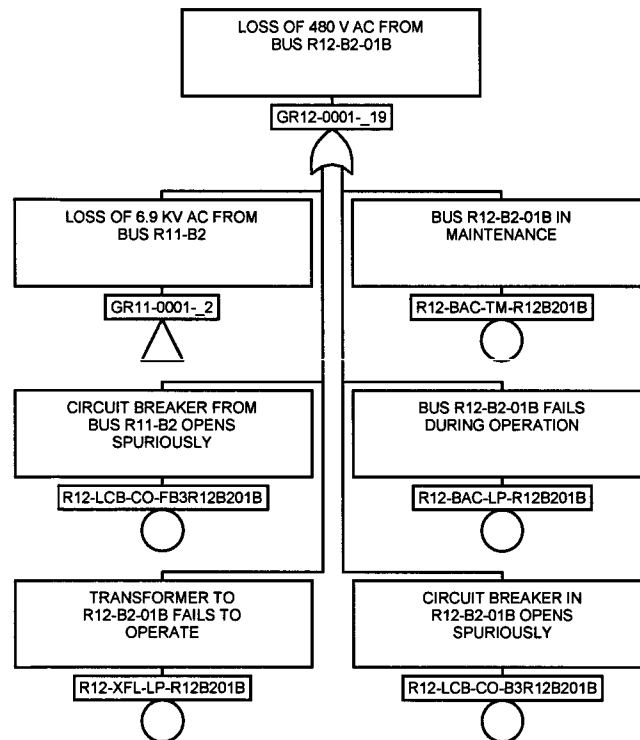
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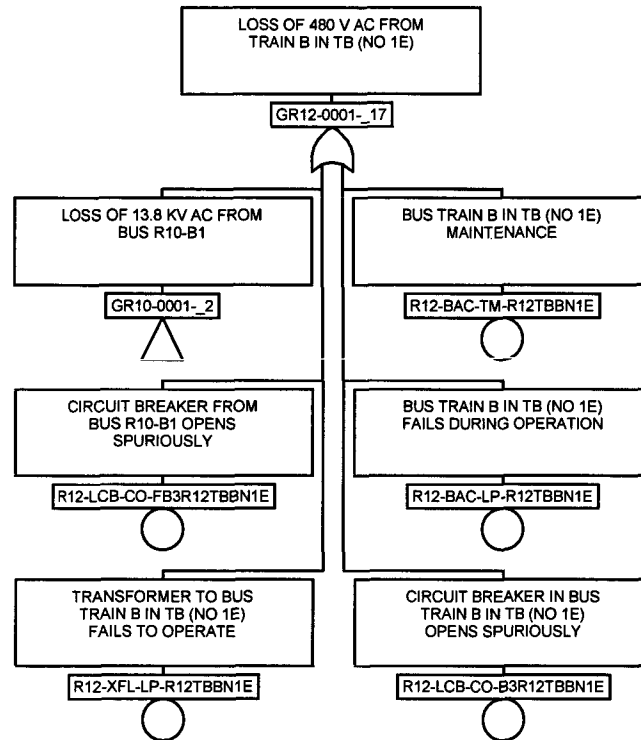
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A.C. Electric Power System

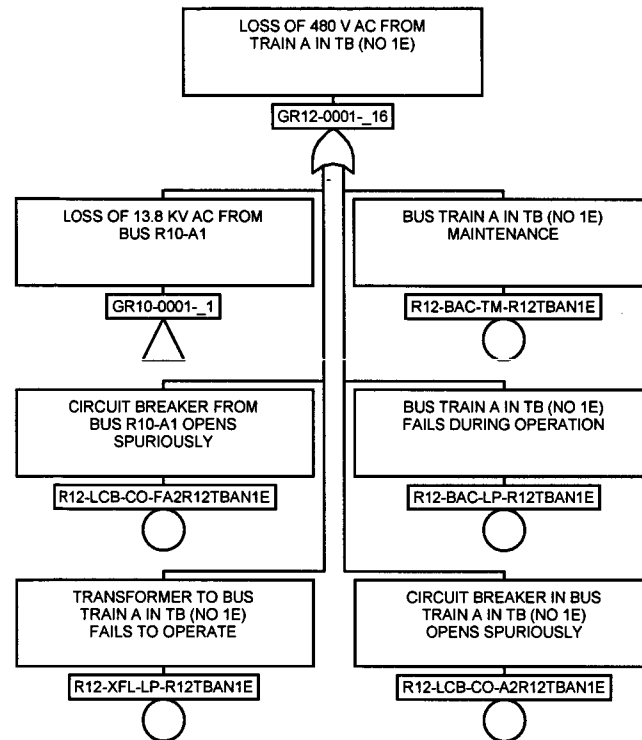
Fault Tree

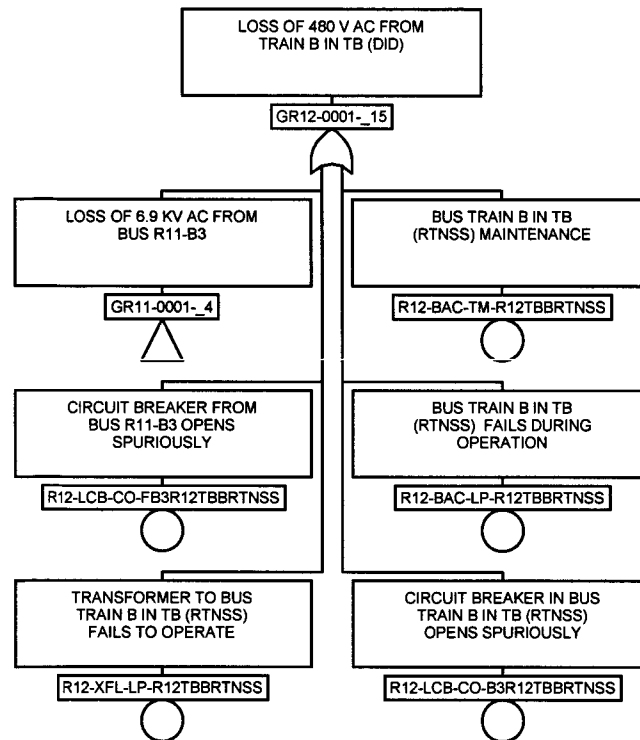




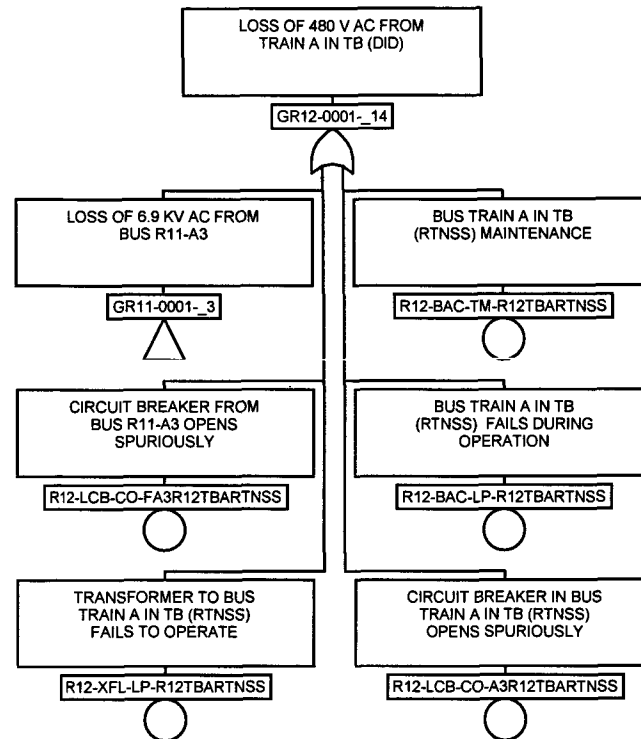


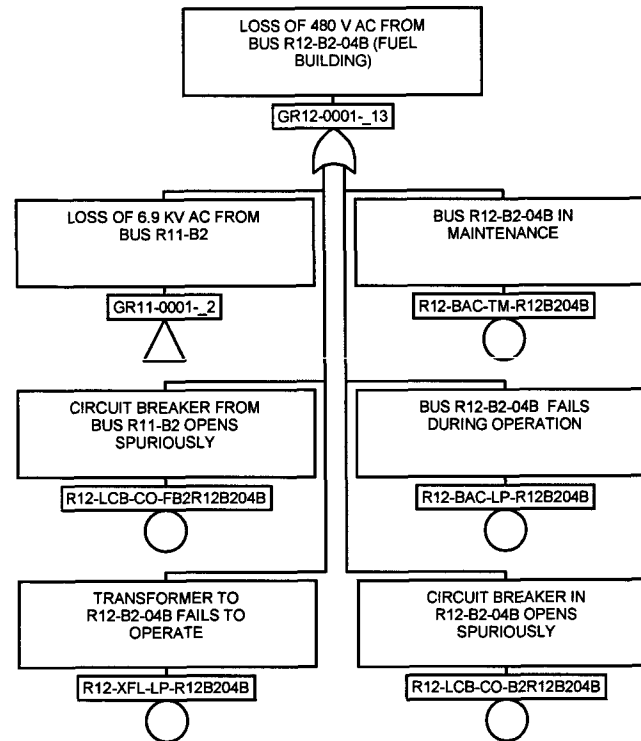
NEDO-33201 Rev 1





NEDO-33201 Rev 1

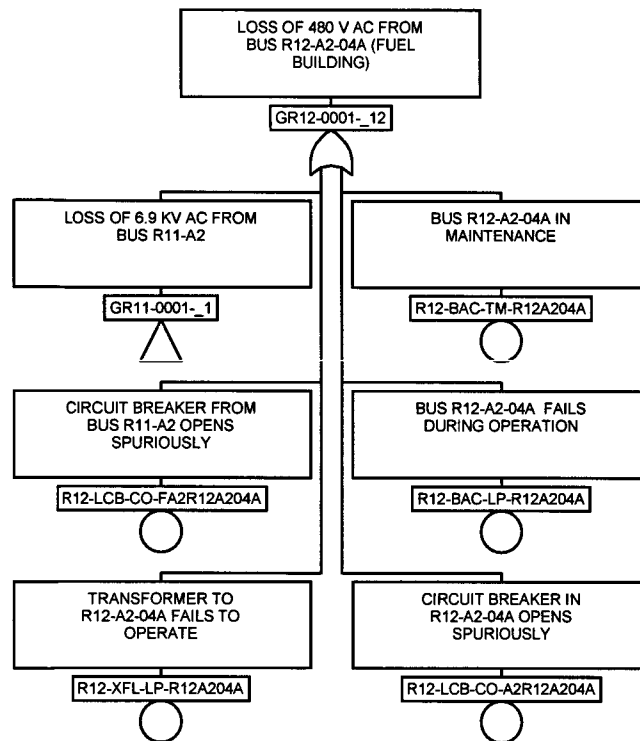


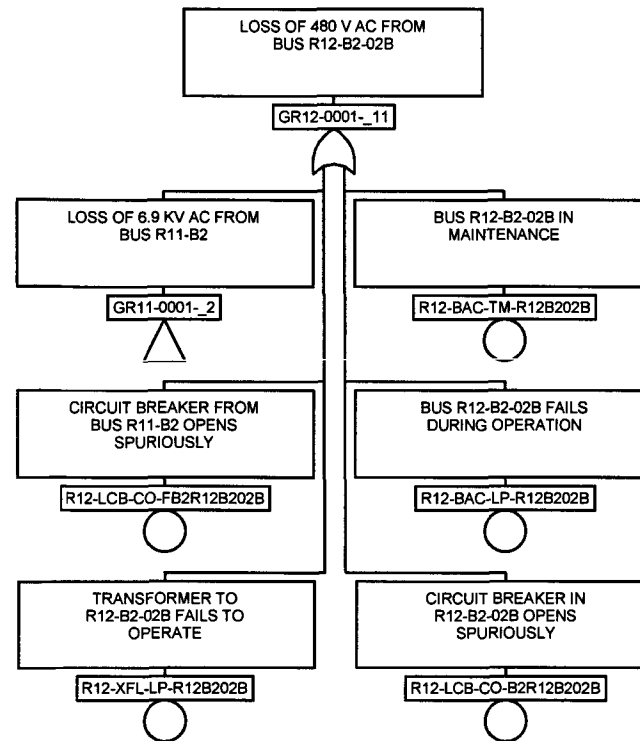


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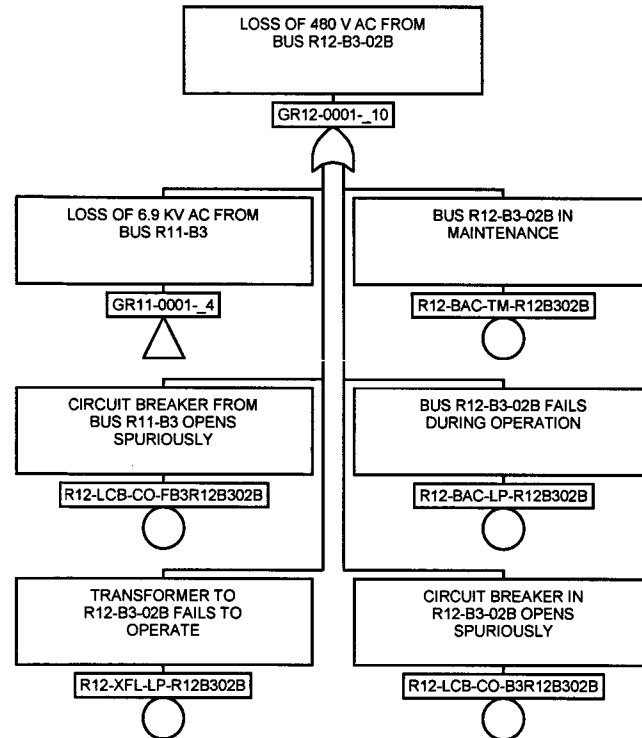
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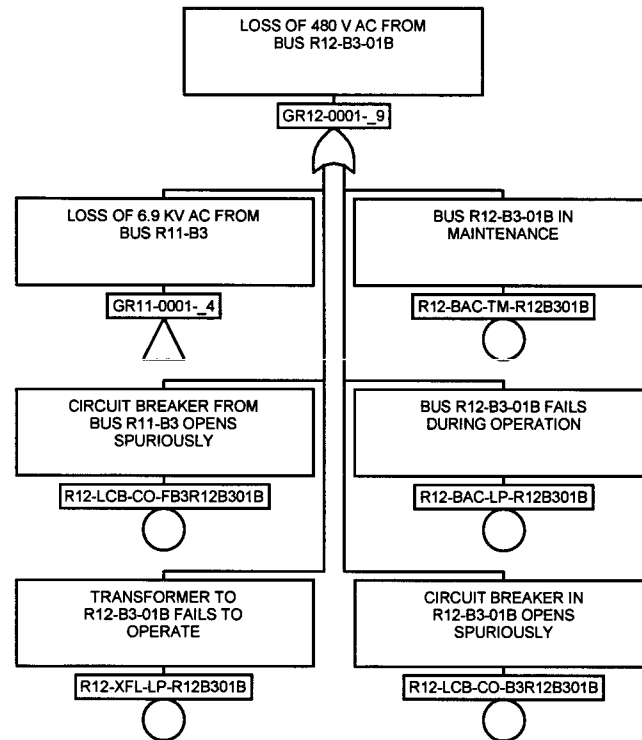
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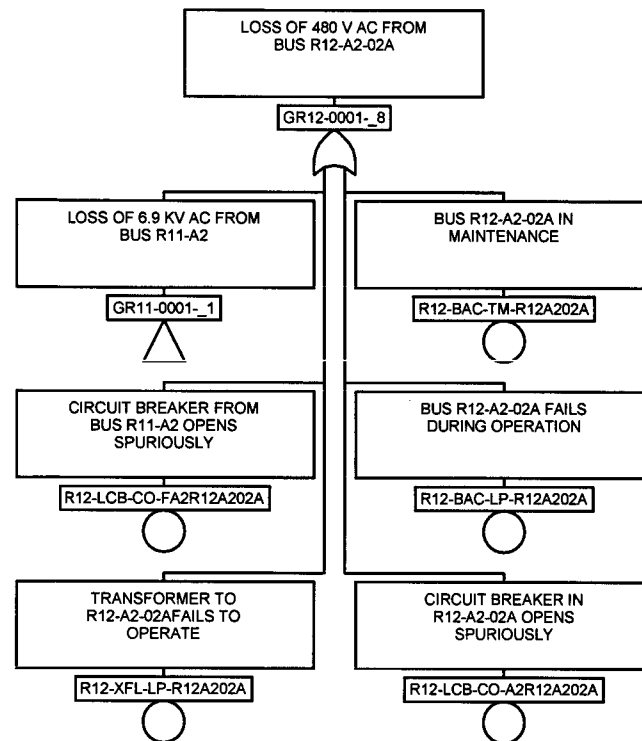


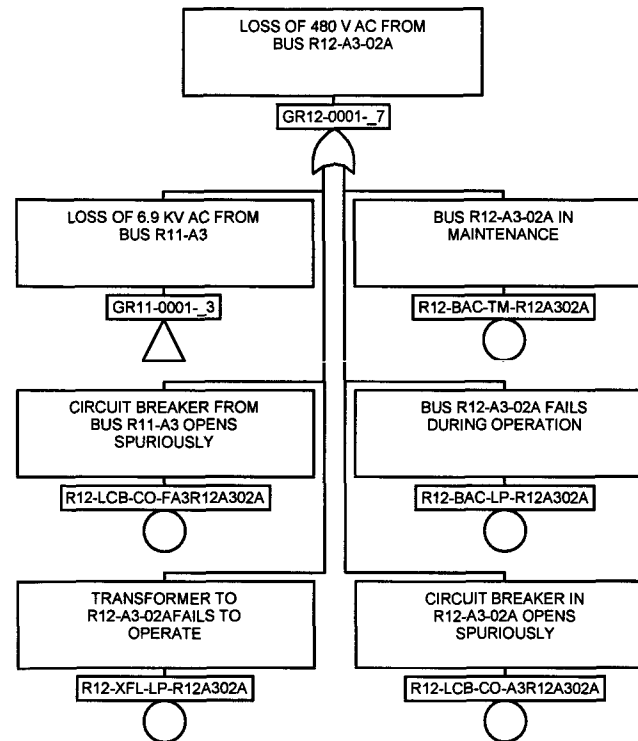


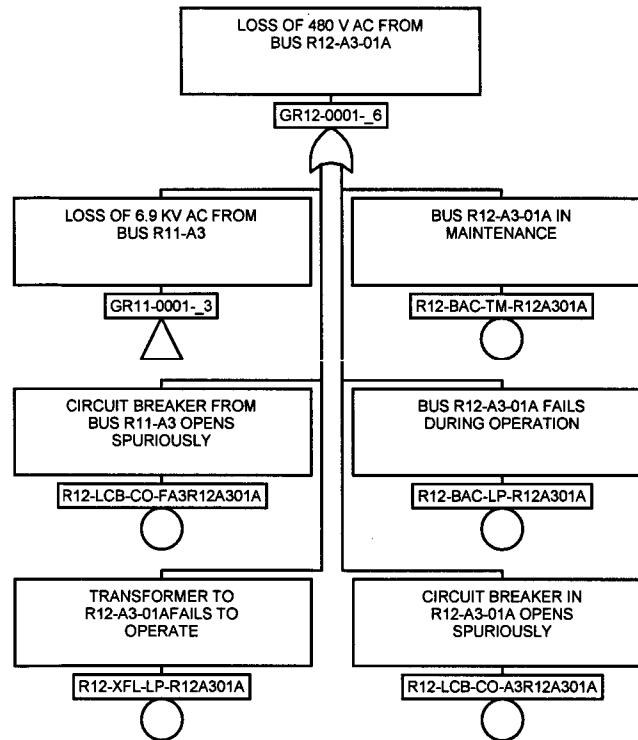
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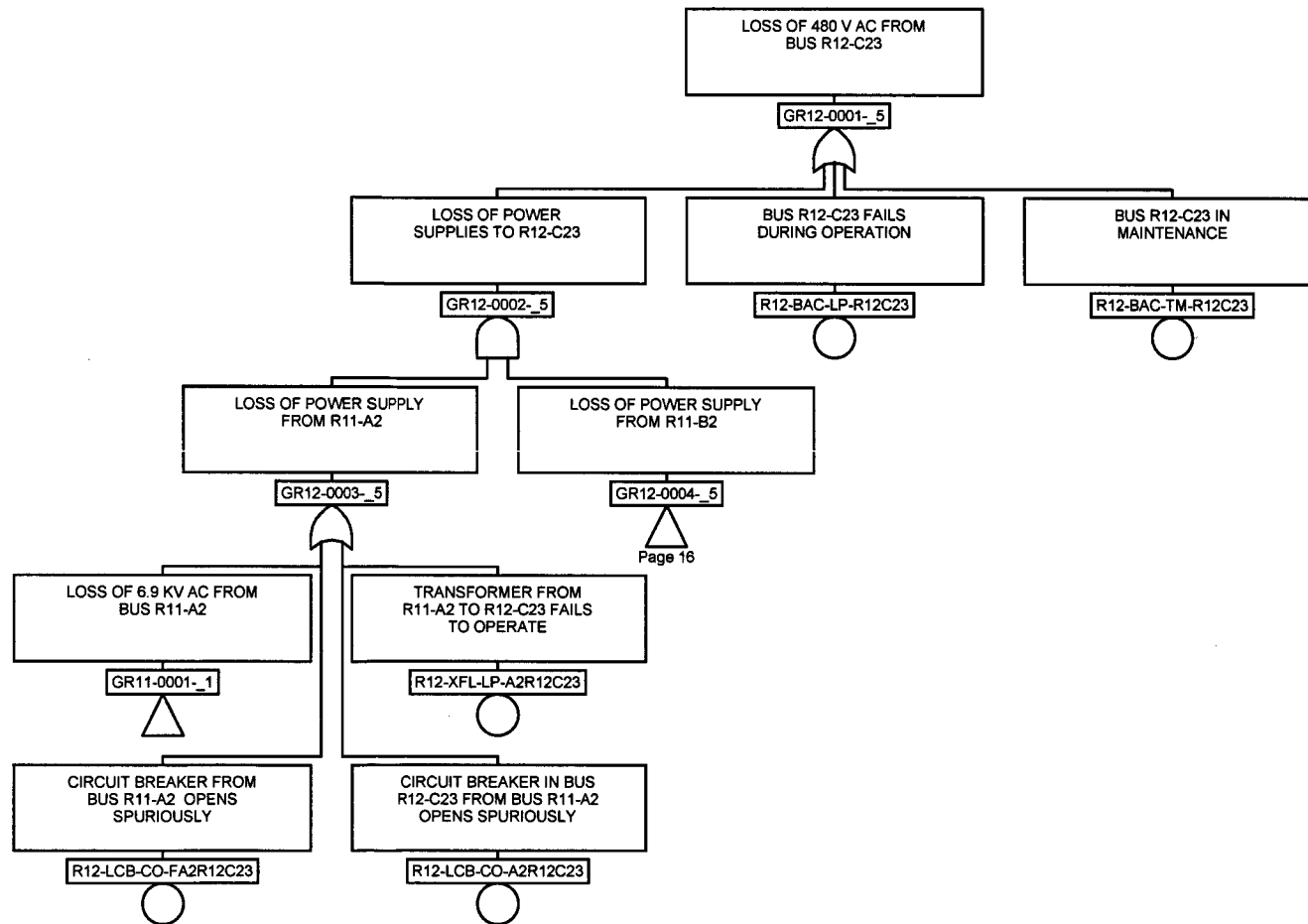


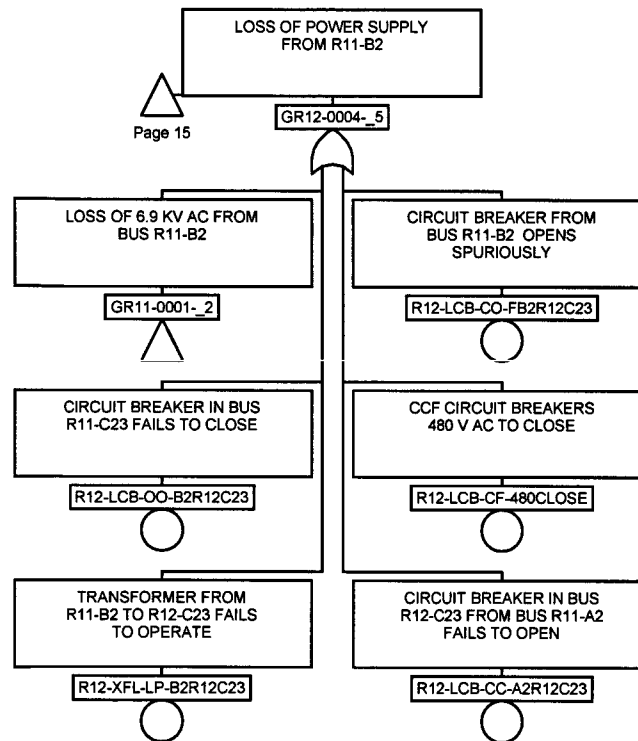


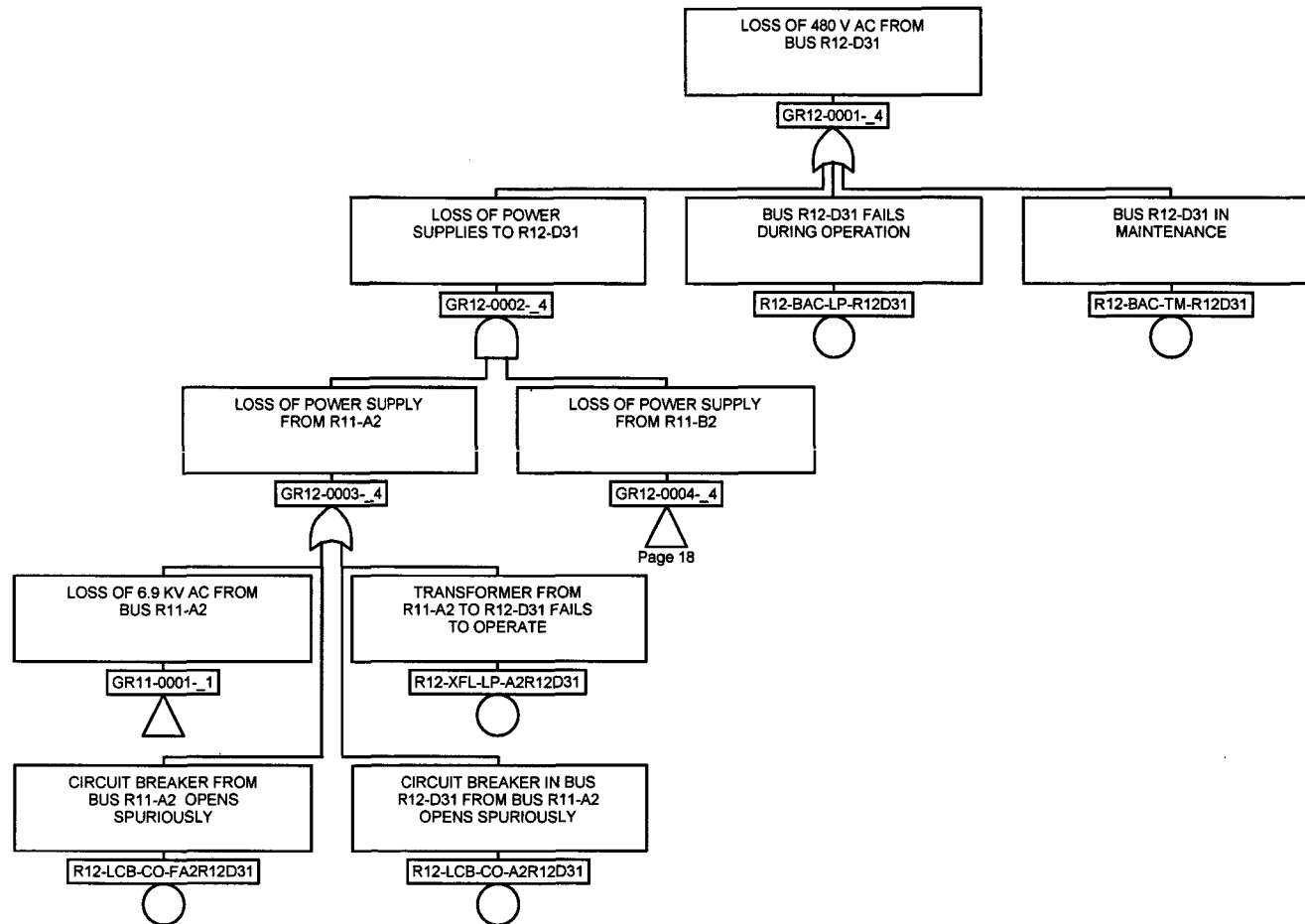




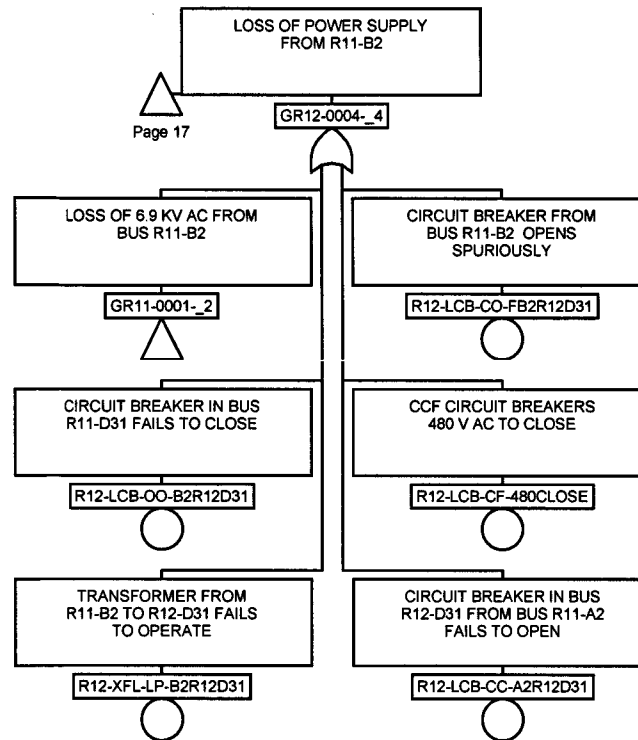


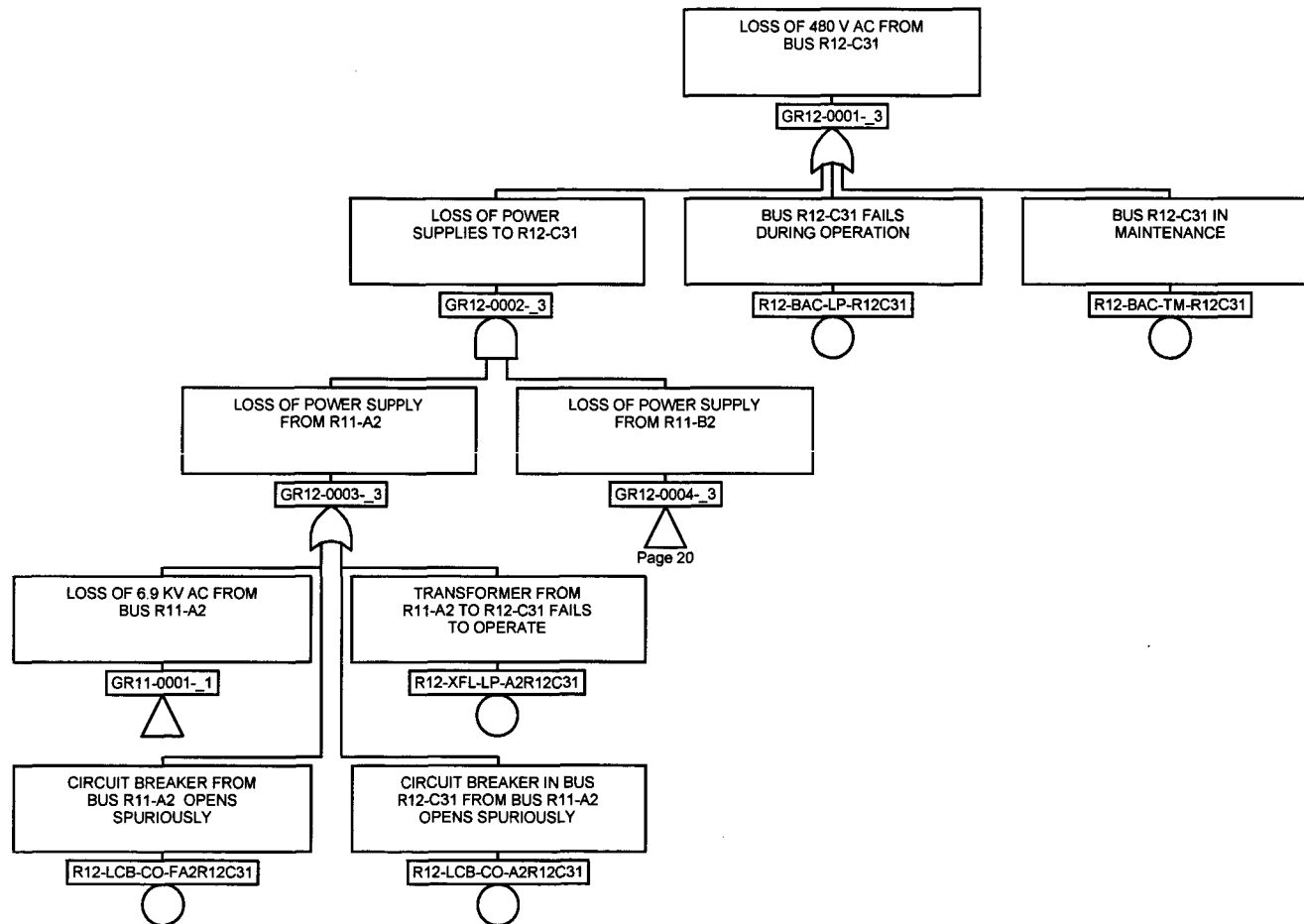


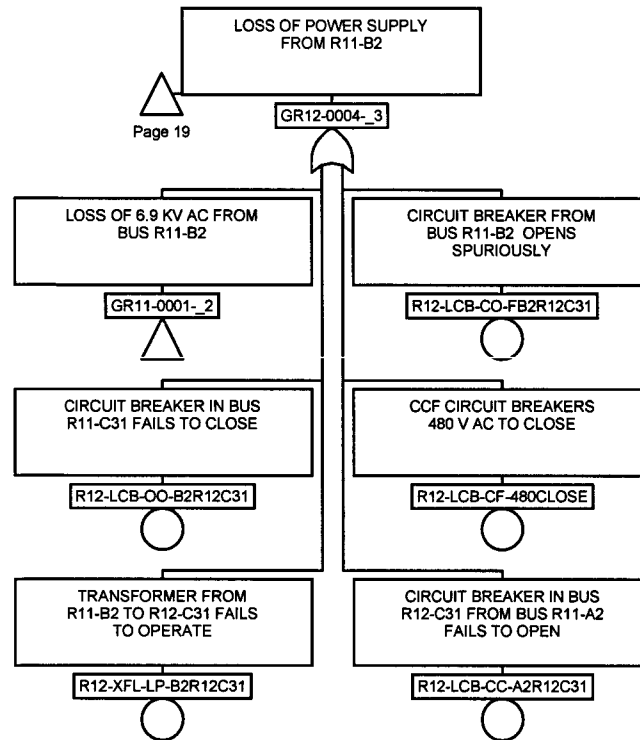




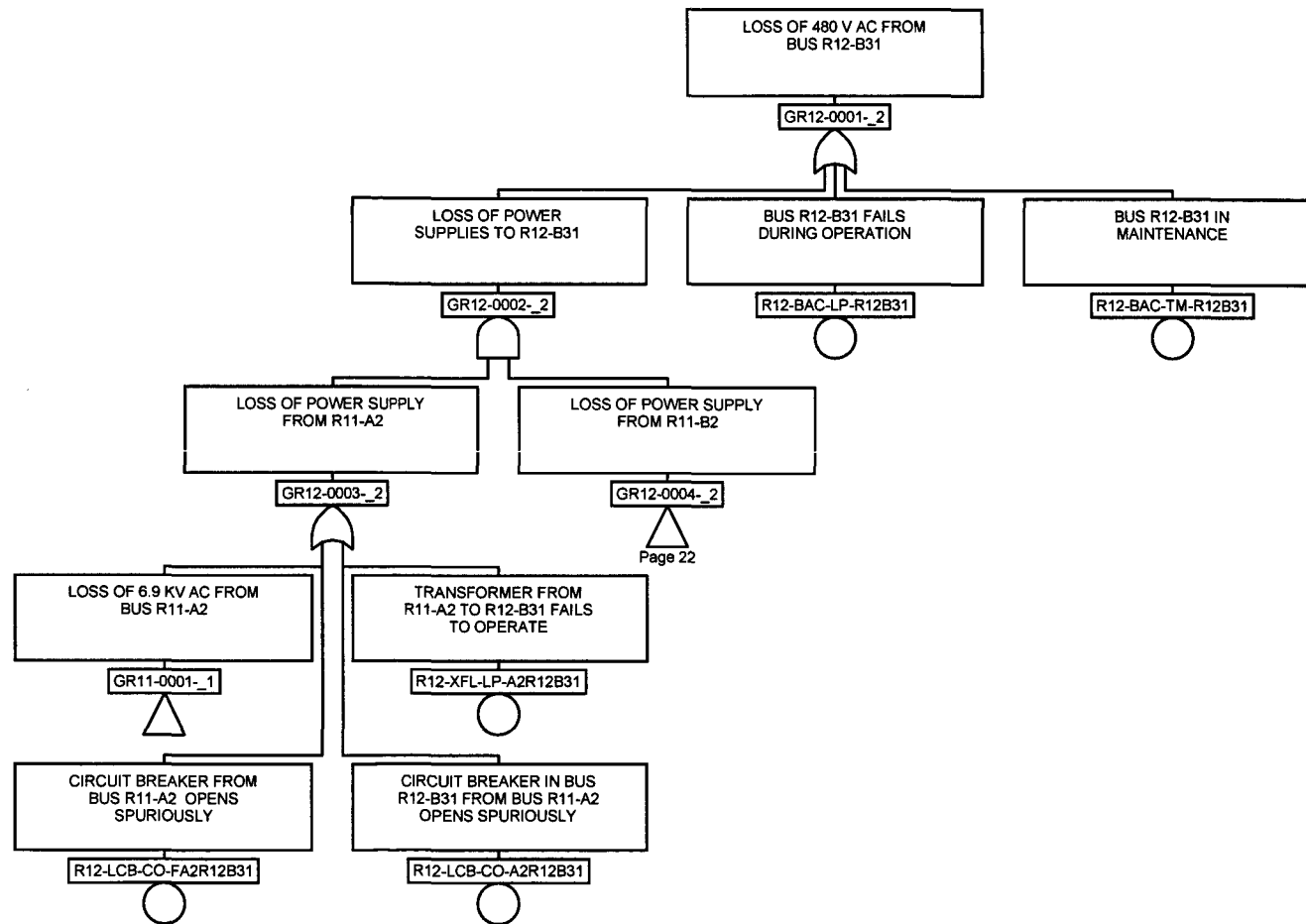
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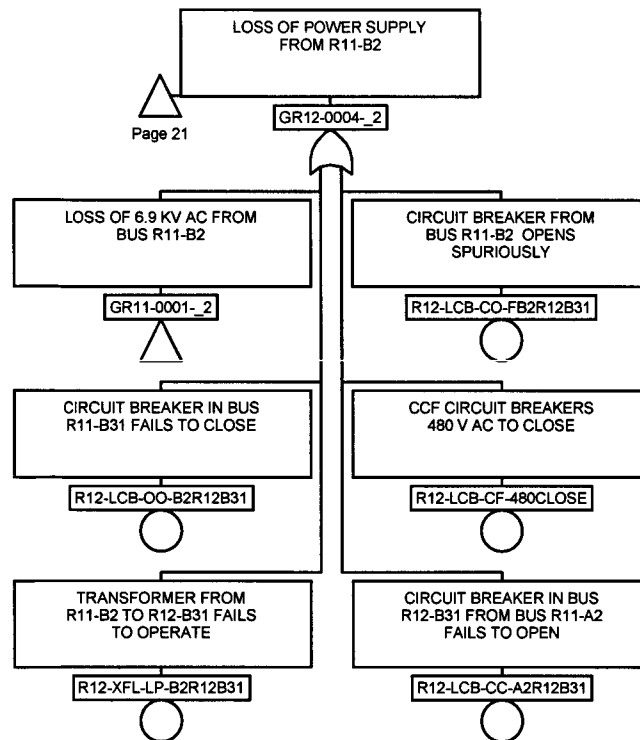


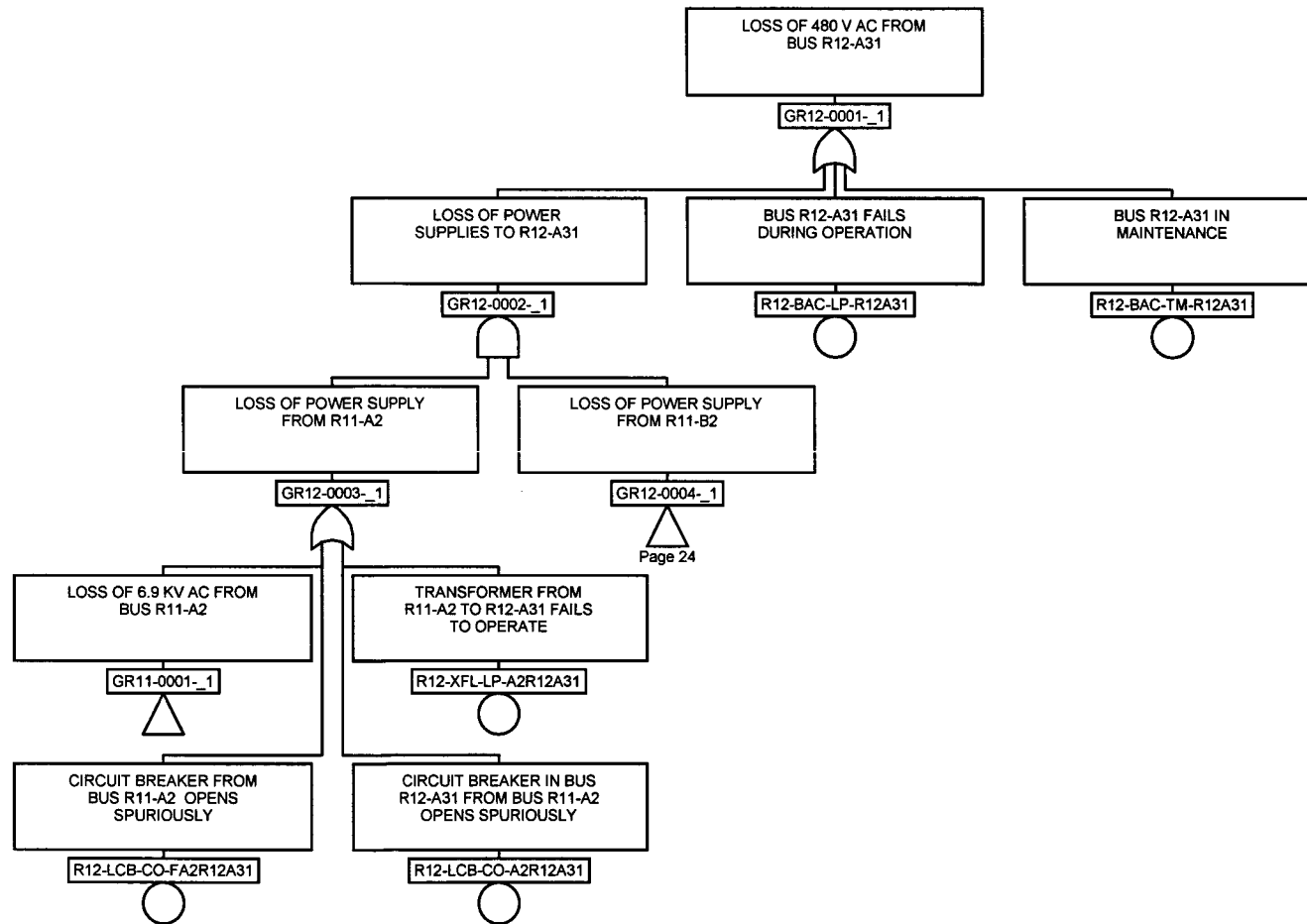


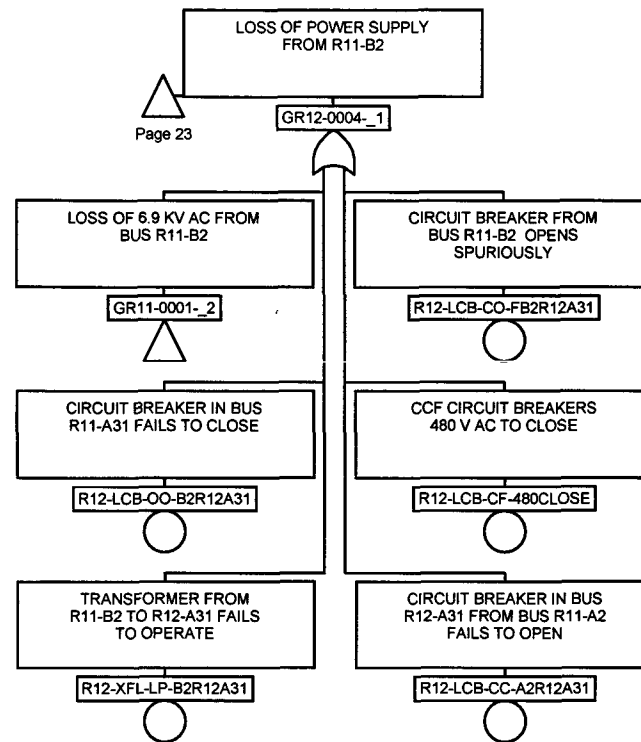


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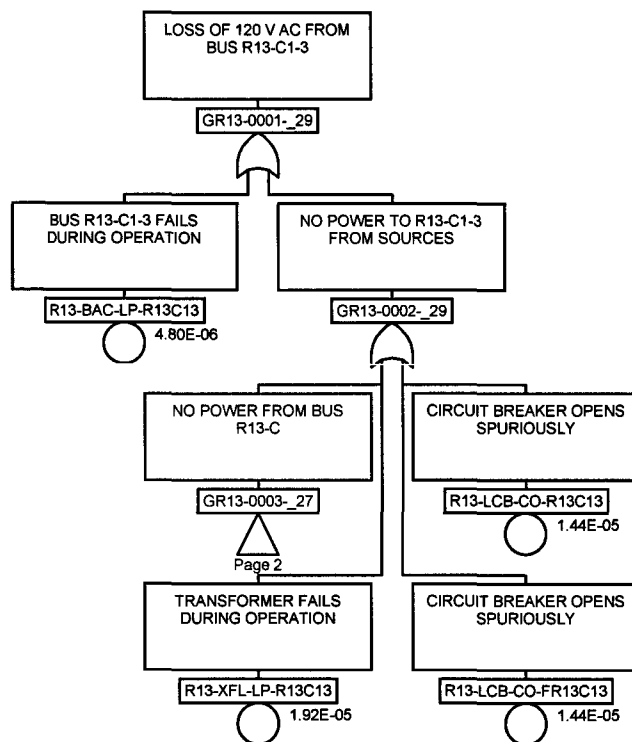


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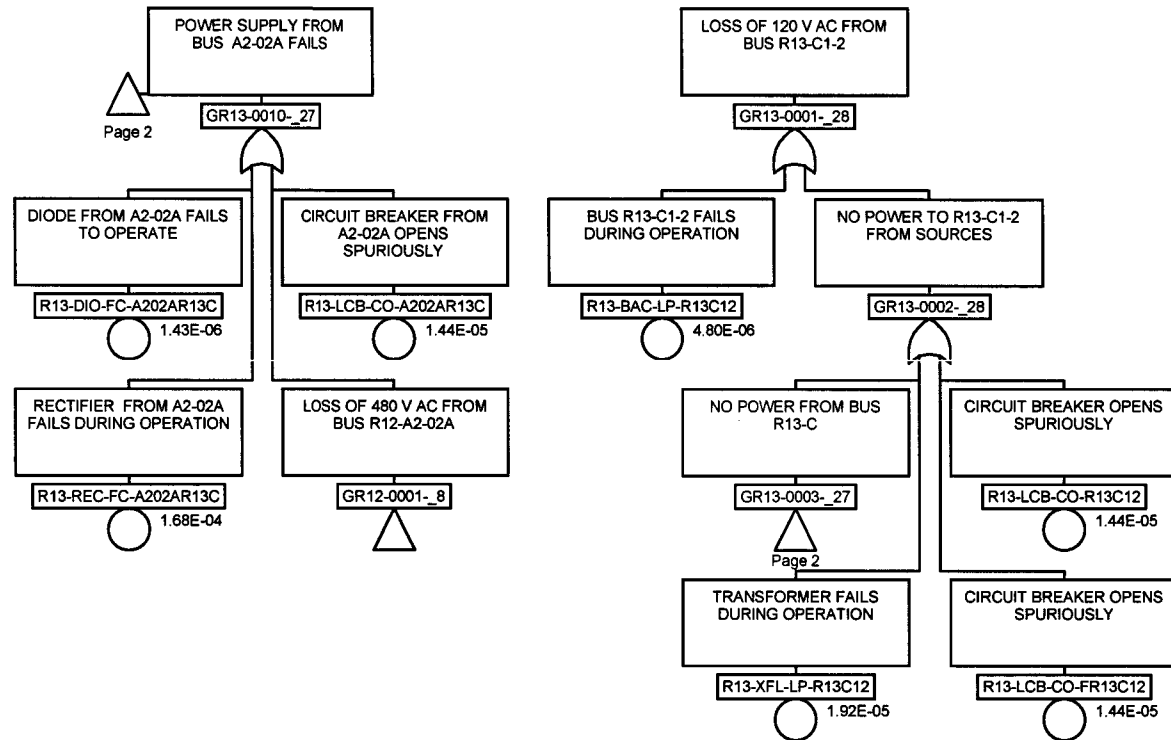
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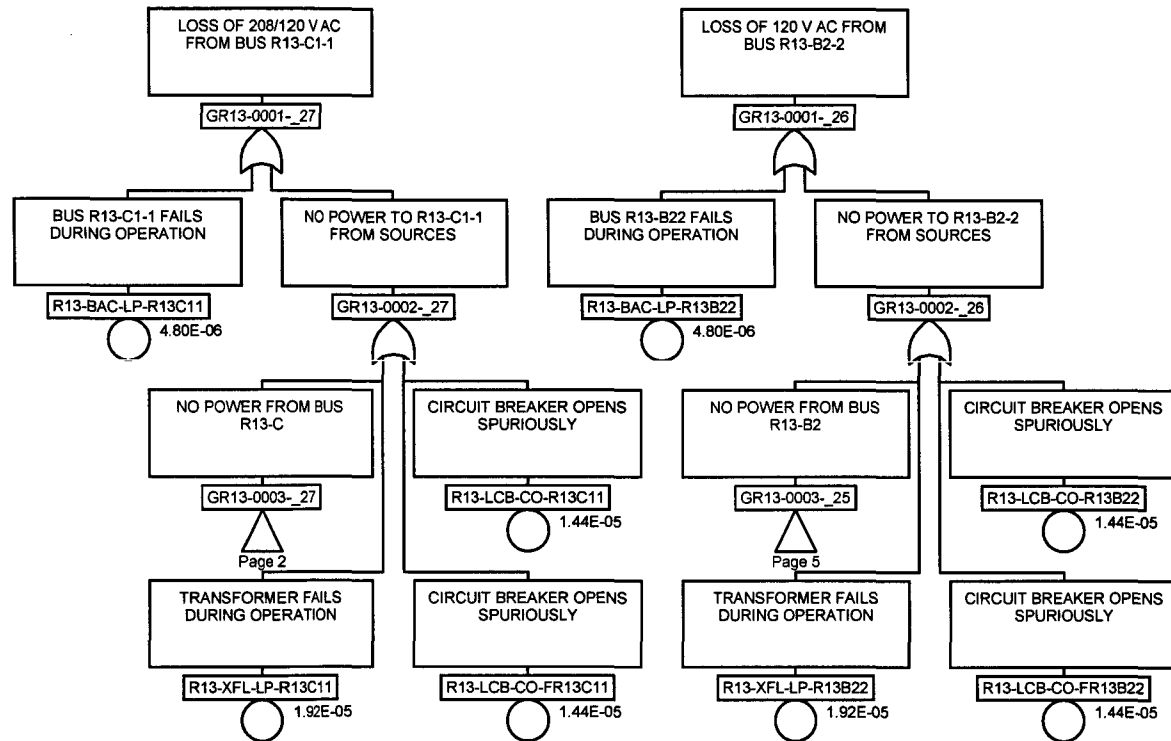
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Appendix B.4.15 Uninterruptible A.C. Power Supply System Fault Tree

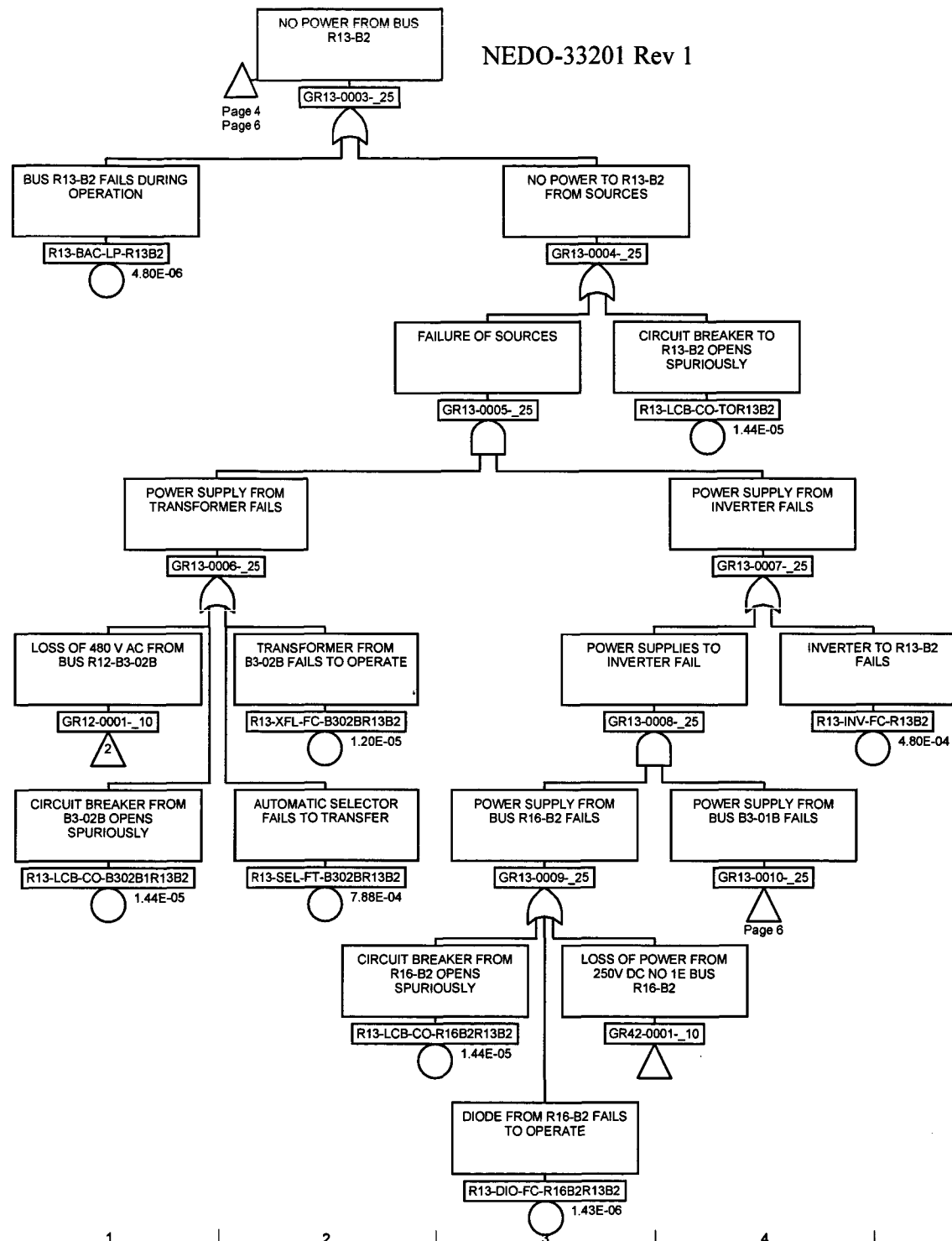


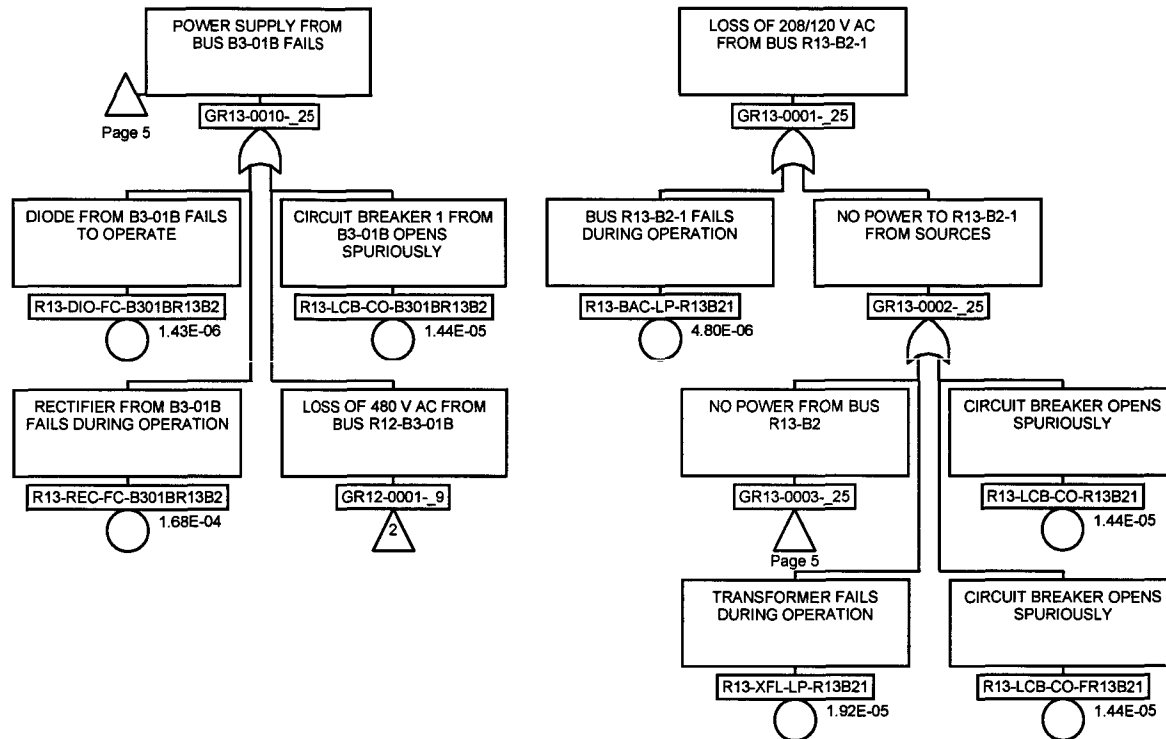


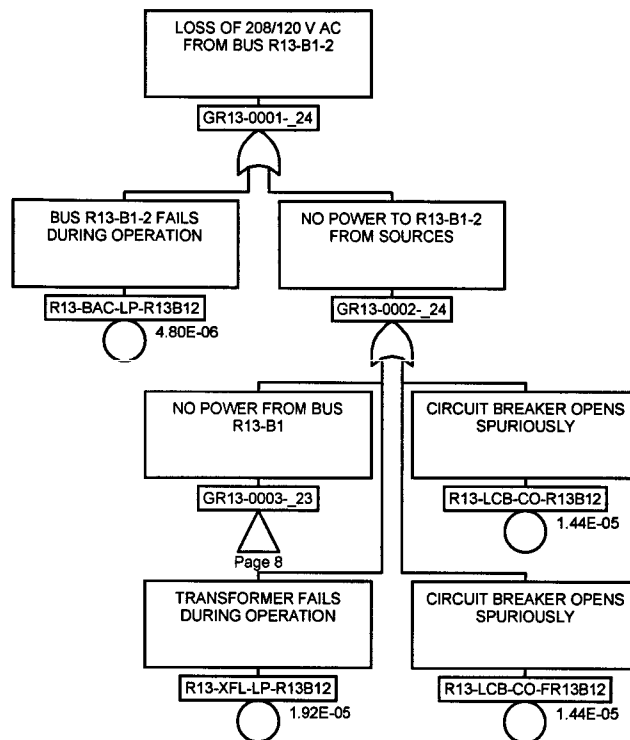




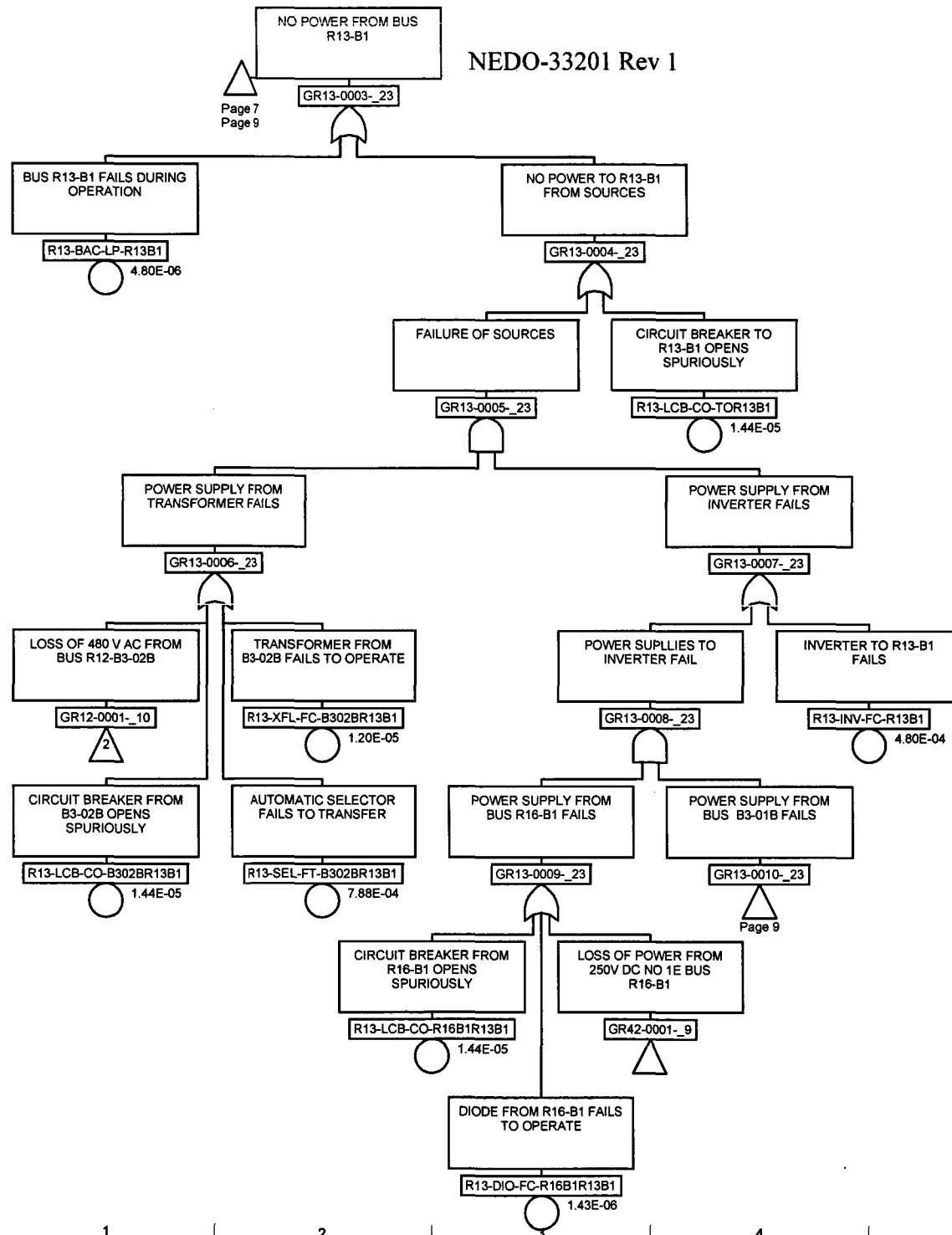
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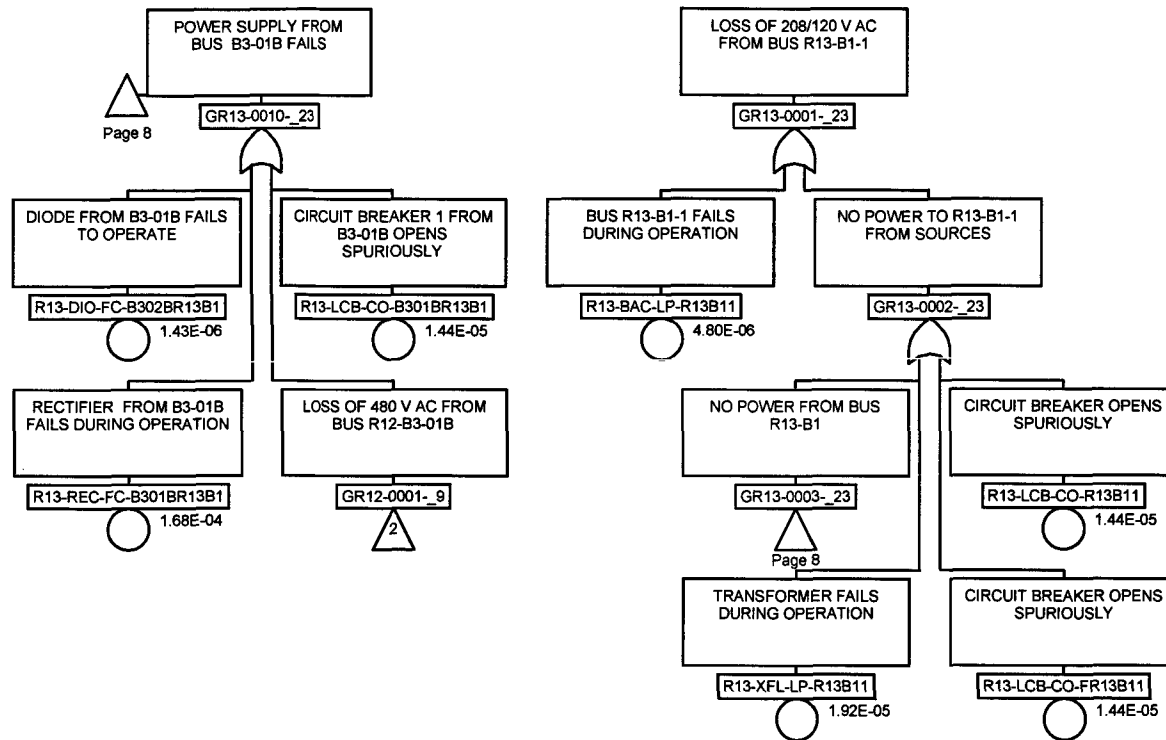


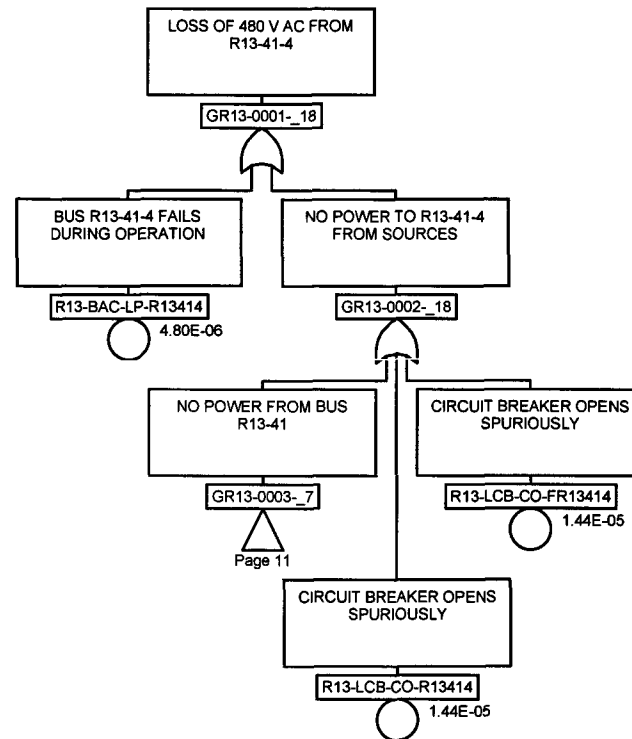
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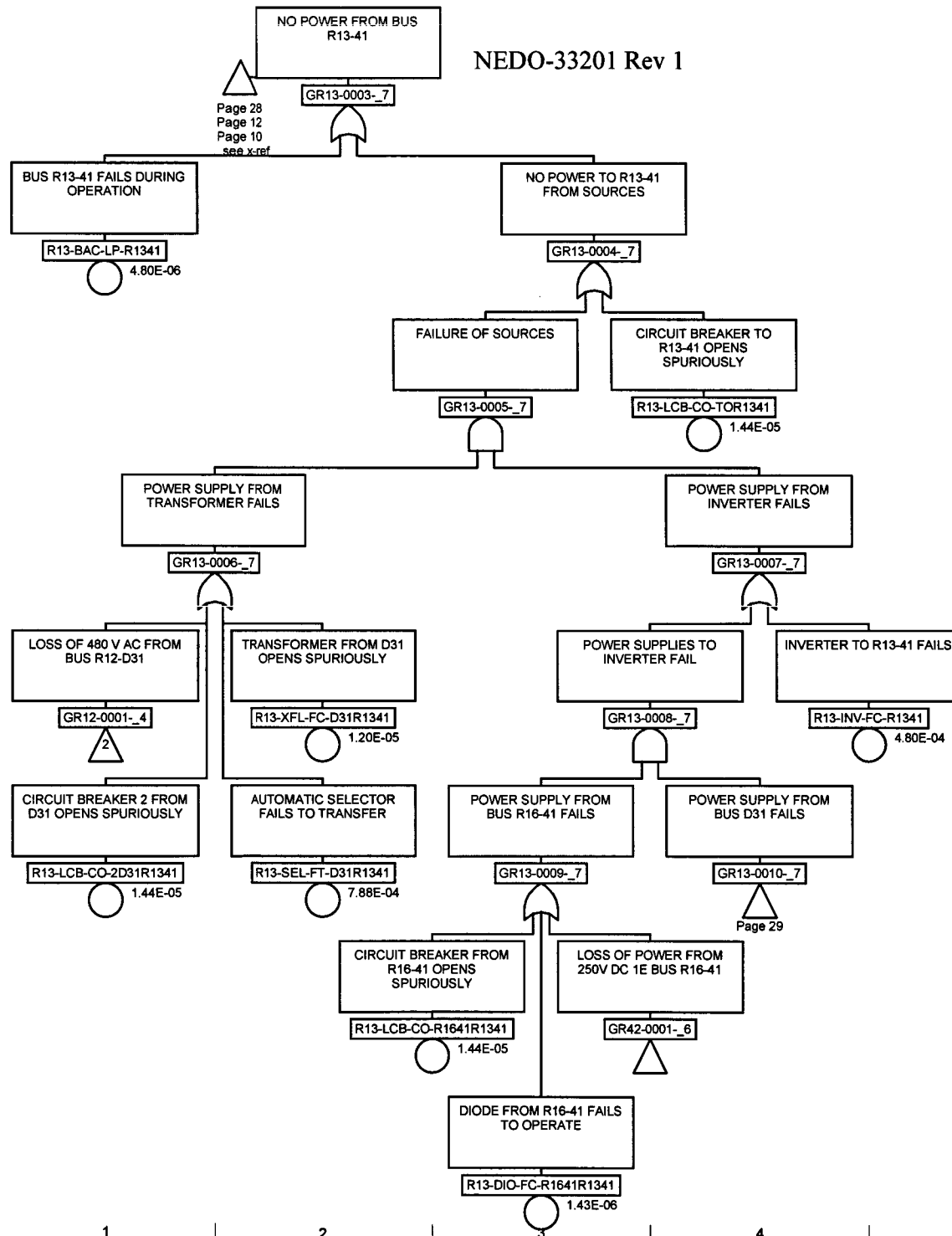
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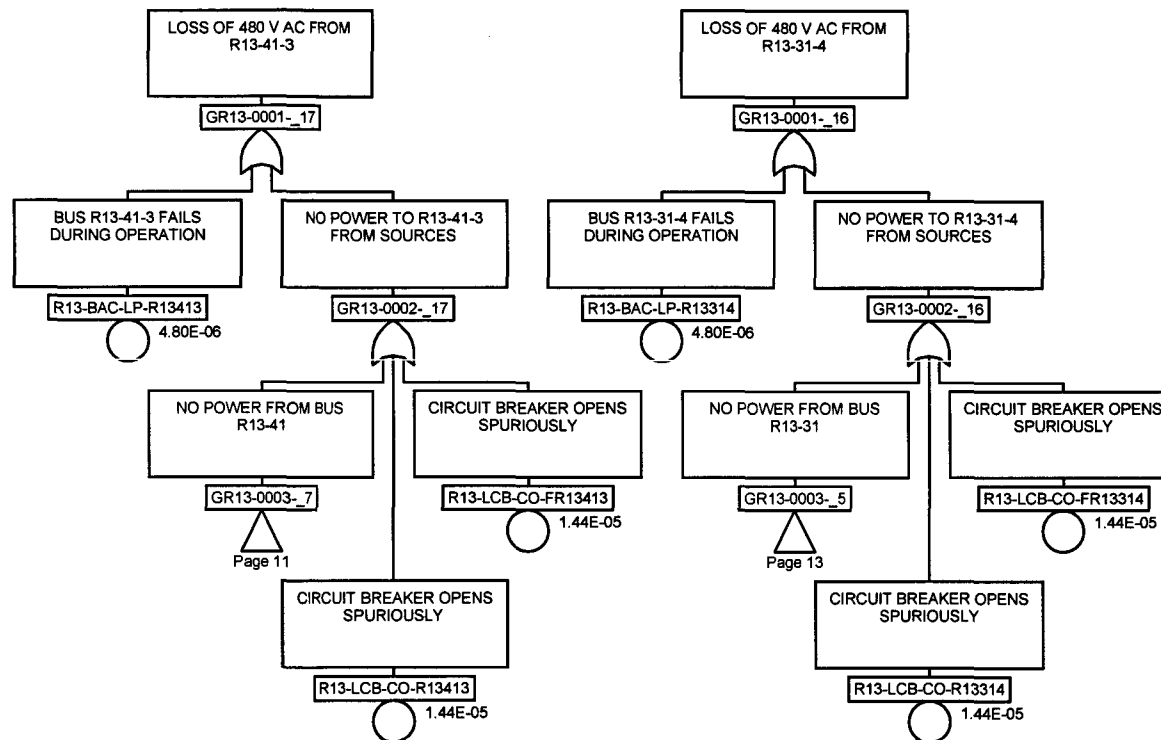


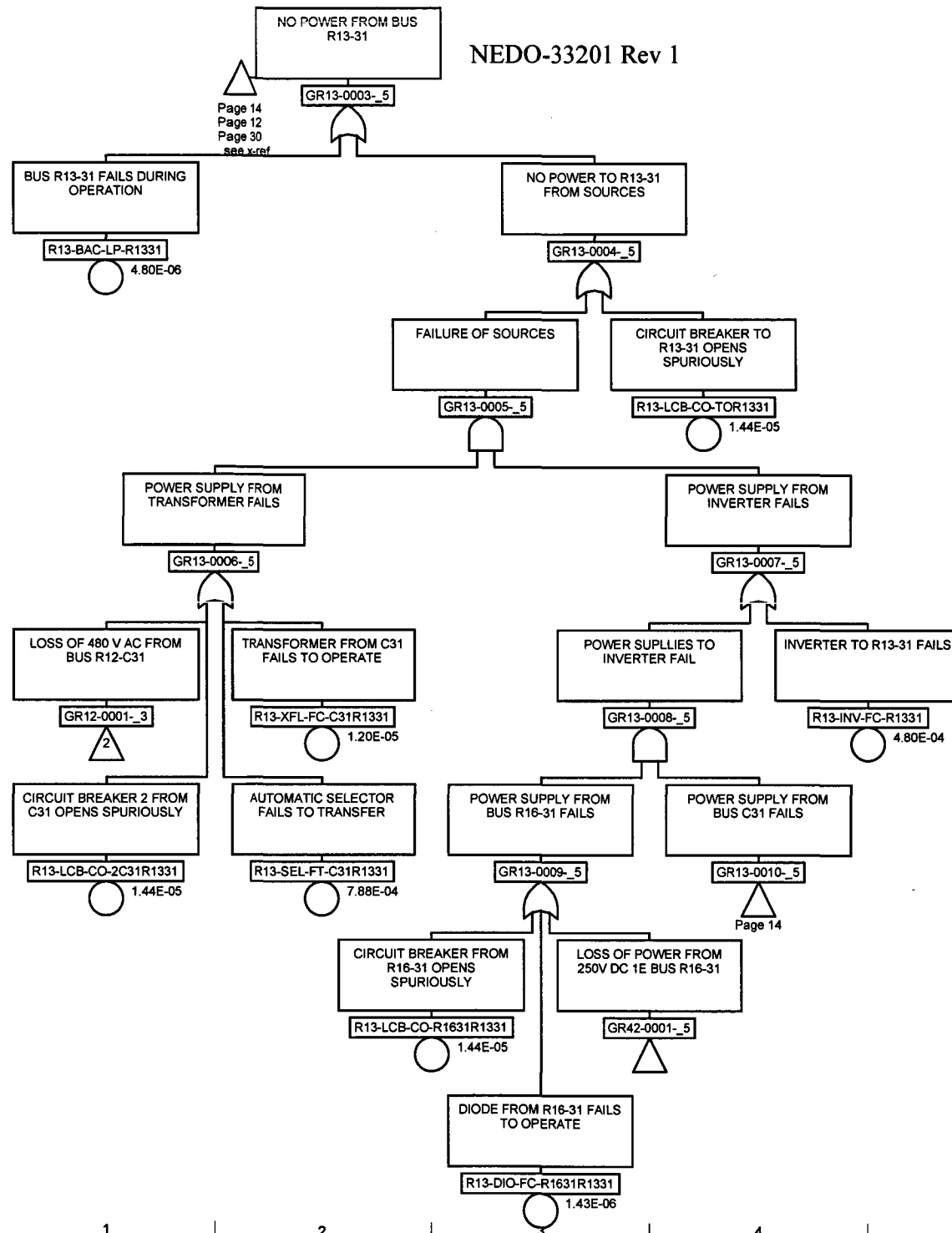
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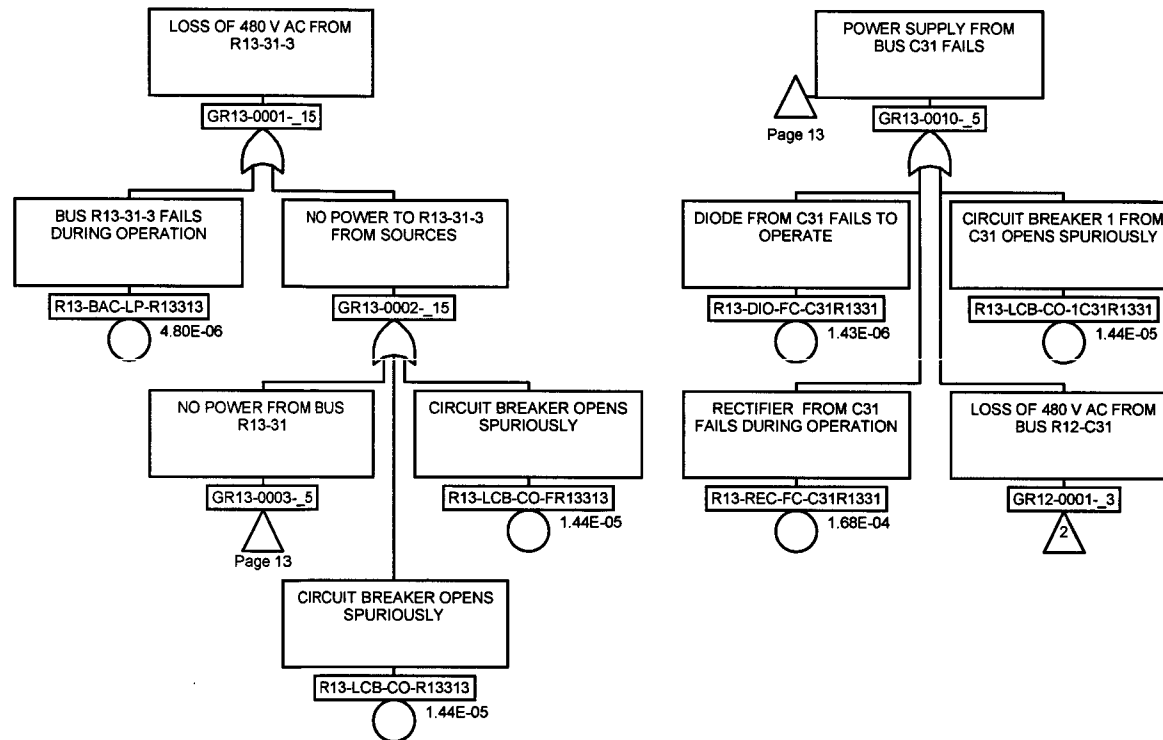


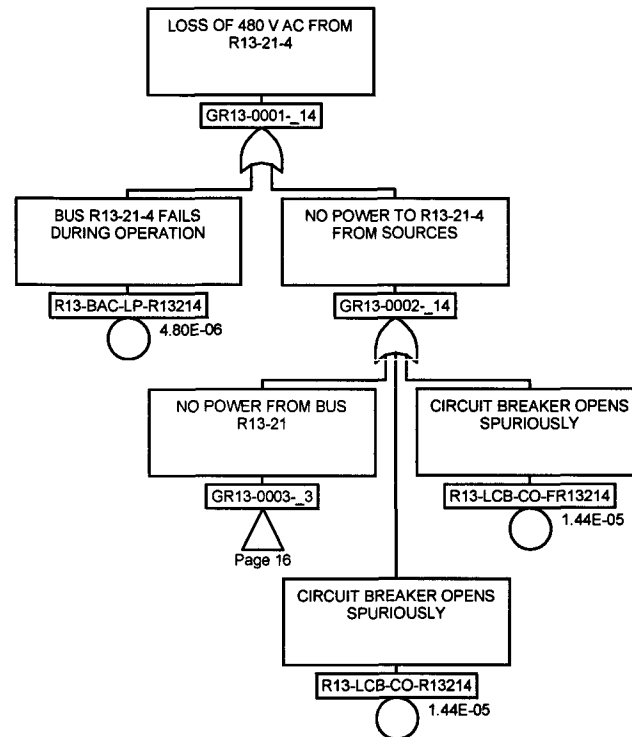
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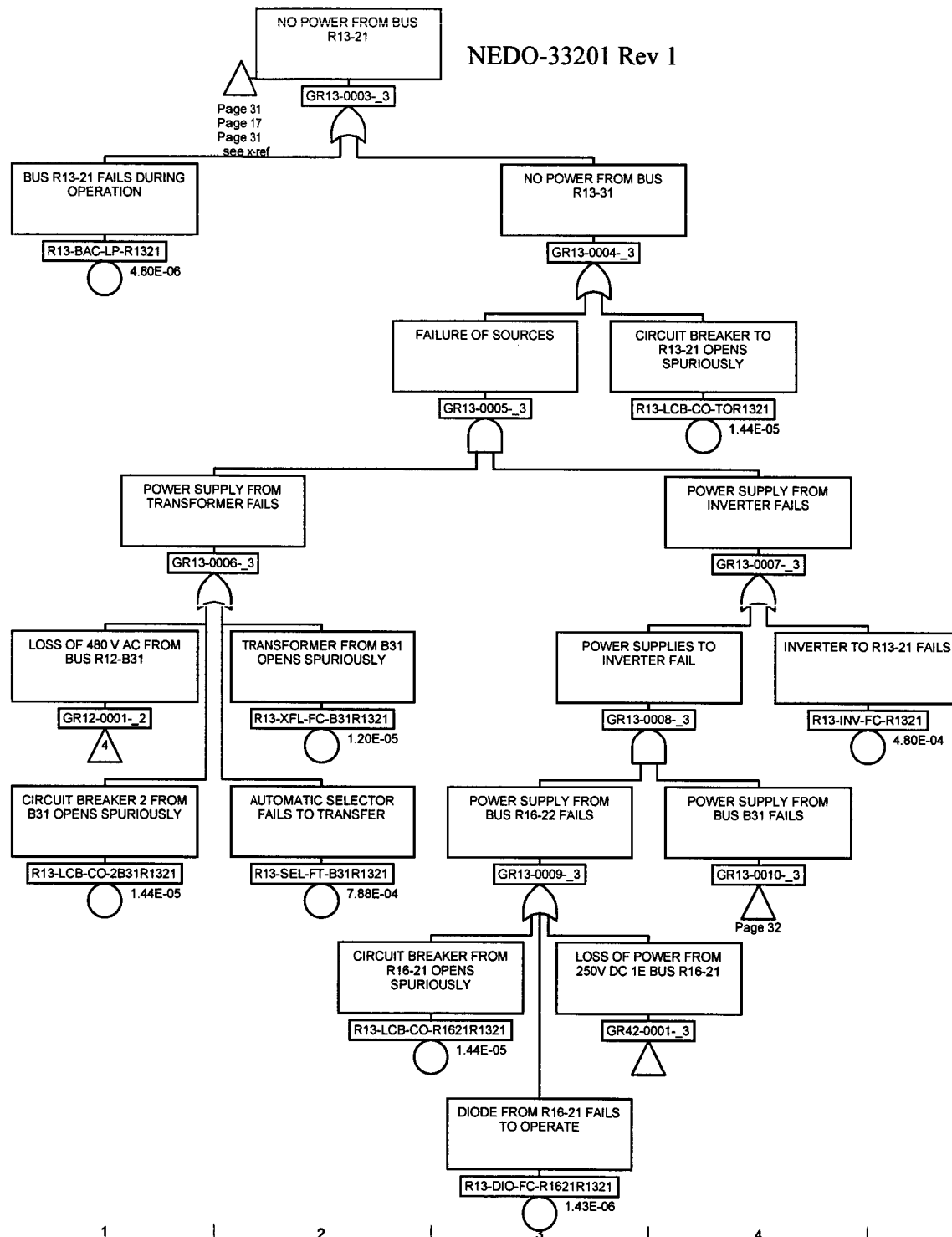
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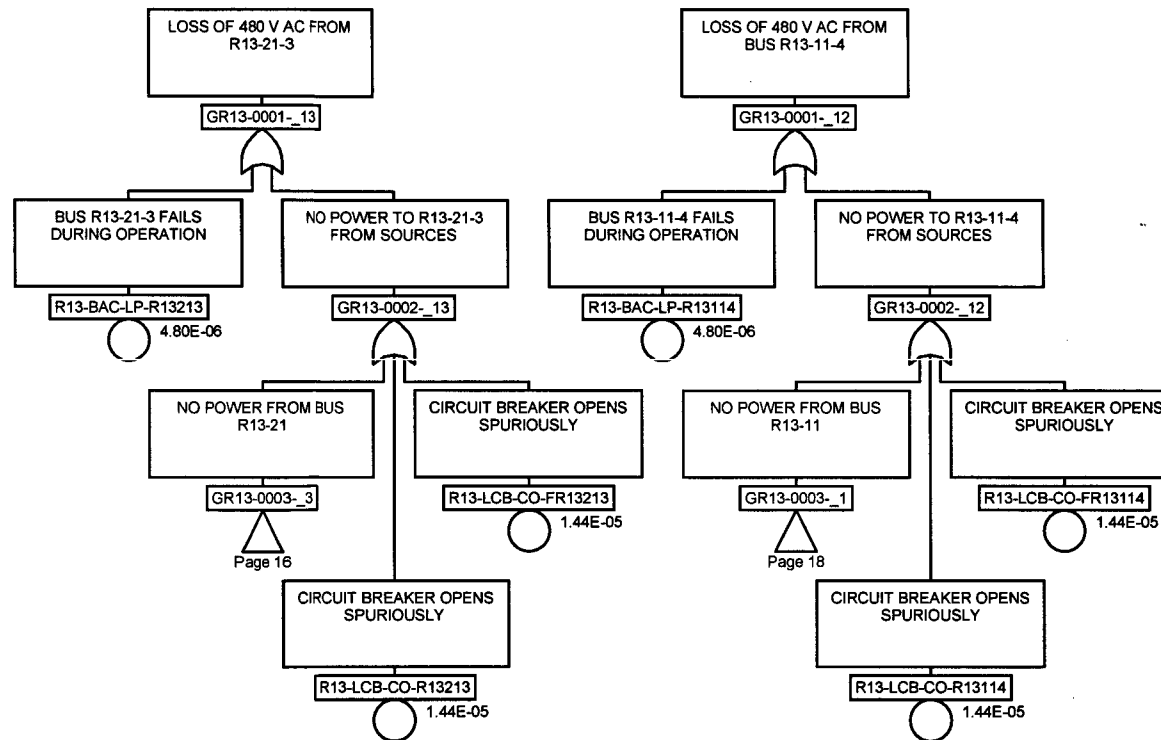
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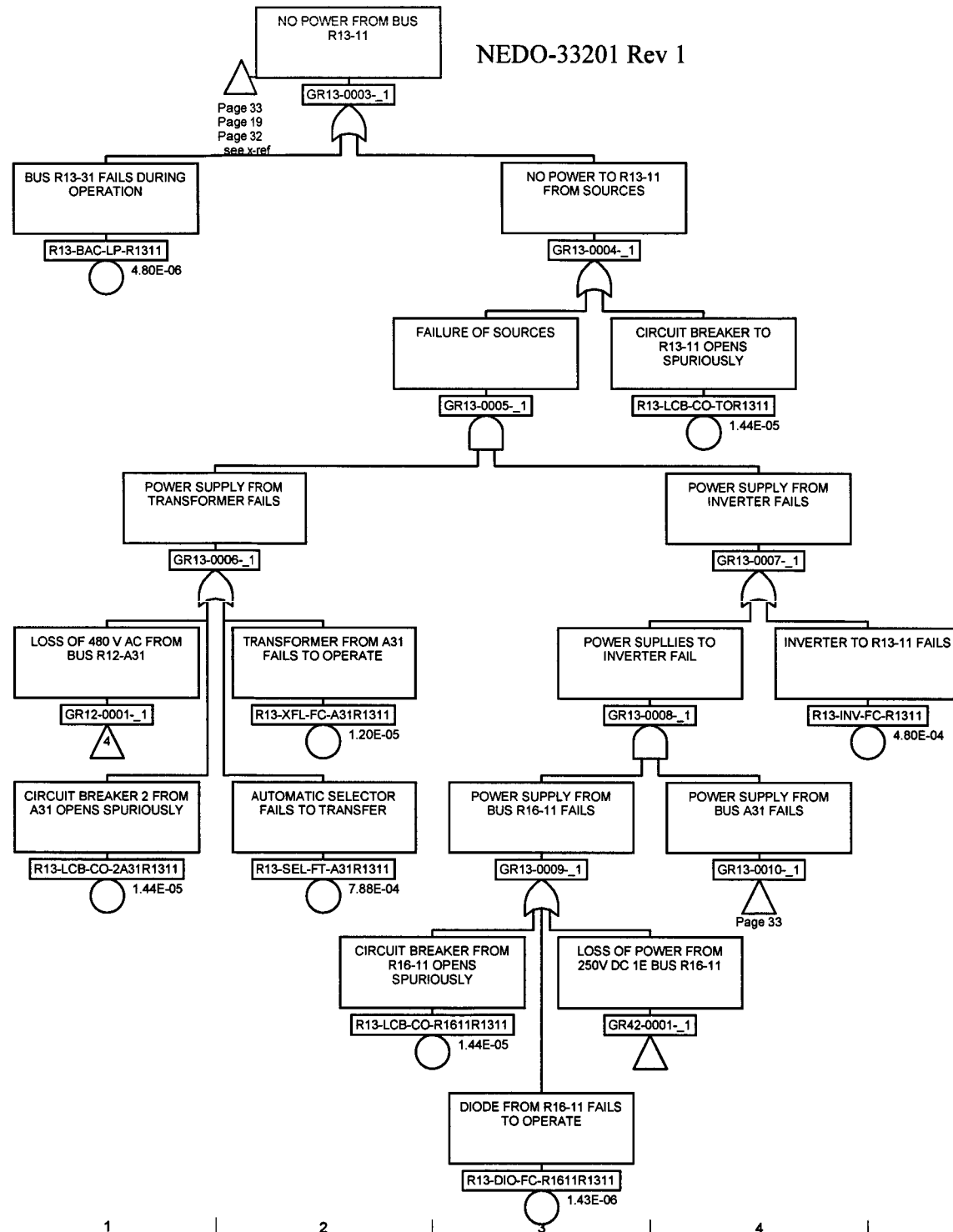




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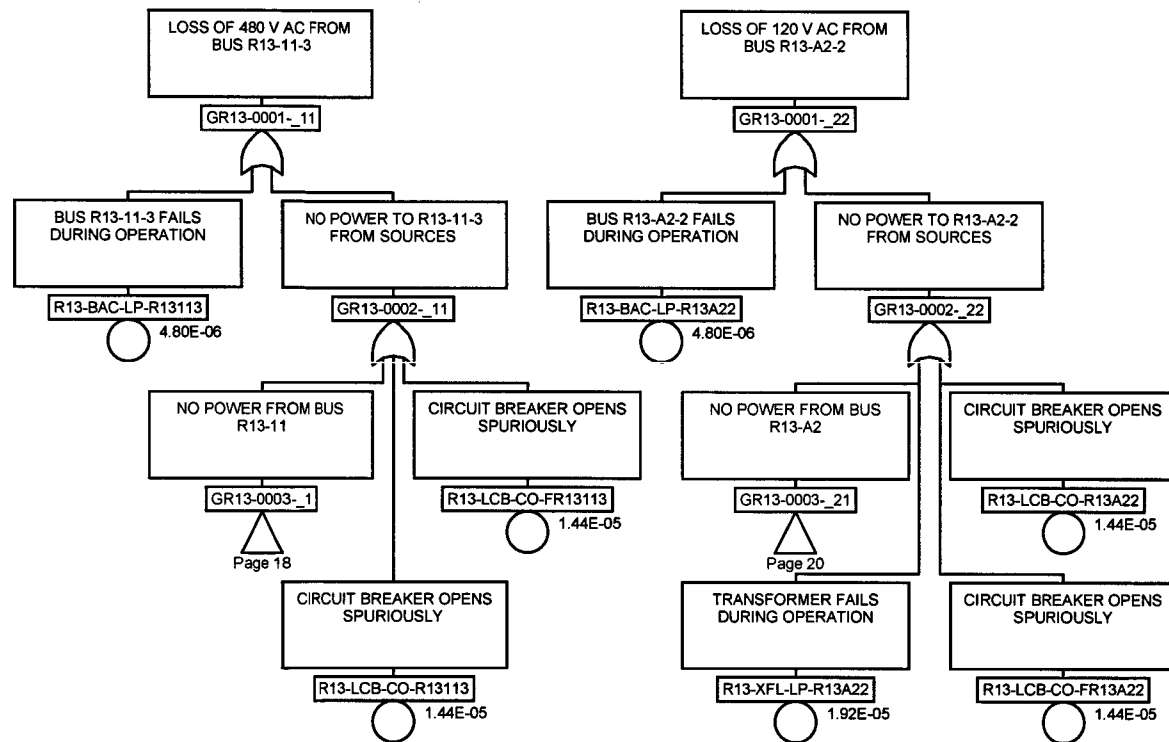


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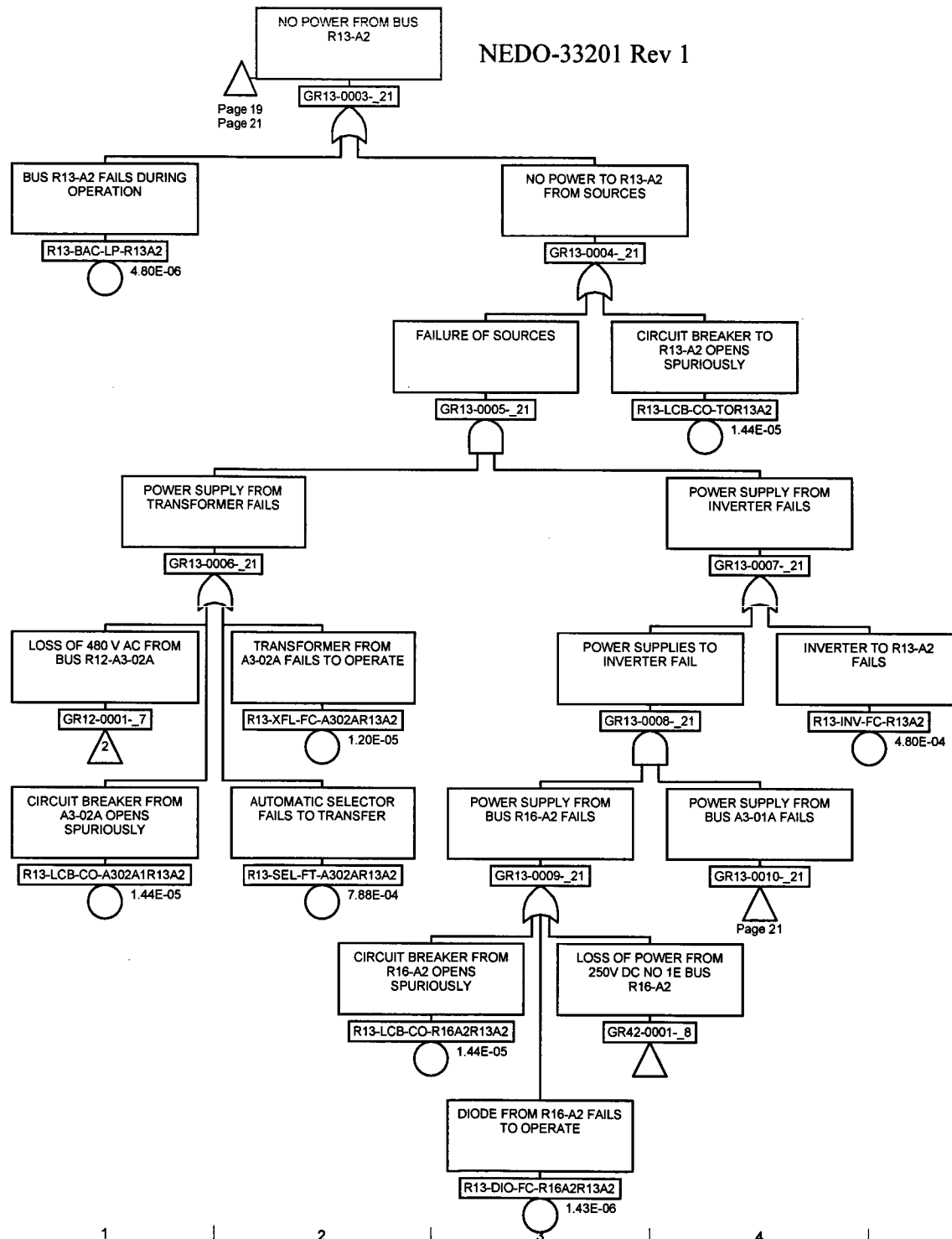
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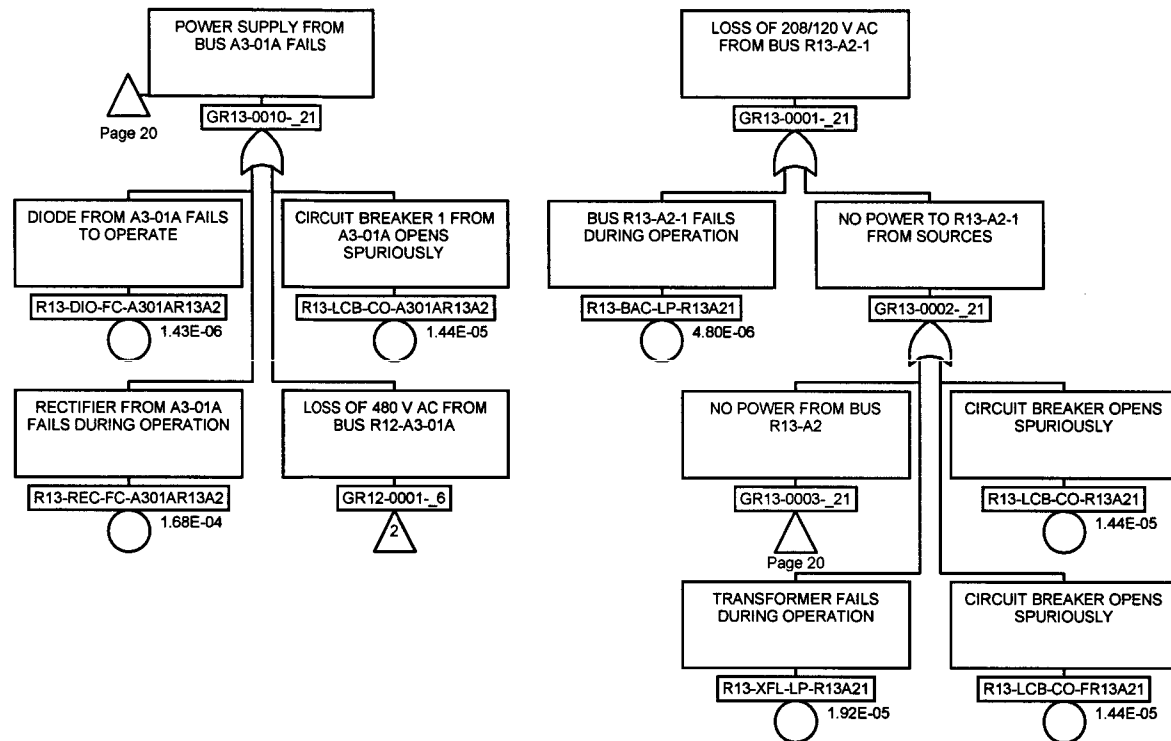


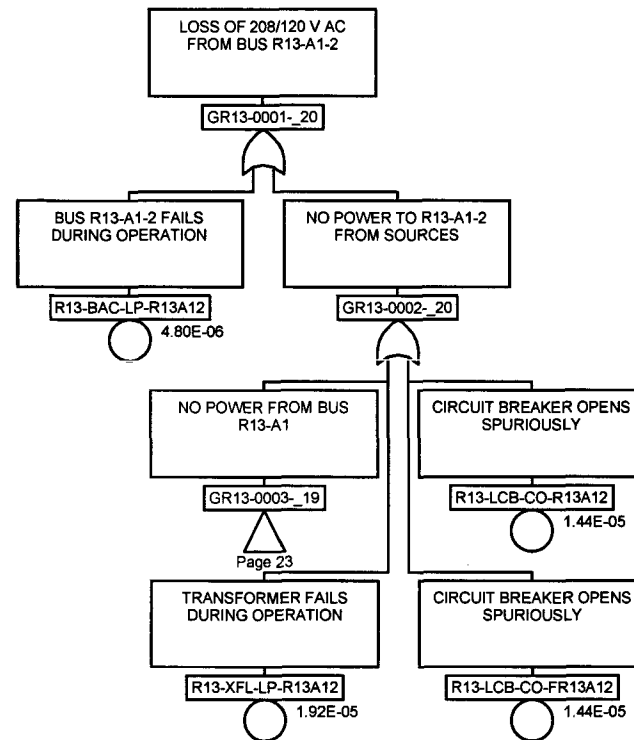
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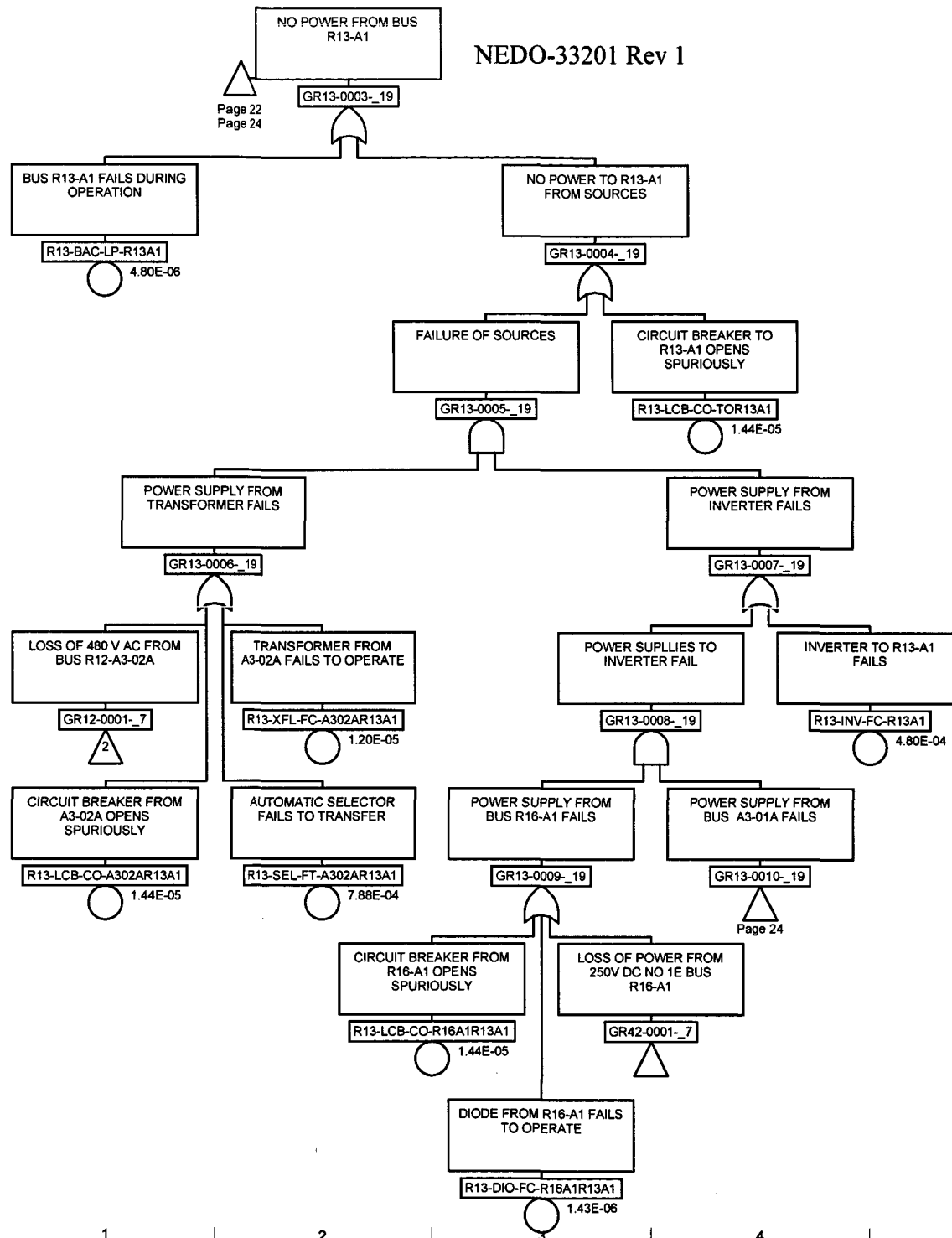
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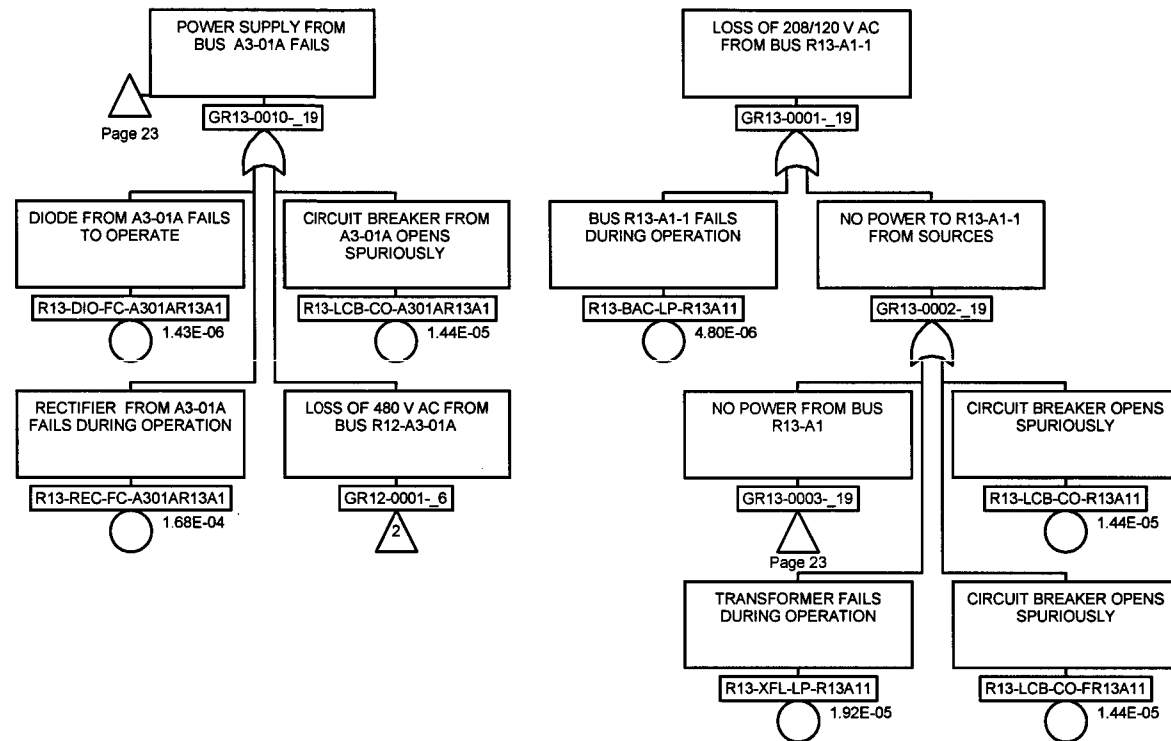


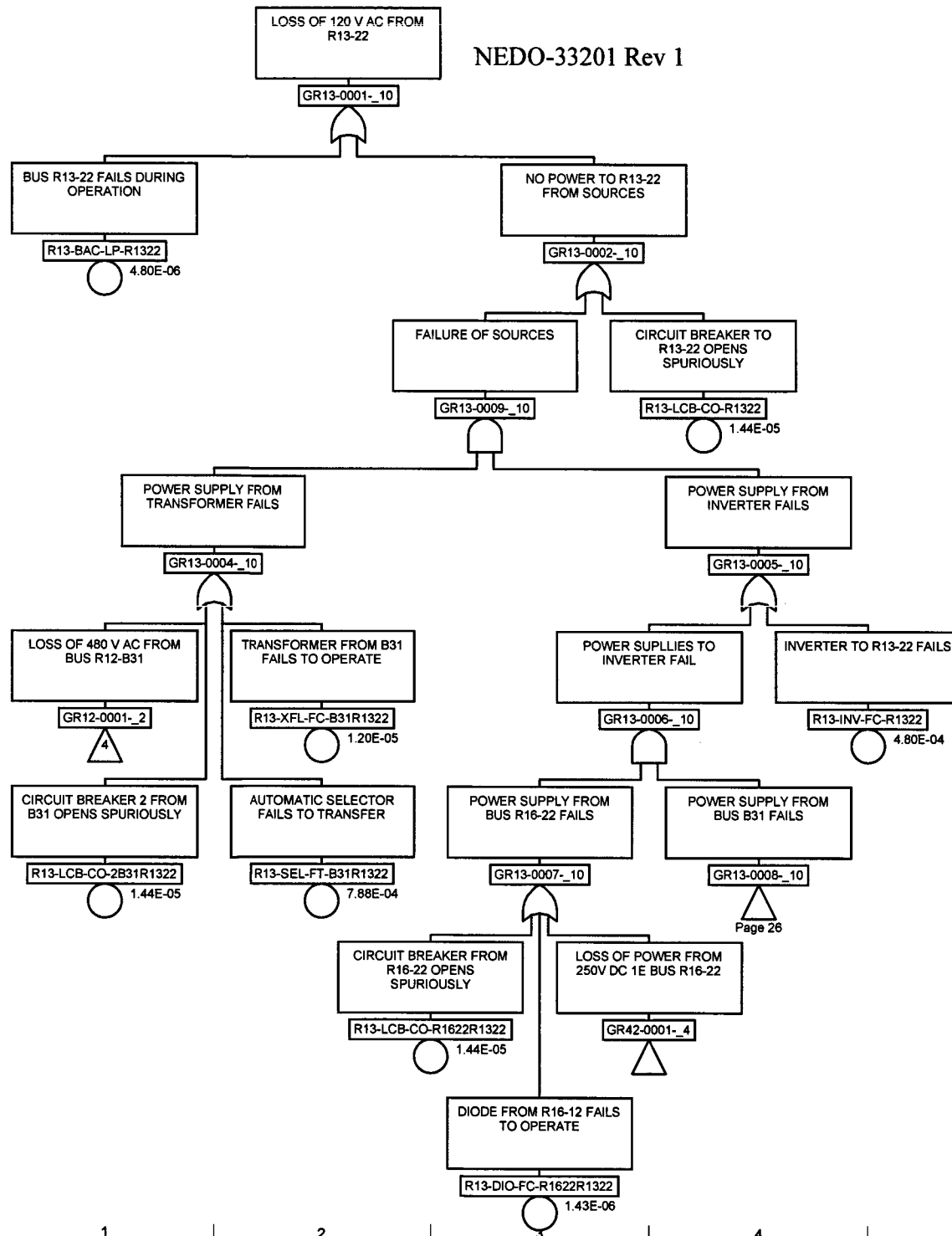
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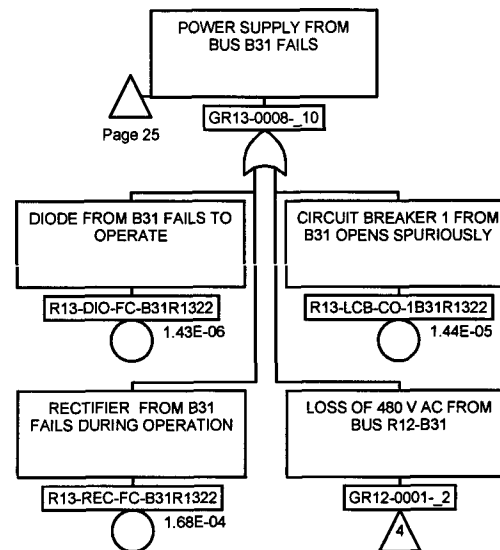


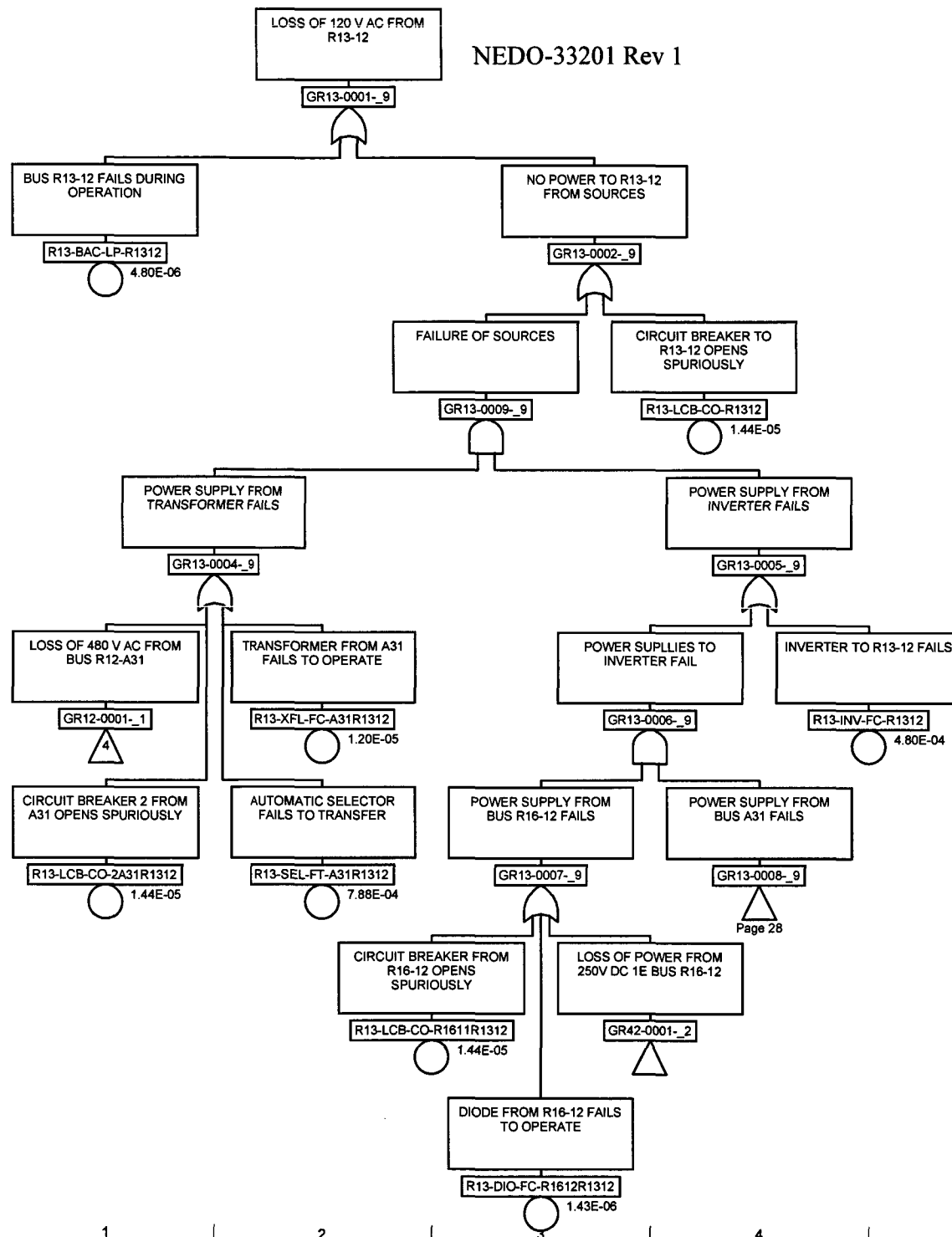
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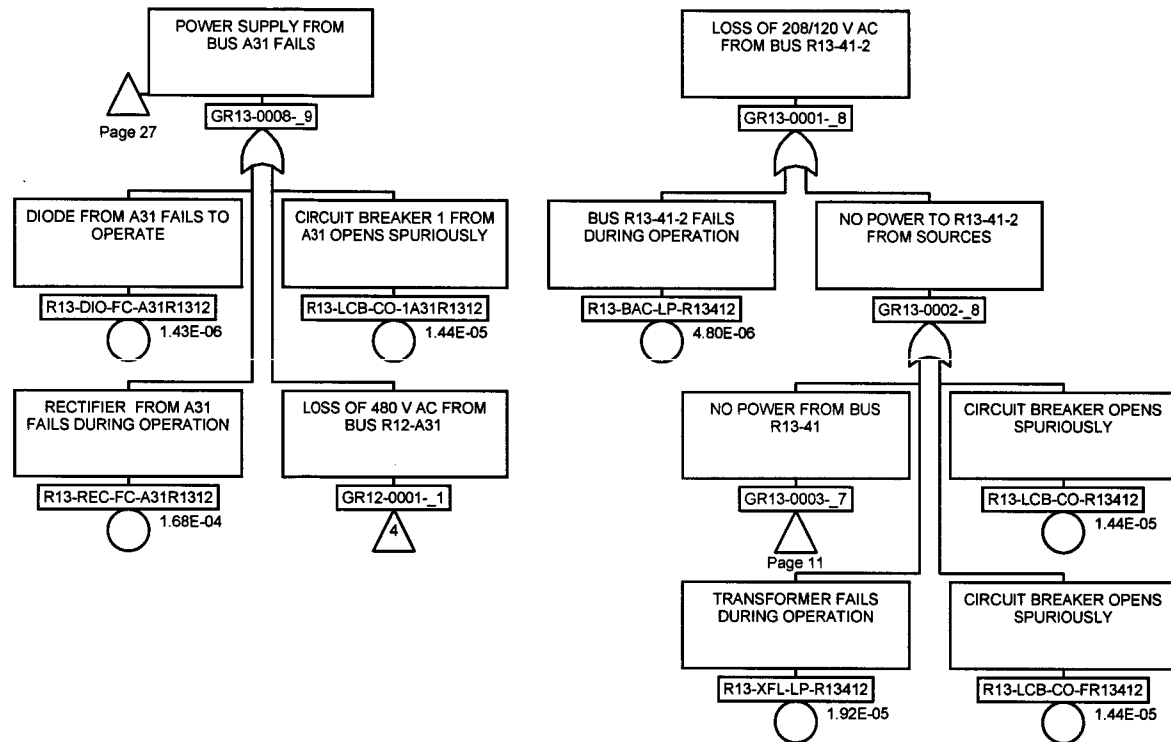
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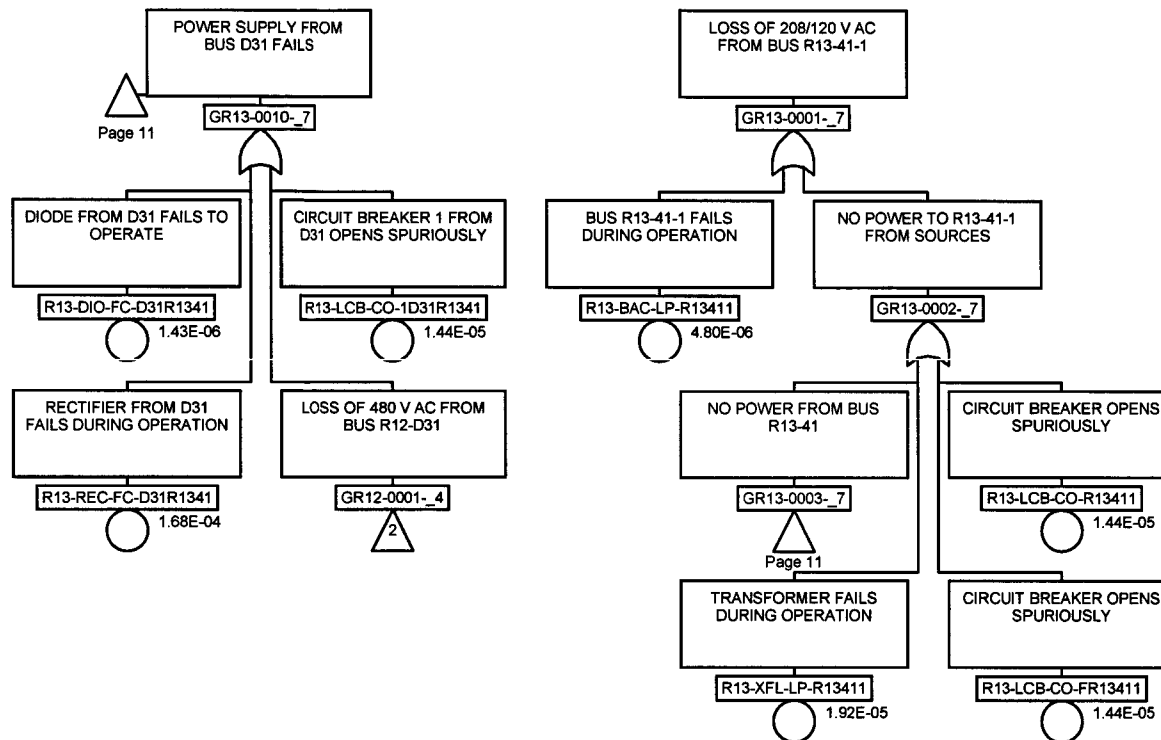
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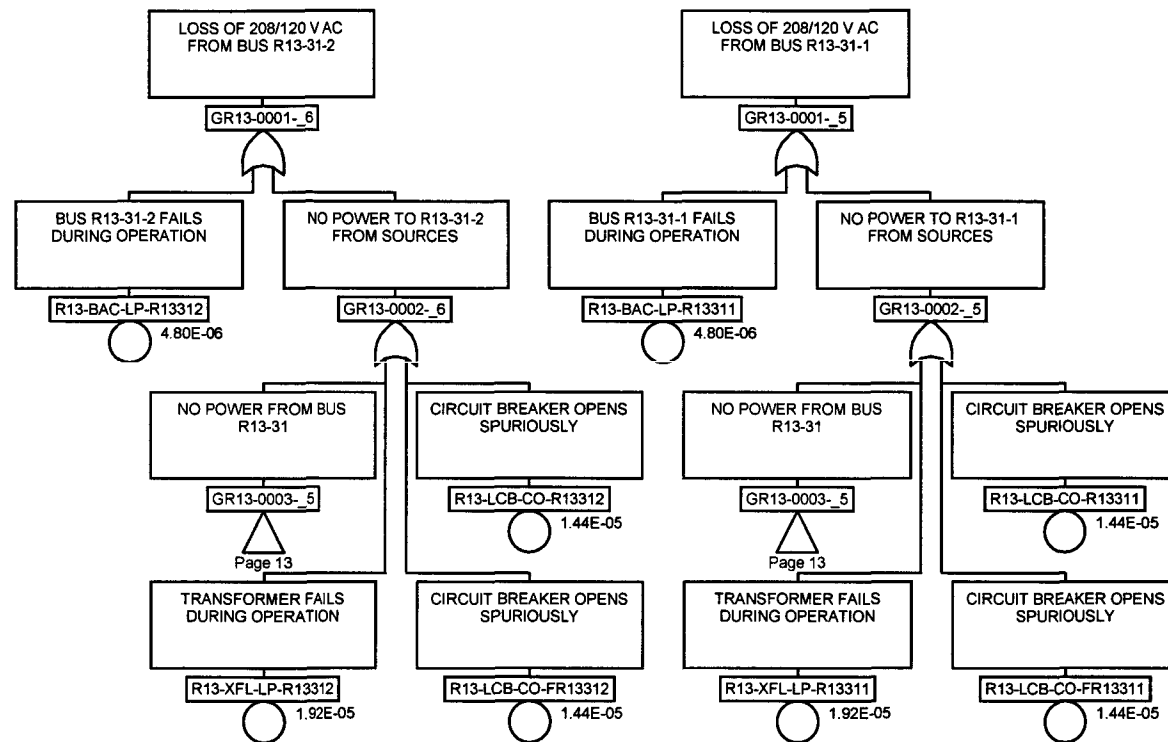




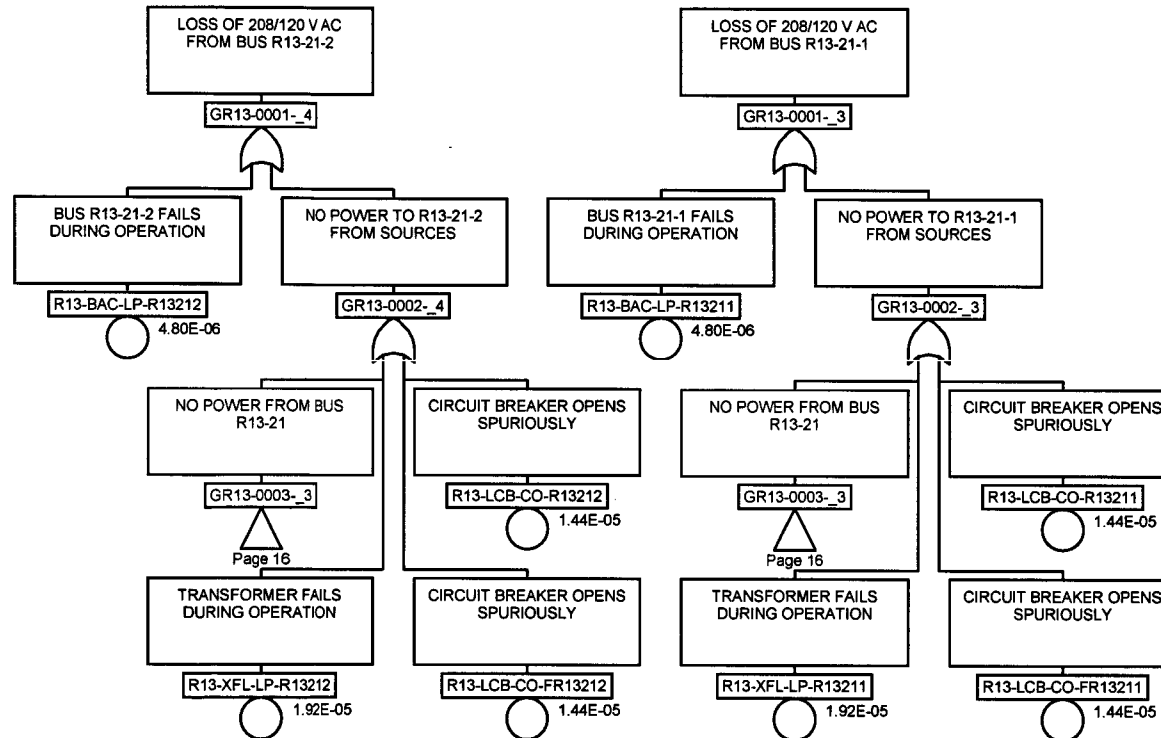
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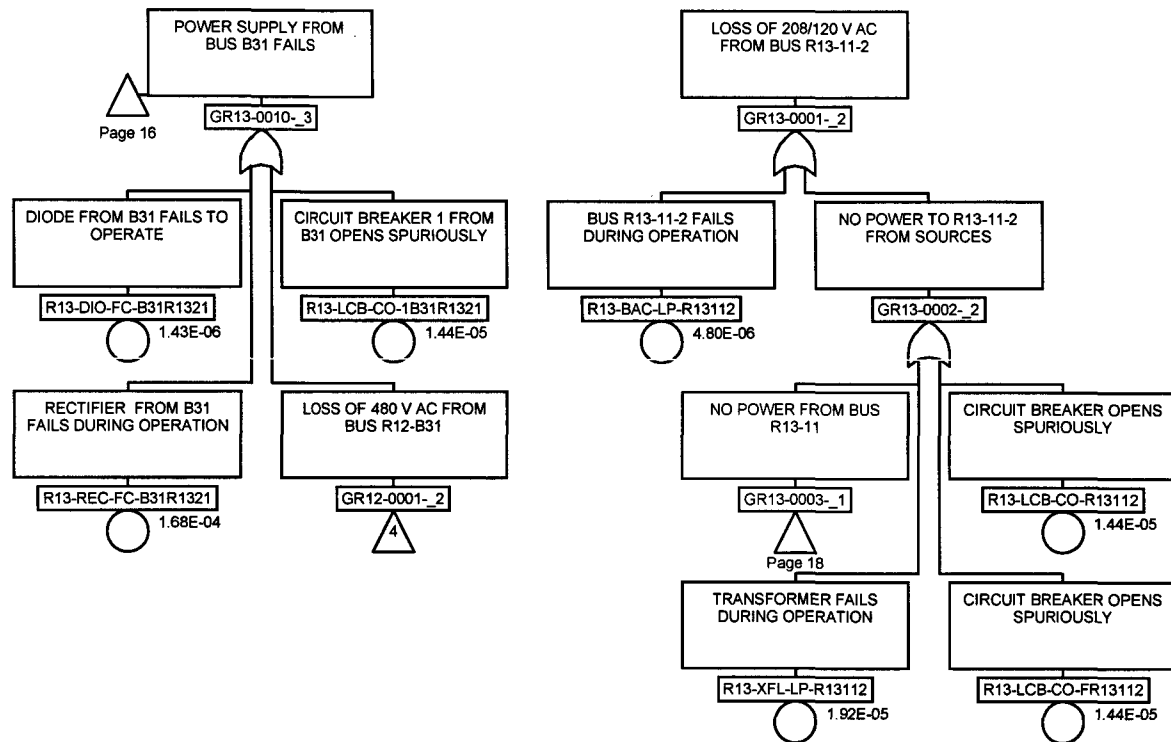


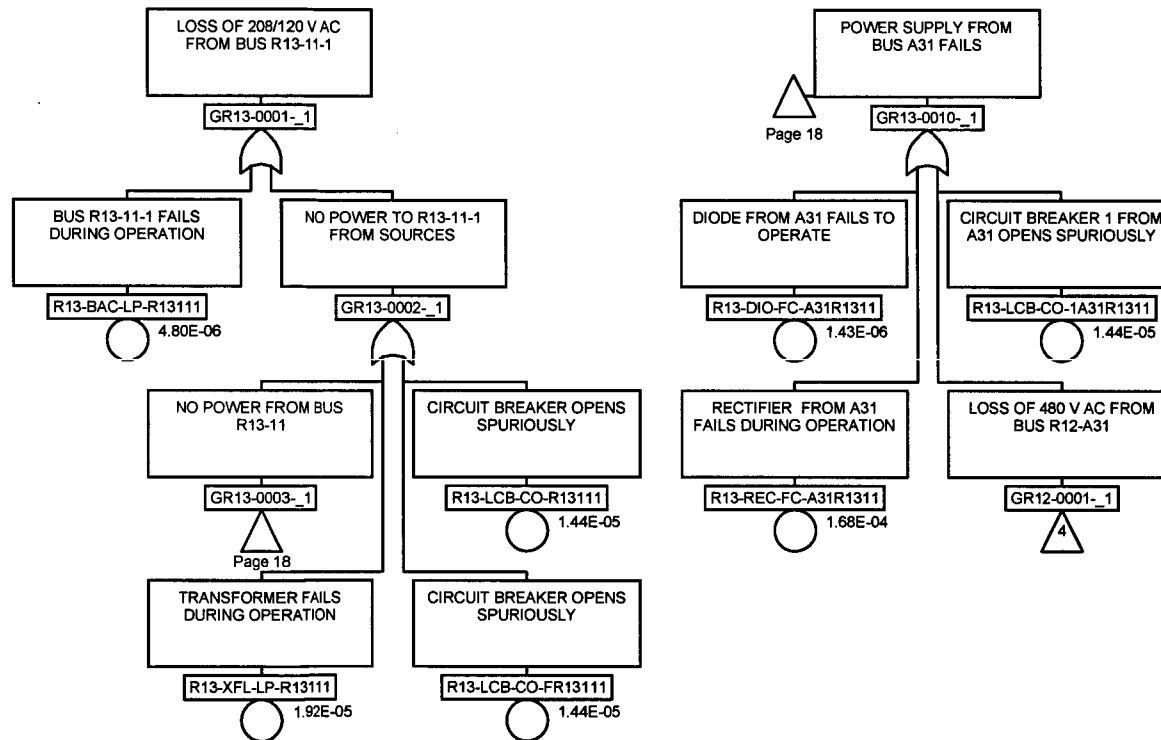




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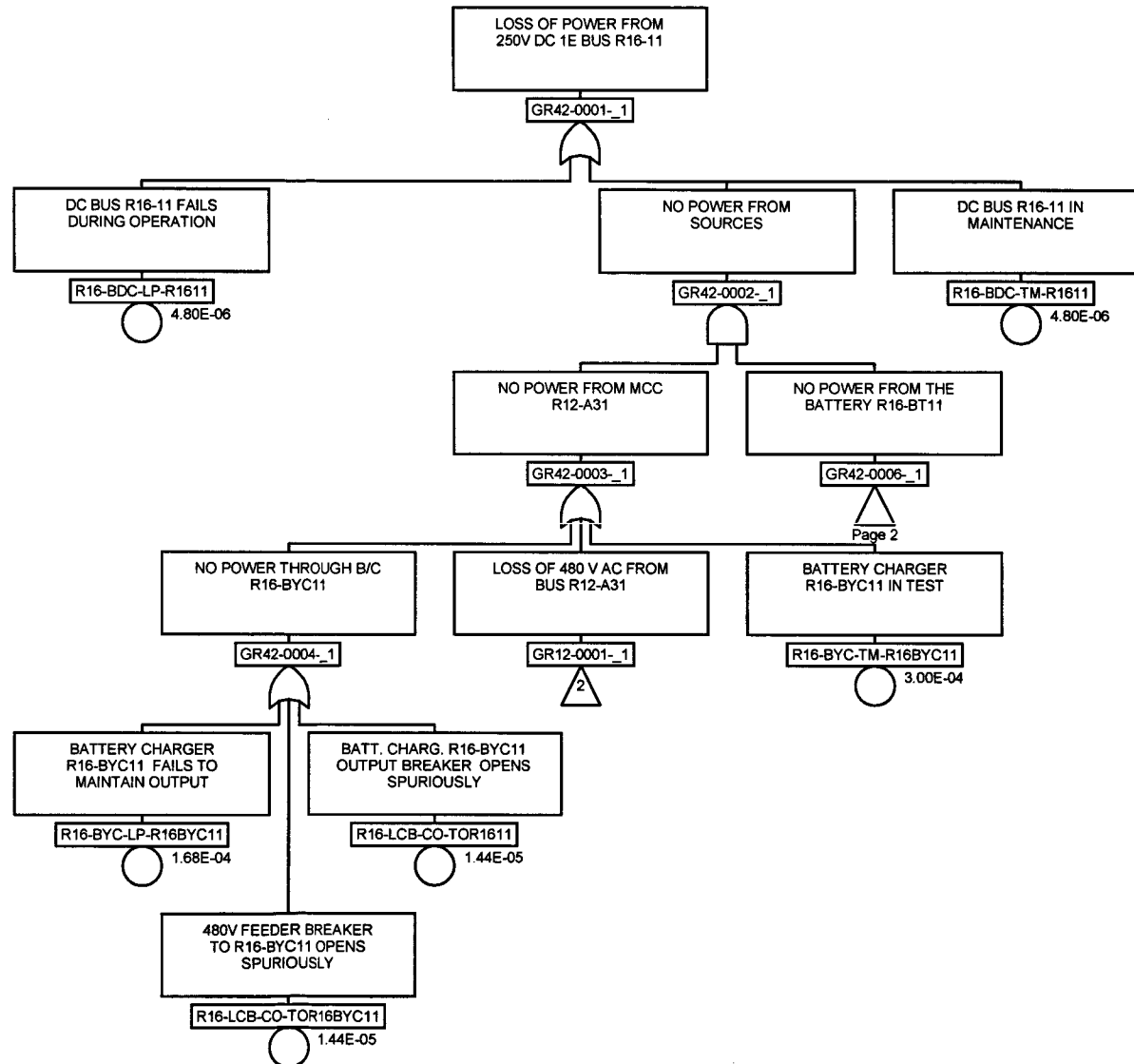
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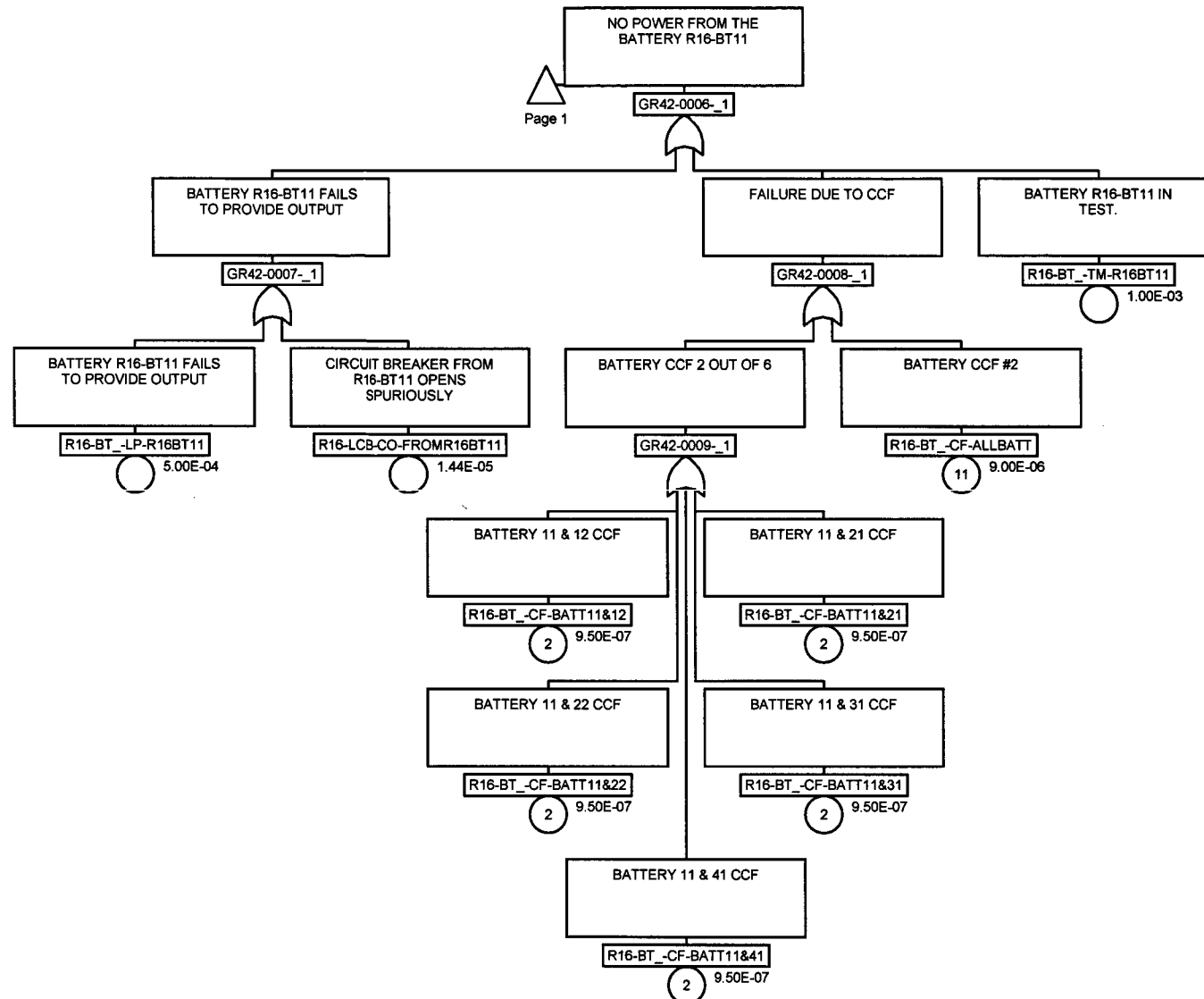
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| R13-LCB-CO-R1631R1331 | 13 | 3 | R13-XFL-LP-R13312 | 30 | 2 | | | |
| R13-LCB-CO-R1641R1341 | 11 | 3 | R13-XFL-LP-R13411 | 29 | 4 | | | |
| R13-LCB-CO-R16A1R13A1 | 23 | 3 | R13-XFL-LP-R13412 | 28 | 4 | | | |
| R13-LCB-CO-R16A2R13A2 | 20 | 3 | R13-XFL-LP-R13A11 | 24 | 4 | | | |
| R13-LCB-CO-R16B1R13B1 | 8 | 3 | R13-XFL-LP-R13A12 | 22 | 2 | | | |
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| R13-LCB-CO-TOR1311 | 18 | 4 | R13-XFL-LP-R13B11 | 9 | 4 | | | |
| R13-LCB-CO-TOR1321 | 16 | 4 | R13-XFL-LP-R13B12 | 7 | 2 | | | |
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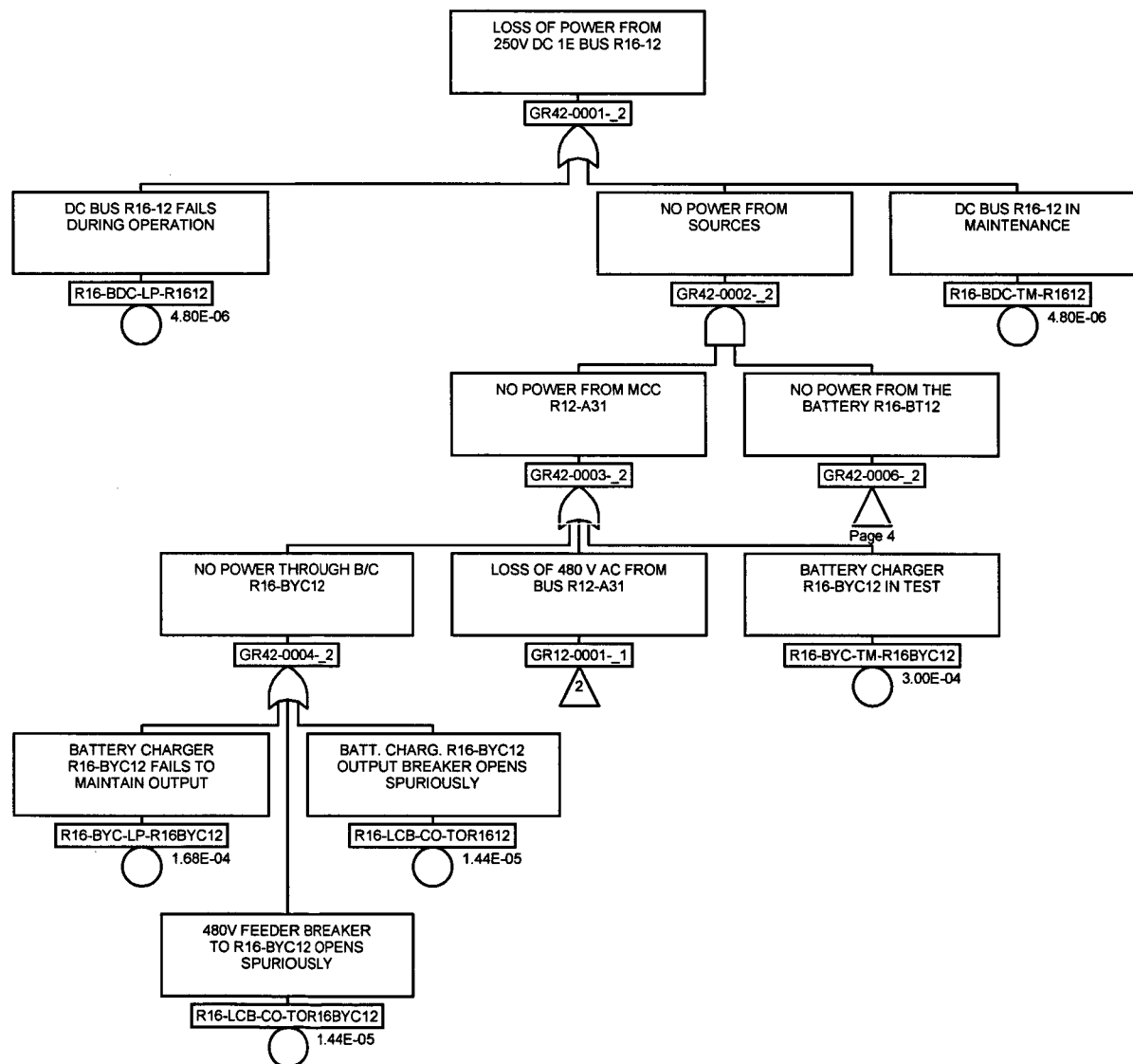
Appendix B.4.17 DC Power Supply System Fault Tree

NEDO-33201 Rev 1

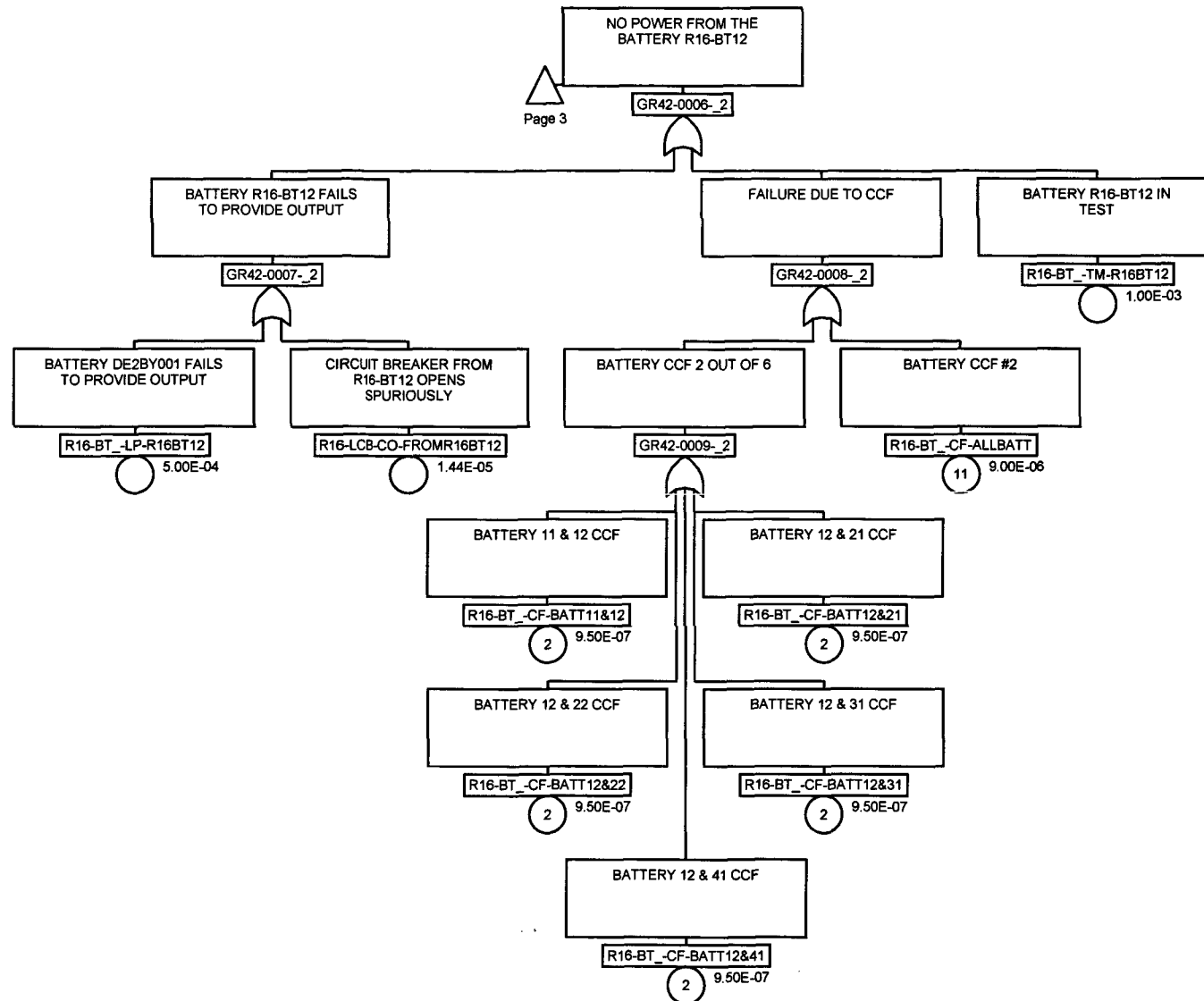




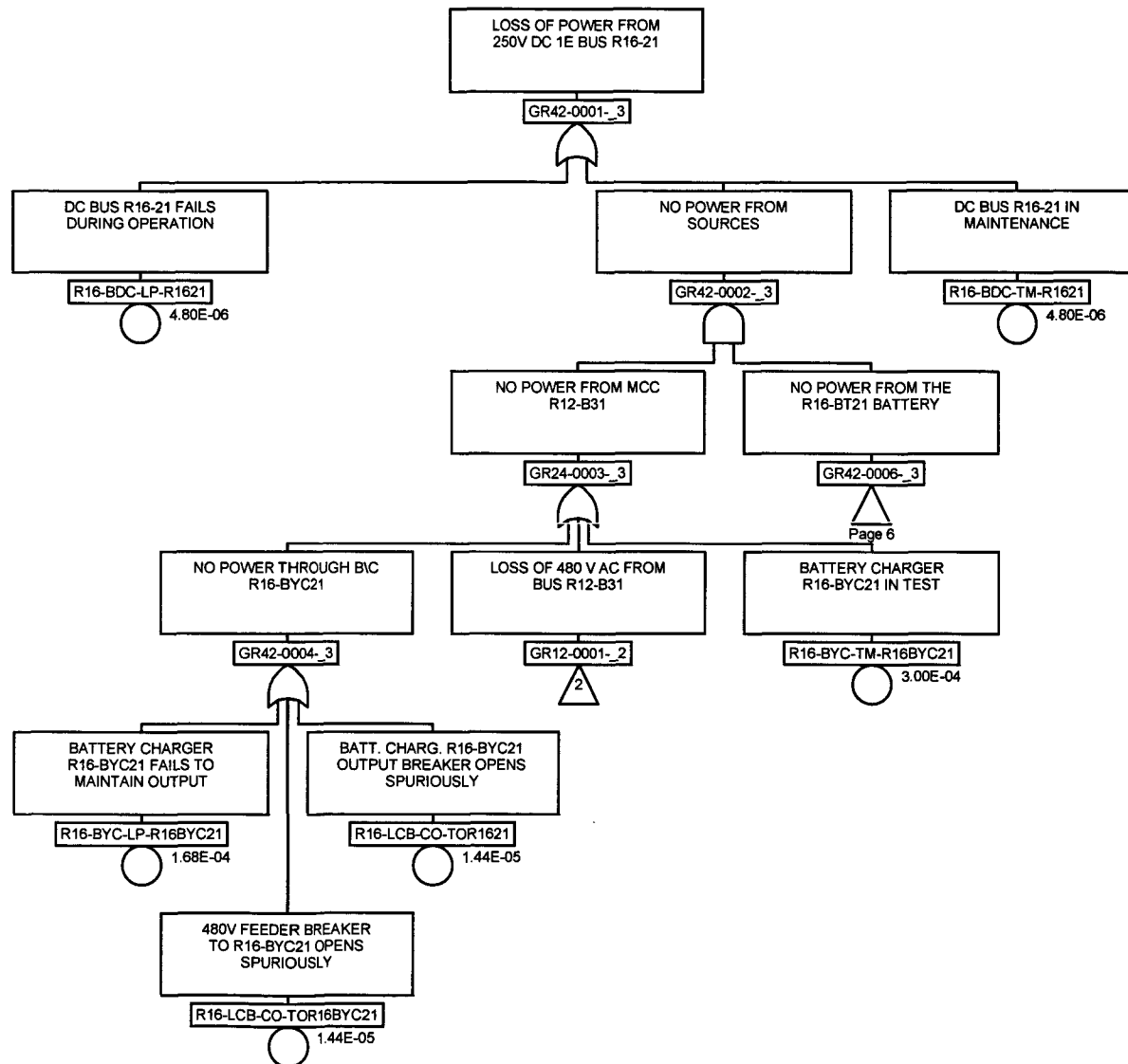
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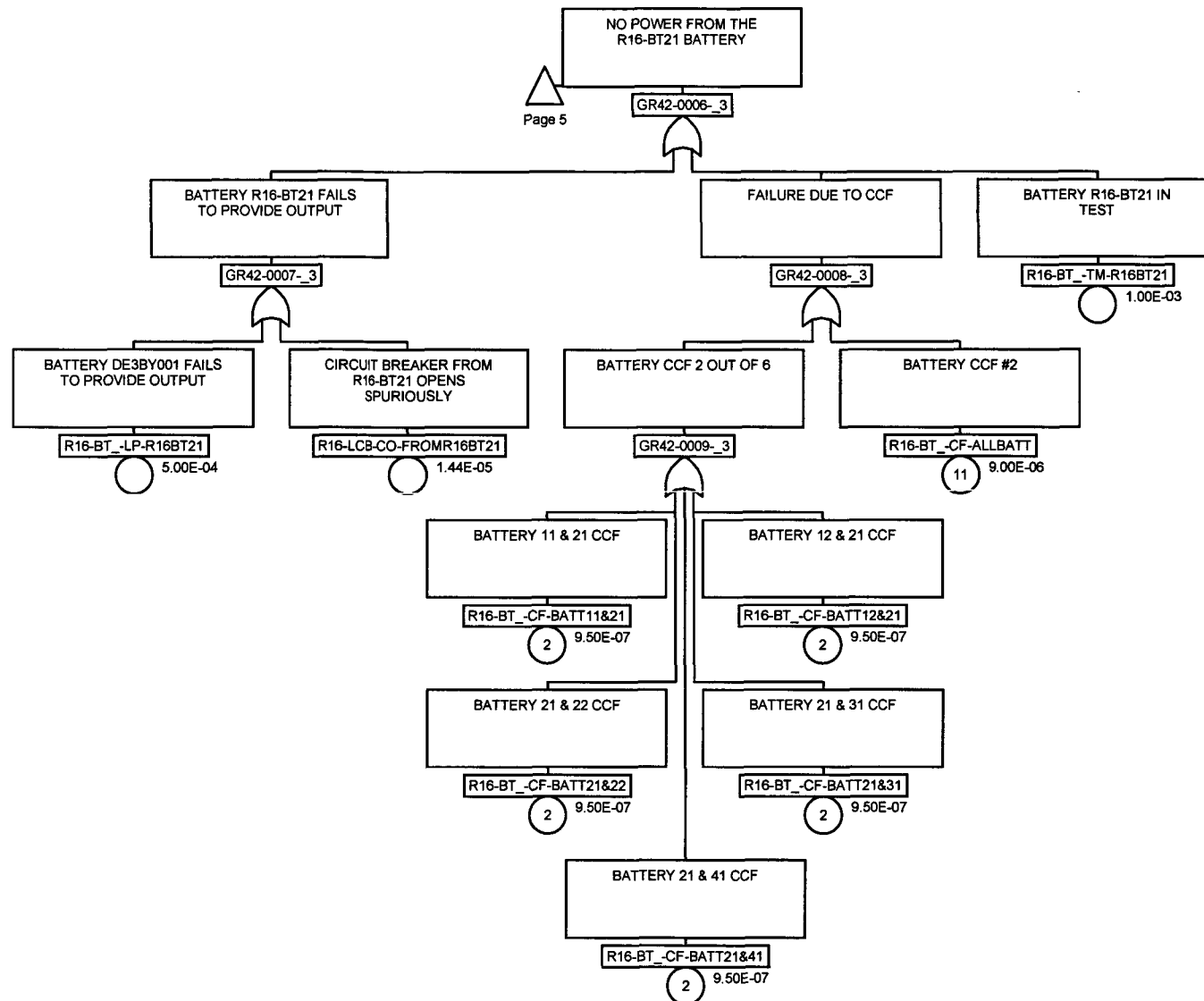


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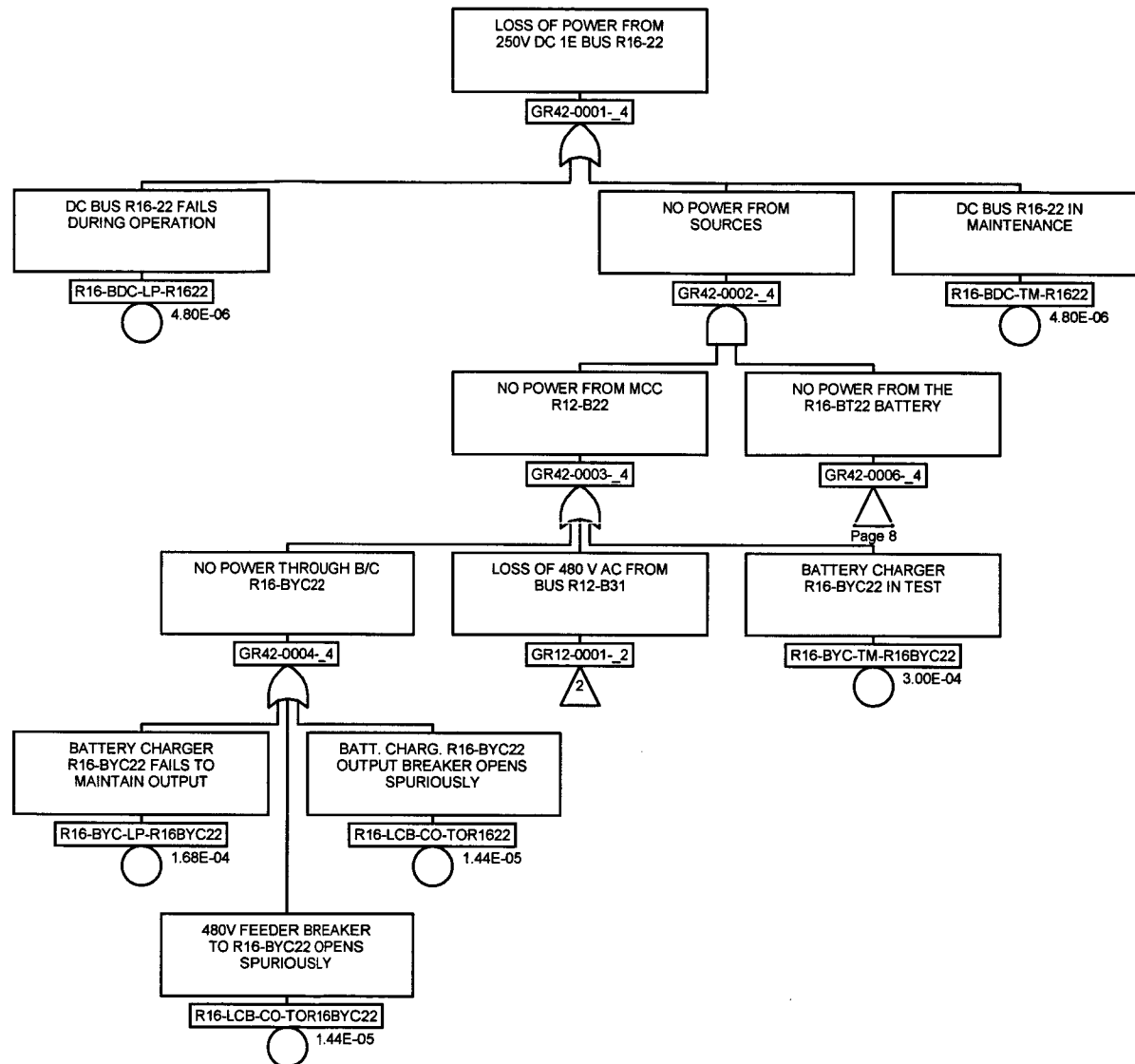


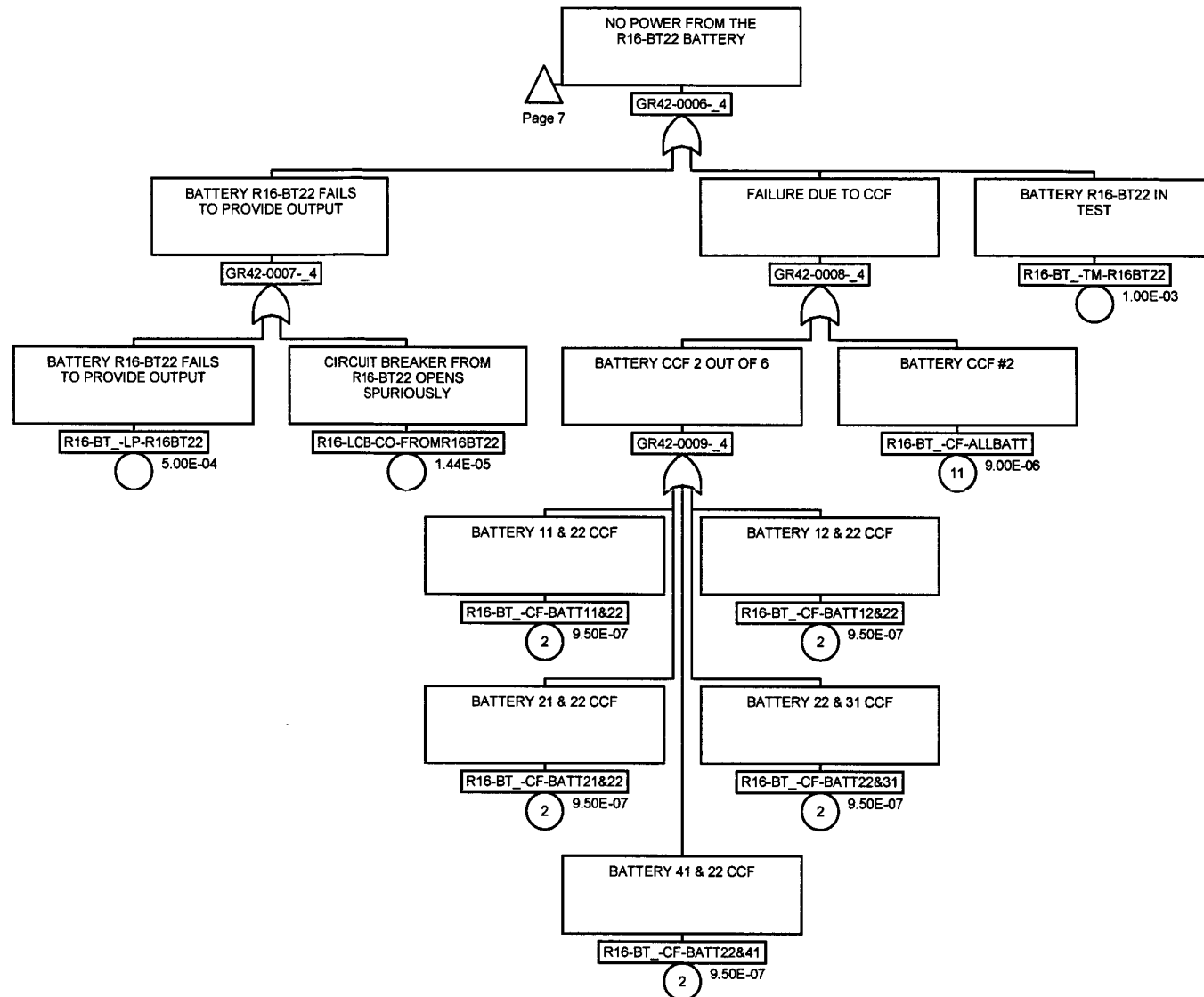
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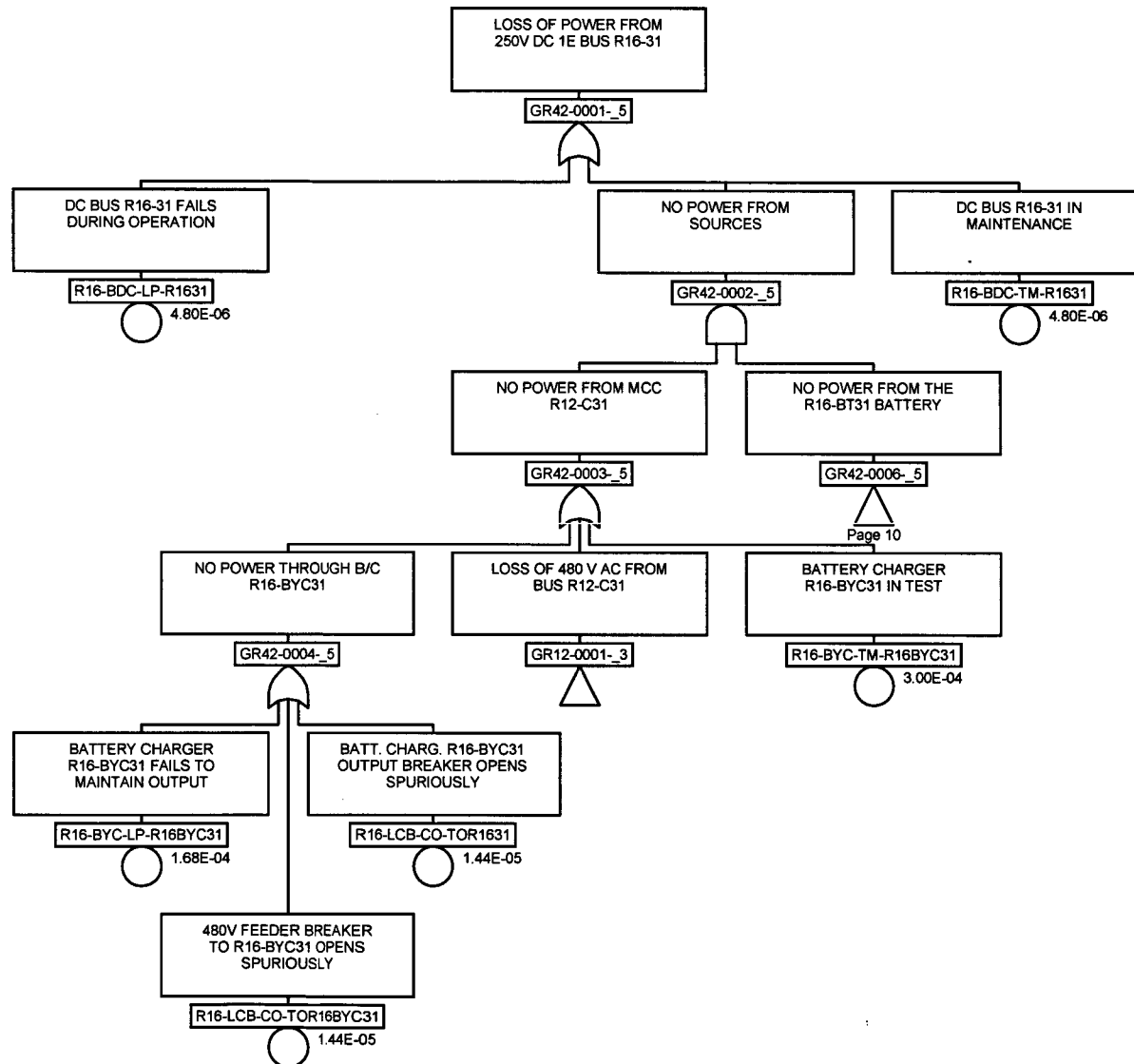


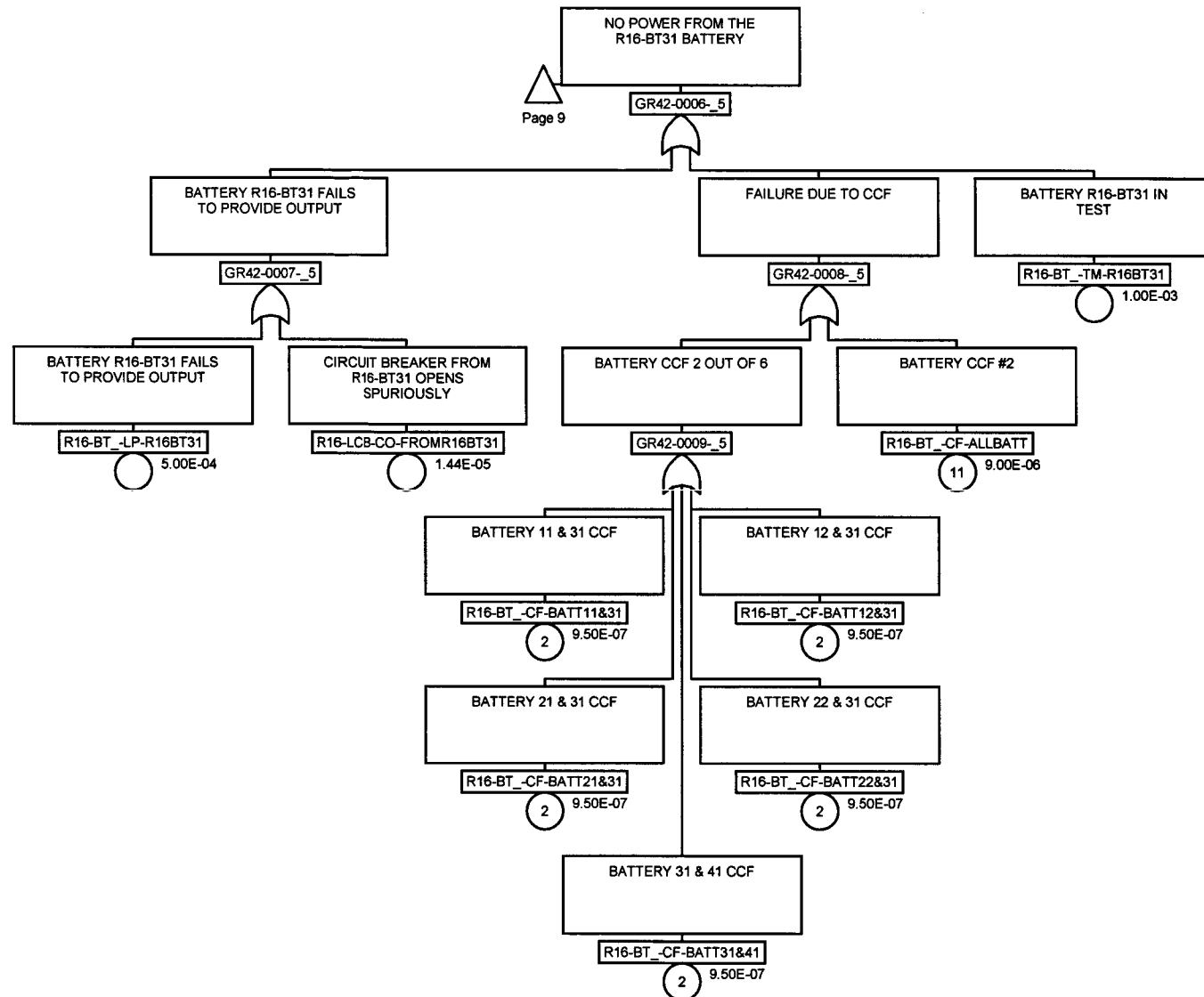
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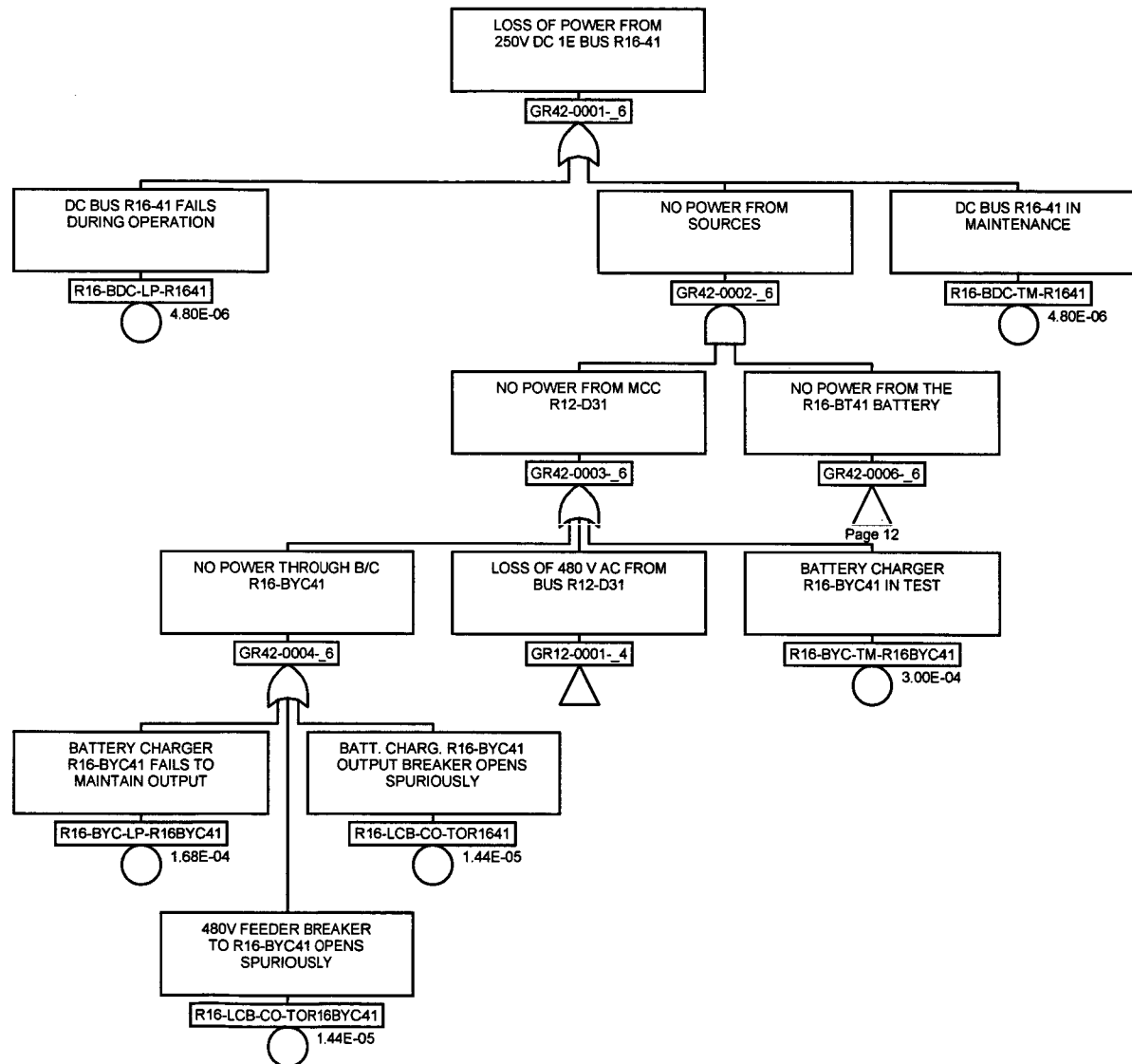


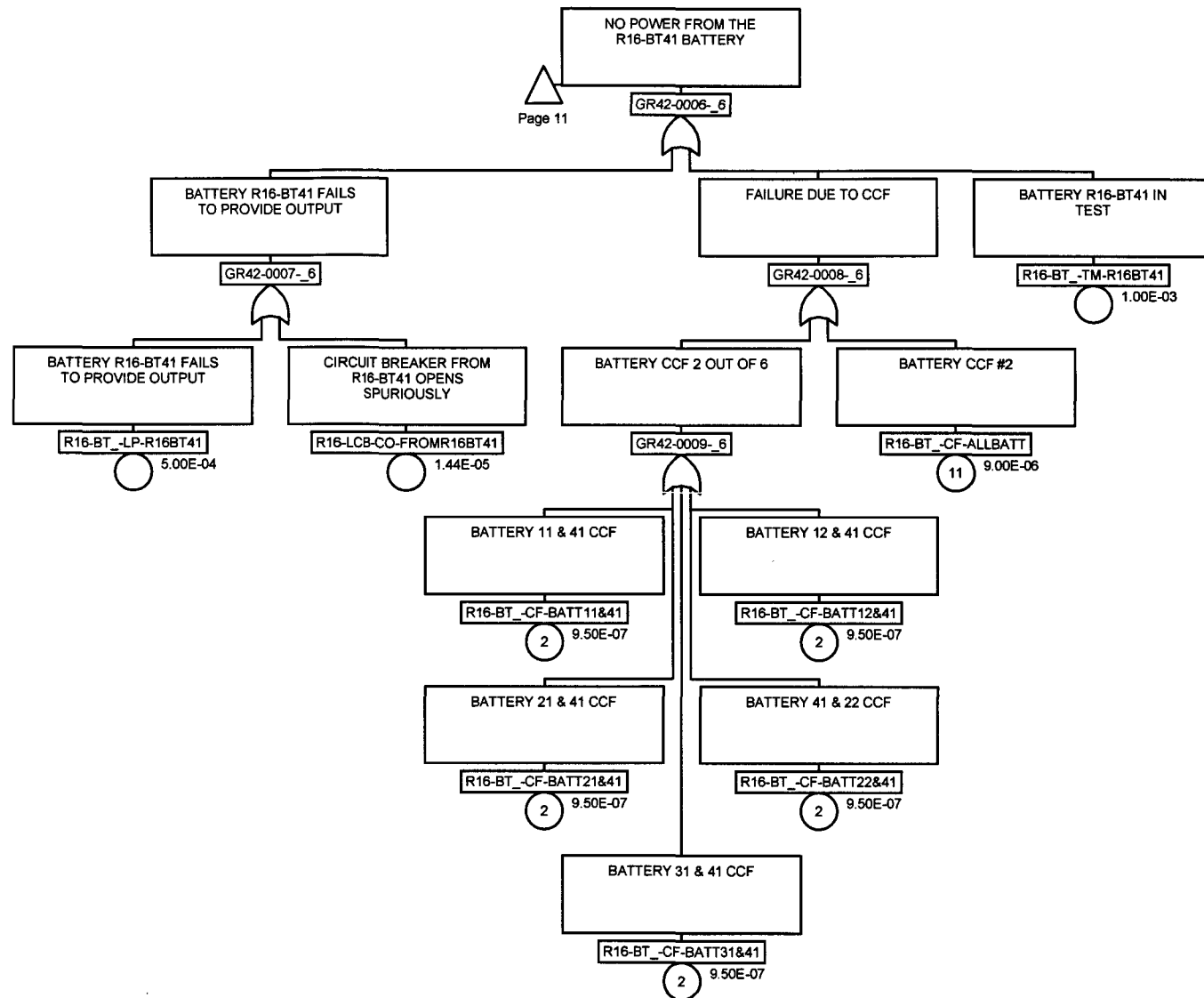
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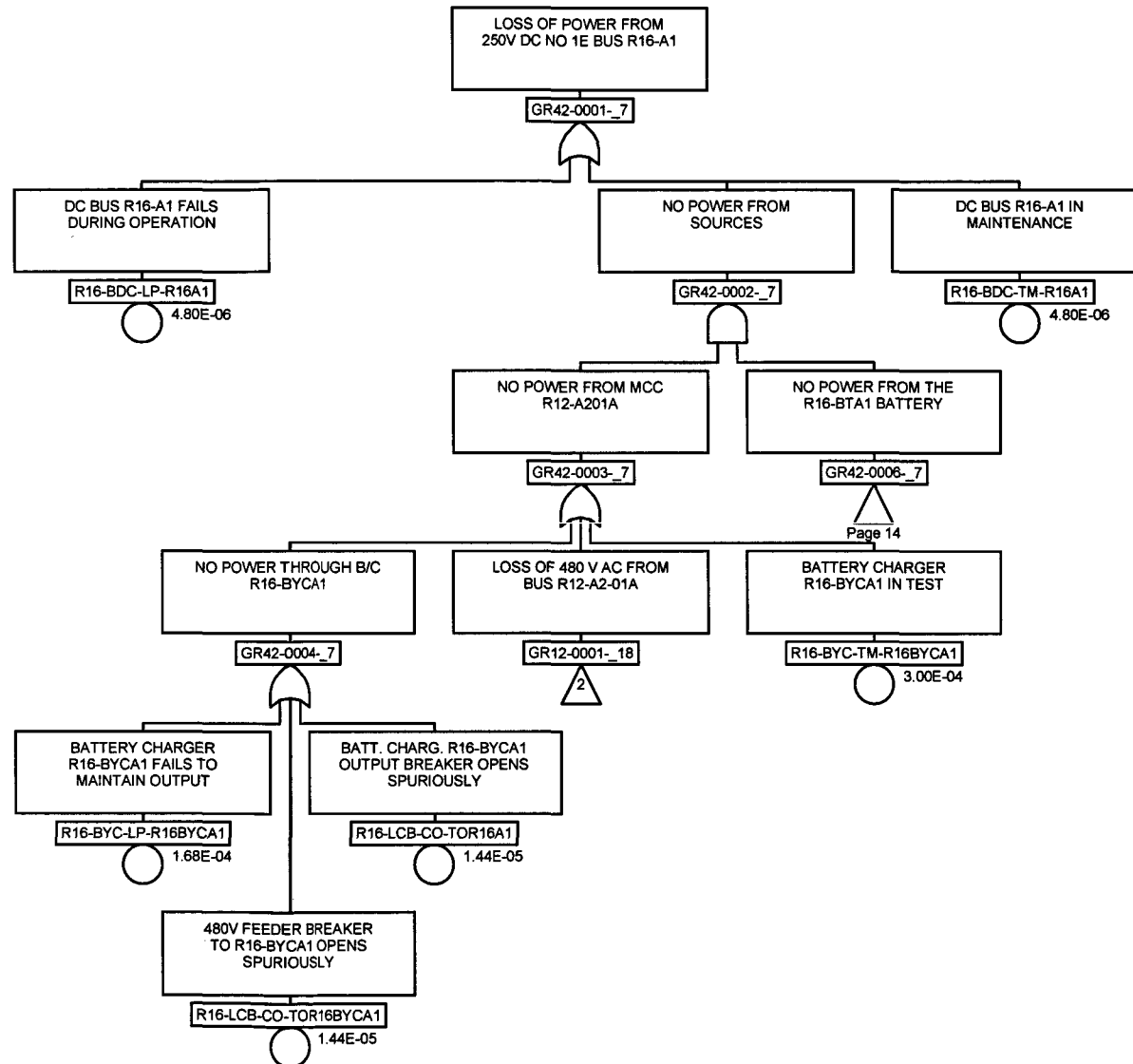


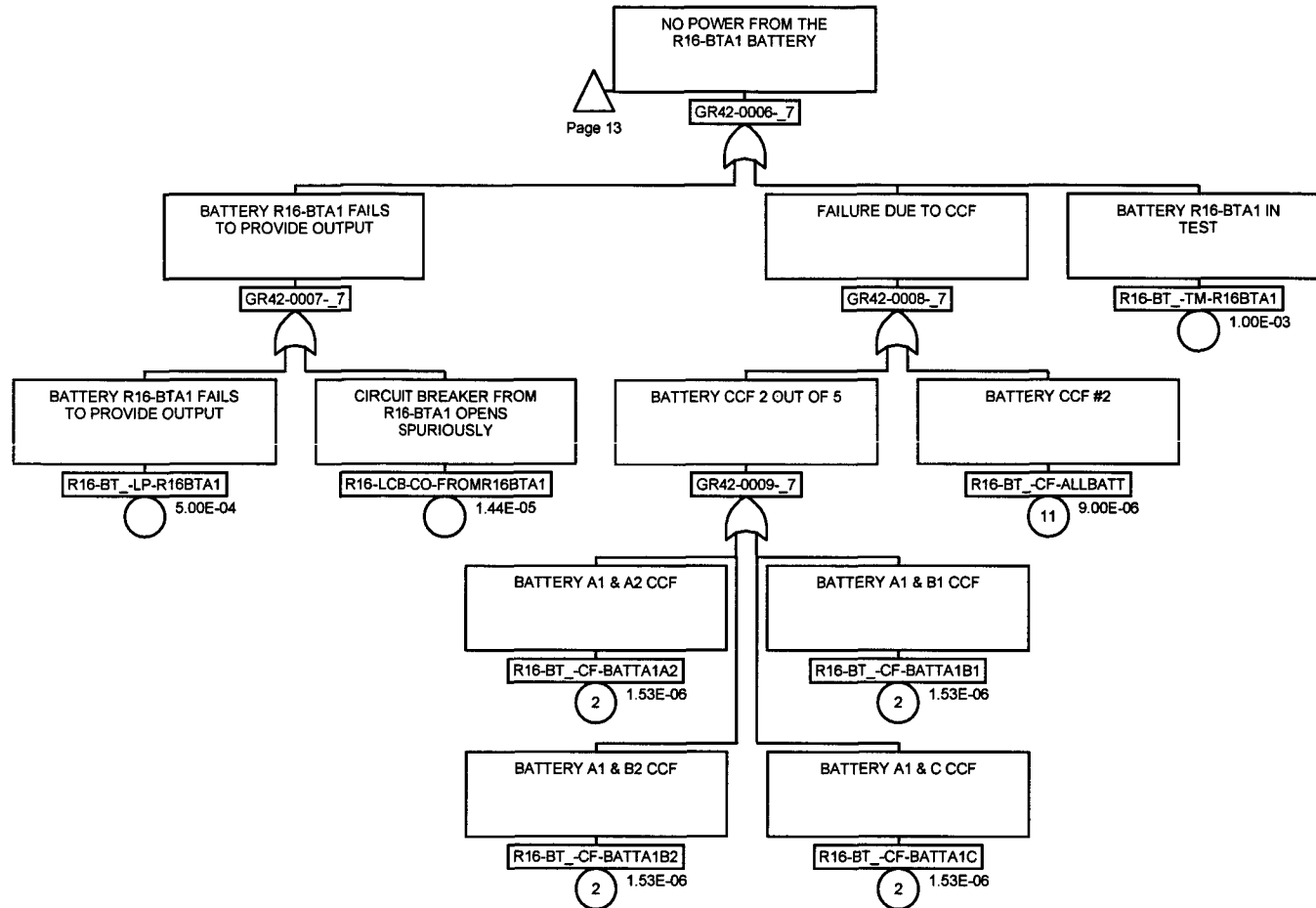
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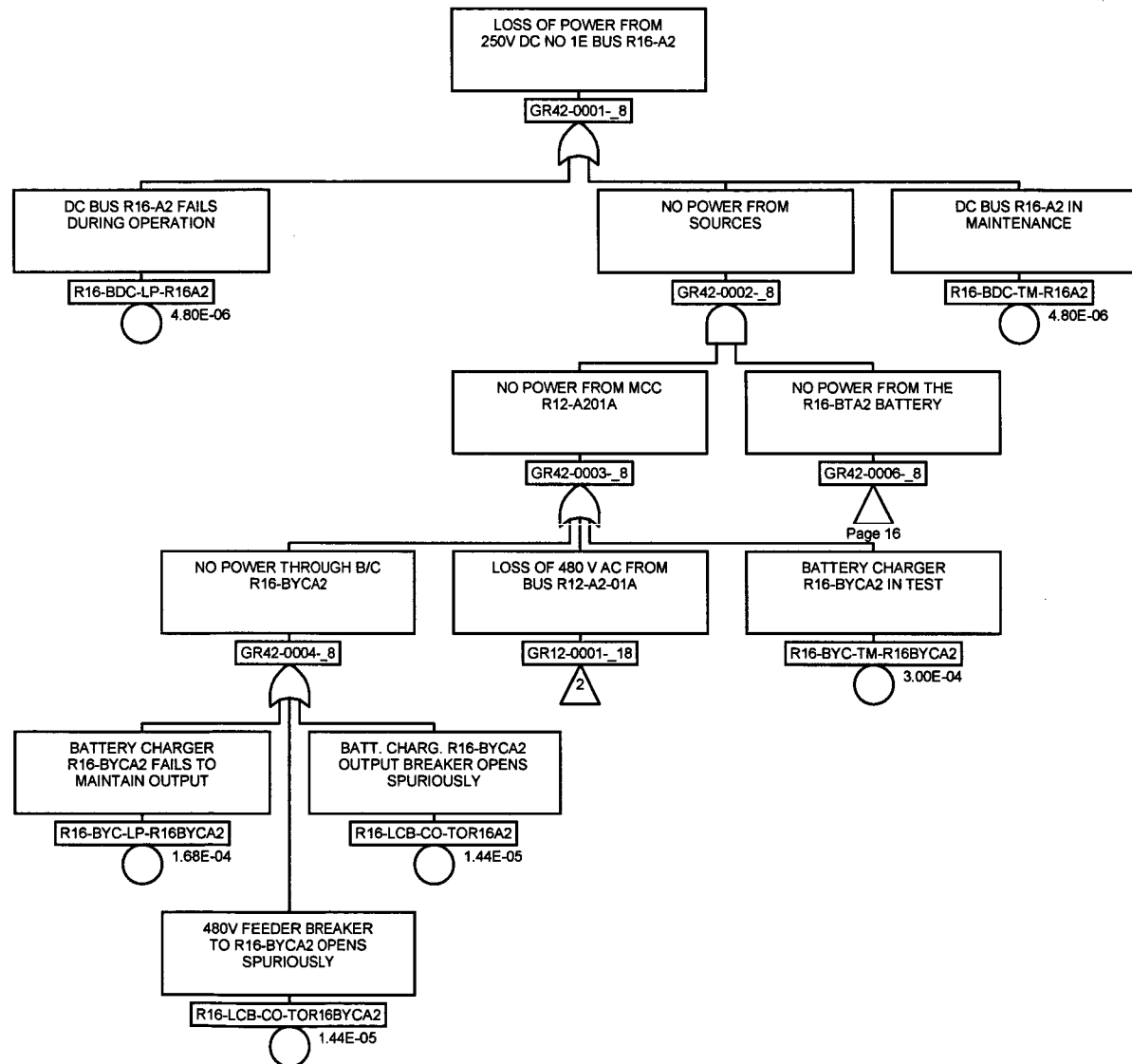


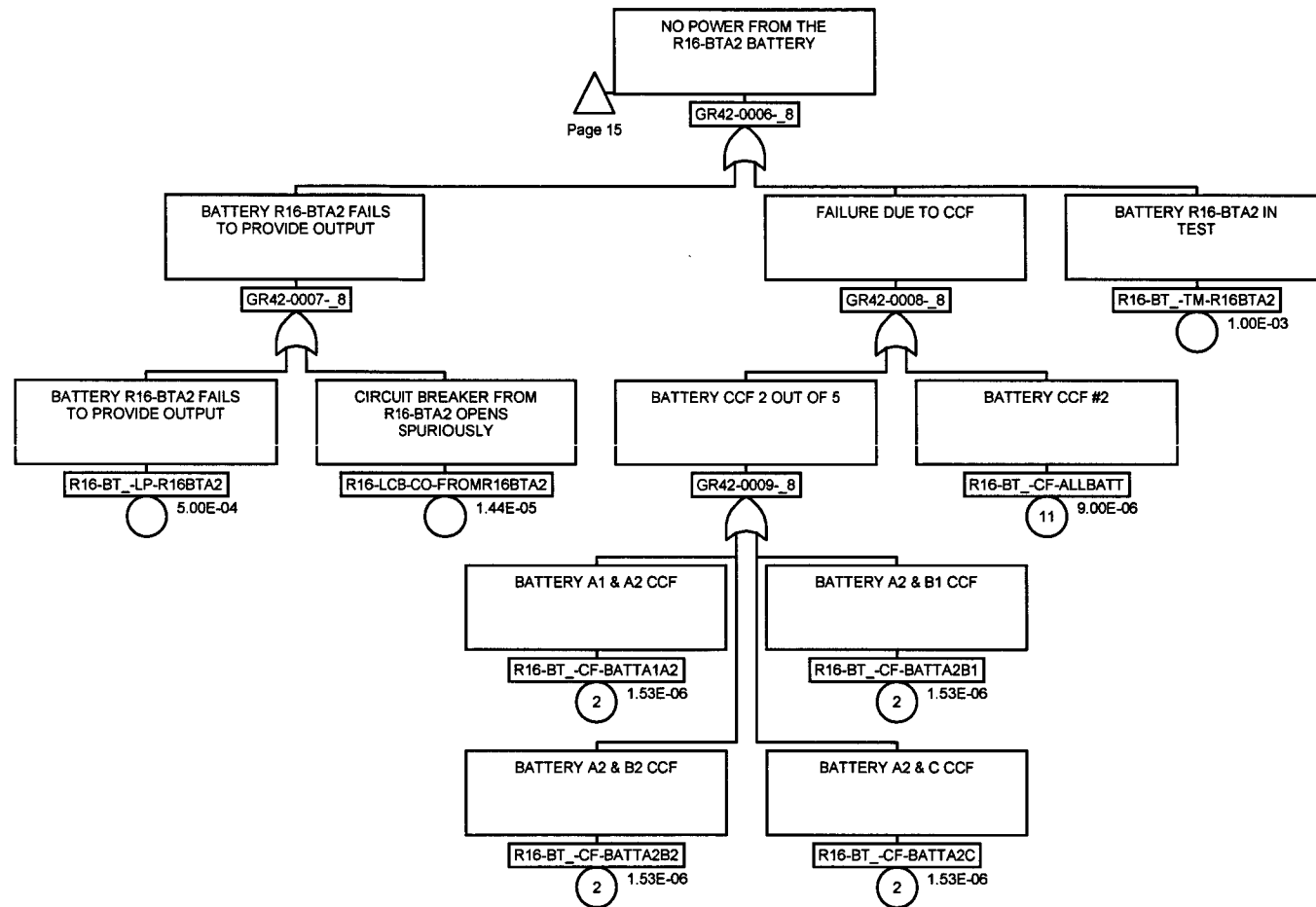
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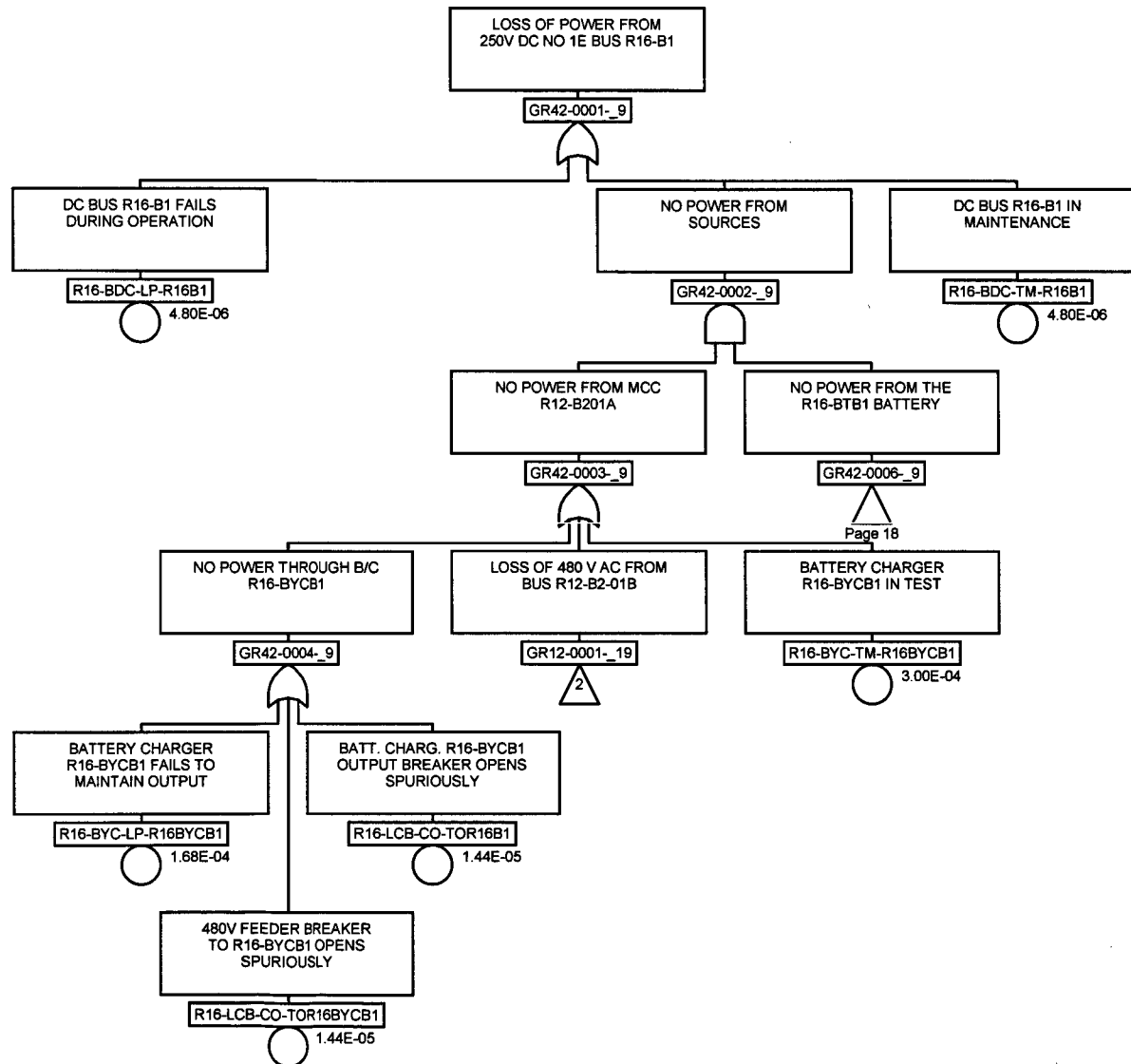


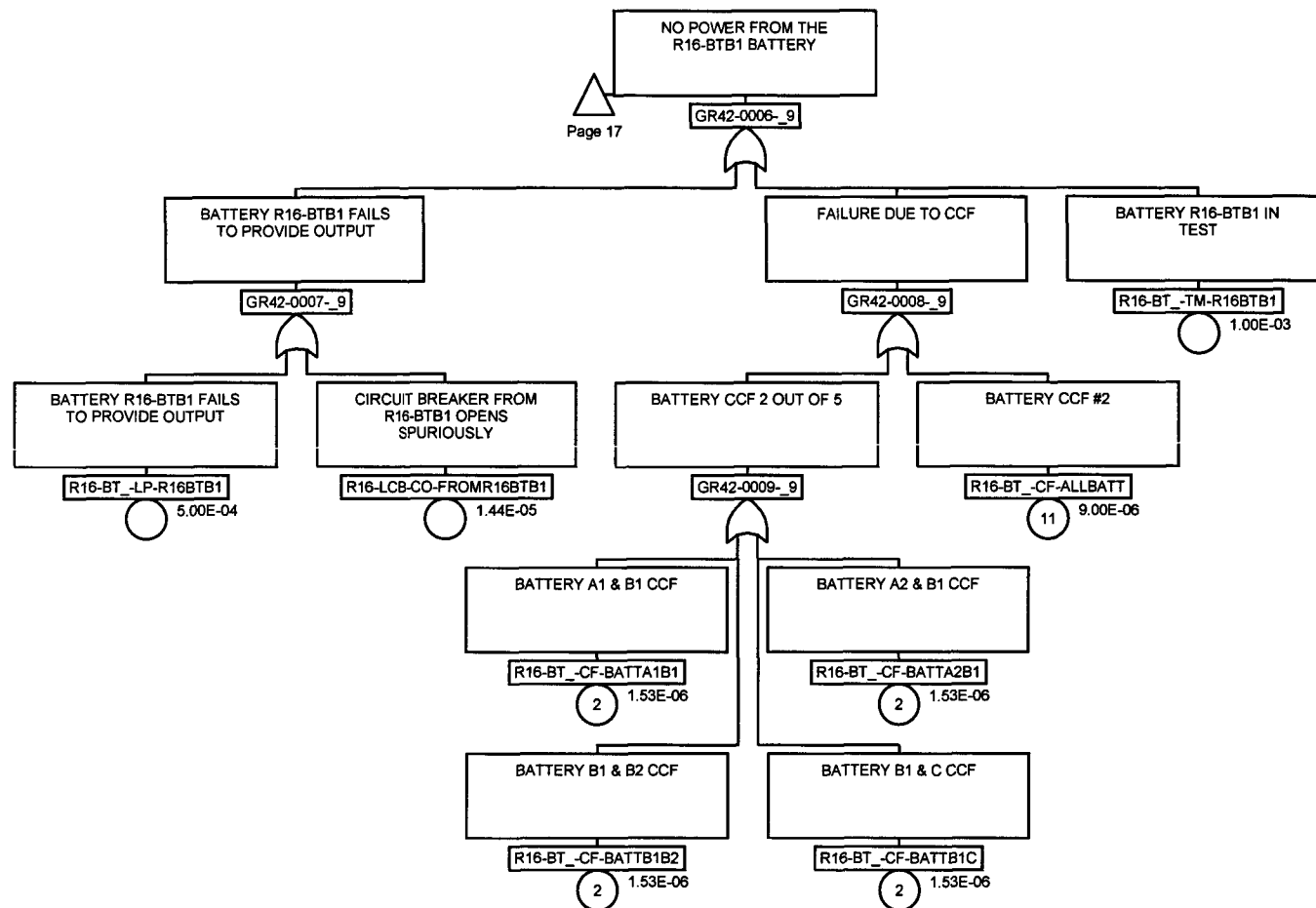
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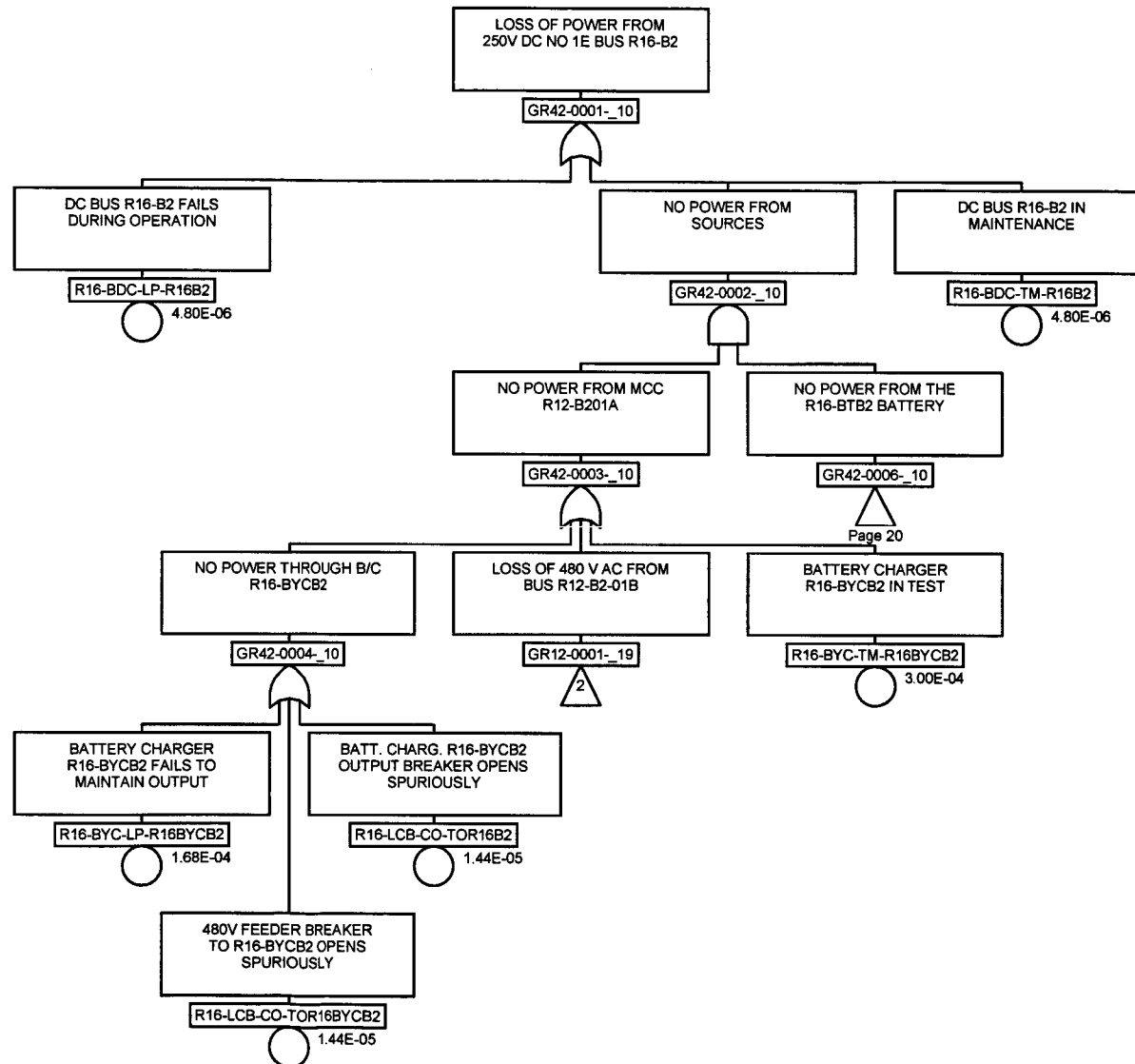


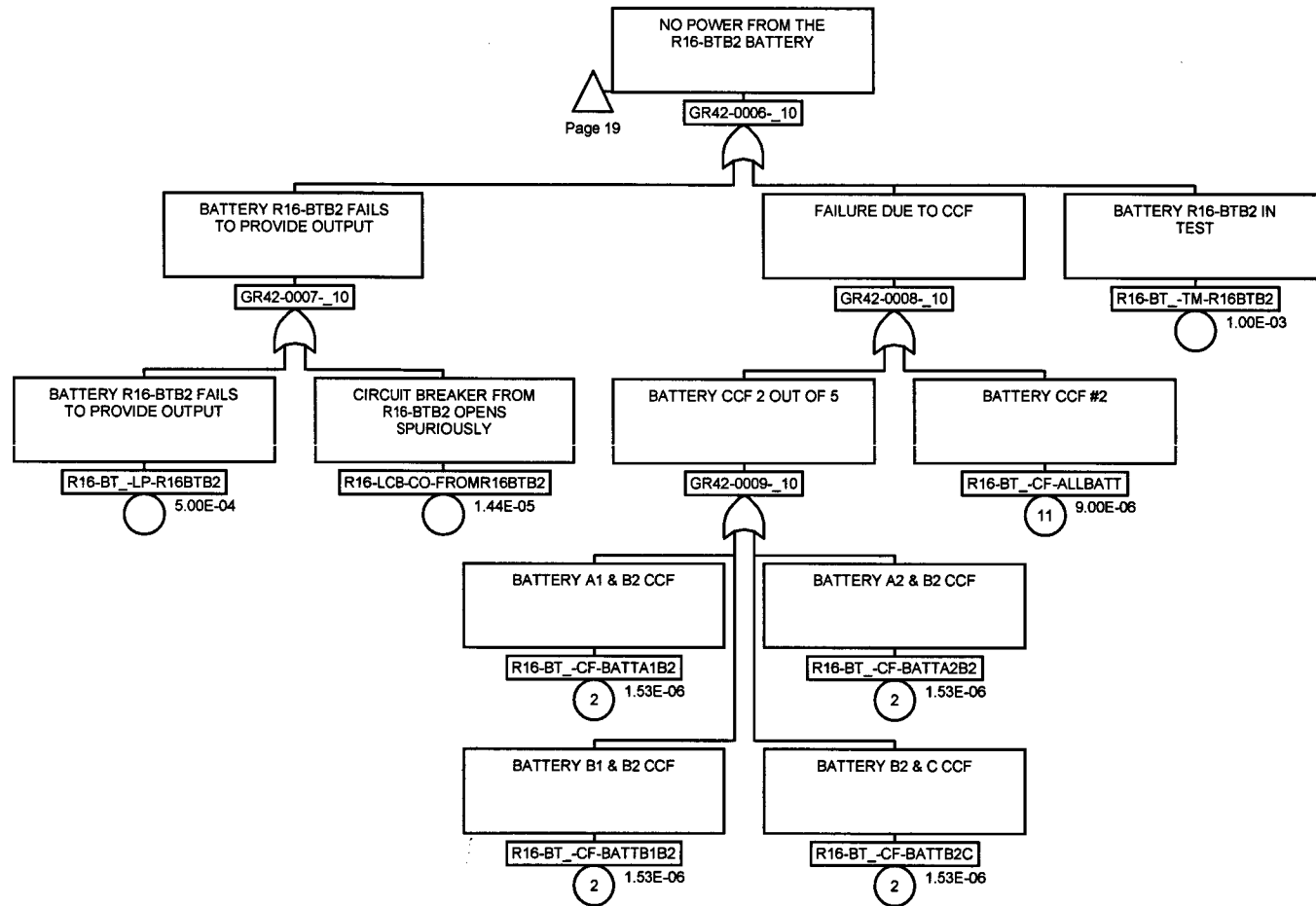
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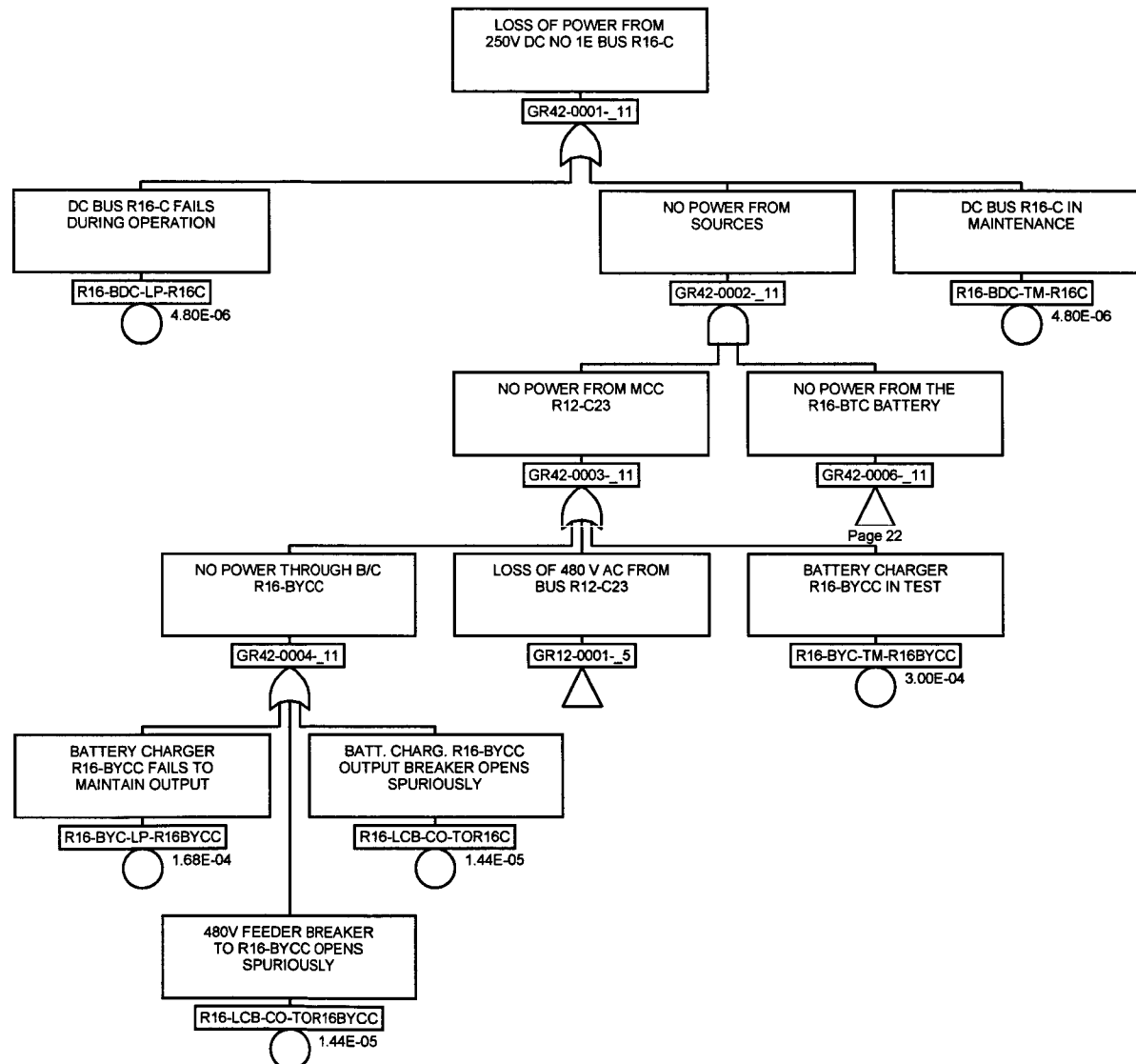


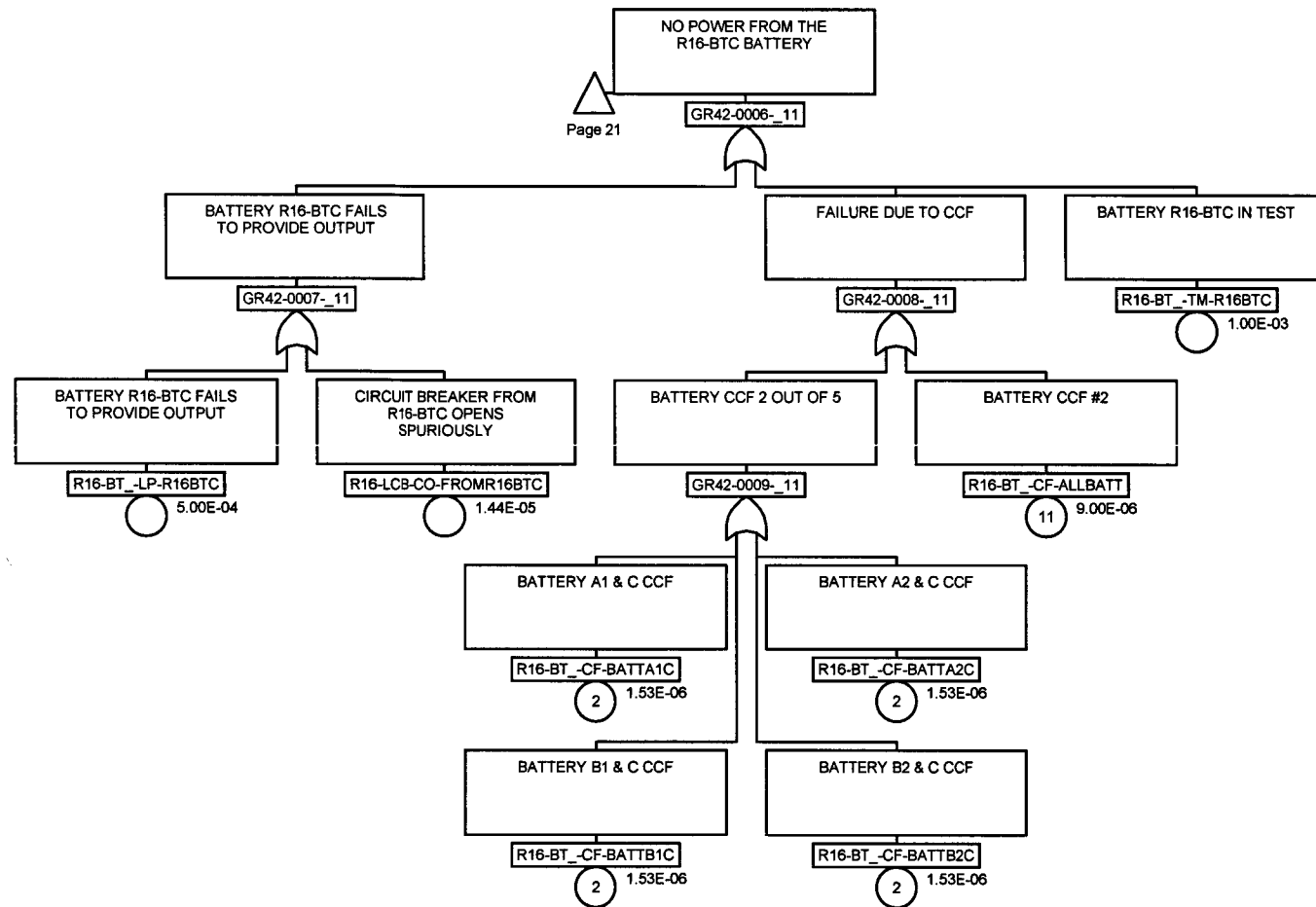
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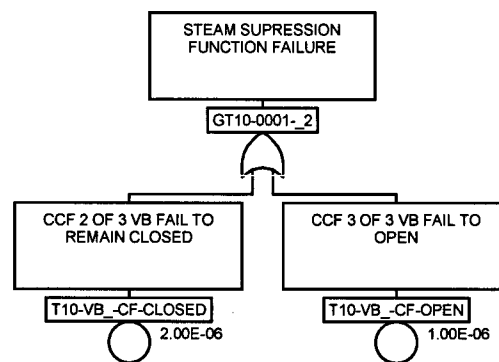


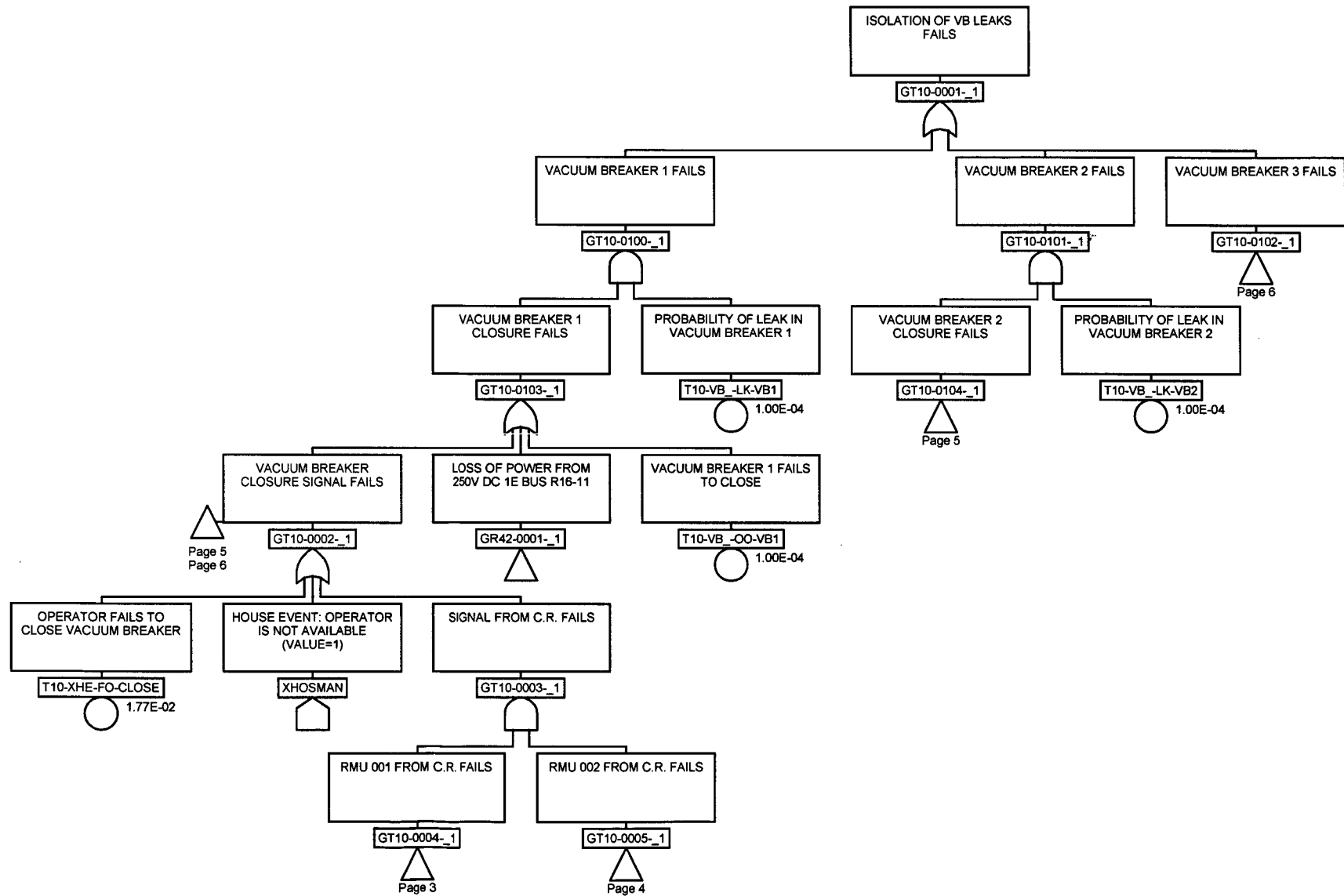
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| R16 Direct Current Power Supply System (DC) | | | X-DC Power.caf | | | Appendix B.4.17 |
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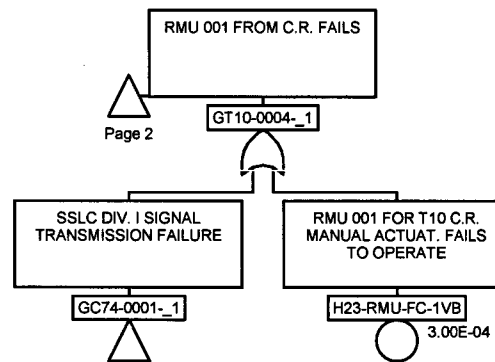
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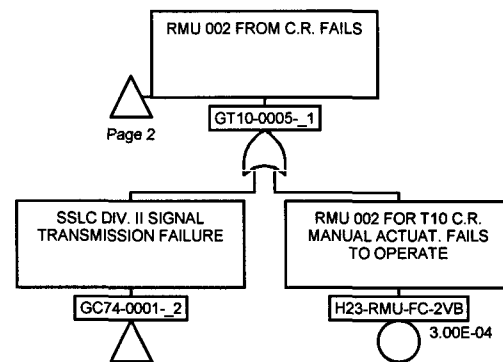
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| R16-BYC-TM-R16BYC41 | 11 | 4 | | | | |
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| R16-LCB-CO-FROMR16BT22 | 8 | 2 | | | | |
| R16-LCB-CO-FROMR16BT31 | 10 | 2 | | | | |
| R16-LCB-CO-FROMR16BT41 | 12 | 2 | | | | |
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| R16-LCB-CO-TOR1612 | 3 | 2 | | | | |
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| R16-LCB-CO-TOR16B2 | 19 | 2 | | | | |
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| R16-LCB-CO-TOR16BYC12 | 3 | 2 | | | | |
| R16-LCB-CO-TOR16BYC21 | 5 | 2 | | | | |
| R16-LCB-CO-TOR16BYC22 | 7 | 2 | | | | |
| R16-LCB-CO-TOR16BYC31 | 9 | 2 | | | | |
| R16-LCB-CO-TOR16BYC41 | 11 | 2 | | | | |
| R16-LCB-CO-TOR16BYCA1 | 13 | 2 | | | | |
| R16-LCB-CO-TOR16BYCA2 | 15 | 2 | | | | |
| R16-LCB-CO-TOR16BYCB1 | 17 | 2 | | | | |
| R16-LCB-CO-TOR16BYCB2 | 19 | 2 | | | | |
| R16-LCB-CO-TOR16BYCC | 21 | 2 | | | | |
| R16-LCB-CO-TOR16C | 21 | 2 | | | | |
| R16 Direct Current Power Supply System (DC) | | | | X-DC Power.caf | Appendix B.4.17 | Page 25 |

Appendix B.4.18-1 Containment Systems Fault Trees

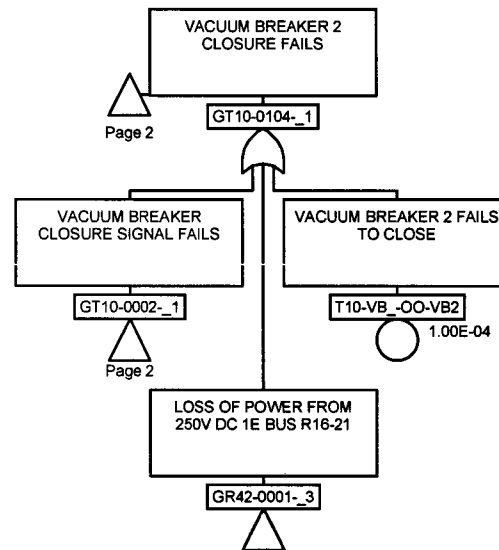






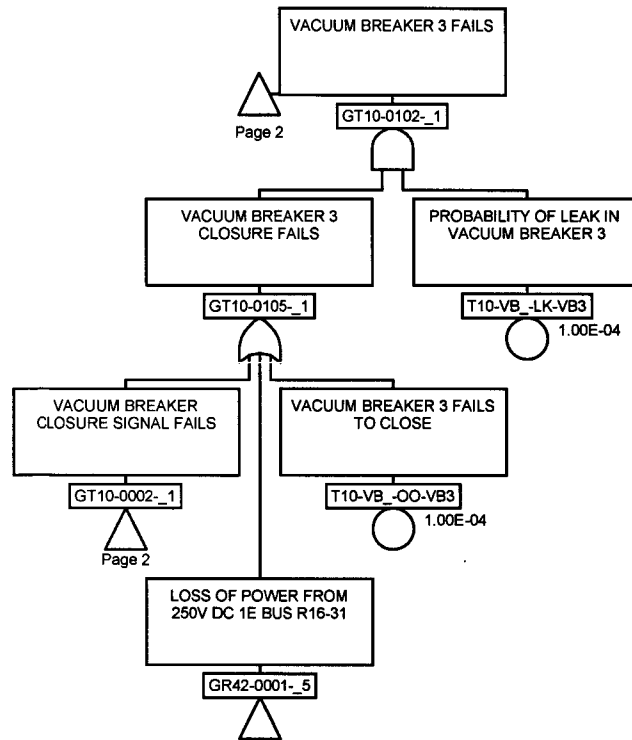


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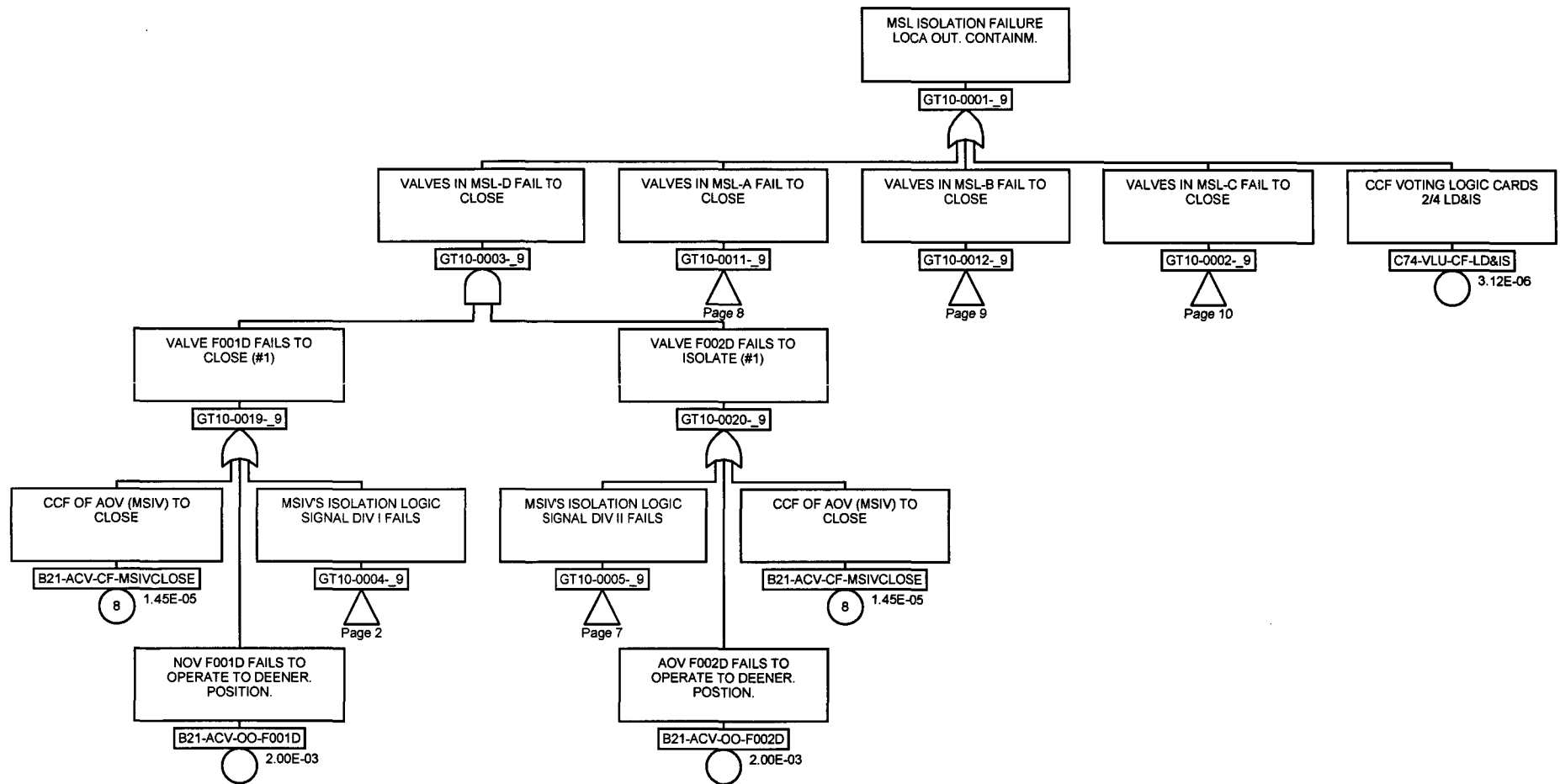
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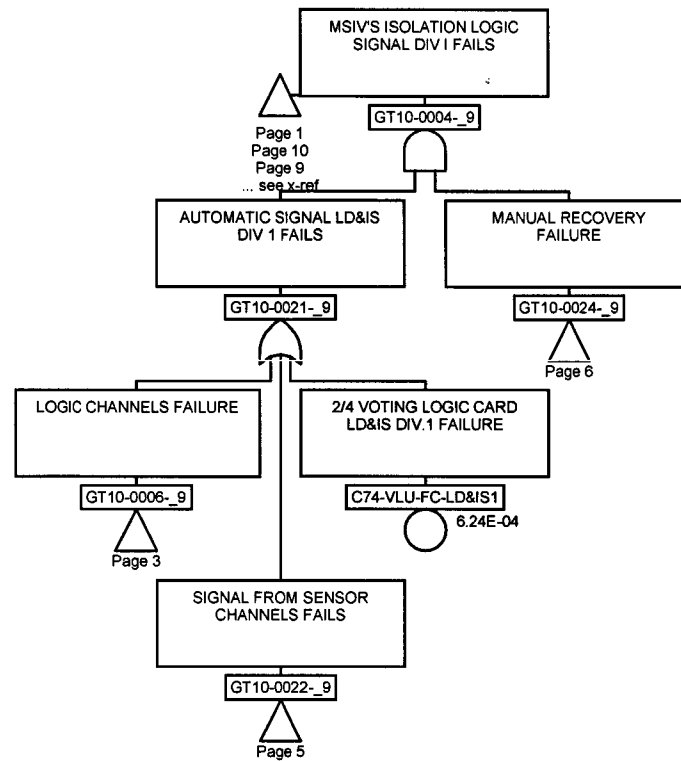
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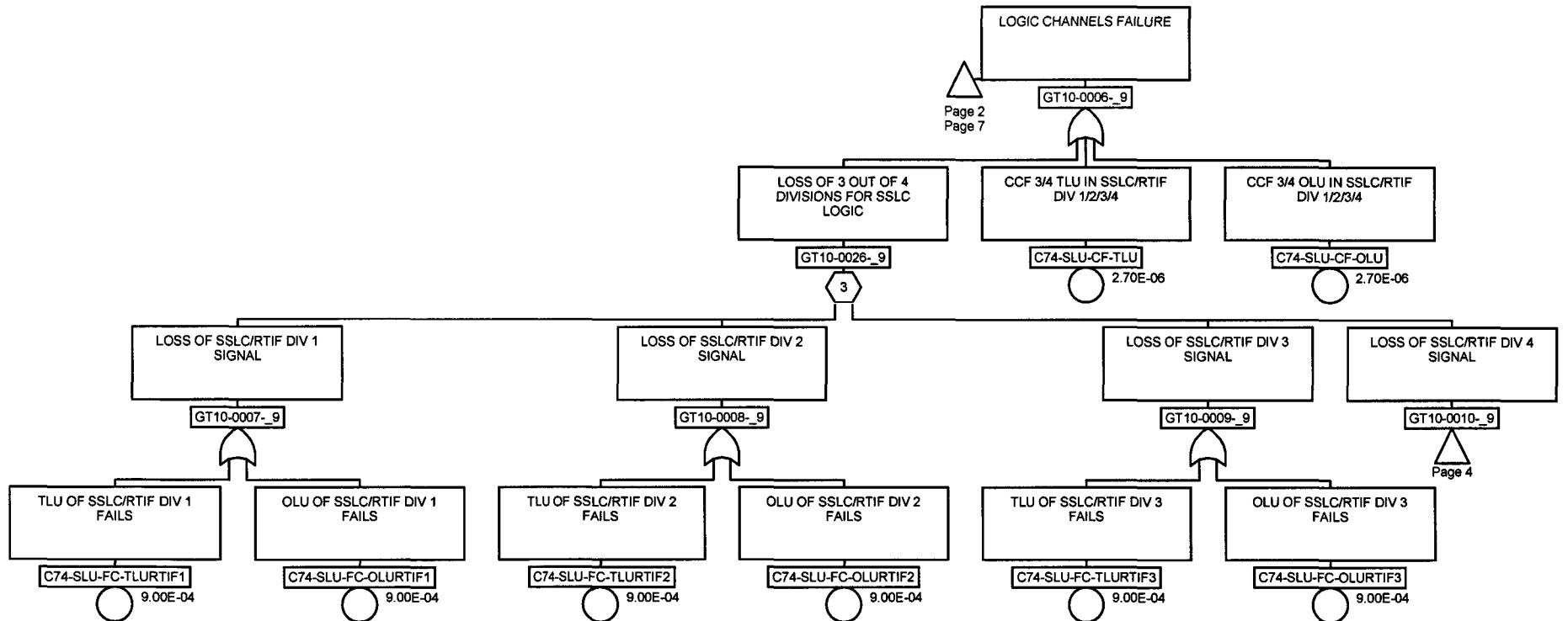


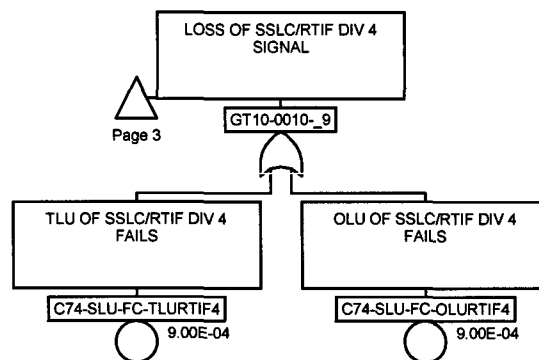
Appendix B.4.18-2
Containment Systems
Fault Trees

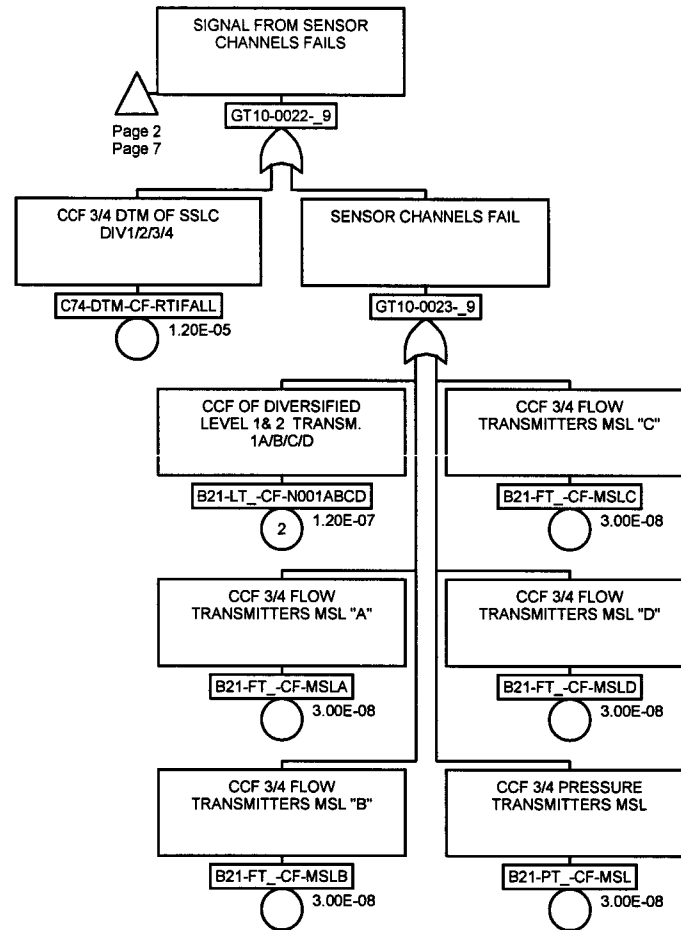
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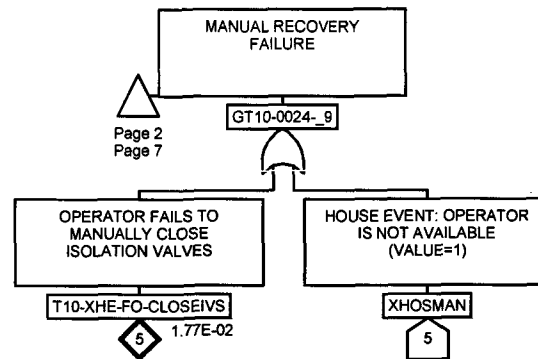


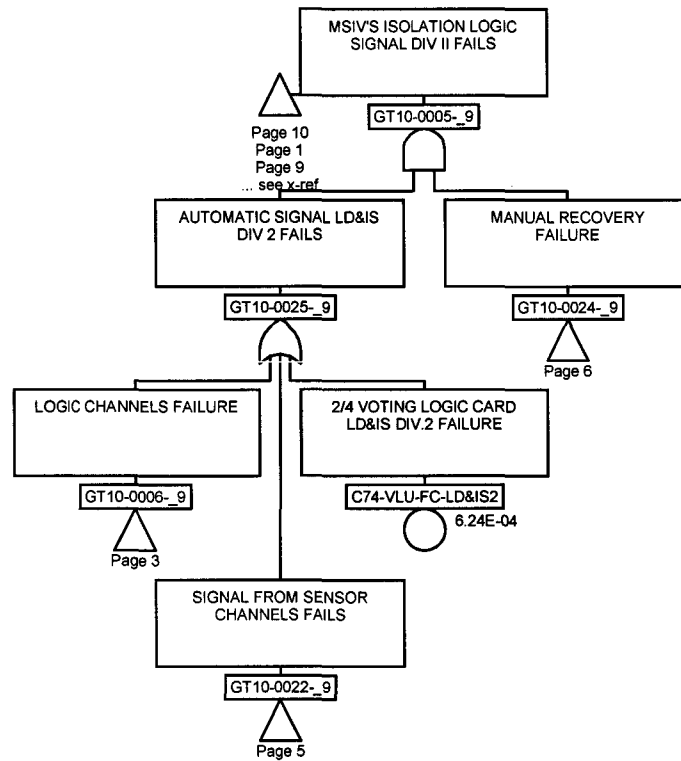


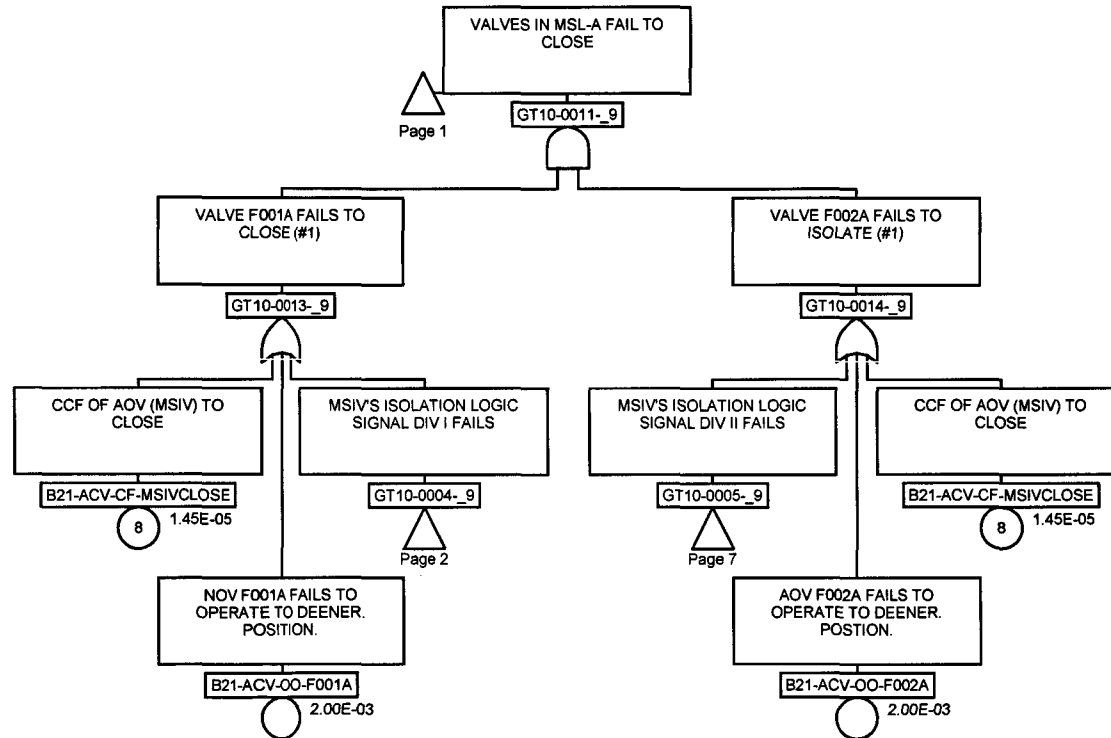


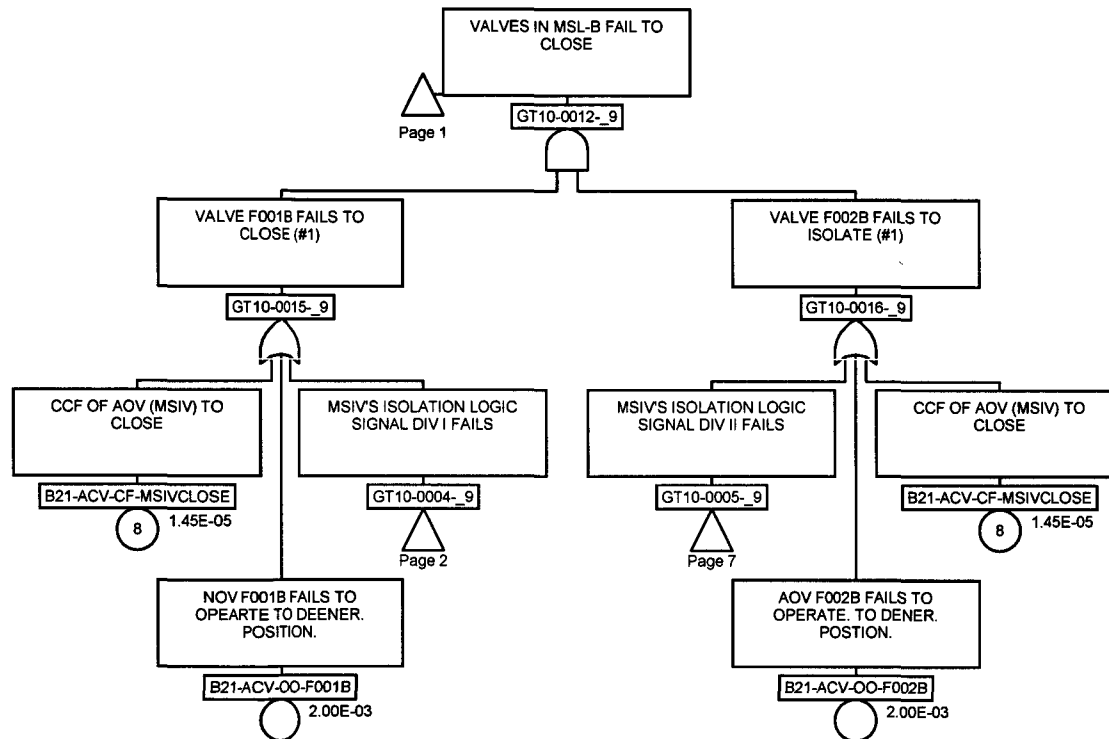


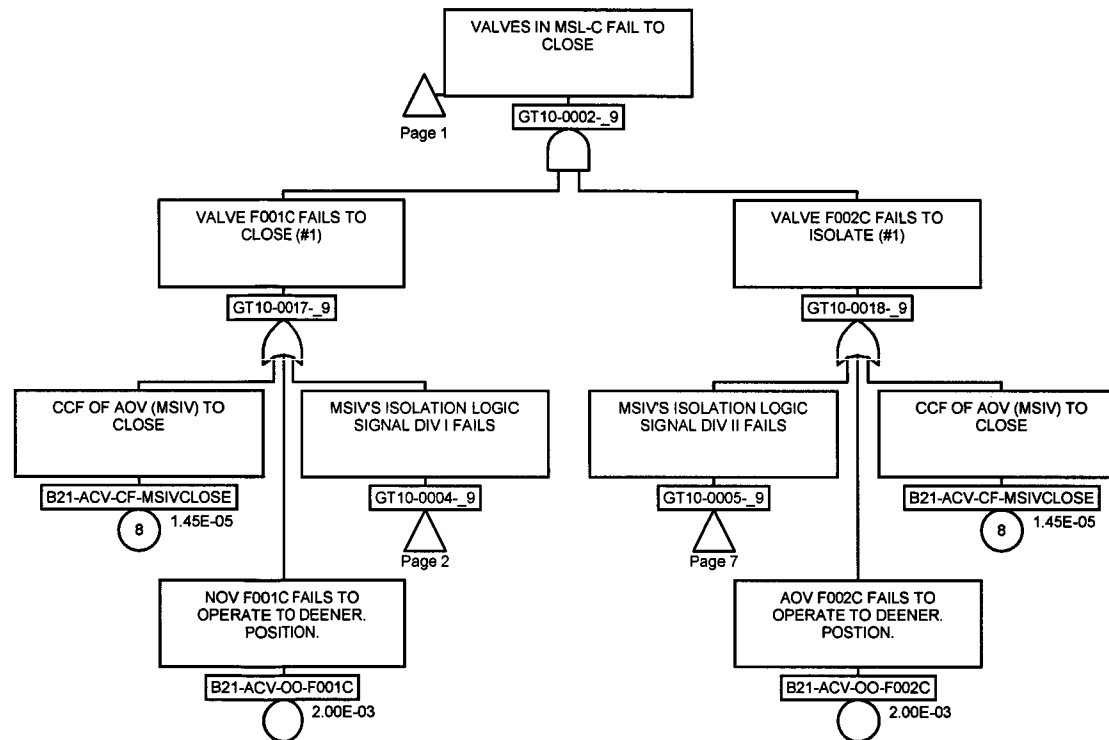


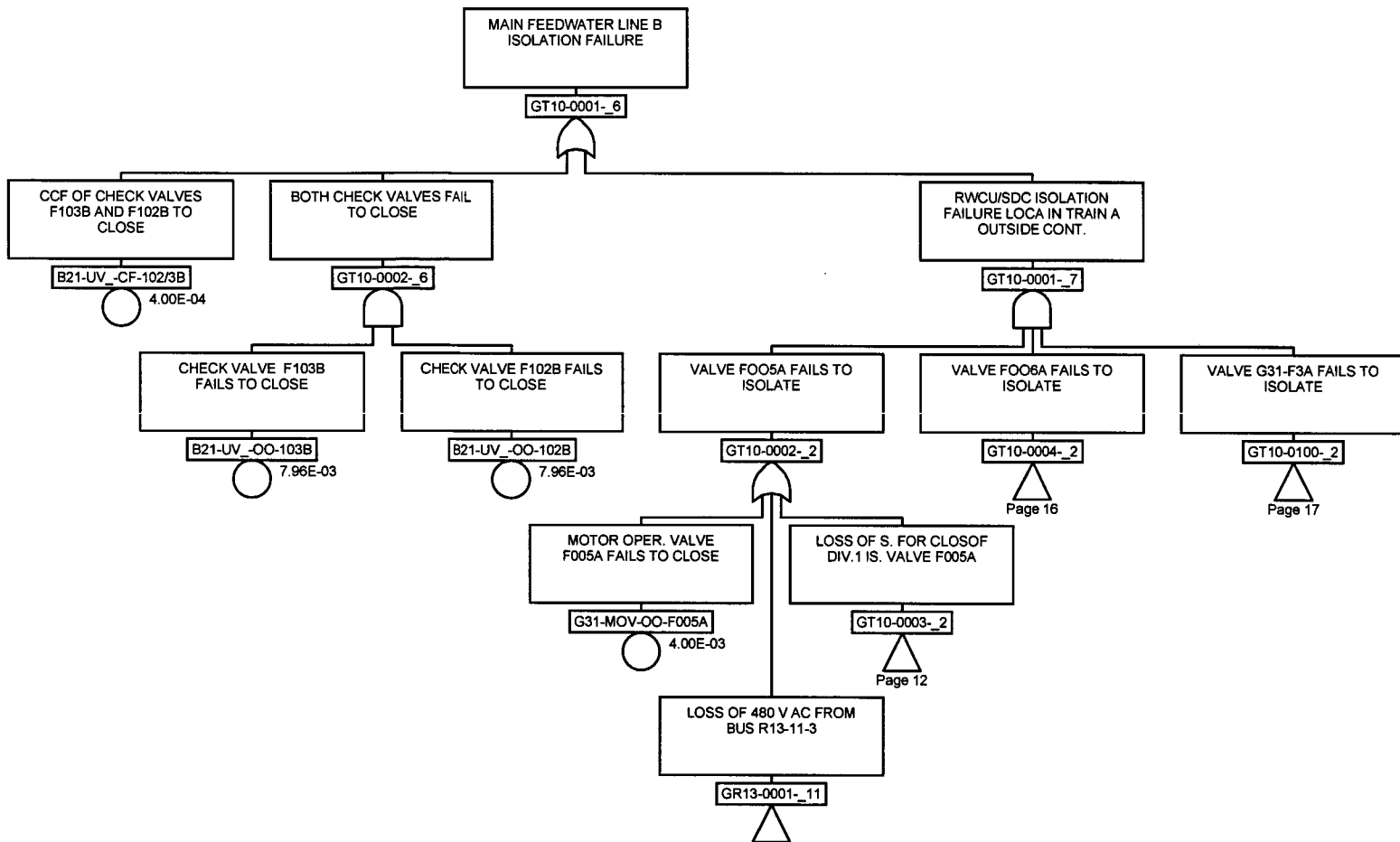


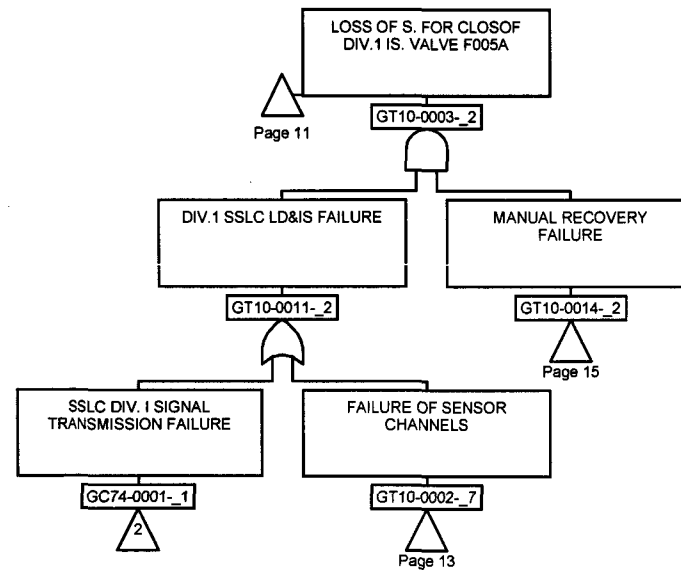


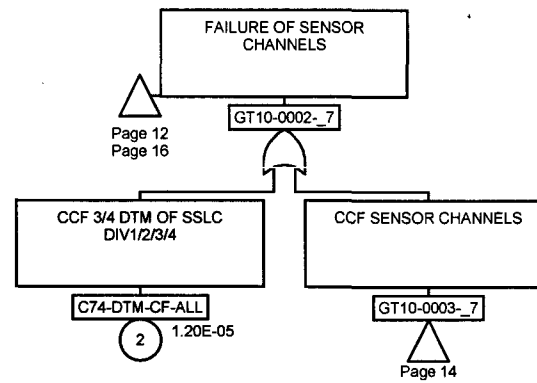


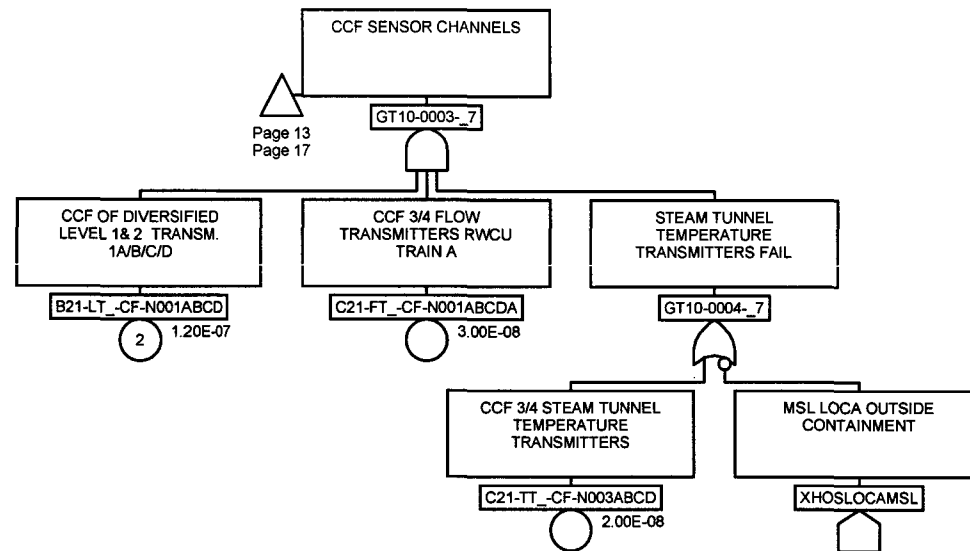


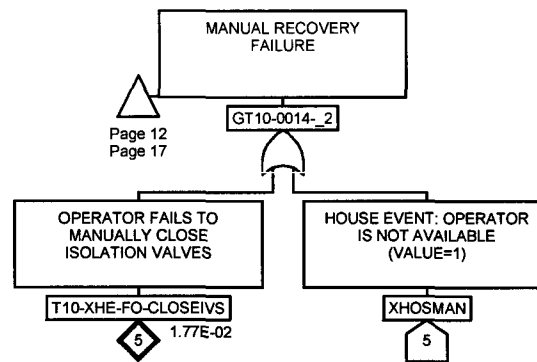


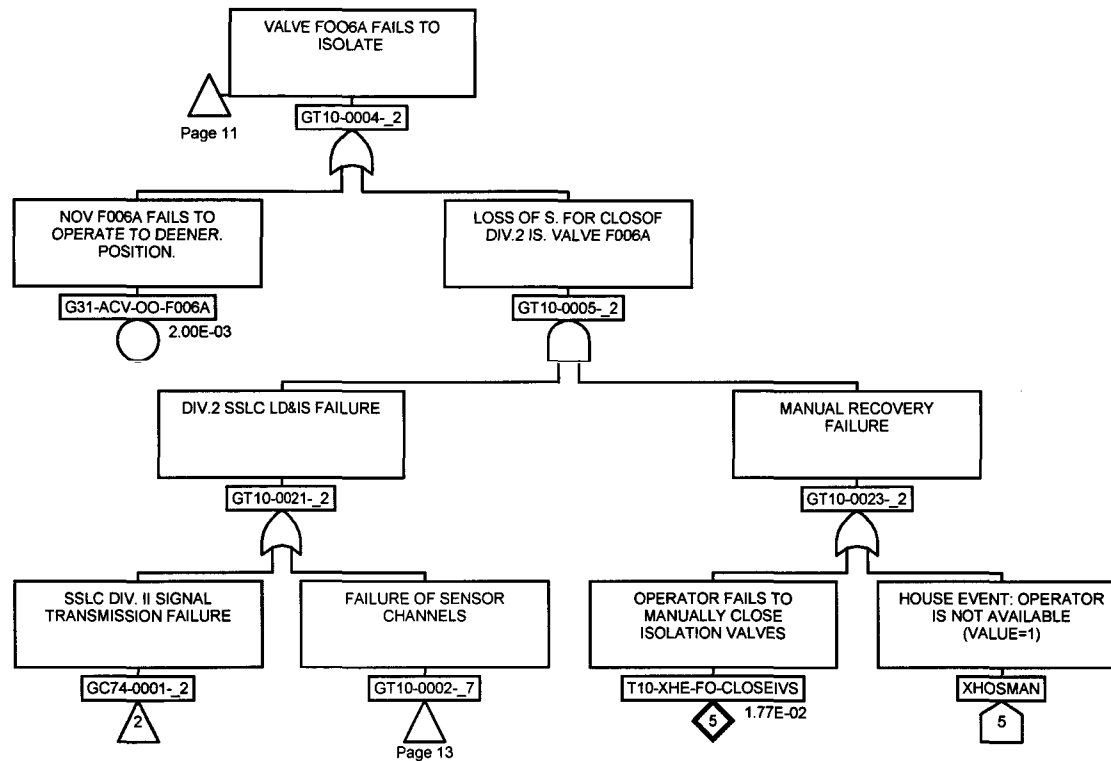


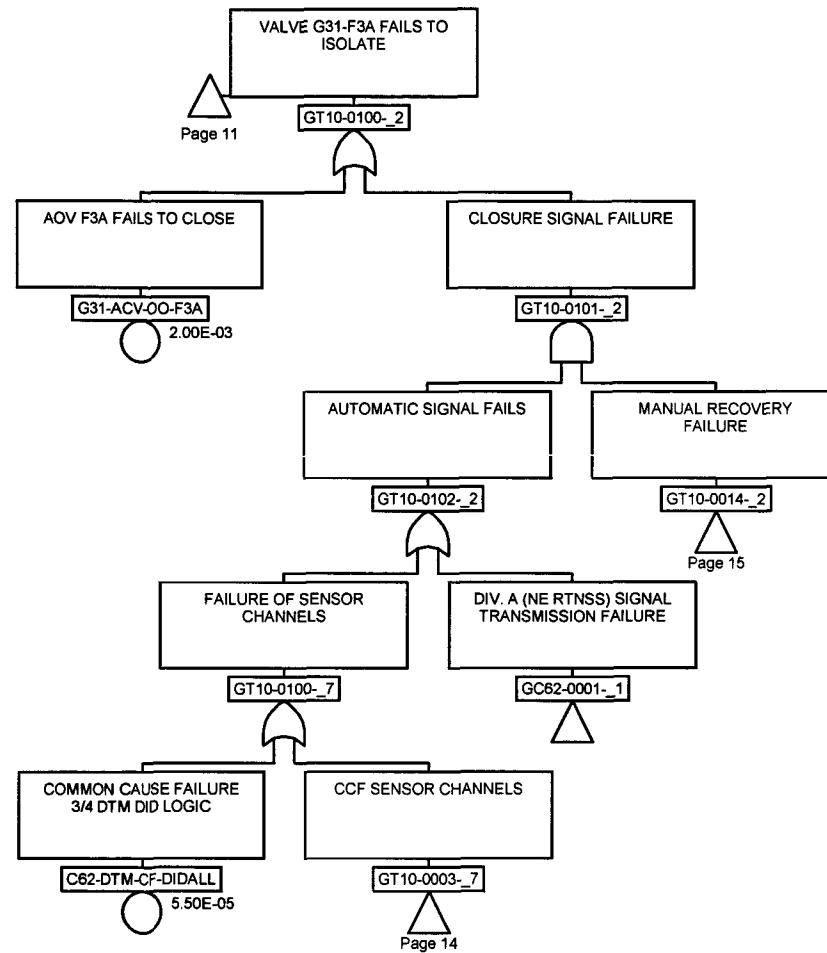


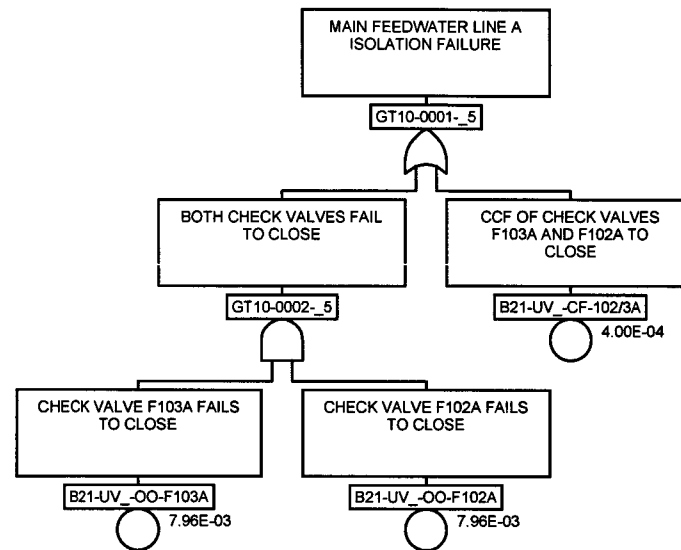




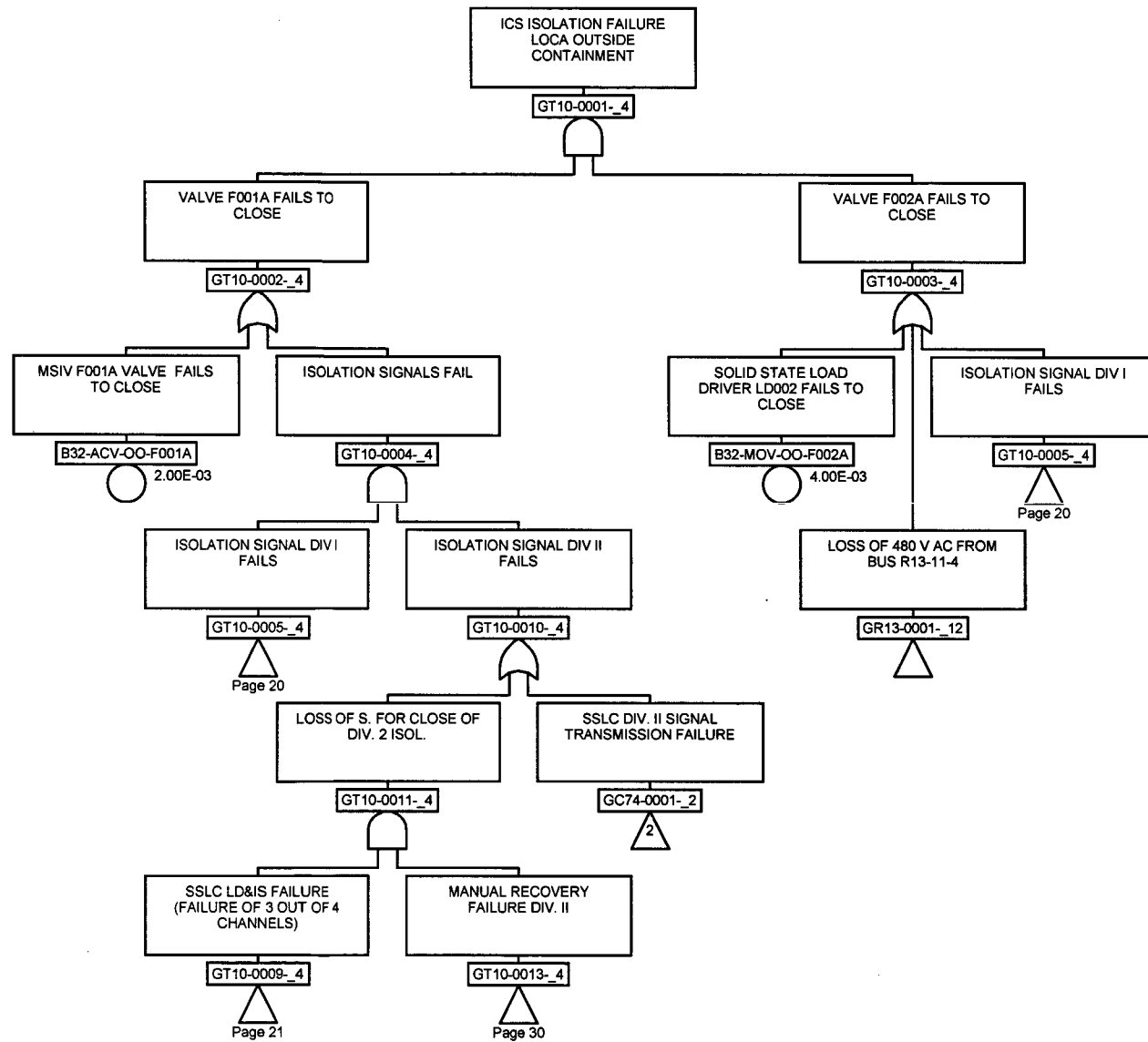


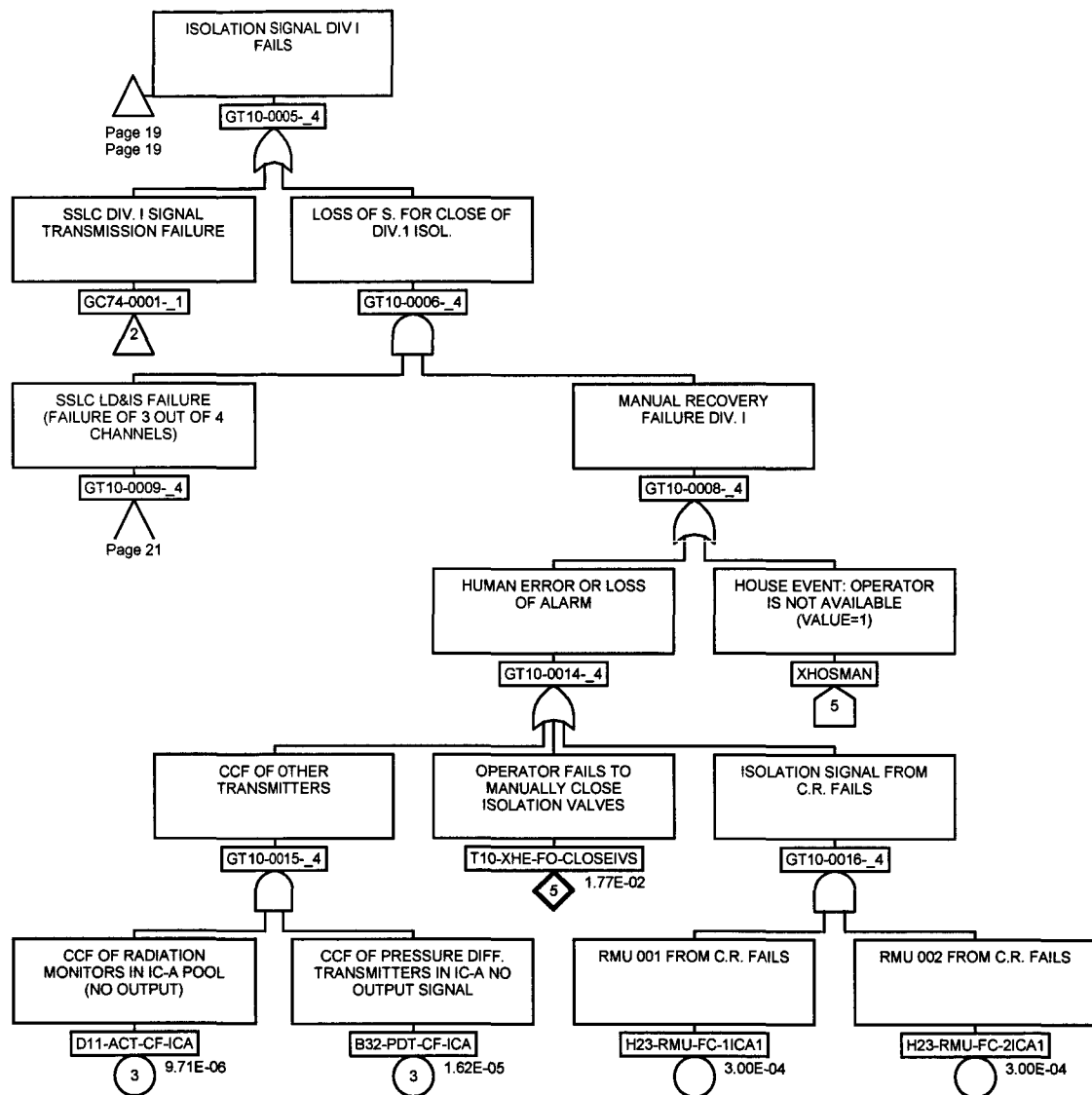


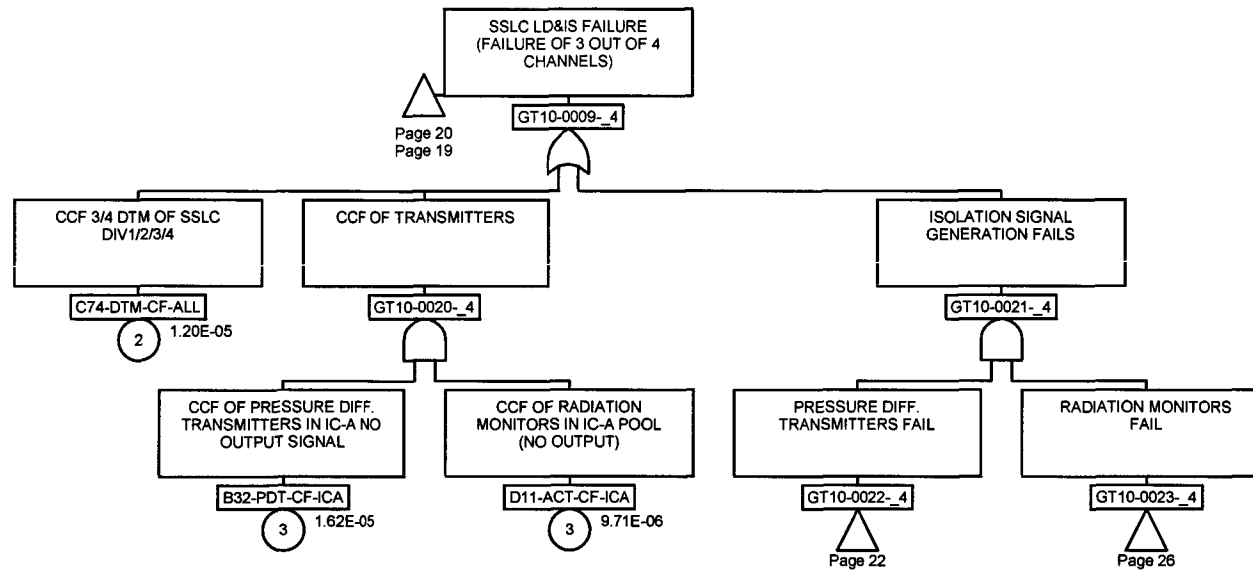


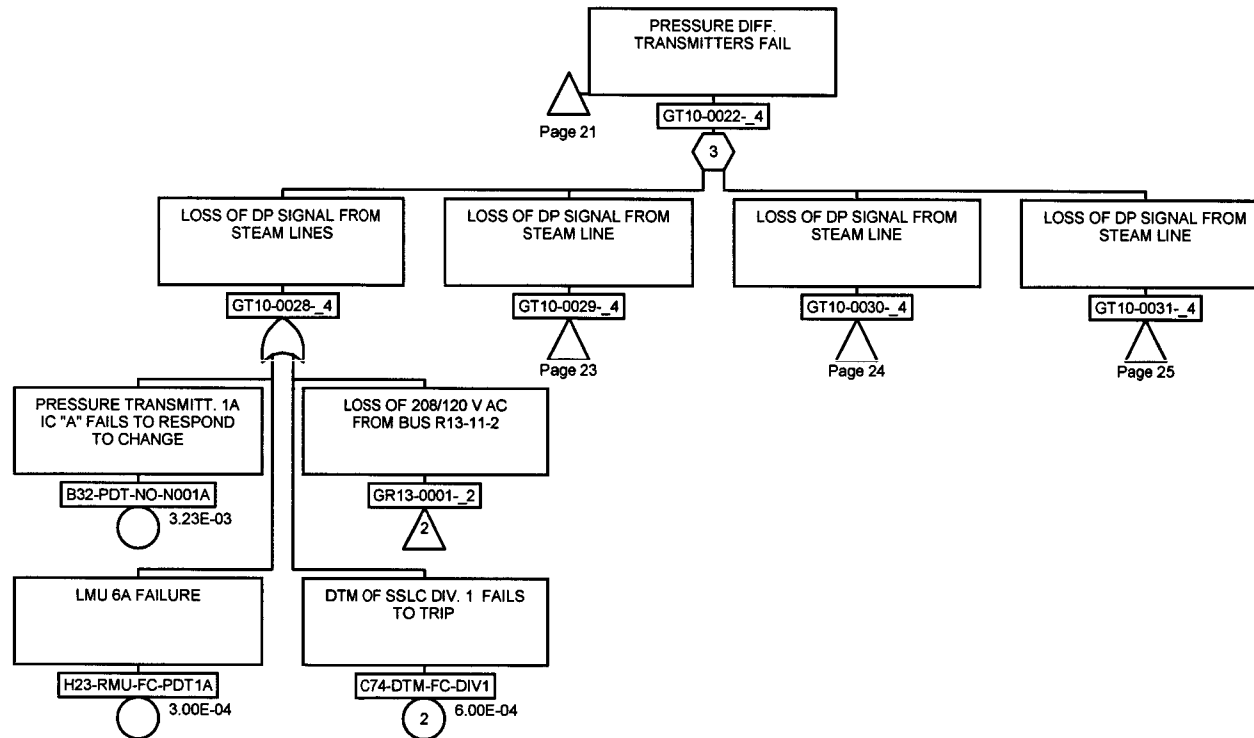


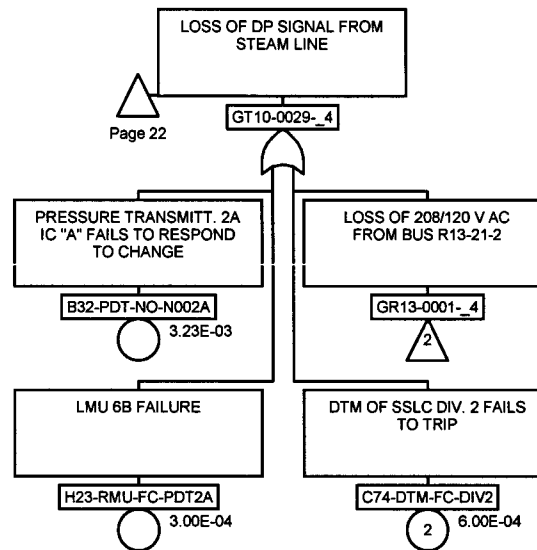
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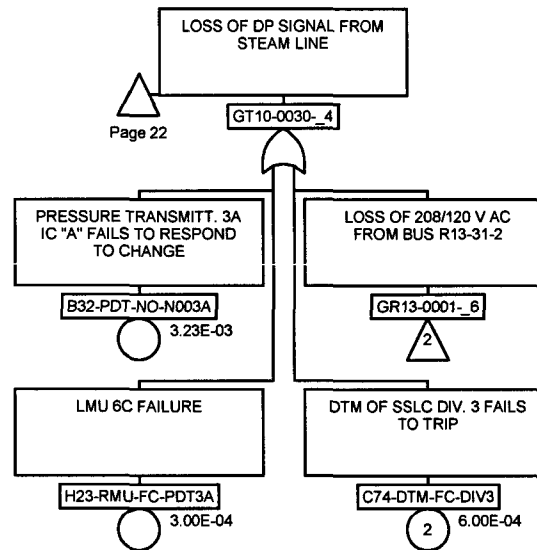


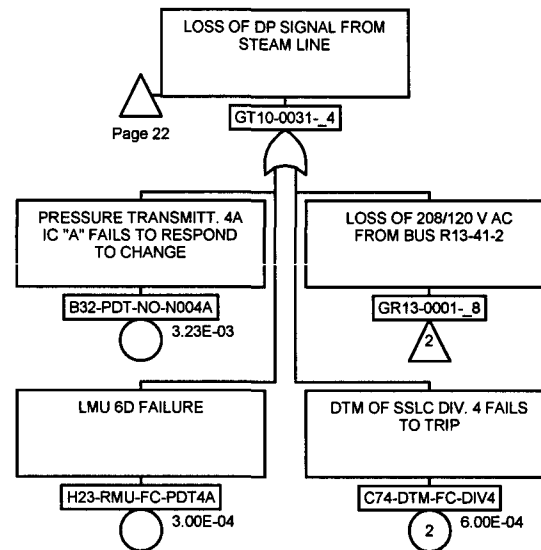


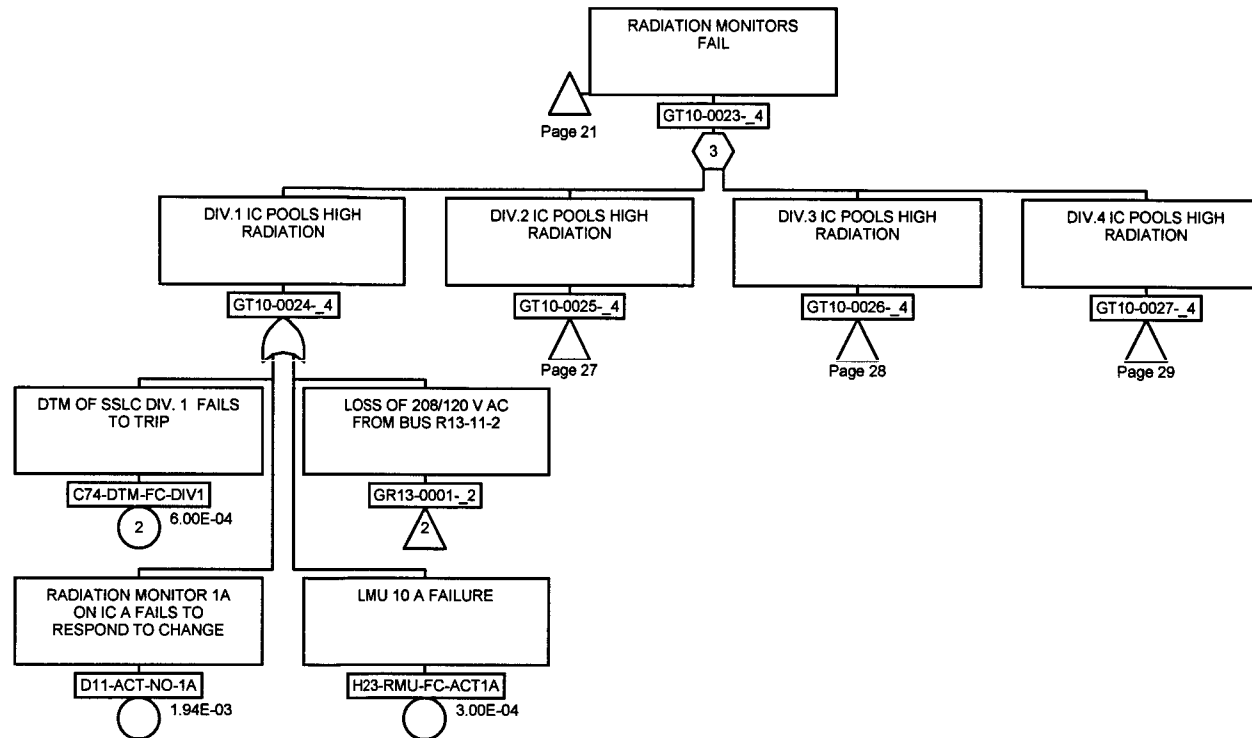


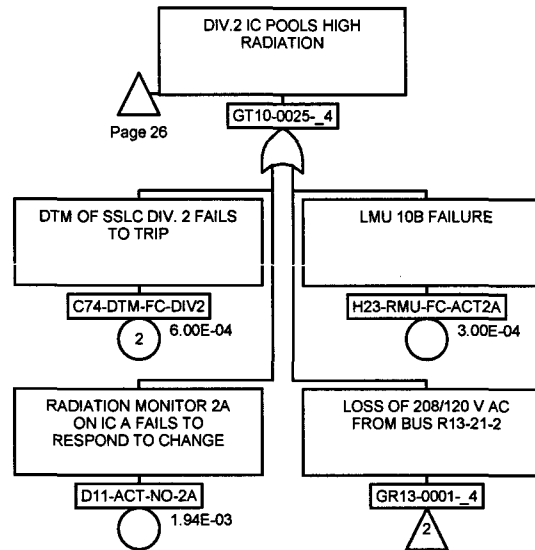


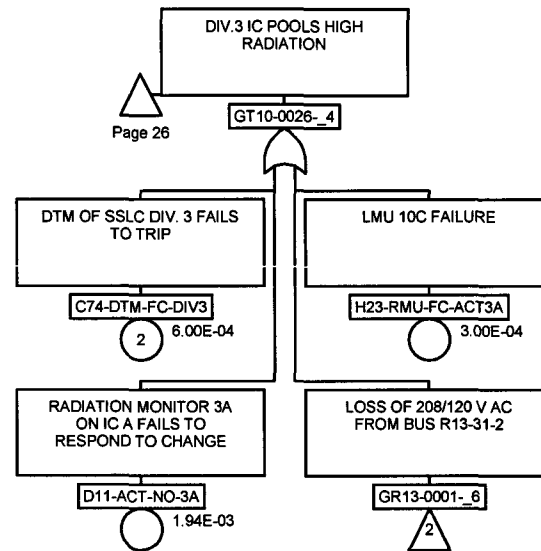


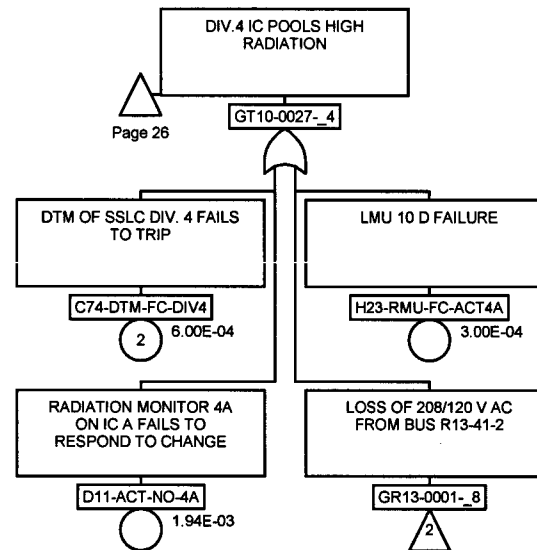


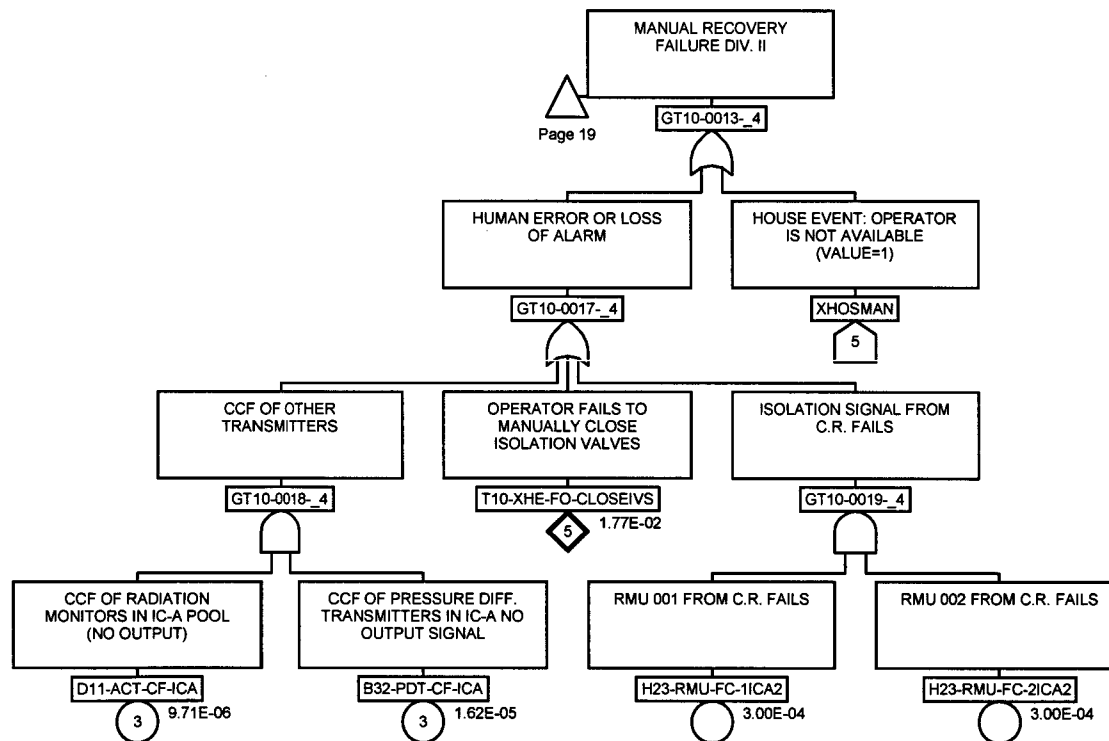








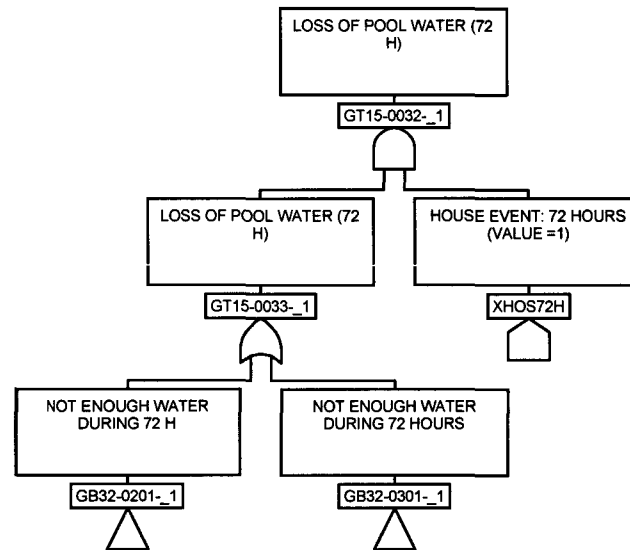


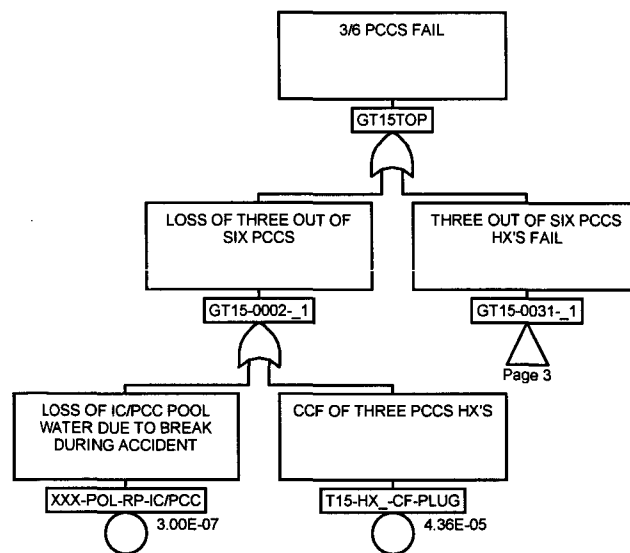


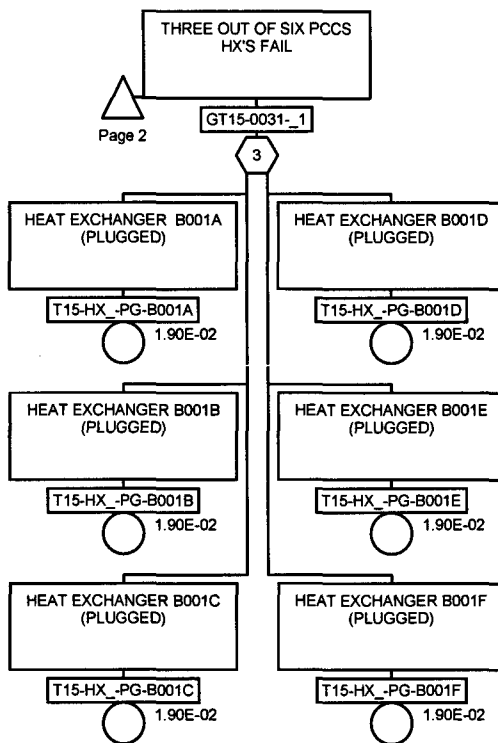
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|----------------------|------|------|---------------------|----------------|------|------|--------------|------|------|
| B21-ACV-CF-MSIVCLOSE | 1 | 1 | C74-SLU-FC-OLURTIF1 | | 3 | 2 | GT10-0003- 7 | 14 | 2 |
| B21-ACV-CF-MSIVCLOSE | 1 | 4 | C74-SLU-FC-OLURTIF2 | | 3 | 4 | GT10-0003- 7 | 17 | 2 |
| B21-ACV-CF-MSIVCLOSE | 8 | 1 | C74-SLU-FC-OLURTIF3 | | 3 | 6 | GT10-0003- 9 | 1 | 3 |
| B21-ACV-CF-MSIVCLOSE | 8 | 4 | C74-SLU-FC-OLURTIF4 | | 4 | 2 | GT10-0004- 2 | 11 | 5 |
| B21-ACV-CF-MSIVCLOSE | 9 | 1 | C74-SLU-FC-TLURTIF1 | | 3 | 1 | GT10-0004- 2 | 16 | 2 |
| B21-ACV-CF-MSIVCLOSE | 9 | 4 | C74-SLU-FC-TLURTIF2 | | 3 | 3 | GT10-0004- 4 | 19 | 2 |
| B21-ACV-CF-MSIVCLOSE | 10 | 1 | C74-SLU-FC-TLURTIF3 | | 3 | 5 | GT10-0004- 7 | 14 | 3 |
| B21-ACV-CF-MSIVCLOSE | 10 | 4 | C74-SLU-FC-TLURTIF4 | | 4 | 1 | GT10-0004- 9 | 1 | 2 |
| B21-ACV-OO-F001A | 8 | 2 | C74-VLU-CF-LD&IS | | 1 | 7 | GT10-0004- 9 | 2 | 2 |
| B21-ACV-OO-F001B | 9 | 2 | C74-VLU-FC-LD&IS1 | | 2 | 2 | GT10-0004- 9 | 8 | 2 |
| B21-ACV-OO-F001C | 10 | 2 | C74-VLU-FC-LD&IS2 | | 7 | 2 | GT10-0004- 9 | 9 | 2 |
| B21-ACV-OO-F001D | 1 | 2 | D11-ACT-CF-ICA | | 20 | 1 | GT10-0004- 9 | 10 | 2 |
| B21-ACV-OO-F002A | 8 | 4 | D11-ACT-CF-ICA | | 21 | 3 | GT10-0005- 2 | 16 | 3 |
| B21-ACV-OO-F002B | 9 | 4 | D11-ACT-CF-ICA | | 30 | 1 | GT10-0005- 4 | 19 | 2 |
| B21-ACV-OO-F002C | 10 | 4 | D11-ACT-NO-1A | | 26 | 1 | GT10-0005- 4 | 19 | 5 |
| B21-ACV-OO-F002D | 1 | 4 | D11-ACT-NO-2A | | 27 | 1 | GT10-0005- 4 | 20 | 2 |
| B21-FT -CF-MSLA | 5 | 2 | D11-ACT-NO-3A | | 28 | 1 | GT10-0005- 9 | 1 | 3 |
| B21-FT -CF-MSLB | 5 | 2 | D11-ACT-NO-4A | | 29 | 1 | GT10-0005- 9 | 7 | 2 |
| B21-FT -CF-MSLC | 5 | 3 | G31-ACV-OO-F006A | | 16 | 1 | GT10-0005- 9 | 8 | 3 |
| B21-FT -CF-MSLD | 5 | 3 | G31-ACV-OO-F3A | | 17 | 1 | GT10-0005- 9 | 9 | 3 |
| B21-LT -CF-N001ABCD | 5 | 2 | G31-MOV-OO-F005A | | 11 | 3 | GT10-0005- 9 | 10 | 3 |
| B21-LT -CF-N001ABCD | 14 | 1 | GC62-0001- 1 | | 17 | 3 | GT10-0006- 4 | 20 | 2 |
| B21-PT -CF-MSL | 5 | 3 | GC74-0001- 1 | | 12 | 1 | GT10-0006- 9 | 2 | 1 |
| B21-UV -CF-102/3A | 18 | 3 | GC74-0001- 1 | | 20 | 1 | GT10-0006- 9 | 3 | 5 |
| B21-UV -CF-102/3B | 11 | 1 | GC74-0001- 2 | | 16 | 1 | GT10-0006- 9 | 7 | 1 |
| B21-UV -OO-102B | 11 | 3 | GC74-0001- 2 | | 19 | 3 | GT10-0007- 9 | 3 | 2 |
| B21-UV -OO-103B | 11 | 2 | GR13-0001- 11 | | 11 | 4 | GT10-0008- 4 | 20 | 3 |
| B21-UV -OO-F102A | 18 | 2 | GR13-0001- 12 | | 19 | 4 | GT10-0008- 9 | 3 | 4 |
| B21-UV -OO-F103A | 18 | 1 | GR13-0001- 2 | | 22 | 2 | GT10-0009- 4 | 19 | 2 |
| B32-ACV-OO-F001A | 19 | 1 | GR13-0001- 2 | | 26 | 2 | GT10-0009- 4 | 20 | 1 |
| B32-MOV-OO-F002A | 19 | 4 | GR13-0001- 4 | | 23 | 2 | GT10-0009- 4 | 21 | 3 |
| B32-PDT-CF-ICA | 20 | 2 | GR13-0001- 4 | | 27 | 2 | GT10-0009- 9 | 3 | 6 |
| B32-PDT-CF-ICA | 21 | 2 | GR13-0001- 6 | | 24 | 2 | GT10-0010- 4 | 19 | 3 |
| B32-PDT-CF-ICA | 30 | 2 | GR13-0001- 6 | | 28 | 2 | GT10-0010- 9 | 3 | 7 |
| B32-PDT-NO-N001A | 22 | 1 | GR13-0001- 8 | | 25 | 2 | GT10-0010- 9 | 4 | 2 |
| B32-PDT-NO-N002A | 23 | 1 | GR13-0001- 8 | | 29 | 2 | GT10-0011- 2 | 12 | 2 |
| B32-PDT-NO-N003A | 24 | 1 | GT10-0001- 4 | | 19 | 3 | GT10-0011- 4 | 19 | 2 |
| B32-PDT-NO-N004A | 25 | 1 | GT10-0001- 5 | | 18 | 2 | GT10-0011- 9 | 1 | 4 |
| C21-FT -CF-N001ABCD | 14 | 2 | GT10-0001- 6 | | 11 | 3 | GT10-0011- 9 | 8 | 3 |
| C21-TT -CF-N003ABCD | 14 | 3 | GT10-0001- 7 | | 11 | 5 | GT10-0012- 9 | 1 | 5 |
| C62-DTM-CF-DIDALL | 17 | 1 | GT10-0001- 9 | | 1 | 5 | GT10-0012- 9 | 9 | 3 |
| C74-DTM-CF-ALL | 13 | 1 | GT10-0002- 2 | | 11 | 4 | GT10-0013- 4 | 19 | 3 |
| C74-DTM-CF-ALL | 21 | 1 | GT10-0002- 4 | | 19 | 2 | GT10-0013- 4 | 30 | 3 |
| C74-DTM-CF-RTIFALL | 5 | 1 | GT10-0002- 5 | | 18 | 2 | GT10-0013- 9 | 8 | 2 |
| C74-DTM-FC-DIV1 | 22 | 2 | GT10-0002- 6 | | 11 | 2 | GT10-0014- 2 | 12 | 3 |
| C74-DTM-FC-DIV1 | 26 | 1 | GT10-0002- 7 | | 12 | 2 | GT10-0014- 2 | 15 | 2 |
| C74-DTM-FC-DIV2 | 23 | 2 | GT10-0002- 7 | | 13 | 2 | GT10-0014- 2 | 17 | 3 |
| C74-DTM-FC-DIV2 | 27 | 1 | GT10-0002- 7 | | 16 | 2 | GT10-0014- 4 | 20 | 3 |
| C74-DTM-FC-DIV3 | 24 | 2 | GT10-0002- 9 | | 1 | 6 | GT10-0014- 9 | 8 | 4 |
| C74-DTM-FC-DIV3 | 28 | 1 | GT10-0002- 9 | | 10 | 3 | GT10-0015- 4 | 20 | 2 |
| C74-DTM-FC-DIV4 | 25 | 2 | GT10-0003- 2 | | 11 | 4 | GT10-0015- 9 | 9 | 2 |
| C74-DTM-FC-DIV4 | 29 | 1 | GT10-0003- 2 | | 12 | 2 | GT10-0016- 4 | 20 | 4 |
| C74-SLU-CF-OLU | 3 | 6 | GT10-0003- 4 | | 19 | 4 | GT10-0016- 9 | 9 | 4 |
| C74-SLU-CF-TLU | 3 | 5 | GT10-0003- 7 | | 13 | 2 | GT10-0017- 4 | 30 | 3 |

| Name | Page | Zone | Name NEDO-33201 Rev 1 | Page | Zone | Name | Page | Zone |
|-----------------------|------|------|-----------------------|------|------|--------------------|-------------------|---------|
| GT10-0017- 9 | 10 | 2 | H23-RMU-FC-PDT4A | 25 | 1 | | | |
| GT10-0018- 4 | 30 | 2 | T10-XHE-FO-CLOSEIVS | 6 | 1 | | | |
| GT10-0018- 9 | 10 | 4 | T10-XHE-FO-CLOSEIVS | 15 | 1 | | | |
| GT10-0019- 4 | 30 | 4 | T10-XHE-FO-CLOSEIVS | 16 | 3 | | | |
| GT10-0019- 9 | 1 | 2 | T10-XHE-FO-CLOSEIVS | 20 | 3 | | | |
| GT10-0020- 4 | 21 | 2 | T10-XHE-FO-CLOSEIVS | 30 | 3 | | | |
| GT10-0020- 9 | 1 | 4 | XHOSLOCAMSL | 14 | 4 | | | |
| GT10-0021- 2 | 16 | 2 | XHOSMAN | 6 | 2 | | | |
| GT10-0021- 4 | 21 | 4 | XHOSMAN | 15 | 2 | | | |
| GT10-0021- 9 | 2 | 2 | XHOSMAN | 16 | 4 | | | |
| GT10-0022- 4 | 21 | 4 | XHOSMAN | 20 | 4 | | | |
| GT10-0022- 4 | 22 | 3 | XHOSMAN | 30 | 4 | | | |
| GT10-0022- 9 | 2 | 2 | | | | | | |
| GT10-0022- 9 | 5 | 2 | | | | | | |
| GT10-0022- 9 | 7 | 2 | | | | | | |
| GT10-0023- 2 | 16 | 4 | | | | | | |
| GT10-0023- 4 | 21 | 5 | | | | | | |
| GT10-0023- 4 | 26 | 3 | | | | | | |
| GT10-0023- 9 | 5 | 2 | | | | | | |
| GT10-0024- 4 | 26 | 2 | | | | | | |
| GT10-0024- 9 | 2 | 3 | | | | | | |
| GT10-0024- 9 | 6 | 2 | | | | | | |
| GT10-0024- 9 | 7 | 3 | | | | | | |
| GT10-0025- 4 | 26 | 3 | | | | | | |
| GT10-0025- 4 | 27 | 2 | | | | | | |
| GT10-0025- 9 | 7 | 2 | | | | | | |
| GT10-0026- 4 | 26 | 4 | | | | | | |
| GT10-0026- 4 | 28 | 2 | | | | | | |
| GT10-0026- 9 | 3 | 4 | | | | | | |
| GT10-0027- 4 | 26 | 5 | | | | | | |
| GT10-0027- 4 | 29 | 2 | | | | | | |
| GT10-0028- 4 | 22 | 2 | | | | | | |
| GT10-0029- 4 | 22 | 3 | | | | | | |
| GT10-0029- 4 | 23 | 2 | | | | | | |
| GT10-0030- 4 | 22 | 4 | | | | | | |
| GT10-0030- 4 | 24 | 2 | | | | | | |
| GT10-0031- 4 | 22 | 5 | | | | | | |
| GT10-0031- 4 | 25 | 2 | | | | | | |
| GT10-0100- 2 | 11 | 6 | | | | | | |
| GT10-0100- 2 | 17 | 2 | | | | | | |
| GT10-0100- 7 | 17 | 2 | | | | | | |
| GT10-0101- 2 | 17 | 3 | | | | | | |
| GT10-0102- 2 | 17 | 2 | | | | | | |
| H23-RMU-FC-1ICA1 | 20 | 3 | | | | | | |
| H23-RMU-FC-1ICA2 | 30 | 3 | | | | | | |
| H23-RMU-FC-2ICA1 | 20 | 4 | | | | | | |
| H23-RMU-FC-2ICA2 | 30 | 4 | | | | | | |
| H23-RMU-FC-ACT1A | 26 | 2 | | | | | | |
| H23-RMU-FC-ACT2A | 27 | 2 | | | | | | |
| H23-RMU-FC-ACT3A | 28 | 2 | | | | | | |
| H23-RMU-FC-ACT4A | 29 | 2 | | | | | | |
| H23-RMU-FC-PDT1A | 22 | 1 | | | | | | |
| H23-RMU-FC-PDT2A | 23 | 1 | | | | | | |
| H23-RMU-FC-PDT3A | 24 | 1 | | | | | | |
| Containment Isolation | | | | | | Cont Isolation.caf | Appendix B.4.18-2 | Page 32 |

Appendix B.4.19 Passive Containment Cooling System Fault Tree







| Name | Page | Zone | Name | NEDO-33201 Rev 1 | Page | Zone | Name | Page | Zone |
|-------------------|------|------|------|------------------|------|------|------|------|------|
| GB32-0201- 1 | 1 | 1 | | | | | | | |
| GB32-0301- 1 | 1 | 2 | | | | | | | |
| GT15-0002- 1 | 2 | 2 | | | | | | | |
| GT15-0031- 1 | 2 | 3 | | | | | | | |
| GT15-0031- 1 | 3 | 2 | | | | | | | |
| GT15-0032- 1 | 1 | 2 | | | | | | | |
| GT15-0033- _1 | 1 | 2 | | | | | | | |
| GT15TOP | 2 | 2 | | | | | | | |
| T15-HX -CF-PLUG | 2 | 2 | | | | | | | |
| T15-HX -PG-B001A | 3 | 1 | | | | | | | |
| T15-HX -PG-B001B | 3 | 1 | | | | | | | |
| T15-HX -PG-B001C | 3 | 1 | | | | | | | |
| T15-HX -PG-B001D | 3 | 2 | | | | | | | |
| T15-HX -PG-B001E | 3 | 2 | | | | | | | |
| T15-HX -PG-B001F | 3 | 2 | | | | | | | |
| XHOS72H | 1 | 3 | | | | | | | |
| XXX-POL-RP-IC/PCC | 2 | 1 | | | | | | | |
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5 DATA ANALYSIS

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5 DATA ANALYSIS

5.1 INTRODUCTION AND SCOPE

This section is an overview of the reliability data used in the ESBWR PRA.

Generic reliability data are based on ALWR URD (Reference 5-1), which is complemented with, ABWR PRA (Reference 5-2) Database, or other database sources, as necessary, following this order of preference.

The use of data from the references is justified by the fact that these data are representative of components used in previous BWRs. Applicability of the data to the ESBWR components in their specific environments is analyzed and adjustments of the values are performed as necessary.

The data and the methodology for Maintenance and Test Unavailability are based on the use of bounding generic values. In particular, the maintenance unavailability for the diesel-generator ($6.0 \cdot 10^{-3}$) is derived from Appendix A of ALWR URD (Reference 5-1). More details about generic unavailability due to Maintenance and Test is included in the ESBWR PRA Report.

Common Cause Failure (CCF) Data derived are used where available. Examples are diesel-generators, batteries, motor-operated valves, and pumps. The sources used for the estimation of the CCF parameters are NUREG/CR-5497 (Reference 5-7), EPRI TR-100382 (Reference 5-6), and NUREG/CR-5801 (Reference 5-8), in this order of preference.

The methodology described in NUREG/CR-4780 (Reference 5-9) is applied. There are several models to estimate the CCF probabilities and failure rates. The beta factor method is often used because of its simplicity; however, the results are conservative, especially for redundancy numbers higher than two (e.g., three or four). The multiple Greek letter method explicitly considers the contributions of subgroups failures, but its correct estimation is complex according to the explanation given in NUREG/CR-4780 (Reference 5-9). The alpha parameter method is considered the most appropriate for this application, because it considers the contributions of subgroups failures and is easily estimated from the system failure information.

5.2 COMPONENT RELIABILITY DATABASE

The data used for random component failure probabilities are included in Table 5.2-1.

Generic reliability data are based on ALWR URD (Reference 5-1). This source is complemented with ABWR PRA Database (Reference 5-2) and other database sources, as necessary. The order of reference for the data is as listed above.

GE data are used for microprocessor based components and discrete logic components; a detailed evaluation of the data for these components is given in Reference 5-2.

The use of generic data for most of the electrical and mechanical components is justified by the following facts:

- In this phase, the specific types of components that will be purchased are not defined.
- These generic data are representative of components used in previous BWR plants.

Applicability of these generic data to the components operating in the specific ESBWR environment is analyzed and an adjustment of the value is made for a few cases.

The component boundaries considered for the case of the 6.9 kV/13.8 kV pumps exclude the breaker. For the diesel generators, associated auxiliary systems such as the startup air system, internal cooling circuit, etc, are included in the quantified diesel generator failure rates.

Rates for "failure to operate" are provided on a per-hour basis. Rates for "failure to start" are provided on a per-demand basis. The analyst chooses which model to apply for a particular case:

- According to the philosophy of NUREG/CR-2728 (Reference 5-4), for components whose test period (TP) is up to six months ($TP \leq 6m$), the demand failure probability (P) is considered adequate and should be used as stated in the database (Table 5.2-1).

Unavailability of the failure basic event = P

- For components whose test period is from six months up to one year ($6M < TP \leq 1 \text{ year}$), it is suggested that the upper bound on demand failure probability be used as the computational median. In a lognormal distribution, the new mean is the generic mean multiplied by the error factor (EF). (Reference 5-4)

Unavailability of the failure basic event = P x EF

- For test periods higher than one year ($TP > 1 \text{ year}$), a standby failure rate is obtained by the following expression (Reference 5-4):

$$\lambda = 2 \times P/TP_g$$

where TP_g is a generic test period considered as 3 months, assuming that test practices of ESBWR design are similar to components of the generic source.

Assuming a standby model:

$$\text{Unavailability of the failure basic event} = 1 - [1 - \exp(-\lambda \times TP)]/(\lambda \times TP).$$

Table 5.2-1 also includes a column containing Error Factor. The values for those Error Factors are derived from the appropriate database source when available and otherwise from the following sources:

- NUREG/CR-4550 (Reference 5-3), or
- IREP (Reference 5-4), or
- If data are not available, an engineering assessment.

A lognormal distribution is assumed for all data. For component outages for Test/Maintenance, an error factor of 10 is used as suggested in NUREG/CR-4550 (Reference 5-3).

Table 5.2-1

Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|---|---------------------|---------------------|--------------|--------|------------------|
| 1. PUMPS/FANS | | | | | |
| 1.1 Pumps | | | | | |
| 1.1.1 Diesel-Driven, Standby | | | | | |
| 1.1.1.1 Failure to start | $2.0 \cdot 10^{-2}$ | d | 10 | EDP-FS | ALWR |
| 1.1.1.2 Failure to run, given start | $1.0 \cdot 10^{-3}$ | hr | 10 | EDP-FR | ALWR |
| 1.1.2 Motor-Driven, Feedwater | | | | | |
| 1.1.2.1 Failure to start | $2.0 \cdot 10^{-3}$ | d | 10 | MPF-FS | ALWR (all types) |
| 1.1.2.2 Failure to run, given start | $2.5 \cdot 10^{-3}$ | hr | 10 | MPF-FR | ALWR (all types) |
| 1.1.2.3 In test or maintenance | $2.0 \cdot 10^{-3}$ | d | 10 | MPF-TM | NUREG/CR-4550 |
| 1.1.3 Motor-Driven CRD | | | | | |
| 1.1.3.1 Failure to start | $2.4 \cdot 10^{-3}$ | d | 10 | MPC-FS | ALWR |
| 1.1.3.2 Failure to run, given start | $2.4 \cdot 10^{-6}$ | hr | 10 | MPC-FR | ALWR |
| 1.1.4 Aux. Oil Pump | | | | | |
| 1.1.4.1 Failure to start | $2.0 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR (all types) |
| 1.1.4.2 In test or maintenance | $2.0 \cdot 10^{-3}$ | d | 10 | MP_-TM | NUREG/CR-4550 |
| 1.1.5 Condensate/CIRC Pump | | | | | |
| 1.1.5.1 Failure to start | $2.0 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR (all types) |
| 1.1.5.2 Failure to run, given start | $2.5 \cdot 10^{-5}$ | hr | 10 | MP_-FR | ALWR (all types) |
| 1.1.5.3 In test or maintenance | $3.0 \cdot 10^{-3}$ | d | 10 | MP_-TM | ALWR |
| 1.1.5.4 Circuit breaker fails to close | $8.0 \cdot 10^{-4}$ | d | 10 | MP_-FS | NUREG-1816 (10) |
| 1.1.6 FAPCS/LPCI Pump | | | | | |
| 1.1.6.1 Failure to start | $3.0 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR |
| 1.1.6.2 Failure to run | $1.5 \cdot 10^{-4}$ | hr | 10 | MP_-FR | ALWR |
| 1.1.7 RWCU/SDC Pump | | | | | |
| 1.1.7.1 Failure to start | $2.3 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR |
| 1.1.7.2 Failure to run | $1.0 \cdot 10^{-5}$ | hr | 10 | MP_-FR | ALWR |
| 1.1.8 PSWS Pump | | | | | |
| 1.1.8.1 Failure to start | $2.4 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR |
| 1.1.8.2 Failure to run | $3.2 \cdot 10^{-5}$ | hr | 10 | MP_-FR | ALWR |
| 1.1.8.3 In test or maintenance | $2.0 \cdot 10^{-3}$ | d | 10 | MP_-TM | NUREG/CR-4550 |
| 1.1.9 RCCWS, TCCWS Pump | | | | | |
| 1.1.9.2 Failure to start | $1.3 \cdot 10^{-3}$ | d | 10 | MP_-FS | ALWR |
| 1.1.9.2 Failure to run | $5.0 \cdot 10^{-6}$ | hr | 10 | MP_-FR | ALWR |
| 1.1.9.3 In test or maintenance | $2.0 \cdot 10^{-3}$ | d | 10 | MP_-TM | NUREG/CR-4550 |
| 1.2. Fans | | | | | |
| 1.2.1 Motor-Driven-Fan | | | | | |
| 1.2.1.1 Failure to start | $6.0 \cdot 10^{-4}$ | d | 3 | FAN-FS | ALWR |
| 1.2.1.2 Failure to run, given start | $1.0 \cdot 10^{-5}$ | hr | 3 | FAN-FR | ALWR |
| 1.3 Instrument and Air Service Compressor | | | | | |

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|--|---------------------|---------------------|--------------|--------|-------------------|
| 1.3.1 Failure to start | $2.0 \cdot 10^{-2}$ | d | 3 | CMP-FS | ALWR |
| 1.3.2 Failure to run, given start | $1.0 \cdot 10^{-4}$ | hr | 10 | CMP-FR | ALWR |
| 1.3.3 In maintenance | $2.0 \cdot 10^{-3}$ | d | 3 | CMP-TM | NUREG/CR-4550 |
| 2. VALVES/DAMPERS | | | | | |
| 2.1 Valves | | | | | |
| 2.1.1 Air operated | | | | | |
| 2.1.1.1 Failure to open | $2.0 \cdot 10^{-3}$ | d | 3 | ACV-CC | ALWR |
| 2.1.1.2 Failure to close | $2.0 \cdot 10^{-3}$ | d | 3 | ACV-OO | ALWR |
| 2.1.1.3 Spuriously transfer to deenergized position | $1.5 \cdot 10^{-7}$ | hr | 10 | ACV-OC | ALWR |
| 2.1.1.4 Spuriously transfer to energized position | $2.0 \cdot 10^{-8}$ | hr | 30 | ACV-CO | IEEE-Std-500 (4) |
| 2.1.1.5 Failure to regulate/transfer | $2.0 \cdot 10^{-3}$ | d | 3 | ACV-FT | NUREG/CR-4550 |
| 2.1.1.6 In test or maintenance | $8.0 \cdot 10^{-4}$ | d | 10 | ACV-TM | NUREG/CR-4550 |
| 2.1.2 Check | | | | | |
| 2.1.2.1 Failure to open | $2.0 \cdot 10^{-4}$ | d | 3 | UV_-CC | ALWR |
| 2.1.2.2 Failure to close | $1.0 \cdot 10^{-3}$ | d | 3 | UV_-OO | ALWR |
| 2.1.2.3 Fails to remain open or plugs | $2.0 \cdot 10^{-7}$ | hr | 30 | UV_-OC | ALWR |
| 2.1.2.4 In test maintenance | $8.0 \cdot 10^{-4}$ | d | 10 | UV_-TM | NUREG/CR-4550 |
| 2.1.3 Motor-Operated | | | | | |
| 2.1.3.1 Failure to open | $4.0 \cdot 10^{-3}$ | d | 10 | MOV-CC | ALWR |
| 2.1.3.2 Failure to close | $4.0 \cdot 10^{-3}$ | d | 10 | MOV-OO | ALWR |
| 2.1.3.3 Failure to remain open | $1.4 \cdot 10^{-7}$ | hr | 3 | MOV-OC | ALWR |
| 2.1.3.4 Failure to remain closed | $5.0 \cdot 10^{-7}$ | hr | 10 | MOV-CO | NUREG/CR-4550 |
| 2.1.3.5 In maintenance, test or unavailable | $8.0 \cdot 10^{-4}$ | d | 10 | MOV-TM | NUREG/CR-4550 |
| 2.1.3.6 Flow control failure | $2.9 \cdot 10^{-6}$ | hr | 10 | MOV-FC | IEEE-Std-500 |
| 2.1.4 Safety/Relief | | | | | |
| 2.1.4.1 Failure to open | $6.0 \cdot 10^{-3}$ | d | 3 | SRV-CC | ALWR |
| 2.1.4.2 Failure to open (safety mode) | $2.0 \cdot 10^{-5}$ | d | 3 | SRV-CC | ALWR (5) |
| 2.1.5 Solenoid | | | | | |
| 2.1.5.1 Failure to operate (energized.) | $1.0 \cdot 10^{-3}$ | d | 3 | SOV-FE | NUREG/CR-2728 |
| 2.1.5.2 Failure to operate (de-energized.) | $1.0 \cdot 10^{-3}$ | d | 3 | SOV-FD | NUREG/CR-2728 |
| 2.1.6 Squib | | | | | |
| 2.1.6.1 Fails to operate ⁽²⁾ | $3.0 \cdot 10^{-3}$ | d | 3 | SQV-CC | ALWR |
| 2.1.6.2 Opens spuriously | $4.0 \cdot 10^{-7}$ | hr | 30 | SQV-CO | ALWR |
| 2.1.6.3 Fails to operate in extreme environmental conditions (equalizing lines) ⁽²⁾ | $6.0 \cdot 10^{-3}$ | d | 30 | SQV-CC | NUREG/CR-2728 (6) |
| 2.1.7 Vacuum Breaker | | | | | |
| 2.1.7.1 Leakage of the vacuum breaker | $1.0 \cdot 10^{-4}$ | d | 3 | VB_-LK | NUREG/CR-2728 (7) |

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|--|---------------------|----------------------------|---------------------|-------------|----------------------|
| 2.1.7.2 Vacuum breaker fails to close | $1.0 \cdot 10^{-4}$ | d | 3 | VB_-OO | NUREG/CR-2728 (7) |
| 2.1.8 Pressure control | | | | | |
| 2.1.8.1 All modes of failure | $2.9 \cdot 10^{-6}$ | hr | 10 | CPV-FC | IEEE-Std-500 |
| 2.1.9 Manual valve | | | | | |
| 2.1.9.1 Fails to remain closed | $1.0 \cdot 10^{-4}$ | d | 3 | BV_-CO | NUREG/CR-4550 |
| 2.1.9.2 Fails to remain open | $1.0 \cdot 10^{-4}$ | d | 3 | BV_-OC | NUREG/CR-4550 |
| 2.1.9.3 Fails to close | $3.0 \cdot 10^{-8}$ | hr | 3 | BV_-OO | ALWR |
| 3. SWITCHES | | | | | |
| 3.1 Pressure Switch | | | | | |
| 3.1.1 Failure to operate | $2.5 \cdot 10^{-4}$ | d | 3 | PS_-CC | ALWR |
| 3.1.2 Operates spuriously | $1.1 \cdot 10^{-7}$ | hr | 3 | PS_-CO | ALWR |
| 4. OTHER MECHANICAL COMPONENTS | | | | | |
| 4.1 Orifices | | | | | |
| 4.1.1 Failure to remain open (plug) | $6.0 \cdot 10^{-7}$ | hr | 3 | OR_-PG | NUREG/CR-2815 (8) |
| 4.2 Heat exchangers (general use) | | | | | |
| 4.2.1 Fails while operating (leaks, plugs) | $1.0 \cdot 10^{-6}$ | hr | 10 | HX_-LK | ALWR |
| 4.2.2 Heater Pressurizer all failures | $2.2 \cdot 10^{-6}$ | hr | 10 | HX_-PG | IEEE-Std-500 |
| 4.2.3 In maintenance | $2.2 \cdot 10^{-3}$ | d | 10 | HX_-TM | NUREG/CR-4550 |
| 4.2.4 Compressor Cooler fails during operation | $1.0 \cdot 10^{-6}$ | hr | 10 | HX_-PG | ALWR |
| 4.3 Heat exchangers (Isolation Condenser/PCCS) | | | | | |
| 4.3.1 Leaks while pressurized | $1.0 \cdot 10^{-7}$ | hr | 10 | HX_-LK | WASH- 1400 (9) |
| 4.4 Strainer/filter | | | | | |
| 4.4.1 Plugs during operation | $1.0 \cdot 10^{-5}$ | hr | 10 | STR-PG | ALWR |
| 4.5 Tanks | | | | | |
| 4.5.1 Leaks catastrophically | $1.0 \cdot 10^{-7}$ | hr | 30 | TNK-RP | ALWR |
| 4.6 Pool | | | | | |
| 4.6.1 Leaks catastrophically | $3.0 \cdot 10^{-7}$ | d | 10 | POL-RP | Engineering Judgment |
| 4.7 Pipe to Essent. Systems ⁽³⁾ | | | | | |
| 4.7.1 Leaks or plugs | $1.7 \cdot 10^{-7}$ | hr | 10 | PSF-RP | (9) |
| 4.8 Dryer | | | | | |
| 4.8.1 In maintenance | $2.0 \cdot 10^{-5}$ | d | 10 | DRY-TM | IEEE-Std-500 |
| 5. ELECTRICAL COMPONENTS | | | | | |
| 5.1 Circuit breakers | | | | | |
| 5.1.1 Circuit breaker (>6 KV) | | | | | |
| 5.1.1.1 Failure to close | $8.0 \cdot 10^{-4}$ | d | 10 | LCB-OO | NUREG-1816 (10) |
| 5.1.1.2 Failure to open | $8.0 \cdot 10^{-4}$ | d | 10 | LCB-CC | NUREG-1816 (10) |

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|--|---------------------|----------------------------|---------------------|-------------|-------------------|
| 5.1.1.3 Opens spuriously | $6.0 \cdot 10^{-7}$ | hr | 10 | LCB-CO | ALWR |
| 5.1.2 Circuit breakers ($\leq 600V$) | | | | | |
| 5.1.2.1 Failure to close | $8.0 \cdot 10^{-4}$ | d | 10 | MCB-OO | NUREG-1816 (10) |
| 5.1.2.2 Failure to open | $8.0 \cdot 10^{-4}$ | d | 10 | MCB-CC | NUREG-1816 (10) |
| 5.1.2.3 Opens spuriously | $5.0 \cdot 10^{-7}$ | hr | 10 | MCB-CO | ALWR |
| 5.2 Electrical buswork | | | | | |
| 5.2.1 AC | | | | | |
| 5.2.1.1 Fails during operation | $2.0 \cdot 10^{-7}$ | hr | 5 | BAC-LP | ALWR |
| 5.2.1.2 In test or maintenance | $8 \cdot 10^{-6}$ | d | 10 | BAC-TM | NUREG/CR-4550 |
| 5.2.2 DC | | | | | |
| 5.2.2.1 Fails during operation | $2.0 \cdot 10^{-7}$ | hr | 5 | BDC-LP | ALWR |
| 5.2.2.2 In test or maintenance | $8 \cdot 10^{-6}$ | d | 10 | BDC-TM | NUREG/CR-4550 |
| 5.3 Transformer | | | | | |
| 5.3.1 High voltage | | | | | |
| 5.3.1.1 Fails to continue operating | $1.2 \cdot 10^{-6}$ | hr | 3 | XFH-LP | ALWR |
| 5.3.1.2 In test or maintenance | $1.0 \cdot 10^{-4}$ | d | 10 | XFH-TM | IEEE-Std-500 |
| 5.3.2 Medium voltage (6.9 KV to 480 V) | | | | | |
| 5.3.2.1 Fails to continue operating | $7.0 \cdot 10^{-7}$ | hr | 3 | XFM-LP | ALWR |
| 5.3.3 Low voltage (480 V and lower) | | | | | |
| 5.3.3.1 Fails to continue operating | $8.0 \cdot 10^{-7}$ | hr | 3 | XFL-LP | ALWR |
| 5.4 Fuse | | | | | |
| 5.4.1 Opens spuriously | $5.0 \cdot 10^{-7}$ | hr | 10 | XFL-FC | ALWR |
| 5.5 Inverter | | | | | |
| 5.5.1 Failure during operation | $2.0 \cdot 10^{-5}$ | hr | 3 | INV-FC | ALWR |
| 5.6 Pressure transmitter | | | | | |
| 5.6.1 Fails to respond to change | $4.8 \cdot 10^{-7}$ | hr | 10 | PT_-NO | ALWR |
| 5.6.2 Fails high | $3.3 \cdot 10^{-7}$ | hr | 10 | PT_-HI | ALWR |
| 5.6.3 Fails low | $3.3 \cdot 10^{-7}$ | hr | 10 | PT_-LO | ALWR |
| 5.7 Temperature transmitter | | | | | |
| 5.7.1 Fails to respond to change | $3.5 \cdot 10^{-7}$ | hr | 10 | TT_-NO | ALWR |
| 5.7.2 Fails high | $1.9 \cdot 10^{-6}$ | hr | 10 | TT_-HI | ALWR |
| 5.7.3 Fails low | $1.9 \cdot 10^{-6}$ | hr | 10 | TT_-LO | ALWR |
| 5.8 Flow transmitter | | | | | |
| 5.8.1 Fails to respond to change | $4.6 \cdot 10^{-7}$ | hr | 10 | FT_-NO | ALWR |
| 5.8.2 Fails high | $1.7 \cdot 10^{-6}$ | hr | 10 | FT_-HI | ALWR |
| 5.8.3 Fails low | $1.7 \cdot 10^{-6}$ | hr | 10 | FT_-LO | ALWR |
| 5.9 Level transmitter | | | | | |
| 5.9.1 Fails to respond to change | $1.0 \cdot 10^{-6}$ | hr | 10 | LT_-NO | ALWR |
| 5.9.2 Fails high | $5.1 \cdot 10^{-7}$ | hr | 10 | LT_-HI | ALWR |
| 5.9.3 Fails low | $5.1 \cdot 10^{-7}$ | hr | 10 | LT_-LO | ALWR |

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|--|---------------------|----------------------------|---------------------|-------------|-------------------|
| 5.10 Radiation Monitors | | | | | |
| 5.10.1 Fails to respond to change | $1.8 \cdot 10^{-6}$ | hr | 10 | ACT-NO | IEEE-Std-500 |
| 5.10.2 Fails high | $2.2 \cdot 10^{-6}$ | hr | 10 | ACT-HI | IEEE-Std-500 |
| 5.10.3 Fails low | $2.2 \cdot 10^{-6}$ | hr | 10 | ACT-LO | IEEE-Std-500 |
| 5.10.4 APRM Channel fails | $6.8 \cdot 10^{-6}$ | hr | 10 | ACT-FC | NUREG/CR-1740 |
| 5.11 Differential Pressure Transmitter | | | | | |
| 5.11.1 Fails to respond to change | $3.0 \cdot 10^{-6}$ | hr | 10 | PDT-NO | NEDE-22056 |
| 5.11.2 Fails high | $3.0 \cdot 10^{-6}$ | hr | 10 | PDT-HI | NEDE-22056 |
| 5.11.3 Fails low | $3.0 \cdot 10^{-6}$ | hr | 10 | PDT-LO | NEDE-22056 |
| 5.12 Transmission Line fails (per circuit) | $3.0 \cdot 10^{-6}$ | hr | 10 | CBU-FC | NUREG/CR-2728 |
| 5.13 Diesel Generator | | | | | |
| 5.13.1 Fails to start | $1.4 \cdot 10^{-2}$ | d | 3 | DG_-FS | ALWR |
| 5.13.2 Fails to run given start | $2.4 \cdot 10^{-3}$ | hr | 3 | DG_-FR | ALWR |
| 5.13.3 In test or maintenance | $6.0 \cdot 10^{-3}$ | d | 10 | DG_-TM | ALWR |
| 5.14 Battery Power System | | | | | |
| 5.14.1 Fails to provide output on demand | $5.0 \cdot 10^{-4}$ | d | 3 | BT_-LP | ALWR |
| 5.14.2 In test or maintenance | $1.0 \cdot 10^{-3}$ | d | 10 | BT_-TM | NUREG/CR-4550 |
| 5.15 Battery Charger | | | | | |
| 5.15.1 Failure to maintain output | $7.0 \cdot 10^{-6}$ | hr | 3 | BYC-LP | ALWR |
| 5.15.2 In test or maintenance | $3.0 \cdot 10^{-4}$ | d | 10 | BYC-TM | NUREG/CR-4550 |
| 5.16 Diode | | | | | |
| 5.16.1 Fails to operate | $1.4 \cdot 10^{-6}$ | d | 10 | DIO-FC | IEEE-Std-500 |
| 5.17 Rectifier | | | | | |
| 5.17.1 Fails during operation | $7.0 \cdot 10^{-6}$ | hr | 3 | REC-FC | ALWR |
| 5.17.2 In test or maintenance | $3.0 \cdot 10^{-4}$ | d | 10 | REC-TM | NUREG/CR-4550 |
| 5.18 Automatic Selector | | | | | |
| 5.18.1 Fails to transfer | $7.3 \cdot 10^{-7}$ | hr | 10 | SEL-FT | NEDC-30851P-A |
| 5.19 Bypass Unit | | | | | |
| 5.19.1 Fails to function | $5.0 \cdot 10^{-6}$ | hr | 10 | BYP-FC | (13) |
| 6. SOLID STATE COMPONENTS | | | | | |
| 6.1 Microprocessor Based Components | | | | | |
| 6.1.1 TLU and TLU Bypass Control Card (Safety Systems) | | | | | |
| 6.1.1.1 TLU fails to trip or TLU Bypass Logic Card fails to transfer | $9.0 \cdot 10^{-4}$ | | 10 | SLU-FC | (11) |
| 6.1.2 DTM (Safety Systems) | | | | | |
| 6.1.2.2 Fails to trip | $6.0 \cdot 10^{-4}$ | d | 10 | DTM-FC | (11) |
| 6.1.3 DTM/TLU and MUX Interface Unit (Non-Safety Systems) | | | | | |

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|-------------------------------|---------------------|---------------------|--------------|--------|-------------------------------|
| 6.1.3.1 Fails to trip | $9.0 \cdot 10^{-4}$ | d | 10 | DTM-FC | (11) |
| 6.1.4 RMU | | | | | |
| 6.1.4.1 Fails to operate | $5.0 \cdot 10^{-6}$ | hr | 10 | RMU-FC | NUMAC Field Data, B. Simon |
| 6.1.5 EMS | | | | | |
| 6.1.5.1 Fails to function | $1.0 \cdot 10^{-5}$ | hr | 10 | EMS-FC | (11) |
| 6.2 Discrete Logic Components | | | | | |
| 6.2.1 2/4 Voting Logic Card | | | | | |
| 6.2.1.1 All modes failure | $7.8 \cdot 10^{-5}$ | d | 10 | VLU-FC | (11) |
| 6.2.2 1/N Logic Card | | | | | |
| 6.2.2.1 All modes failure | $3.0 \cdot 10^{-4}$ | d | 10 | LOG-FC | (11) |
| 6.3 Relay (electromechanical) | | | | | |
| 6.3.1 Fails to operate | $1.0 \cdot 10^{-4}$ | d | 10 | RE_-FD | ALWR |

Notes: (1) Base, d = demand, hr = hour

- (2) For these components, periods between demands are usually irregular and long. The failure rates therefore adequately reflect standby failures, and an hourly rate is not estimated.
- (3) A piping section is defined as a section of pipe between major discontinuities (i.e., pumps, valves, elbows, bends...) up to 3 m (10 ft) in length. The fraction of this rate corresponding to leakage severe enough to constitute a rupture should be taken as 5 %. The 5 % value is consistent with WASH-1400. Of this 5 %, 10 % should be taken to be essentially a complete break; 30 % should be taken to be a large rupture; and the remaining 60 % a small rupture. This is the breakdown used in the Oconee PRA, which NRC reviewers found to be acceptable.
- (4) For spurious transfer to the energized position, the value for manual valves is recommended.
- (5) Data for Safety-Relief Valves (other than PWR Primary nor BWR, actuation mode).
- (6) To account for those extreme environmental conditions a failure probability double that for other squib valves is used.
- (7) The failure data, obtained from NUREG/CR-2728 for a generic test interval, is $1.25\text{E-}5/\text{d}$; considering a PRA correction criteria, the failure probability assumed is $1.0\text{E-}4$.
- (8) For continuously flushed lines.
- (9) Considering a failure rate of $8.5\text{E-}9/\text{h}$, obtained from WASH-1400 for a pipe with a diameter less or equal to 3", and 248 pipes per IC, and taking into account that the 5% of those failures can cause problems (according to the WASH-1400), the failure rate estimated is $1\text{E-}7/\text{h}$.
- (10) NUREG-1816, Table C.2, recommends $8\text{E-}4/\text{d}$ for failure to open/close of circuit breakers.
- (11) a. Two types of failure rates are used: (1) $5\text{E-}6/\text{h}$ for component consisting of one or more circuit boards, like DTMs, RMUs, OLUs, VLUs, SLUs, TLUs, etc., based on a required MTBF of 200,000 hours, and (2) $1\text{E-}5/\text{h}$ for the overall multiplexing system (EMS), based on a required MTBF of 100,000 hours.
b. For obtaining the failure probability on demand, it is assumed that 95% of the component failures will be detected by self-testing; the LMU, DTM, and TLU cards are self-tested every 30 minutes (Tself-test). The remaining 5% will be detected only during surveillance tests performed quarterly (Ttest = 2190 hr). In both cases, it is assuming a MTTR of five hours. The expression used is:

$$P = \lambda \times [0.95 \times ((T_{\text{self-test}}/2 + \text{MTTR}) + 0.05 \times (T_{\text{test}}/2 + \text{MTTR})], \text{ where: } \lambda \text{ is the failure rate}$$

Table 5.2-1
Database

| Component and Failure Mode | Mean | Base ⁽¹⁾ | Error Factor | Code | References |
|--|-------------|----------------------------|---------------------|-------------|-------------------|
| (12) The Failure rate used is 1.3 E-06/hr obtained from the document "Printed circuit Card Failure Analysis Report No. 9 GE Nuclear Energy, December 16, 1988", and the estimation of the probability is based on the part b of Note (11) above. | | | | | |
| (13) Joint Study Report of SSLC Reliability Analysis, No. IF-R-389 and MIL-HDBK-217C | | | | | |

5.3 COMMON CAUSE FAILURE ANALYSIS

This section contains the results of the analysis performed on the potential common cause failures (CCF) within the systems of the ESBWR plant. The analysis is performed to identify potential CCFs within each system and quantify their probabilities.

5.3.1 Scope

The scope of this section is to consider the component failure types which are within the event trees of each system and represent groups of main components with a redundant function.

Common cause failures are not postulated among components of different systems. Therefore, common cause failures are considered among equal active equipment and with similar functions to provide redundancy.

The types of components where common cause failures have been analyzed are:

- Diesel Generators (DG_)
- Pumps (MP_, MPC, MPF)
- Explosive Valves (SQV)
- Motor Operated Valves (MOV)
- Solenoid Operated Valves (SOV)
- Air Operated Valves (ACV)
- Safety/Relief Valves (SRV)
- Pressure Control Valves (CPV)
- Check Valves (UV_)
- Vacuum Breakers (VB_)
- Batteries (BT_)
- Bypass Units (BYP)
- Compressors (CMP)
- Fans (FAN)
- Heat exchangers (HX_)
- Strainers (STR)
- Circuit Breakers (LCB, MCB)
- Temperature, pressure, level, differential pressure, flow, radiation transmitters (TT_, PT_, LT_, PDT_, FT_, ACT)
- Digital Trip Modules (DTM)
- Essential Multiplexing Systems (EMS)
- Discrete Logic Cards (LOG)

- Remote Multiplexing Units (RMU)
- Safety System Logic Unit (SLU)
- Voting Logic Unit (VLU)
- Orifices (OR_)

The systems where common cause failures are postulated are the following:

- B21 Reactor Depressurization System
- B32 Isolation Condenser System
- C12 Control Rod Drive System
- C41 Standby Liquid Control System
- C62 Instrument and Control System
- C74 Safety Logic and Control System
- E50 Gravity Driven Cooling System
- G21 Fuel and Auxiliary Pool Cooling System
- G31 Reactor Water Cleanup System
- N21 Feedwater and Condensate System
- N71 Circulating Water System
- P21 Reactor Component Cooling Water System
- P41 Plant Service Water System
- P51 Service Air System
- P52 Instrument Air System
- P54 High Pressure Nitrogen Supply System
- R10 13.8 kV AC Power Distribution System
- R11 6.9 kV AC Power Distribution System
- R12 480 V AC Power Distribution System
- R16 250 V DC Power Distribution System
- T15 Passive Containment Cooling System
- U43 Fire Protection System

5.3.2 METHODOLOGY

The systematic method described in NUREG/CR-4780 (Reference 5-9) is used. The common cause failure databases from NUREG/CR-5497 (Reference 5-7), EPRI TR-100382 (Reference 5-6), and from NUREG/CR-5801 (Reference 5-8) are used to create a database that provides the common cause factors for use in this analysis.

There are several models for estimating CCF probabilities and failures rates. The beta factor method is often used because of its simplicity; however, the results are conservative, especially for redundancy numbers higher than two (e.g., three or four). The multiple Greek letter method explicitly considers the contributions of subgroup failures, but its correct estimation is complex according to the explanation given in NUREG/CR-4780 (Reference 5-9). The alpha factor method is considered the most appropriate for this application, because it considers the contributions of subgroup failures and is easily estimated from system failure information, as described below.

5.3.3 ALPHA FACTOR METHOD

The following probabilities are defined considering a common cause group of m components.

Q_k = probability or failure rate, depending on the model being in demand or based on time respectively, of specific k components due to a common cause event where $1 \leq k \leq m$.

Q_t = probability or total failure rate of a specific component which is obtained from the Q_k values by the following expression:

$$Q_t = \sum_{k=1}^m \binom{m-1}{k-1} Q_k$$

where the binomial term:

$$\binom{m-1}{k-1} = \frac{(m-1)!}{(m-k)!(k-1)!}$$

represents the number of different combinations in which a specific component can fail together with other $(k-1)$ components in a group of similar (m) components.

This model, which uses Q_k parameters in the way defined above to calculate probabilities or system failure rates, is called the basic parameter model. Given that, in general, the necessary information to estimate the Q_k directly is not available, therefore, other models have been developed that require additional hypotheses. These other models simplify the estimation of parameters from historical information.

The first hypothesis made is one of symmetry, which postulates that the probability of CCF in any k number of components belonging to a group of m components only depends on the k number and not on which components fail.

The alpha factor model is multiparametric and the parameters are calculated directly from system failure information. The fraction of the total frequency of the failure events, which take place in the system affecting any of the k components in a group of m components, is defined as $\alpha_k(m)$. In terms of probabilities of basic events, the parameters are defined as:

$$\alpha_k^{(m)} = \frac{\binom{m}{k} Q_k^{(m)}}{\sum_{k=1}^m \binom{m}{k} Q_k^{(m)}}$$

Using this last equation and the one previously written for Q_t , the probabilities or failure rates of basic events can be expressed based on Q_t and the alpha factors.

$$Q_k^{(m)} = \frac{k \alpha_k^{(m)}}{\binom{m-1}{k-1} \alpha_t^{(m)}} Q_t = \frac{m}{\binom{m}{k}} \frac{\alpha_k^{(m)}}{\alpha_t^{(m)}} Q_t \quad (1)$$

where

$$\alpha_t^{(m)} = \sum_{k=1}^m K \alpha_k^{(m)}$$

To calculate Q_k not only is α_k necessary, but also all the alphas are necessary, from $k=1$ to $k=m$, according to the $\alpha_t(m)$ definition.

The CCF common cause factors for non-staggered and staggered tests respectively are the following:

$$Q_k^{(m)} = \frac{k \alpha_k^{(m)}}{\binom{m-1}{k-1} \alpha_t^{(m)}} Q_t = f_{cc} Q_t \quad Q_k^{(m)} = \frac{\alpha_k^{(m)}}{\binom{m-1}{k-1}} Q_t = f_{cc} Q_t$$

Non- staggered tests

Staggered tests

5.3.4 GENERIC SOURCES

Generic data sources provide alpha factors for most basic events. Generic alpha factors are used for basic events for which no information is found.

The sources used are:

- (1) NUREG/CR-5497 (Reference 5-7).

The α factors are taken directly from this source. If information is not available here, the data sources listed below are used.

⁽¹⁾ This expression applies to a "non-staggered" case.

(2) EPRI TR-100382 (Reference 5-6).

An update of the 1985 EPRI database of common cause failures, published as EPRI NP-3967. This database is specifically common cause failures and the most complete regarding analyzed information. This study includes failure events that have taken place in commercial nuclear power plants in the USA.

This document dedicates one section to each type of component, where it describes, firstly, the physical limits of the component, e.g. identifying which subcomponents would be included in the consideration and which would be left out.

EPRI then presents the data sheets, which summarize the common cause events that have occurred.

(3) NUREG/CR-5801 (Reference 5-8).

When information in the EPRI database is not available and it is not possible to assimilate a component within those defined in the previous sources (NUREG/CR-5497 or EPRI TR-100382), NUREG/CR-5801 (Reference 5-8) is used, because it advises the use of generic alpha parameters for cases where information is not available in the databases (see Section 5.5.2, Reference 5-8).

NUREG/CR-5801 (Reference 5-8) recommends that the α_2 , α_3 and α_4 factors above are divided by two when the rate of common cause faults in operation is determined.

These values are slightly higher, apart from some cases, which are of the same order as those obtained by the alpha parameter method based on the analysis of the EPRI and NUREG/CR-5497 databases. Because it is not usually practical to obtain information for redundancies higher than four, in these situations, values corresponding to four redundant components are used. This may be conservative.

5.3.5 CCF ESTIMATIONS

In Table 5.3-1 the CCF basic events are identified with a description, their probabilities and failure rates (Q_k), CCF factor (fcc), and total probability (Q_t). Refer to Table 5.2-1 for the data source.

For squib valves, periods between demands are unusually irregular and long. Therefore, the demand failure probabilities are adequate for use, and an hourly rate is not estimated, (i.e., $Q_t = 3.00E-3$).

For a software failure, a common cause fault is assumed in the microprocessor programming of an added value of $1.00E-05$ in the various common cause faults that affect the DTM: C62-DTM-CF-DIDALL, C62-DTM-CF-N1EALL, C74-DTM-CF-ALL, C74-DTM-CF-ATWSALL, and C74-DTM-CF-RTIFALL.

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|------------------------|---|-------------------------|------------------|----------------|
| B21-ACV-CF-MSIVCLOSE | CCF OF AOV (MSIV) TO CLOSE | 7.26E-03 | 2.00E-03 | 1.45E-05 |
| B21-FT_-CF-MSLA | CCF 3/4 FLOW TRANSMITTERS MSL "A" | 3.00E-03 | 1.10E-05 | 3.31E-08 |
| B21-FT_-CF-MSLB | CCF 3/4 FLOW TRANSMITTERS MSL "B" | 3.00E-03 | 1.10E-05 | 3.31E-08 |
| B21-FT_-CF-MSLC | CCF 3/4 FLOW TRANSMITTERS MSL "C" | 3.00E-03 | 1.10E-05 | 3.31E-08 |
| B21-FT_-CF-MSLD | CCF 3/4 FLOW TRANSMITTERS MSL "D" | 3.00E-03 | 1.10E-05 | 3.31E-08 |
| B21-LT_-CF-ATWS | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 1A/B/C/D | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-DPSWR | CCF OF DPS WR LEVEL TRANSMITTERS | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-L8NO | CCF LEVEL TRANSMITTERS L8 (FAILURE TO RESPOND TO CHANGE) | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-L9SPU | SPURIOUS L9 SIGNAL CCF | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-L9SPU1 | SPURIOUS L9 SIGNAL CCF | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-N001A/B/C/D | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 10A/B/C/D | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-N001ABCD | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 1A/B/C/D | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-N001ABCDLOW | CCF LEVEL TRANSMITTERS (L2) FAIL LOW | 5.00E-03 | 1.22E-05 | 6.12E-08 |
| B21-LT_-CF-N010ABCD | CCF OF DIVERSIFIED LEVEL 1 & 2 TRANSM. 10A/B/C/D | 5.00E-03 | 2.40E-05 | 1.20E-07 |
| B21-LT_-CF-N012ABCD | CCF 3/4 LEVEL TRANSMITTERS N012A/B/C/D | 3.00E-03 | 2.40E-05 | 7.20E-08 |
| B21-OR_-CF-N012ABCD | CCF 3/4 ORIFICES LINES 12 A/B/C/D (PLUG) | 3.00E-03 | 1.44E-05 | 4.32E-08 |
| B21-PT_-CF-02ALL | CCF 3 OUT OF 4 RPV PRESSURE TRANSMITTERS | 3.00E-03 | 1.15E-05 | 3.46E-08 |
| B21-PT_-CF-02ALLNO | CCF OF RPV PRESSURE TRANSMITTERS | 3.00E-03 | 1.15E-05 | 3.46E-08 |
| B21-PT_-CF-02HIGH | CCF OF PRESSURE RPV TRANSMITTERS HIGH | 5.00E-03 | 7.92E-06 | 3.96E-08 |
| B21-PT_-CF-ATWS | CCF 3 OUT OF 4 RPV PRESSURE TRANSMITTERS | 3.00E-03 | 1.15E-05 | 3.46E-08 |
| B21-PT_-CF-MSL | CCF 3/4 PRESSURE TRANSMITTERS MSL | 3.00E-03 | 1.15E-05 | 3.46E-08 |
| B21-SQV-CF-DPVOPEN | CCF OF DPV'S TO OPEN | 5.00E-03 | 3.00E-03 | 1.50E-05 |
| B21-SQV-CF-F004AB | CCF OF DPV A AND B TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004ABC | CCF OF DPV A,B & C TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ABD | CCF OF DPV A,B & D TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ABE | CCF OF DPV A,B & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ABF | CCF OF DPV A,B & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ABG | CCF OF DPV A,B & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ABH | CCF OF DPV A,B & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AC | CCF OF DPV A AND C TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004ACD | CCF OF DPV A,C & D TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ACE | CCF OF DPV A,C & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ACF | CCF OF DPV A,C & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|--------------------|----------------------------|-------------------------|------------------|----------------|
| B21-SQV-CF-F004ACG | CCF OF DPV A,C & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ACH | CCF OF DPV A,C & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AD | CCF OF DPV A AND D TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004ADE | CCF OF DPV A,D & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ADF | CCF OF DPV A,D & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ADG | CCF OF DPV A,D & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004ADH | CCF OF DPV A,D & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AE | CCF OF DPV A AND E TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004AEF | CCF OF DPV A,E & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AEG | CCF OF DPV A,E & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AEH | CCF OF DPV A,E & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AF | CCF OF DPV A AND F TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004AFG | CCF OF DPV A,F & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AFH | CCF OF DPV A,F & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AG | CCF OF DPV A AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004AGH | CCF OF DPV A,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004AH | CCF OF DPV A AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BC | CCF OF DPV B AND C TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BCD | CCF OF DPV B,C & D TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BCE | CCF OF DPV B,C & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BCF | CCF OF DPV B,C & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BCG | CCF OF DPV B,C & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BCH | CCF OF DPV B,C & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BD | CCF OF DPV B AND D TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BDE | CCF OF DPV B,D & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BDF | CCF OF DPV B,D & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BDG | CCF OF DPV B,D & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BDH | CCF OF DPV B,D & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BE | CCF OF DPV B AND E TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BEF | CCF OF DPV B,E & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BEG | CCF OF DPV B,E & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BEH | CCF OF DPV B,E & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BF | CCF OF DPV B AND F TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BFG | CCF OF DPV B,F & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BFH | CCF OF DPV B,F & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BG | CCF OF DPV B AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004BGH | CCF OF DPV B,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004BH | CCF OF DPV B AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004CD | CCF OF DPV C AND D TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004CDE | CCF OF DPV C,D & E TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CDF | CCF OF DPV C,D & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CDG | CCF OF DPV C,D & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CDH | CCF OF DPV C,D & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CE | CCF OF DPV C AND E TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|--------------------|--|-------------------------|------------------|----------------|
| B21-SQV-CF-F004CEF | CCF OF DPV C,E & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CEG | CCF OF DPV C,E & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CEH | CCF OF DPV C,E & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CF | CCF OF DPV C AND F TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004CFG | CCF OF DPV C,F & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CFH | CCF OF DPV C,F & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CG | CCF OF DPV C AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004CGH | CCF OF DPV C,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004CH | CCF OF DPV C AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004DE | CCF OF DPV D AND E TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004DEF | CCF OF DPV D,E & F TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DEG | CCF OF DPV D,E & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DEH | CCF OF DPV D,E & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DF | CCF OF DPV D AND F TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004DFG | CCF OF DPV D,F & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DFH | CCF OF DPV D,F & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DG | CCF OF DPV D AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004DGH | CCF OF DPV D,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004DH | CCF OF DPV D AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004EF | CCF OF DPV E AND F TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004EFG | CCF OF DPV E,F & G TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004EFH | CCF OF DPV E,F & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004EG | CCF OF DPV E AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004EGH | CCF OF DPV E,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004EH | CCF OF DPV E AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004FG | CCF OF DPV F AND G TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004FGH | CCF OF DPV F,G & H TO OPEN | 3.00E-03 | 3.00E-03 | 9.00E-06 |
| B21-SQV-CF-F004FH | CCF OF DPV F AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SQV-CF-F004GH | CCF OF DPV G AND H TO OPEN | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| B21-SRV-CF-9OPEN | CCF TO OPEN 9 SRVs | 5.28E-03 | 3.60E-02 | 1.90E-04 |
| B21-UV_-CF-102/3A | CCF OF CHECK VALVES F103A AND F102A TO CLOSE | 5.00E-02 | 8.00E-03 | 4.00E-04 |
| B21-UV_-CF-102/3B | CCF OF CHECK VALVES F103B AND F102B TO CLOSE | 5.00E-02 | 8.00E-03 | 4.00E-04 |
| B32-ACV-CF-2ICABCD | CCF TO OPEN 2/4 ACV VALVES TRAINS A,B,C,D | 7.74E-03 | 2.00E-03 | 1.55E-05 |
| B32-LT_-CF-LT72HA | CCF 3 OUT OF FOUR A LEVEL TRANSMITTERS | 3.00E-03 | 2.40E-05 | 7.20E-08 |
| B32-LT_-CF-LT72HB | CCF 3 OUT OF FOUR B LEVEL TRANSMITTERS | 3.00E-03 | 2.40E-05 | 7.20E-08 |
| B32-MOV-CF-2ICABCD | CCF TO OPEN 2/4 MOV VALVES TRAINS A,B,C,D | 2.02E-03 | 4.00E-03 | 8.08E-06 |
| B32-MOV-CF-F005A/B | CCF TO OPERATE BETWEEN F005 A & B | 3.60E-03 | 4.00E-03 | 1.44E-05 |
| B32-MOV-CF-F005A/C | CCF TO OPERATE BETWEEN F005 A & C | 3.60E-03 | 4.00E-03 | 1.44E-05 |
| B32-MOV-CF-F005A/D | CCF TO OPERATE BETWEEN F005 A & D | 3.60E-03 | 4.00E-03 | 1.44E-05 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|-----------------------|---|-------------------------|------------------|----------------|
| B32-MOV-CF-F005B/C | CCF TO OPERATE BETWEEN F005 B & C | 3.60E-03 | 4.00E-03 | 1.44E-05 |
| B32-MOV-CF-F005B/D | CCF TO OPERATE BETWEEN F005 B & D | 3.60E-03 | 4.00E-03 | 1.44E-05 |
| B32-MOV-CF-F005C/D | CCF TO OPERATE BETWEEN F005 C & D | 3.60E-03 | 4.00E-03 | 1.44E-05 |
| B32-MOV-CF-F72HADOPEN | CCF VALVES F72HA AND D TO OPEN | 5.00E-02 | 4.00E-03 | 2.00E-04 |
| B32-MOV-CF-F72HBCOPEN | CCF VALVES F72HB AND C TO OPEN | 5.00E-02 | 4.00E-03 | 2.00E-04 |
| B32-PDT-CF-3ICAHIGH | CCF 3/4 PDT'S ISOLATION CONDENSER A SPURIOUS ACTUATION | 3.00E-03 | 3.24E-03 | 9.72E-06 |
| B32-PDT-CF-3ICBHIGH | CCF 3/4 PDT'S ISOLATION CONDENSER B SPURIOUS ACTUATION | 3.00E-03 | 3.24E-03 | 9.72E-06 |
| B32-PDT-CF-3ICC HIGH | CCF 3/4 PDT'S ISOLATION CONDENSER C SPURIOUS ACTUATION | 3.00E-03 | 3.24E-03 | 9.72E-06 |
| B32-PDT-CF-3ICD HIGH | CCF 3/4 PDT'S ISOLATION CONDENSER D SPURIOUS ACTUATION | 3.00E-03 | 3.24E-03 | 9.72E-06 |
| B32-PDT-CF-ICA | CCF OF PRESSURE DIFF. TRANSMITTERS IN IC-A NO OUTPUT SIGNAL | 5.00E-03 | 3.24E-03 | 1.62E-05 |
| C12-MCB-CF-CLOSE | CIRCUIT BREAKER CCF TO CLOSE | 9.29E-02 | 1.00E-03 | 9.29E-05 |
| C12-MOV-CF-OPEN | CCF MOV TO OPEN | 4.44E-02 | 4.00E-03 | 1.78E-04 |
| C12-MP_-CS-C001A/BOIL | CCF AUX. OIL PUMPS TO START | 8.54E-02 | 2.40E-03 | 2.05E-04 |
| C12-MPC-CR-C001AB | CCF OF CRD PUMPS TO RUN | 6.21E-02 | 5.76E-05 | 3.57E-06 |
| C12-MPC-CS-C001A/B | CCF PUMPS TO START | 8.54E-02 | 2.40E-03 | 2.05E-04 |
| C12-PT_-CF-N001A/BLOW | CCF PRESSURE TRANSMITTERS IN SUCTION LINES FAIL LOW | 5.00E-03 | 7.92E-06 | 3.96E-08 |
| C21-FT_-CF-N001ABCD A | CCF 3/4 FLOW TRANSMITTERS RWCU TRAIN A | 3.00E-03 | 1.10E-05 | 3.31E-08 |
| C21-TT_-CF-N003ABCD | CCF 3/4 STEAM TUNNEL TEMPERATURE TRANSMITTERS | 3.00E-03 | 8.40E-06 | 2.52E-08 |
| C41-LT_-CF-N001TALOW | CCF 2 OUT OF 3 OF LEVEL TRANSMITTERS LOW | 2.00E-02 | 1.22E-05 | 2.45E-07 |
| C41-LT_-CF-N001TBLOW | CCF 2 OUT OF 3 LEVEL TRANSMITTERS LOW | 2.00E-02 | 1.22E-05 | 2.45E-07 |
| C41-SQV-CF-F003AC | CCF TO OPERATE OF SQUIB VALVES ON SLCS-A | 5.00E-02 | 3.00E-03 | 1.50E-04 |
| C41-SQV-CF-F003BD | CCF TO OPERATE OF SQUIB VALVES ON SLCS-B | 5.00E-02 | 3.00E-03 | 1.50E-04 |
| C51-ACT-CF-1PRM | CCF APRM NEUTRON CHANNELS | 5.00E-03 | 5.96E-02 | 2.98E-04 |
| C51-ACT-CF-APRMSTUCK | CCF APRM DETECTORS STUCK AT POWER LEVEL | 5.00E-03 | 4.32E-05 | 2.16E-07 |
| C51-ACT-CF-SRNM | CCF OF SRNM CORE FLUX CHANNELS | 5.00E-03 | 5.96E-02 | 2.98E-04 |
| C51-VLU-CF-1PRM | PRNM DIV I 2/4 MODULES FAILS | 1.20E-02 | 7.80E-05 | 9.36E-07 |
| C51-VLU-CF-2PRM | PRNM DIV II 2/4 MODULES FAILS | 1.20E-02 | 7.80E-05 | 9.36E-07 |
| C51-VLU-CF-APRM | CCF 2/4 MODULES APRM | 1.20E-02 | 7.80E-05 | 9.36E-07 |
| C51-VLU-CF-SRNM | CCF 3/4 OF SRNM 2/4 VLU | 1.20E-02 | 6.24E-04 | 7.49E-06 |
| C62-BYP-CF-N1EALL | CCF OF BYPASS UNITS (N1E) | 5.00E-02 | 3.00E-04 | 1.50E-05 |
| C62-DTM-CF-DIDALL | COMMON CAUSE FAILURE 3/4 DTM DID LOGIC | 5.00E-02 | 9.00E-04 | 5.50E-05 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|-----------------------|--|-------------------------|------------------|----------------|
| C62-DTM-CF-NIEALL | CCF OF DIGITAL TRIP MODULES NO 1E | 5.00E-02 | 9.00E-04 | 5.50E-05 |
| C62-VLU-CF-DIDALL | CCF OF VOTER LOGIC UNITS | 5.00E-02 | 6.24E-04 | 3.12E-05 |
| C62-VLU-CF-NIEALL | CCF OF VOTER LOGIC UNITS | 5.00E-02 | 6.24E-04 | 3.12E-05 |
| C72-BYP-CF-DPSALL | CCF OF BYPASS UNITS | 5.00E-03 | 3.00E-04 | 1.50E-06 |
| C72-DTM-CF-DPSALL | CCF 3/4 DTM OF DPS DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.20E-06 |
| C72-VLU-CF-DPSALL | CCF OF VOTER LOGIC UNITS | 5.00E-03 | 6.24E-04 | 3.12E-06 |
| C74-BYP-CF-ALL | CCF OF BYPASS UNITS | 5.00E-03 | 3.00E-04 | 1.50E-06 |
| C74-BYP-CF-DIDALL | CCF OF BYPASS UNITS (DID) | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| C74-DTM-CF-ALL | CCF OF ¼ DTM OF SSLC DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.20E-05 |
| C74-DTM-CF-ATWSALL | CCF 3 OUT OF 4 DIGITAL TRIP MODULES | 5.00E-03 | 6.00E-04 | 1.30E-05 |
| C74-DTM-CF-RTIFALL | CCF 3/4 DTM OF SSLC DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| C74-SLU-CF-DLTD | CCF's TO OPERATE OF DISCRETE LOGIC TIME DELAYS | 1.20E-02 | 9.00E-04 | 1.08E-05 |
| C74-SLU-CF-OLU | CCF 3/4 OLU IN SSLC/RTIF DIV 1/2/3/4 | 3.00E-03 | 9.00E-04 | 2.70E-06 |
| C74-SLU-CF-TLU | CCF 3/4 TLU IN SSLC/RTIF DIV 1/2/3/4 | 3.00E-03 | 9.00E-04 | 2.70E-06 |
| C74-VLU-CF-ALL | CCF OF VOTER LOGIC UNITS | 5.00E-03 | 6.24E-04 | 3.12E-06 |
| C74-VLU-CF-ATWS | CCF 3/4 VOTING LOGIC UNIT ATWS DIVISIONS | 5.00E-03 | 6.24E-04 | 3.12E-06 |
| C74-VLU-CF-LD&IS | CCF VOTING LOGIC CARDS 2/4 LD&IS | 5.00E-03 | 6.24E-04 | 3.12E-06 |
| C74-VLU-CF-SLCS | CCF 2/2 VOTING CARD SLCS LOGIC | 1.20E-02 | 6.24E-04 | 7.49E-06 |
| D11-ACT-CF-3ICAHIGH | CCF 3/4 RADIATION MONITORS IC-A SPURIOUS ACTUATION | 3.00E-03 | 5.28E-05 | 1.58E-07 |
| D11-ACT-CF-3ICBHIGH | CCF 3/4 RADIATION MONITORS IC-B SPURIOUS ACTUATION | 3.00E-03 | 5.28E-05 | 1.58E-07 |
| D11-ACT-CF-3ICC HIGH | CCF 3/4 RADIATION MONITORS IC-C SPURIOUS ACTUATION | 3.00E-03 | 5.28E-05 | 1.58E-07 |
| D11-ACT-CF-3ICD HIGH | CCF 3/4 RADIATION MONITORS IC-D SPURIOUS ACTUATION | 3.00E-03 | 5.28E-05 | 1.58E-07 |
| D11-ACT-CF-ICA | CCF OF RADIATION MONITORS IN IC-A POOL (NO OUTPUT) | 5.00E-03 | 1.94E-03 | 9.72E-06 |
| E50-LT_-CF-N005ABCLOW | CCF 2/3 LEVEL TRANSMITTERS E50-N005A/B/C LOW | 2.00E-02 | 1.22E-05 | 2.45E-07 |
| E50-OR_-CF-4EQUA | CCF OF 4 ORIFICES EQUALIZING LINES (PLUG) | 3.00E-03 | 1.44E-05 | 4.32E-08 |
| E50-OR_-CF-4PLUG | CCF OF 4 OR MORE ORIFICES TO PLUG | 5.00E-03 | 1.44E-05 | 7.20E-08 |
| E50-OR_-CF-7PLUG | CCF OF 7 ORIFICES TO PLUG | 5.00E-03 | 1.44E-05 | 7.20E-08 |
| E50-OR_-CF-D001AE | CCF 2/2 ORIFICES LINES A AND E (PLUG) | 5.00E-02 | 1.44E-05 | 7.20E-07 |
| E50-OR_-CF-D001BCFG | CCF 4/4 ORIFICES LINES B, C, F AND G (PLUG) | 5.00E-03 | 1.44E-05 | 7.20E-08 |
| E50-OR_-CF-D001DH | CCF 2/2 ORIFICES LINES D AND H (PLUG) | 5.00E-02 | 1.44E-05 | 7.20E-07 |
| E50-OR_-CF-PLUGALL | CCF OF ALL ORIFICES TO PLUG | 5.00E-03 | 1.44E-05 | 7.20E-08 |
| E50-SQV-CF-4OPEN | CCF OF 4 OR MORE SQUIB VALVES TO OPEN | 5.00E-03 | 3.00E-03 | 1.50E-05 |
| E50-SQV-CF-EQALLOPEN | CCF OF ALL 4 SQUIB VALVES TO | 5.00E-03 | 6.00E-03 | 3.00E-05 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|---------------------------|--|-------------------------|------------------|----------------|
| | OPEN | | | |
| E50-SQV-CF-F002A/2E | CCF OF SQUIB VALVES F002A/ F002E | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| E50-SQV-CF-F002B/2C/2F/2G | CCF OF SQUIB VALVES F002B/ F002C/ F002F/ F002G | 5.00E-03 | 3.00E-03 | 1.50E-05 |
| E50-SQV-CF-F002D/2H | CCF OF SQUIB VALVES F002D/ F002H | 1.20E-02 | 3.00E-03 | 3.60E-05 |
| E50-SQV-CF-GDCS7OPEN | CCF OF 7 SQUIB VALVES IN GDCS LINES TO OPEN | 5.00E-03 | 3.00E-03 | 1.50E-05 |
| E50-SQV-CF-OPENALL | CCF OF ALL SQUIB VALVES TO OPEN | 5.00E-03 | 3.00E-03 | 1.50E-05 |
| E50-STR-CF-SPPLUG | CCF FILTER/STRAINER IN PSP TO PLUG | 3.48E-02 | 1.08E-02 | 3.75E-04 |
| G21-ACV-CF-SUCTION | CCF OPEN ACV SUCTION PUMPS LINE | 6.66E-03 | 2.00E-03 | 1.33E-05 |
| G21-FT_-CF-N014A/BLOW | CCF DISCHARGE FLOW TRANSMITTERS | 5.00E-03 | 1.66E-04 | 8.28E-07 |
| G21-HX_-CF-PLUG/LEAK | CCF H/Xs TO PLUG/LEAK | 4.98E-02 | 2.40E-05 | 1.19E-06 |
| G21-MCB-CF-C001A/BCLOSE | CCF CIRCUIT BREAKERS TO CLOSE | 9.29E-02 | 1.00E-03 | 9.29E-05 |
| G21-MOV-CF-CLOSEA | CCF TO CLOSE MOV TRAIN A | 1.51E-02 | 4.00E-03 | 6.06E-05 |
| G21-MOV-CF-OPENA/B | CCF TO OPEN MOV TRAINS A AND B | 2.16E-03 | 4.00E-03 | 8.65E-06 |
| G21-MP_-CR-C001A/B | CCF PUMPS TO RUN | 6.21E-02 | 3.60E-03 | 2.23E-04 |
| G21-MP_-CS-C001A/B | CCF PUMPS TO START | 8.54E-02 | 3.00E-03 | 2.56E-04 |
| G21-PT_-CF-N002A/BLOW | CCF SUCTION PRESSURE TRANSMITTERS | 5.00E-02 | 1.19E-04 | 5.94E-06 |
| G21-STR-CF-SPPLUG | CCF FILTER/STRAINER IN PSP TO PLUG | 9.34E-02 | 1.08E-02 | 1.01E-03 |
| G31-ACV-CF-DEENERGA/B | AOV/NOV SPURIOUS TRANSF. TO DEENERG. POSITION | 2.25E-01 | 1.62E-04 | 3.64E-05 |
| G31-FT_-CF-N002ABLOW | CCF SUCTION FLOW TRANSMITTERS FAIL LOW | 5.00E-02 | 6.12E-04 | 3.06E-05 |
| G31-MCB-CF-CLOSE | CCF CIRCUIT BREAKER TO CLOSE | 7.62E-02 | 1.00E-03 | 7.62E-05 |
| G31-MOV-CF-F018/62/63A | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018A, F062A AND F063A TO CLOSE | 1.51E-02 | 4.00E-03 | 6.06E-05 |
| G31-MOV-CF-F018/62/63B | CCF 3 OUT OF 3 MOTOR OPERATED VALVES F018B, F062B AND F063B TO CLOSE | 1.51E-02 | 4.00E-03 | 6.06E-05 |
| G31-MOV-CF-OPENA | MOV _s (3) TRAIN A CCF TO OPEN | 5.28E-03 | 4.00E-03 | 2.11E-05 |
| G31-MOV-CF-OPENA/B | CCF MOV TO OPEN | 2.16E-03 | 4.00E-03 | 8.65E-06 |
| G31-MOV-CF-OPENB | MOV _s (3) TRAIN B CCF TO OPEN | 5.28E-03 | 4.00E-03 | 2.11E-05 |
| G31-MP_-CR-RUN | CCF OF RWCU PUMPS TO RUN | 6.21E-02 | 2.40E-04 | 1.49E-05 |
| G31-MP_-CS-C001A/B | CCF TO RESTART PUMPS | 8.54E-02 | 2.30E-03 | 1.96E-04 |
| H23-EMS-CF-ALL | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| H23-EMS-CF-ATWSALL | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| H23-EMS-CF-DIDALL | CCF OF ALL DIVISION OF THE EMS | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| H23-EMS-CF-DPSALL | CCF OF ESSENTIAL MULTIPLEXING SYSTEM DIV 1/2/3/4 | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| H23-EMS-CF-N1EALL | CCF OF ALL DIVISION OF THE EMS | 3.00E-03 | 6.00E-04 | 1.80E-06 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|------------------------------|---|-------------------------|------------------|----------------|
| H23-RMU-CF-ALL | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| H23-RMU-CF-ATWSALL | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| H23-RMU-CF-DIDALL | CCF OF REMOTE MULTIPLEXING UNITS (DID) | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| H23-RMU-CF-DPSALL | CCF OF REMOTE MULTIPLEXING UNITS TO OPERATE | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| H23-RMU-CF-N1EALL | CCF OF REMOTE MULTIPLEXING UNITS (NO 1E) TO OPERATE | 3.00E-03 | 3.00E-04 | 9.00E-07 |
| H23-RMU-CF-P41A | CCF TO OPERATOR MU 001 AND RMU 002 P41 TRAIN A FOR MANUAL ACTUAT. | 1.20E-02 | 3.00E-04 | 3.60E-06 |
| H23-RMU-CF-P41B | CCF TO OPERATOR MU 001 AND RMU 002 P41 TRAIN B FOR MANUAL ACTUAT. | 1.20E-02 | 3.00E-04 | 3.60E-06 |
| N21-ACV-CF-MKUPA/B | CCF TO OPERATE AOVs TRAINS A & B | 6.75E-03 | 1.60E-02 | 1.08E-04 |
| N21-LT_-CF-FWTKLOW | CCF OF LEVEL TRANSMITTERS (LOW) IN FW STORAGE TANK | 5.00E-03 | 1.22E-05 | 6.12E-08 |
| N21-LT_-CF-FWTKNO | CCF FEEDWATER STORAGE TANK LEVEL TRANSMITTERS | 5.00E-03 | 8.76E-03 | 4.38E-05 |
| N21-LT_-CF-HWLOW | CCF HOTWELL LEVEL TRANSMITTERS TO STOP PUMPS | 5.00E-03 | 1.22E-05 | 6.12E-08 |
| N21-LT_-CF-HWNO | CCF HOTWELL LEVEL TRANSMITTERS | 5.00E-03 | 8.76E-03 | 4.38E-05 |
| N21-MP_-CR-COND | CONDENSATE PUMPS CCF TO RUN | 5.00E-03 | 6.00E-04 | 3.00E-06 |
| N71-MP_-CR-PUMP | CCF PUMPS CWS TO RUN | 1.32E-02 | 6.00E-04 | 7.92E-06 |
| N71-STR-CF-CWS | CCF STRAINER TO RUN | 4.44E-02 | 2.40E-04 | 1.07E-05 |
| P21-ACV-CF-F010/13 | CCF AIR OPERATED VALVES F010 AND F013 TO REMAIN OPEN | 2.50E-02 | 3.60E-06 | 9.00E-08 |
| P21-ACV-CF-F022/25A | CCF VALV ACV F022A/ F025A TRAIN A | 2.50E-02 | 2.00E-03 | 5.00E-05 |
| P21-ACV-CF-F022/25B | CCF VALV ACV F022B/F025B TRAIN B | 2.50E-02 | 2.00E-03 | 5.00E-05 |
| P21-ACV-CF-F027/32 | CCF AIR OPERATED VALVES F027 AND F032 TO REMAIN OPEN | 2.50E-02 | 1.82E-06 | 4.56E-08 |
| P21-MP_-CR-3 ^a | CCF 3 PUMPS TO RUN (INTRADIVISON A) | 2.00E-02 | 1.20E-04 | 2.40E-06 |
| P21-MP_-CR-3B | CCF 3 PUMPS TO RUN (INTRADIVISON B) | 1.00E-02 | 1.20E-04 | 1.20E-06 |
| P21-MP_-CR-5 ^a LL | CCF TO RUN PUMPS TRAINS A AND B | 5.00E-03 | 1.20E-04 | 6.00E-07 |
| P21-MP_-CR-TRAINA | CCF TO RUN RCCW PUMPS TRAIN A | 1.00E-02 | 1.20E-04 | 1.20E-06 |
| P21-MP_-CR-TRAINAB | CCF TO RUN PUMPS TRAINS A AND B | 5.00E-02 | 1.20E-04 | 6.00E-06 |
| P21-MP_-CR-TRAINB | CCF TO RUN RCCW PUMPS TRAIN B | 1.00E-02 | 1.20E-04 | 1.20E-06 |
| P21-MP_-CS-2 ^a | CCF 2 PUMPS (INTRADIVISON A) TO START | 2.00E-02 | 1.30E-03 | 2.60E-05 |
| P21-MP_-CS-2B | CCF 2 PUMPS (INTRADIVISON B) TO START | 2.00E-02 | 1.30E-03 | 2.60E-05 |
| P21-MP_-CS-5 ^a LL | CCF TO START PUMPS DIVISIONS A AND B | 5.00E-03 | 1.30E-03 | 6.50E-06 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|------------------------------|--|-------------------------|------------------|----------------|
| P21-MP_-CS-TRAINA | CCF TO START RCCW PUMPS TRAIN A | 1.00E-02 | 1.30E-03 | 1.30E-05 |
| P21-MP_-CS-TRAINAB | CCF TO START PUMPS TRAINS A AND B | 5.00E-02 | 1.30E-03 | 6.50E-05 |
| P21-MP_-CS-TRAINB | CCF TO START RCCW PUMPS TRAIN B | 1.00E-02 | 1.30E-03 | 1.30E-05 |
| P21-TT_-CF-N038A/B | CCF OF TEMPERATURE XMTR | 5.00E-03 | 8.40E-06 | 4.20E-08 |
| P22-MCB-CF-2 ^o LL | CCF 2 OUT OF 3 CIRCUIT BREAKERS FAIL TO CLOSE | 9.29E-02 | 1.00E-03 | 9.29E-05 |
| P22-MP_-CR-C001ABC | CCF TO RUN 2 PUMPS | 2.00E-02 | 1.20E-04 | 2.40E-06 |
| P22-MP_-CS-2 ^o LL | CCF 2 OUT OF 3 PUMPS FAIL TO START | 2.00E-02 | 1.30E-03 | 2.60E-05 |
| P22-PT_-CF-N002LOW | CCF SUCTION PRESSURE 2/ 3 TRANSMITTERS | 5.00E-03 | 7.92E-06 | 3.96E-08 |
| P22-TT_-CF-N003LOW | CCF 2/ 3 TEMPERATURE TRANSMITTERS | 5.00E-03 | 4.56E-05 | 2.28E-07 |
| P41-FAN-CR-3ALL | CCF TO RUN 3 FAN UNITS | 3.00E-03 | 2.40E-04 | 7.20E-07 |
| P41-FAN-CS-2ALL | CCF TO START 2 FAN UNITS | 1.20E-02 | 6.00E-04 | 7.20E-06 |
| P41-FAN-CS-3ALL | CCF TO START 3 FAN UNITS | 3.00E-03 | 6.00E-04 | 1.80E-06 |
| P41-MP_-CR-2ALL | CCF TO RUN 2 PUMPS TRAINS A AND B | 4.42E-03 | 7.68E-04 | 3.40E-06 |
| P41-MP_-CR-3ALL | CCF TO RUN 3 PUMPS TRAINS A AND B | 1.53E-02 | 7.68E-04 | 1.17E-05 |
| P41-MP_-CS-2ALL | CCF TO START 2 PUMPS TRAINS A AND B | 1.79E-02 | 2.40E-03 | 4.29E-05 |
| P41-STR-CF-2ALL | CCF 2 STRAINERS PLUGGED | 4.44E-02 | 2.40E-04 | 1.07E-05 |
| P41-STR-CF-3ALL | CCF 3 STRAINERS PLUGGED | 4.44E-02 | 2.40E-04 | 1.07E-05 |
| P51-ACV-CF-OPEN | CCF OF AOV TO OPEN | 6.75E-03 | 2.00E-03 | 1.35E-05 |
| P51-CMP-CR-RUN | CCF OF P51 COMPRESSORS TO RUN | 6.21E-02 | 2.40E-03 | 1.49E-04 |
| P51-CMP-CS-START | CCF OF P51 COMPRESSORS TO START | 8.54E-02 | 2.00E-02 | 1.71E-03 |
| P52-ACV-CF-DEENERG | CCF OF AOVs TO OPERATE TO OTHER THEN DEENERG. POS. | 1.10E-02 | 2.00E-03 | 2.20E-05 |
| P52-ACV-CF-OPEN | CCF OF AOV TO OPEN | 6.75E-03 | 2.00E-03 | 1.35E-05 |
| P52-CMP-CR-C001AB | CCF TO RUN COMPRESSORS LINES 1 & 2 | 6.21E-02 | 2.40E-03 | 1.49E-04 |
| P52-CMP-CR-P52/P51 | CCF OF P52/P51 COMPRESSORS TO RUN | 5.00E-03 | 2.40E-03 | 1.20E-05 |
| P52-CMP-CS-C001AB | CCF TO START COMPRESSORS LINES 1 & 2 | 8.54E-02 | 2.00E-02 | 1.71E-03 |
| P52-CMP-CS-P52/P51 | CCF OF P52/P51 COMPRESSORS TO START | 5.00E-03 | 2.00E-02 | 1.00E-04 |
| P52-PT_-CF-P51/P52 | CCF OF PRESSURE TRANSMITTERS | 5.00E-03 | 5.18E-04 | 2.59E-06 |
| P54-CPV-CF-CONTROL | CCF OF PRESSURE CONTROL VALVES FAILURES | 5.00E-02 | 3.13E-03 | 1.57E-04 |
| R10-MCB-CF-138CLOSE | CCF CIRCUIT BREAKERS TO CLOSE | 9.29E-02 | 8.00E-03 | 7.43E-04 |
| R11-MCB-CF-69CLOSE | CCF CIRCUIT BREAKERS 6.9 KV TO CLOSE | 9.29E-02 | 1.00E-03 | 9.29E-05 |
| R12-LCB-CF-480CLOSE | CCF CIRCUIT BREAKERS 480 V AC TO CLOSE | 9.29E-02 | 3.00E-04 | 2.79E-05 |
| R12-LCB-CF-480OPEN | CCF CIRCUIT BREAKERS 480 V AC TO OPEN | 9.50E-02 | 5.00E-04 | 4.75E-05 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|----------------------|---|----------------------------|---------------------|-------------------|
| R16-BT_-CF-125NO1E | CCF OF 125 V NO 1E BATTERIES | 2.39E-02 | 5.00E-04 | 1.20E-05 |
| R16-BT_-CF-ALLBATT | BATTERY CCF #2 | 1.80E-02 | 5.00E-04 | 9.01E-06 |
| R16-BT_-CF-BATT11&12 | BATTERY 11 & 12 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT11&21 | BATTERY 11 & 21 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT11&22 | BATTERY 11 & 22 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT11&31 | BATTERY 11 & 31 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT11&41 | BATTERY 11 & 41 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT12&21 | BATTERY 12 & 21 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT12&22 | BATTERY 12 & 22 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT12&31 | BATTERY 12 & 31 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT12&41 | BATTERY 12 & 41 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT21&22 | BATTERY 21 & 22 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT21&31 | BATTERY 21 & 31 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT21&41 | BATTERY 21 & 41 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT22&31 | BATTERY 22 & 31 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT22&41 | BATTERY 41 & 22 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATT31&41 | BATTERY 31 & 41 CCF | 1.91E-03 | 5.00E-04 | 9.56E-07 |
| R16-BT_-CF-BATTA1A2 | BATTERY A1 & A2 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA1B1 | BATTERY A1 & B1 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA1B2 | BATTERY A1 & B2 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA1C | BATTERY A1 & C CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA2B1 | BATTERY A2 & B1 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA2B2 | BATTERY A2 & B2 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTA2C | BATTERY A2 & C CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTB1B2 | BATTERY B1 & B2 CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTB1C | BATTERY B1 & C CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R16-BT_-CF-BATTB2C | BATTERY B2 & C CCF | 3.08E-03 | 5.00E-04 | 1.54E-06 |
| R21-DG_-CR-ALLDG | CCF OF DIESEL GENERATORS TO RUN | 7.73E-02 | 5.75E-02 | 4.44E-03 |
| R21-DG_-CS-ALLDG | CCF OF DIESEL GENERATORS TO START AND LOAD | 6.09E-02 | 1.40E-02 | 8.52E-04 |
| R21-MP_-CR-FUELTRANS | CCF TO RUN MOTOR-DRIVEN FUEL TRANSFER PUMPS | 6.21E-02 | 6.00E-04 | 3.72E-05 |
| R21-MP_-CS-FUELTRANS | CCF TO START MOTOR-DRIVEN FUEL TRANSFER PUMPS | 8.54E-02 | 2.00E-03 | 1.71E-04 |
| T10-VB_-CF-CLOSED | CCF 2 OF 3 VB FAIL TO REMAIN CLOSED | 2.00E-02 | 1.00E-04 | 2.00E-06 |
| T10-VB_-CF-OPEN | CCF 3 OF 3 VB FAIL TO OPEN | 1.00E-02 | 1.00E-04 | 1.00E-06 |
| T15-HX_-CF-PLUG | CCF OF THREE PCCS HX'S | 4.98E-02 | 8.76E-04 | 4.36E-05 |
| T62-TT_-CF-SPALL | CCF OF SUPPRESSION POOL TEMP. TRANSMITTERS | 5.00E-03 | 8.40E-06 | 4.20E-08 |
| T64-TT_-CF-HIGH | CCF TEMPERATURE TRANSMITTERS IN STEAM TUNNEL FAIL HIGH | 5.00E-02 | 4.56E-05 | 2.28E-06 |
| XXX-MP_-CR-SWS/CWS | CCF PUMPS TO RUN SWS/CWS | 1.32E-02 | 6.00E-04 | 7.92E-06 |

Table 5.3-1
Common Cause Failure Probabilities

| Event | Description | CCF Factor (f_{cc}) | Ptotal (Q_i) | Prob (Q_k) |
|---|-------------|----------------------------|---------------------|-------------------|
| (1) Per hour event failure rates are multiplied by 24 hours to obtain the value given in the (Q_i) column of Table 5.3-1. | | | | |

5.4 SPECIAL EVENTS

During the development of the PRA, in the Systems Analysis as well as the Accident Sequence Analysis, certain basic events are identified that do not correspond to any of the following categories, which are called special events:

- a. Probabilities or rates of individual failure of components.
- b. Unavailability for testing and maintenance.
- c. Probabilities of human error.
- d. Probabilities or rates of common cause failure.

Each case of these special events is treated differently. Some special events represent the maintenance policy of the plant, for example, the case of redundant trains. Other special events need to be treated like the events calculated from situations produced in the plant, for example, loss and recuperation of the electric power supply.

Table 5.4-1 includes all the events considered as special in the models, the probabilities, descriptions, and data source references.

Table 5.4-1
Special Events

| Special Event | Prob | Description | References |
|----------------------|-------------|---|---|
| B21-SYS-FF-1/9OPEN | 5.85E-02 | 1 OUT OF 9 SRV FAIL TO CLOSE AFTER OPENING | ALWR |
| B21-SYS-FF-10/18SRV | 7.87E-08 | 10 OUT OF 18 SRV FAIL TO OPEN IN SAFETY MODE | ALWR |
| B21-SYS-FF-18/18SRV | 7.87E-08 | 18 OUT OF 18 SRV FAIL TO OPEN IN SAFETY MODE | ALWR |
| B32-SYS-TM-ICA | 4.16E-02 | IC "A" UNAVAILABLE | NUREG/CR-4550 |
| C12-SYS-TM-TRAINB | 3.00E-03 | TRAIN B IN MAINTENANCE | ALWR |
| C41-SYS-FF-MAKEUP | 1.00E-01 | INVENTORY MAKE-UP BORATION FAILURE | Bounding Value Engineering Judgment |
| C71-SYS-FF-SCRAM | 1.00E-08 | SCRAM FAILURE | (2) |
| E50-SYS-FF-MLLOCA | 3.79E-01 | PROBABILITY OF MEDIUM LIQUID LOCA IN GDCS LINES | Initiating Events, section 2 |
| E50-SYS-FF-SLLOCA | 0.00E+00 | PROBABILITY OF SLLOCA IN GDCS LINES | Initiating Events, section 2 |
| G21-SYS-TM-TRAINB | 3.00E-03 | TRAIN B IN MAINTENANCE | ALWR |
| G31-SYS-TM-B | 3.00E-03 | RWCU/SDCS TRAIN B IN MAINTENANCE OR OUT OF SERVICE | ALWR |
| N21-SYS-FF-BYPASS | 1.00E-02 | TURBINE BYPASS FAILS | Bounding Value, Engineering Judgment |
| N21-SYS-FF-FWAL | 5.00E-01 | PROBABILITY OF LSLOCA IN FW TRAIN A PIPE INSIDE CONTAINMENT | |
| N21-SYS-FF-FWBL | 5.00E-01 | PROBABILITY OF LSLOCA IN FW TRAIN B PIPE INSIDE CONTAINMENT | |
| P41-SYS-FF-2CTMP | 5.04E-05 | INSUFFICIENT FLOW FROM CTM PUMPS (2 PUMPS FAIL) | ALWR |
| P41-SYS-FF-3CTMP | 9.20E-06 | INSUFFICIENT FLOW FROM CTM PUMPS (3 PUMPS FAIL) | ALWR |
| R10-SYS-FF-230KV | 5.00E-04 | 230 KV SWITCHYARD FAILS DURING OPERATION | ALWR |
| R10-SYS-FF-500KV | 1.20E-06 | 500KV SWITCHYARD FAILS DURING OPERATION | ALWR |
| R10-SYS-TM-230KV | 1.00E-02 | 230 KV SWITCHYARD IN MAINTENANCE | ALWR |
| R11-SYS-FF-NOREC | 6.13E-01 | FAILURE IN OFFSITE POWER RECOVERY | NUREG/CR-5496,NUREG/CR-6823 |
| R21-SYS-FC-AIRDG4 | 1.00E-03 | AIR STARTING SYSTEM FAILURE [#13] | NUREG/CR-2728 |
| R21-SYS-FC-AIRDG5 | 1.00E-03 | AIR STARTING SYSTEM FAILURE | NUREG/CR-2728 |
| R21-SYS-FC-FUELDG4 | 4.00E-03 | FUEL OIL STORAGE & TRANSFER SYSTEM FAILURE[#14] | ALWR |
| R21-SYS-FC-FUELDG5 | 4.00E-03 | FUEL OIL STORAGE & TRANSFER SYSTEM FAILURE[#14] | ALWR |
| T11-SYS-FF-OPEN | 5.69E-02 | ALL OVERPRESSURE PROTECTION VALVES FAIL TO OPEN | Bounding Value, Engineering Judgment |
| U43-SYS-FF-LPCI | 2.40E-02 | U43 HARDWARE FAILURES | ALWR |
| U43-SYS-FF-YARD | 2.00E-03 | HARDWARE FAILURES IN YARD AREA | ALWR |
| XXX-SYS-FF- | 7.80E-05 | LOSS OF SCRAM FOLLOW SIGNAL. (2/4 | (1) |

Table 5.4-1
Special Events

| Special Event | Prob | Description | References |
|---|-------------|----------------------|-------------------|
| SCRAMFOLLOW | | RELAY LOGIC FAILURE) | |
| <p>(1). This value corresponds to the probability of the 2/4 Voting Logic Card "all modes failure".</p> <p>(2). This value was derived for the ABWR using a fault tree model of the Control Rods, including the FMCRD. Because the system is identical to the ESBWR design and it is independent of other systems in the ESBWR, the point estimate from the ABWR analysis was deemed appropriate.</p> | | | |

5.5 REFERENCES

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6 HUMAN RELIABILITY ANALYSIS

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6 HUMAN RELIABILITY ANALYSIS

6.1 INTRODUCTION AND SCOPE

The purpose of this section is to document the Human Reliability Analysis performed as part of the ESBWR PRA.

Three types of human actions are described in this chapter: actions that take place prior to the initiating event, actions that cause an initiating event, and actions that are taken to mitigate an initiating event. The evaluation is done using bounding Human Error Probabilities (HEP) in order to demonstrate that the ESBWR does not overly rely on human actions to maintain a low Core Damage Frequency (CDF).

Section 6.2 describes the methods used in the analysis. Section 6.3 describes the specific calculations for the HEPs used in the ESBWR PRA and how dependencies are treated. Section 6.4 outlines COL action items related to refinement of HEPs.

6.2 METHODOLOGY

This section describes the methodology applied in evaluating the human interactions (HIs) with the plant systems both during normal operation and during accidents. HIs during normal plant operation are those causing an initiating event and those which fail to restore equipment to their normal condition following a test and/or maintenance action.

The methodology used in the present study is in agreement with the Systematic Human Action Reliability Procedure (SHARP) (Reference 6-1).

In the design phase of ESBWR, generic screening values of human reliability are used. These generic HEPs are obtained from EPRI NP-3583 (Reference 6-1), NUREG/CR-1278 (Reference 6-2), and NUREG/CR-4772 (Reference 6-3).

Human interactions analyzed in the ESBWR PRA are categorized in the following types:

- Type A: Pre-Initiating Event HIs

Before an initiating event, plant personnel can affect availability of standby systems by inadvertently disabling equipment during the performance of operational activities in the plant.

These HIs are associated with testing, instrument calibration, preventive maintenance, corrective maintenance, replacement of components and realignment of systems. These activities are assumed to take place prior to the initiating event.

- Type B: Initiating Event-Related HIs

Plant personnel can initiate an event through various interactions with plant equipment. These would typically occur due to a misposition or maloperation of equipment that inadvertently trips equipment or inserts false control signals.

This type of human action is implicitly included in the ESBWR model as part of the frequency of the initiating events. The design PRA uses generic initiating event frequencies that include the contribution of operator-initiated events. Therefore, Type B HIs are not explicitly modeled in the ESBWR design PRA.

- Type C: Post Initiating Event HIs

The nature of the passive ESBWR is such that post initiator operator actions are not as strong of a contributor to the risk profile as in current LWRs. As such, a conservative assessment of HEPs is judged to be acceptable during the design phase. Probabilities used in the PRA are based on generic values based on the time frame for the operators to successfully diagnose situations and perform the necessary actions prior to core damage.

Two major subsets are identified as follows:

- (1) CP – These are actions that are specifically directed by plant operating and emergency procedures. By following procedures during the course of an accident, plant personnel can operate standby equipment that will terminate the accident.
- (2) CR – These are actions that have traditionally been called recovery actions. In the ESBWR, only actions to recover equipment from misalignment (Type A actions) are included. These require diagnosis and realignment of plant equipment in order

to perform procedure directed actions. Plant personnel can recover and operate initially unavailable equipment to terminate an accident.

Errors of commission have not been included in the ESBWR design PRA.

To analyze these types of HIs, the Systematic Human Action Reliability Procedure (SHARP) has been adopted (Reference 6-9).

6.3 QUANTIFICATION OF HUMAN ERROR PROBABILITIES

A conservative assessment of the HEPs is used for the ESBWR design PRA. Generic screening probabilities are estimated from References 6-1, 6-2, and 6-3. A more detailed analysis of the values could be performed, however this would tend to further reduce the calculated probabilities. In this phase of the design, screening values are appropriate. The values taken from the references are median values. The ESBWR PRA model requires mean values. These were calculated by combining the median values with the associated error factors, assuming a lognormal distribution.

6.3.1 Type A Human Actions

The systems modeled in the PRA are reviewed to determine likely test and maintenance activities. Because procedures for these activities are not available at the design phase, likely configurations for these activities are assumed. Failure to restore from these configurations is identified as potential Type A actions.

For all the components for which there is an alarm or an indication that is checked at least once every day, it is not necessary to consider the probability of an incorrect positioning. In addition, for all the components for which there are a total check (such as a flow test) after manipulation, it is not necessary to consider the probability of an incorrect position or state before postulated accidents. The potential for misalignment of each system is described within each system description in Section 4.

For the screening analysis of human reliability, generic values are used as reported in Table 6.3-1. These HEPs are extracted from documents EPRI NP-3583 (Reference 6-1), NUREG/CR-1278 (Reference 6-2), and NUREG/CR-4772 (Reference 6-3). The detection interval in Table 6.3-1 corresponds to the frequency of the maintenance activity anticipated for the system.

There are three different levels of Type A actions modeled as follows:

- Skill Based

Skill based actions include the manipulation of equipment that is performed on a routine basis (i.e. every shift) during the operation of the plant. Personnel are trained and well practiced in these manipulations.

An example is the closure of the associated discharge valve during the maintenance of a pump.

- Rule Based

Rule based actions include the manipulation of equipment that is performed occasionally. Personnel perform the manipulations under the direction of procedures and following training on those tasks.

An example is the restoration of isolations following the mixing of a batch of sodium pentaborate SLCS solution.

- Knowledge Based

Knowledge based actions include the manipulation of equipment that is performed during unique situations.

An example is the immediate reconfiguration of a system following a failure of an operating pump.

Table 6-3 presents the final Type A human error probabilities used in the PRA model.

6.3.2 Type B Human Actions

Due to the way that the initiating event frequencies were developed in the ESBWR design PRA; no Type B actions are included as basic events in the models. The generic initiating events includes operator actions that have historically caused plant transients and accidents. The ESBWR design does not include any features that are not covered by the analysis presented in Section 2.

6.3.3 Type C Human Actions

The systems modeled in the PRA were reviewed to determine the need for human interactions. These would be necessary to provide backup actuations for automatic systems, manual alignment of systems that do not automatically start, and to recover from the misalignments identified in the Type A analysis. Actuations of systems are modeled as "CP" sub-type HEPs and recovery are modeled as "CR" sub-type HEPs. These sub-types are defined in Section 6.2.

There are four time frames considered in the ESBWR design PRA, and the Type C HEPs are based on these time frames. The screening probabilities are assigned, in part, according to these time frames. No credit is taken for actions that must be completed very quickly (i.e., within a few minutes of the cue).

HEPs are estimated in two parts, the cognitive part (or diagnosis phase) and the manual part (or action phase). The total HEP for an action is the sum of these two parts. The HEPs for the cognitive parts are assigned based on the following criteria:

- Skill Based

The skill based actions are those, which are routine and frequently practiced. An example would be for the operators to insert a backup scram signal following an initiating event.

- Rule Based

The rule based actions are those, which are performed in response to a specific set of procedures. An example would be to start FAPCS in SPC mode when the suppression pool water temperature reaches a preset limit.

- Knowledge Based

The knowledge based actions are those where the operators need to deduce the proper course of action based on indirect indications or a complex combination of individual indications. An example would be injection SLCS via the boron mixing system following a failure of a SLCS division.

The HEPs for the manual part (action) are assigned based on the following criteria:

- Skill Based

The skill based actions are those, which are routine, frequently practiced, and require a few simple actions. An example would be for the operators to manual initiate depressurization from the control room.

- Rule Based

The rule based actions are those which are more complex and involve specific sequences of manipulations, however procedures and training are readily available. An example would be starting FAPCS in suppression pool cooling mode (non-automated).

- Knowledge Based

The knowledge based actions are those that are complex and not routine. Procedures are available, but training and drills are infrequent. An example would be controlling a system from outside the control room.

For the screening analysis of human reliability, generic values are taken from documents EPRI NP-3583 (Reference 6-1), NUREG/CR-1278 (Reference 6-2), and NUREG/CR-4772 (Reference 6-3). These are presented in Table 6.3-1 and the final HEPs are presented in Table 6.3-4.

6.3.4 HEP Dependencies

The screening analysis accounts for the dependencies among HIs in the following manner:

Type A human errors associated with improper realignments following tests or maintenance typically do not have strong dependence between multiple actions. Dependencies of this kind usually arise from incorrect procedures or poorly trained maintenance personnel. It is anticipated that the COL holder will institute programmatic activities to minimize this type of dependence. Because of this, it is not considered part of the design PRA.

Type A human errors associated with miscalibration are not important in ESBWR. The design of the DCIS incorporates self detection of miscalibrated channels. Therefore, as long as the calibration of the redundant channels is performed on a staggered schedule, this type of human error will not be important and will not be susceptible to dependence.

For Type C human errors that affect various systems, the logic model includes separate basic events representing the cognitive part of the action, and various basic events representing the manipulation of the systems involved. If more than one human action shares a cognitive or manual part, the basic event representing the dependent part is included in both locations in the fault tree model. For the purposes of the design PRA, even partially dependent actions are modeled as fully dependent.

Figure 6-1 shows an example of this modelling. Depressurization using the SRVs is modeled using separate cognitive and manual basic events. This is because initiating low pressure injection using FAPCS is dependent on depressurization and must occur in the same time frame as the depressurization action. As is seen in the figure, the decision to depressurize the plant is modeled under both logic sub-sections.

After quantification, all the human actions that appear combined in the same minimal cutsets whose accumulated frequencies are, at the most, 99% of the core damage frequency, were reviewed. Based on this review, it is considered unnecessary to model any special new dependencies. This is because either no dependence is identified or the dependencies are already

explicitly taken into consideration in the models. For those cases where a degree of dependence is considered possible, it is determined that the values of the probabilities of the human actions combined in the same minimal cutsets at this stage are sufficiently conservative that the impact of the possible dependency is considered to be adequately.

Table 6.3-1
Generic Error Probability Values for Type A Actions

| Human Action | Detection Interval | Behavior Type ⁽¹⁾ | | |
|-----------------------|---|------------------------------|-----------------------------|------------------------------|
| | | Skill | Rule | Knowledge |
| Type A ⁽²⁾ | 720 h | 5×10^{-4} (EF=5) | 2.5×10^{-3} (EF=5) | 1.25×10^{-2} (EF=5) |
| | 2190 h | 1.5×10^{-3} (EF=5) | 7.5×10^{-3} (EF=5) | 3.75×10^{-2} (EF=5) |
| | 6380 h | 3×10^{-3} (EF=5) | 1.5×10^{-2} (EF=5) | 7.5×10^{-2} (EF=5) |
| | 8640 h | 6×10^{-3} (EF=5) | 3×10^{-2} (EF=5) | 1.5×10^{-1} (EF=3) |
| Notes: | (1) Values given are the Median HEP and the associated Error Factor (2) Source: EPRI NP-3583, NUREG/CR-1278, NUREG/CR 4772 | | | |

Table 6.3-2
Type A Human Action Probabilities

| Identification | Description | Type | Behavior Type | Actuation Interval (hr) | Value | Mean Value |
|------------------|-----------------------------|------|---------------|-------------------------|----------|------------|
| P54-XHE-MH-F003 | Misalignment of valve F003 | A | Rule | 720 | 2.50E-03 | 4.02E-03 |
| C12-XHE-MH-F013A | Misalignment of valve F013A | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| C12-XHE-MH-F013B | Misalignment of valve F013B | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| C12-XHE-MH-F015A | Misalignment of valve F015A | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| C12-XHE-MH-F015B | Misalignment of valve F015B | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| C12-XHE-MH-F003B | Misalignment of valve F003B | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| C12-XHE-MH-F018A | Misalignment of valve F018A | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| C12-XHE-MH-F018B | Misalignment of valve F018B | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| C12-XHE-MH-F021A | Misalignment of valve F021A | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| C12-XHE-MH-F021B | Misalignment of valve F021B | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| P21-XHE-MH-F022B | Misalignment of valve F022B | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| P21-XHE-MH-F023B | Misalignment of valve F023B | A | Rule | 2190 | 7.50E-03 | 1.20E-02 |
| P30-XHE-MH-F015 | Misalignment of valve F01T | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| G21-XHE-MH-F308 | Misalignment of valve F308 | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| G21-XHE-MH-F320 | Misalignment of valve F320 | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| G21-XHE-MH-F334 | Misalignment of valve F334 | A | Rule | 8640 | 3.00E-02 | 4.83E-02 |
| C41-XHE-MH-F002A | Misalignment of valve F002A | A | Rule | 720 | 2.50E-03 | 4.03E-03 |
| C41-XHE-MH-F002B | Misalignment of valve F002B | A | Rule | 720 | 2.50E-03 | 4.03E-03 |

Table 6.3-3
Generic Error Probability Values for Type C Actions

| Human Action | Cognitive HEP (Time) | Behavior Type ⁽¹⁾ | | |
|------------------------|---|------------------------------|-------------------|-------------------|
| | | Skill | Rule | Knowledge |
| Type CP ⁽²⁾ | 30 minutes | 10^{-2} (EF=5) | 10^{-1} (EF=5) | 1 |
| | 60 minutes | 10^{-3} (EF=5) | 10^{-2} (EF=5) | 10^{-1} (EF=5) |
| | 24 hours | 10^{-4} (EF=5) | 10^{-3} (EF=5) | 10^{-2} (EF=5) |
| | Manipulation HEP | 10^{-3} (EF=5) | 10^{-2} (EF=5) | 10^{-1} (EF=5) |
| | 72 hours | 10^{-5} (EF=10) | 10^{-4} (EF=10) | 10^{-3} (EF=10) |
| | Manipulation HEP | 10^{-4} (EF=5) | 10^{-3} (EF=5) | 10^{-2} (EF=5) |
| Type CR ⁽²⁾ | 30 minutes | 10^{-2} (EF=10) | 10^{-1} (EF=10) | 1 |
| | 24 hours | 10^{-3} (EF=10) | 10^{-2} (EF=10) | 10^{-1} (EF=10) |
| | Manipulation HEP | 10^{-3} (EF=10) | 10^{-2} (EF=10) | 10^{-1} (EF=10) |
| | 72 hours | 10^{-4} (EF=10) | 10^{-3} (EF=10) | 10^{-2} (EF=10) |
| | Manipulation HEP | 10^{-4} (EF=10) | 10^{-3} (EF=10) | 10^{-3} (EF=10) |
| Notes: | (1) Values given are the Median HEP and the associated Error Factor (2) Source: EPRI NP-3583, NUREG/CR-1278, NUREG/CR-4772 | | | |

Table 6.3-4
Type C Human Action Probabilities

| Identification | Description | Type | Cognitive | Time | Value | Manipulation | Value | Mean Value |
|----------------------|---|------|---------------------|------------|----------|--------------|----------|------------|
| B21-XHE-FO-9OPEN | Operator fails to manually open 9/10 SRV | CP | XXX-XHE-FO-DESPRESS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| B21-XHE-FO-ADS | Operator fails to back up the ADS actuation | CP | Skill | 30 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| B32-XHE-FO-F72HA | Operator fails to manually back-up F72HA valve opening | CP | XXX-XHE-FO-ICPCCS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| B32-XHE-FO-F72HB | Operator fails to manually back-up F72HB valve opening | CP | XXX-XHE-FO-ICPCCS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| C12-XHE-FO-LEVEL2 | Operator fails to back-up CRD actuation | CP | Skill | 30 minutes | 1.00E-02 | Rule | 1.00E-02 | 3.22E-02 |
| C41-XHE-FO-INI | Operator fails to back up SLCS initiation | CP | Skill | 30 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| C41-XHE-FO-INISLCS | Operator fails to initiate SLCS alternative | CP | Rule | 30 minutes | 1.00E-01 | Rule | 1.00E-02 | 1.77E-01 |
| C41-XHE-FO-OPENF002A | Operator fails to recover the opening of valve F002A | CR | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| C41-XHE-FO-OPENF002B | Operator fails to recover the opening of valve F003A | CR | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| E50-XHE-FO-EQU | Operator fails to manually back up Equalizing lines actuation | CP | XXX-XHE-FO-RPVLDE | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| E50-XHE-FO-GDCS | Operator fails to manually back up GDCS actuation | CP | XXX-XHE-FO-RPVLDE | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| G21-XHE-FO-LPCI | Operator fails to manually align and actuate FAPCS in LPCI mode | CP | XXX-XHE-FO-DESPRESS | N/A | | Skill | 1.00E-03 | 1.61E-03 |

Table 6.3-4
Type C Human Action Probabilities

| Identification | Description | Type | Cognitive | Time | Value | Manipulation | Value | Mean Value |
|----------------------|---|------|---------------------|------------|----------|--------------|----------|------------|
| G21-XHE-FO-LPCIADS | Operator fails to manually align and actuate FAPCS in LPCI mode | CP | XXX-XHE-FO-LPMAKEUP | N/A | | Rule | 1.00E-02 | 1.61E-02 |
| G21-XHE-FO-SPC | Operator fails to manually initiate FAPCS in SPC mode. | CP | N/A | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| G21-XHE-FO-SPCADS | Operator fails to manually initiate FAPCS in SPC mode after MDS | CP | N/A | N/A | | Rule | 1.00E-02 | 1.61E-02 |
| G31-XHE-FO-SDC | Operator fails to actuate RWCU/SDC mode | CP | Rule | 24 hours | 1.00E-03 | Rule | 1.00E-02 | 1.77E-02 |
| G31-XHE-FO-SDCSLCS | Operator fails to actuate RWCU/SDC mode in ATWS | CP | Knowledge | 24 hours | 1.00E-02 | Knowledge | 1.00E-01 | 1.77E-01 |
| G31-XHE-FO-MIBOC | Operator fails to manually isolate a RWCU/SDC broken line | CR | Skill | 72 hours | 1.00E-04 | Skill | 1.00E-04 | 5.32E-04 |
| G31-XHE-FO-SDCMSL | Operator fails to actuate RWCU/SDC mode in MSL | CP | Knowledge | 24 hours | 1.00E-02 | Rule | 1.00E-02 | 3.22E-02 |
| N21-XHE-FO-CONDPUMPD | Operator fails to start condensate pump D. | CP | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| N21-XHE-FO-FWPUMPD | Operator fails to start feedwater pump D | CP | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| P21-XHE-FO-AIRVALVES | Operator fails in manual opening | CP | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| P21-XHE-FO-STDBYPUMP | Operator fails to back up the start of the standby pump | CP | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |
| P22-XHE-FO-HXB | Operator fails to align HXB | CP | Rule | 30 minutes | 1.00E-01 | Skill | 1.00E-03 | 2.69E-01 |

Table 6.3-4
Type C Human Action Probabilities

| Identification | Description | Type | Cognitive | Time | Value | Manipulation | Value | Mean Value |
|------------------------|---|------|-------------------------|------------|----------|--------------|----------|------------|
| P41-XHE-FO-STDBYPUMP | Operator fails to back up the start of the standby pump | CP | Skill | 30 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| P51-XHE-FO-STDBYCOMP | Operator fails to back-up standby compressor SAS | CP | Rule | 60 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| P52-XHE-FO-ALIGNDRYER2 | Operator fails to manually actuate valve F015B to failing Dryer 2 | CP | P52-XHE-FO- IAS/SAS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| P52-XHE-FO-ALIGNSAS | Operator fails to manual actuation SAS for IAS use | CP | P52-XHE-FO- IAS/SAS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| P52-XHE-FO-IAS/SAS | Operator fails to recognize back-up IAS | CP | Rule | 60 minutes | 1.00E-02 | N/A | | 1.61E-02 |
| P52-XHE-FO-STDBYCOMP | Operator fails to manually actuate standby compressor IAS | CP | P52-XHE-FO- IAS/SAS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| P54-XHE-FO-OPENF005 | Operator fails to back-up F005 valve opening | CP | Rule | 60 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| T10-XHE-FO-CLOSE | Operator fails to close vacuum breakers | CP | Rule | 60 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| T10-XHE-FO-CLOSEIVS | Operator fails to back up closed isolation valves | CP | Skill | 30 minutes | 1.00E-02 | Skill | 1.00E-03 | 1.77E-02 |
| U43-XHE-FO-LPCI | Operator fails to manually align and actuate FPS in LPCI mode | CP | XXX-XHE-FO- DESPRESS | N/A | | Skill | 1.00E-03 | 1.61E-03 |
| U43-XHE-FO-LPCIADS | Operator fails to manually align and actuate FPS in LPCI mode | CP | XXX-XHE-FO- LPMAKEUP | N/A | | Rule | 1.00E-02 | 1.61E-02 |
| U43-XHE-FO-MAKEUP | Operator fails to manually actuate U43 for back-up PCCS | CP | XXX-XHE-FO- ICPCCS | N/A | 0.00E+00 | Rule | 1.00E-02 | 1.61E-02 |

Table 6.3-4
Type C Human Action Probabilities

| Identification | Description | Type | Cognitive | Time | Value | Manipulation | Value | Mean Value |
|---------------------|---|------|-----------|------------|----------|--------------|----------|------------|
| U43-XHE-FO-YARD | Operator fails to supply PCCS from yard area | CP | Rule | 72 hours | 1.00E-04 | Rule | 1.00E-03 | 1.77E-03 |
| XXX-XHE-FO-DEPRESS | Operator fails to recognize the need for SRVs for depressurization | CP | Rule | 30 minutes | 1.00E-01 | N/A | | 1.61E-01 |
| XXX-XHE-FO-ICPCCS | Operator fails to recognize back-up F72HA/B opening | CP | Rule | 24 hours | 1.00E-03 | N/A | | 1.61E-03 |
| XXX-XHE-FO-LPMAKEUP | Operator fails to recognize the need for low-pressure makeup with RPV depressurized | CP | Rule | 30 minutes | 1.00E-01 | N/A | | 1.61E-01 |
| XXX-XHE-FO-RPVLDE | Operator fails to recognize back-up Passive System | CP | Skill | 30 minutes | 1.00E-02 | N/A | | 1.61E-02 |
| XXX-XHE-FO-SPC | Operator fails to recognize the need for SPC | CP | Rule | 60 minutes | 1.00E-02 | N/A | | 1.61E-02 |

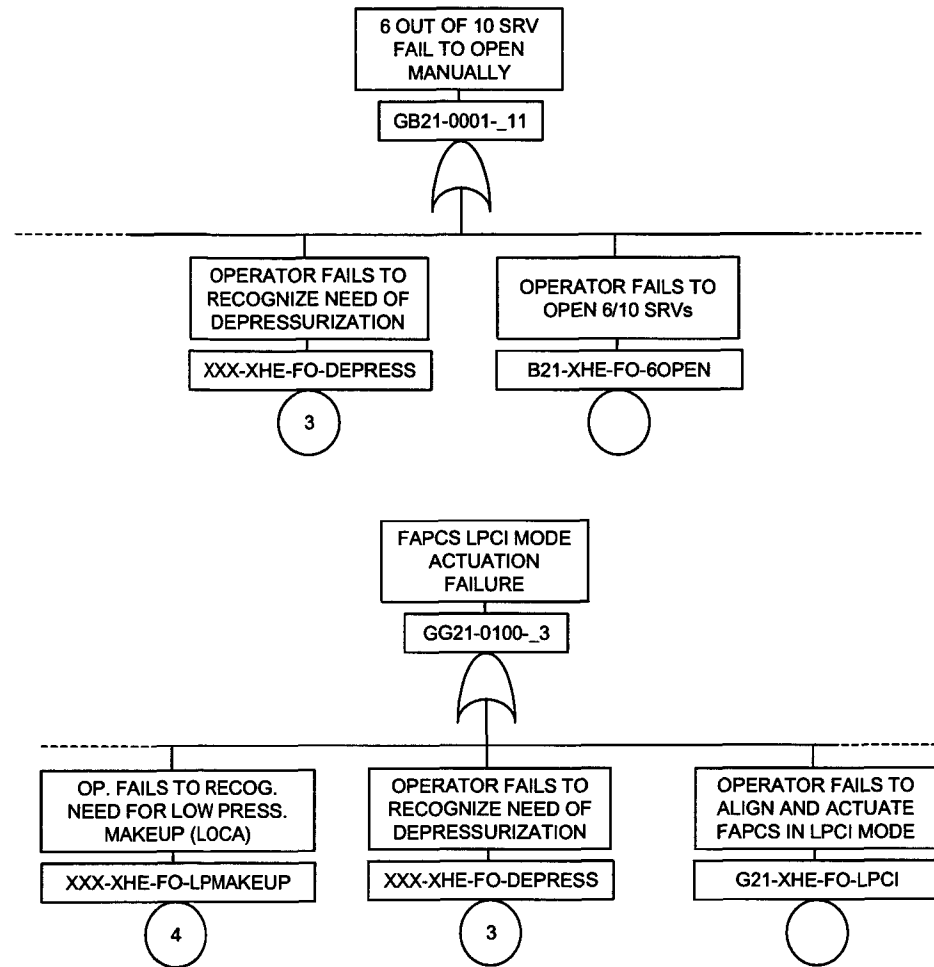


Figure 6.3-1
Human Action Dependence Example

6.4 REFINEMENT OF HUMAN ERROR PROBABILITIES

In order to reduce the conservative contribution of human actions in the ESBWR PRA, human actions that prove to be important can be analyzed in detail.

The criteria for selecting the human actions for detailed analyses are the following:

- Any Type A human action which could potentially cause a failure of more than one redundant/train/division of one or more systems
- Any single Type C human action of an operator which could directly cause damage to the core

The purpose of the detailed analysis is to remove unnecessary conservatism from the PRA results. Because of the way the screening values were chosen, more detailed analysis will reduce the HEPs. As a consequence, detailed analysis is not considered necessary in the DCD phase.

In order to perform a detailed analysis of the human actions modeled in NEDO-33201 Section 6.4, several procedures and data are required. The development and availability of most of these items are expected in the COL phase.

The type of procedures and information necessary to develop detailed analysis are listed below:

Type A human actions

Tagging procedures

Test and calibration procedures

Periodic Operational equipment realignment procedures

Equipment position checking procedures

Walk around inspection procedures

Type C human actions

- Emergency Operating Procedures
- Abnormal Operating Procedures
- Integrated Operating Procedures
- Annunciator Response Procedures
- System operation procedures
- Plant staff organization procedures
- Any other Plant special or auxiliary procedures

Finally, to fully assess the final HEPs, it is necessary to evaluate other performance shaping factors (PSF) that could influence steps in each important human action basic events (e.g., training, man-machine interface, operator experience, etc.). Much of this information will not be available prior to the final construction phase of the plant. Therefore, during the detailed design and COL phase, PSFs will be estimated based on data from existing BWR PRA models.

6.5 REFERENCES

- 6-1 EPRI: Systematic Human Action Reliability Procedure (SHARP), NP-3583, June 1984.
- 6-2 NUREG/CR-1278 "Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications Final Report", August 1983.
- 6-3 NUREG/CR-4772 "Accident Sequence Evaluation Program – Human Reliability Analysis Procedure", February 1987.
- 6-4 EPRI Project RP2170-3 (NUS Report 4531) "Human Cognitive Reliability Model for PRA Analysis" December 1984.