

February 27, 2006

Mr. James H. Lash
Vice President
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
P. O. Box 4
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SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (BVPS-1 AND 2) -
ISSUANCE OF AMENDMENT RE: RELAXED AXIAL OFFSET CONTROL
(RAOC) AND F_Q SURVEILLANCE METHODOLOGIES (TAC NOS. MC5904
AND MC5905)

Dear Mr. Lash:

The Commission has issued the enclosed Amendment No. 274 to Facility Operating License No. DPR-66 and Amendment No. 155 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated February 11, 2005, as supplemented August 8, 2005.

These amendments approve the adoption of the RAOC and F_Q surveillance methodologies in accordance with Nuclear Regulatory Commission (NRC)-approved Topical Report WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control F_Q Surveillance Technical Specification." TS 3.2.1, "Axial Flux Difference (AFD)," and TS 3.2.2, "Heat Flux Hot Channel Factor - $F_Q(Z)$," are being revised to adopt the RAOC calculational procedure of NUREG-1431, "Standard Westinghouse Technical Specifications for Westinghouse Plants, Revision 3, June 2004." Changes to TS 3.2.3, "Nuclear Enthalpy Hot Channel Factor - $F_{\Delta H}^N$," TS 3.2.4, "Quadrant Power Tilt Ratio (QPTR)," TS 3.3.1, "Reactor Trip System Instrumentation (Table 4.3-1, Note 3)," and TS 6.9.5, "Core Operating Limits Report (COLR)," have been made to provide consistency with the changes made to TSs 3.2.1 and 3.2.2.

J. Lash

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A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Timothy G. Colburn, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures:

1. Amendment No. 274 to DPR-66
2. Amendment No. 155 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

J. Lash

-2-

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cc w/encls: See next page

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*SE input provided. No substantive changes made.

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FIRSTENERGY NUCLEAR OPERATING COMPANY

FIRSTENERGY NUCLEAR GENERATION CORP.

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 274
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee), dated February 11, 2005, as supplemented August 8, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 274, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to entry into Mode 4 upon restart from the spring 2006 refueling outage which begins on or about February 10, 2006.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 27, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 274

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

IV
3/4 2-1
3/4 2-2
3/4 2-3
3/4 2-5
3/4 2-6
3/4 2-6a
3/4 2-8
3/4 2-10
3/4 3-13
6-18
6-19

Insert

IV
3/4 2-1
3/4 2-2
3/4 2-3
3/4 2-4

3/4 2-8
3/4 2-10
3/4 3-13
6-18
6-19

FIRSTENERGY NUCLEAR OPERATING COMPANY

FIRSTENERGY NUCLEAR GENERATION CORP.

OHIO EDISON COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee), dated February 11, 2005, as supplemented August 8, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 155, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. FENOC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to entry into Mode 4 from startup following the fall 2006 refueling outage which begins in October 2006.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 27, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 155

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

3/4 2-1
3/4 2-2
3/4 2-3
3/4 2-4
3/4 2-5
3/4 2-6
3/4 2-7
3/4 2-9
3/4 3-13
6-19
6-20

Insert

3/4 2-1

3/4 2-4
3/4 2-5
3/4 2-6
3/4 2-7
3/4 2-9
3/4 3-13
6-19
6-20

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 274 AND 155 TO FACILITY OPERATING
LICENSE NOS. DPR-66 AND NPF-73
FIRSTENERGY NUCLEAR OPERATING COMPANY
FIRSTENERGY NUCLEAR GENERATION CORP.
OHIO EDISON COMPANY
THE TOLEDO EDISON COMPANY
BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (BVPS-1 and 2)
DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By application dated February 11, 2005 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML050480264), as supplemented August 8, 2005 (ADAMS Accession No. ML052220351), the FirstEnergy Nuclear Operating Company (FENOC, the licensee), requested changes to the Technical Specifications (TSs) for BVPS-1 and 2. The supplement dated August 8, 2005, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on April 26, 2005 (70 FR 21457).

The proposed changes would approve the adoption of the relaxed axial offset control (RAOC) and F_Q surveillance methodologies in accordance with NRC-approved Topical Report, WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control - F_Q Surveillance Technical Specification." TS 3.2.1, "Axial Flux Difference (AFD)," and TS 3.2.2, "Heat Flux Hot Channel Factor - $F_Q(Z)$," would be revised to adopt the RAOC calculational procedure of NUREG-1431, "Standard Westinghouse Technical Specifications for Westinghouse Plants, Revision 3, June 2004." Changes to TS 3.2.3, "Nuclear Enthalpy Hot Channel Factor - $F_{\Delta H}^N$," TS 3.2.4, "Quadrant Power Tilt Ratio (QPTR)," TS 3.3.1, "Reactor Trip System Instrumentation (Table 4.3-1, Note 3)," and TS 6.9.5, "Core Operating Limits Report (COLR)," would be made to provide consistency with the changes made to TSs 3.2.1 and 3.2.2.

2.0 REGULATORY EVALUATION

BVPS-1 and 2 TSs currently stipulate the use of the constant axial offset control (CAOC) methodology for analyzing and controlling the AFD parameter. In the power range, CAOC methodology calls for a constant band about a reference AFD, without regard to reactor power. In the power range, RAOC methodology generates a power dependent AFD curve. Generally, the RAOC-allowed AFD increases with decreasing power. In some plants the CAOC methodology results in significant margin between the CAOC-generated AFD and the maximum permitted AFD in the loss-of-coolant accident (LOCA) analysis. RAOC methodology allows a licensee to transfer some of that margin to operating flexibility.

The Commission's regulatory requirements related to the content of the TSs are set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical specifications." This regulation requires that the TSs include items in five specific categories. These categories include (1) safety limits, limiting safety system settings and limiting control settings, (2) limiting conditions for operation, (3) surveillance requirements, (4) design features, and (5) administrative controls. Appendix A to 10 CFR Part 50, "General Design Criteria [GDC] for Nuclear Power Plants," establishes minimum requirements for the principal design criteria for water-cooled nuclear power plants. GDC 10, "Reactor design," states, "The reactor core and associated coolant, control, and protection systems shall be designed with appropriate margin to assure that specified acceptable fuel design limits are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences."

The NRC approved WCAP-10216-P-A on November 26, 1993, for referencing in license applications to the extent specified and under the limitations stated in the topical report and NRC safety evaluation. The NRC staff did not herein repeat its review of the matters described in the report and that were found acceptable. The NRC staff's review herein was to assure that WCAP-10216-P-A is applicable to BVPS-1 and 2, and that WCAP-10216-P-A is being implemented appropriately.

3.0 TECHNICAL EVALUATION

The NRC staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendment, as described in the licensee's February 11, 2005, application (Reference 1) and supplemented by letter dated August 8, 2005 (Reference 2). The evaluation below supports the following conclusions: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

The NRC's approval of WCAP-10216-P-A was limited to Westinghouse-designed pressurized-water reactors. BVPS-1 and 2 meets these applicability criteria. The NRC approval did not impose any additional limitations beyond those already present in WCAP-10216-P-A. WCAP-10216-P-A is comprised of two parts. Part A covers the relaxation of constant axial offset control. Part B covers the F_Q surveillance TS.

Part A describes a method for determining an acceptable AFD profile. AFD is a measure of the power differential between the top and bottom halves of the core. If power is shifted too much in either direction it can exacerbate local power peaking, resulting in challenges to fuel integrity. The RAOC methodology generates a range of AFD shapes at different power levels. Those shapes are used in LOCA analysis, loss-of-flow accident analysis, and anticipated operational occurrence analysis. Unacceptable shapes are eliminated by each subsequent step. The remaining AFD shapes constitute a power dependent AFD profile that is used to operate the reactor. As noted previously, the RAOC methodology identifies margin in the analysis and transfers it to operational flexibility. In References 1 and 2, the licensee demonstrated and affirmed its ability to implement the WCAP-10216-P-A, RAOC methodology.

Part B describes an alternate means for monitoring the heat flux hot-channel factor (F_Q). The WCAP-10216-P-A methodology measures F_Q directly under equilibrium conditions. The methodology then increases the measured F_Q to account for manufacturing tolerance and measurement uncertainties. A further increase is made to account for the effect of normal operating transients. This final F_Q is compared to TS limits to determine whether this parameter is out of specification. In References 1 and 2, the licensee demonstrated and affirmed its ability to implement the WCAP-10216-P-A, F_Q surveillance methodology.

In References 1 and 2, the licensee stated the supporting analysis assumes Best-Estimate LOCA (BELOCA) methodology and extended power uprate (EPU) conditions. These are the subject of separate license amendment requests (LARs), References 5 and 6, respectively. The BELOCA methodology LAR was approved on February 6, 2006. The EPU LAR is still under review by the NRC staff. While the use of WCAP-10216-P-A is not predicated on a particular reactor rated thermal power, plant configuration, or LOCA methodology, changes thereto could alter the acceptable AFD profile results. The licensee affirmed that the reload methodology in use at BVPS-1 and 2, (WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," (Reference 7)) would verify the use of the AFD profile and identify any non-bounding conditions. Therefore, the NRC staff has reasonable assurance that the WCAP-10216-P-A, (relaxation of constant axial offset control and F_Q surveillance TS) methodologies will be implemented appropriately.

When implementing the WCAP-10216-P-A methodologies there are several changes to the TSs that must be made. The licensee has described those changes and they are consistent with the WCAP-10216-P-A recommended TSs and the Westinghouse Standard Technical Specifications (STS) in NUREG-1431 (Reference 8). Therefore, the proposed changes are acceptable. However, the licensee has also requested two TS changes that are not a direct byproduct of incorporating WCAP-10216-P-A. Those changes are addressed separately below.

The licensee's current TS 3.2.2 Action a., requires the Power Range Neutron Flux-High Trip Setpoints be reduced within 4 hours of a power reduction due to the Heat Flux Hot Channel Factor (F_Q) being out of limits. The licensee has requested that the time required to reduce the Power Range Neutron Flux-High Trip Setpoints be extended from 4 hours to 72 hours. The rationale for this change is consistent with Technical Specification Task Force (TSTF)-95, "Revise completion time for reducing Power Range High Trip Setpoint from 8 to 72 hours," (Reference 9). TSTF-95 extended the completion time for changing the Power Range Neutron Flux-High Trip Setpoints when the setpoint changes were associated with power reductions

directed by STSs 3.2.1, "Heat Flux Hot Channel Factor (F_Q)" and 3.2.2, "Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}$)" (Note: The licensee did not reference TSTF-95 in its LAR). BVPS-1 and 2 have a Power Range Neutron Flux, High Positive Rate Trip to provide protection against a large positive reactivity addition event during the proposed extended time to reduce the power range neutron flux-high trip setpoints. Therefore, the NRC staff has reasonable assurance that the extended time will not adversely affect safety margin.

The license proposed an editorial change to TS 3.2.3, "Nuclear Enthalpy Hot Channel Factor ($F_{\Delta H}$)."

 The change is purely editorial and is, therefore, acceptable.

Based on the above evaluations, the NRC staff determined that the licensee's request to implement WCAP-10216-P-A and associated byproduct TS changes, the extension of time requested by changing the Power Range Neutron Flux-High Trip Setpoints from 4 hours to 72 hours, and the editorial change to TS 3.2.3 do not reduce any margins of safety and the revised TSs continue to meet NRC regulatory requirements.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (70 FR 21457). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. FirstEnergy Nuclear Operating Company (FENOC), letter L-05-009 dated February 11, 2005, from L. William Pearce, Site Vice President to USNRC, re: Beaver Valley Power Station, Unit Nos. 1 and 2, BV-1 Docket No. 50-334, License No. DPR-66, BV-2 Docket No. 50-412, License No. NPF-73, License Amendment Request Nos. 310 and 182 (ADAMS Accession no. ML050480264).
2. FirstEnergy Nuclear Operating Company (FENOC), letter L-05-133 dated August 8, 2005, from L. William Pearce, Site Vice President to USNRC, re: Beaver Valley Power Station, Unit Nos. 1 and 2, BV-1 Docket No. 50-334, License No. DPR-66, BV-2 Docket No. 50-412, License No. NPF-73, Supplement to License Amendment Request Nos. 310 and 182 (ADAMS Accession No. ML052220351).
3. WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control/ F_Q Surveillance Technical Specification," Revision 1, February 1994.
4. Letter from Ashok C. Thadani, Director, Division of Systems Safety and Analysis, U.S. Nuclear Regulatory Commission to Nicholas J. Liparulo, Manager, Nuclear Safety and Regulatory Activities, Westinghouse Electric Corporation, "Acceptance for Referencing of Revised Version of Licensing Topical Report WCAP-10216-P-A, Rev. 1, "Relaxation of Constant Axial Offset Control - F_Q Surveillance Technical Specification," (TAC No. M88206), dated November 26, 1993.
5. FirstEnergy Nuclear Operating Company (FENOC), letter L-04-124 dated October 4, 2004 from L. William Pearce, Site Vice President to USNRC, re: Beaver Valley Power Station, Unit Nos. 1 and 2, BV-1 Docket No. 50-334, License No. DPR-66 BV-2 Docket No. 50-412, License No. NPF-73 License Amendment Request Nos. 318 and 191 (ADAMS Accession No. ML042860374).
6. FirstEnergy Nuclear Operating Company (FENOC), letter L-04-125 dated October 4, 2004 from L. William Pearce, Site Vice President to USNRC, re: Beaver Valley Power Station, Unit Nos. 1 and 2, BV-1 Docket No. 50-334, License No. DPR-66 BV-2 Docket No. 50-412, License No. NPF-73 License Amendment Request Nos. 302 and 173 (ADAMS Accession No. ML042920300).
7. WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," July 1985.
8. NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 3.0.
9. TSTF-95, "Revise completion time for reducing Power Range High trip setpoint from 8 to 72 hours," Revision 0, September 18, 1996.

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Date: February 27, 2006

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