

February 7, 2006

Mr. R. T. Ridenoure  
Vice President - Chief Nuclear Officer  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
Post Office Box 550  
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT 1 - REQUEST FOR ADDITIONAL  
INFORMATION RELATED TO LICENSE AMENDMENT REQUEST FOR  
UPDATED SAFETY ANALYSIS REPORT CLARIFICATION OF OPERATOR  
ACTION DURING LOSS OF MAIN FEEDWATER EVENT (TAC NO. MC7524)

Dear Mr. Ridenoure:

By letter dated July 1, 2005, Omaha Public Power District (OPPD) submitted a license amendment request for the Fort Calhoun Station, Unit 1. The license amendment proposed to revise the Updated Safety Analysis Report (USAR), Section 14.10, to credit operator action during the Loss of Main Feedwater Event. The proposed change will add a description of an existing Emergency Operating Procedure operator action to isolate steam generator blowdown within 15 minutes of reactor trip during a loss of main feedwater event to the USAR.

The Nuclear Regulatory Commission (NRC) staff has reviewed OPPD's submittal and had a teleconference on January 13, 2006, with OPPD to discuss this submittal. The NRC staff has determined that additional information is needed to complete our review. A request for additional information (RAI) is enclosed. This request was discussed with Thomas Matthews of your staff on January 13, 2006, and it was agreed that a response would be provided within 14 days of receipt of this letter.

If you have any questions, please contact me at (301) 415-1445.

Sincerely,

**/RAI/**

Alan B. Wang, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION  
REGARDING UPDATED SAFETY ANALYSIS REPORT (USAR) CLARIFICATION  
OMAHA PUBLIC POWER DISTRICT  
FORT CALHOUN STATION  
DOCKET NO. 50-285

By letter dated July 1, 2005, Omaha Public Power District (OPPD), the licensee for the Fort Calhoun Station, Unit 1 (FCS), requested a license amendment to permit FCS to revise the USAR, Section 14.10, to credit operator action during the Loss of Main Feedwater (LMFW) Event. The proposed change will add a description of an existing Emergency Operating Procedure operator action to isolate steam generator blowdown within 15 minutes of reactor trip during a loss of main feedwater event to the USAR. The operator action is intended to assist the auxiliary feedwater system in performing its design function of maintaining adequate steam generator (SG) water level for decay heat removal once the auxiliary feedwater actuation signal is actuated. OPPD requests approval of the proposed change to support plant operations following the 2006 outage during which the steam generators, pressurizer, and reactor vessel head will be replaced.

The Nuclear Regulatory Commission staff has performed a preliminary review of the human performance associated changes in the license amendment request. The licensee's responses to the following request for additional information with regard to the human performance aspects of the license amendment are needed for the staff to complete its review.

1. What actions for the two discrete manipulations are performed by the operator for this LMFW event? Do the actions include the operator closing the SG blowdown isolation valves by means of control switches in the control room? Are there any actions taken locally that must be completed within the 8 minute time limit?
2. What are the other operator manual actions, if any, to be performed during the LMFW event prior to isolating the SG blowdown valves? How were these actions considered in the facility's conclusion that "FCS operators complete this action [manual isolation of SG blowdown] within 8 minutes following reactor trip"?
3. How was the simulator modeled to accurately attain a plant response to an LMFW event? Are the 8 minutes for completion of the operator actions and subsequent system response on the "simulator training observations" representative of an actual LMFW event, including operating crew response?
4. How was the training conducted to capture the 8 minutes completion time of manual actions after the reactor trip for the LMFW event? What was the composition of the crews that performed this exercise? Were these evaluation scenarios run by crews with no prior knowledge of the simulated LMFW event(s) or were the training scenarios run with crews knowledgeable regarding the LMFW training scenario event(s)? Did the simulator training observation scenarios that resulted in SG blowdown isolation "within

8 minutes following reactor trip” involve loss of AC power as the initiating event for the LMFWR event? Please describe the factors that were used to provide a worst case scenario in obtaining the 8 minute mark for being able to generalize that all crews can perform the required actions in the available time. Additionally, what other factors or hindrances to the operators could be in place to possibly extend this 8 minute completion time up to 15 minutes?

Ft. Calhoun Station, Unit 1

cc:

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January 2006