

September 19, 2006

Mr. Michael R. Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: NRC RECEIPT OF INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2
AND 3 RESPONSE TO NRC GENERIC LETTER 2003-01, "CONTROL ROOM
HABITABILITY" (TAC NOS. MB9813 AND MB9814)

Dear Mr. Kansler:

Entergy has provided the following responses to Nuclear Regulatory Commission (NRC)
Generic Letter (GL) 2003-01:

- 60-day response for Unit 3 - Entergy letter NL-03-129, dated August 6, 2003, Agencywide Documents Access and Management System (ADAMS) Accession No. ML032260036).
- 180-day response for Unit 2 - Entergy Letter NL-03-181, dated November 25, 2003, ADAMS Accession No. ML033350400.
- Revised schedule for Unit 3 - Entergy letter NL-04-118, dated September 20, 2004, ADAMS Accession No. ML042710270.
- Supplemental 60-day response for Unit 3 - Entergy Letter NL-05-085, dated June 28, 2005, ADAMS Accession No. ML051890070.

This letter provides a status of Entergy's response and describes any actions that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that Entergy confirm that the control rooms meet their design bases (e.g., General Design Criterion (GDC) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments; and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2).

Entergy reported the results of ASTM E741 tracer gas tests for Indian Point Nuclear Generating Unit Nos. 2 and 3 (IP2 and IP3) control rooms, which are pressurized for accident mitigation. You determined that the maximum tested value for inleakage into the Control Room Envelope (CRE) for IP2 was 589 (+/- 69) standard cubic feet per minute (scfm), which does not exceed the value of 700 scfm assumed in the design basis radiological analyses for IP2 control room habitability. You determined that the maximum tested value for inleakage into the CRE for IP3

was 88 (+/- 33) scfm, which does not exceed the value of 400 scfm assumed in the design basis radiological analyses for IP3 control room habitability.

Entergy indicated that the tracer gas test performed for hazardous chemicals resulted in a maximum inleakage of 1251 (+/- 43) scfm for the IP2 CRE and 301(+/- 17) actual cubic feet per minute for the IP3 CRE, which is less than the value of 1300 scfm assumed in your design basis analyses for hazardous chemical releases. Entergy also indicated that reactor control capability is maintained from either the control rooms or various local control stations in the event of smoke.

The GL further requested that you assess your Technical Specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for control room habitability, in light of the demonstrated inadequacy of a differential pressure measurement to alone provide such verification (GL Item 1.c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TSs to reference an acceptable surveillance methodology. In your November 25, 2003, response you stated you would submit a license amendment request (LAR) for proposed changes to your TSs based on the Technical Specification Task Force (TSTF) document TSTF-448 including a new surveillance requirement for periodic verification of unfiltered inleakage. You committed to submit proposed changes to the TSs within 9 months of NRC approval of TSTF-448.

The information you provided indicated there are no compensatory measures currently in place to demonstrate control room habitability and also supported the conclusion that you are committed to meet the GDC regarding control room habitability as documented in your Updated Final Safety Analysis Reports.

Your commitment to submit LARs based on TSTF-448 following our formal review and approval of TSTF-448 is acceptable for purposes of closing out your response to GL 2003-01. The NRC staff will monitor submission of the LARs and interact with you as necessary during the amendment process.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

cc: See next page

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Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

cc: See next page

ADAMS ACCESSION NUMBER: ML053490104

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DATE	9/08/06	9/11/06	9/11/06	9/18/06	9/19/06

OFFICIAL RECORD COPY

DATED: September 19, 2006

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LPL1-1 R/F

RidsNrrDorlLpla

RidsOGCMailCenter

RidsNrrDssScvb

RidsNrrDprPgcb

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