



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

37-00497-15

March 10, 1999

Mr. A. Joseph Nardi
Supervisor Engineer, Regulatory Affairs
Westinghouse Electric Company, a division of CBS
Corporation
Box 355
Pittsburgh, Pennsylvania 15230-0355

SUBJECT: APPLICATION TO TRANSFER AND AMEND WESTINGHOUSE MATERIALS
LICENSES, QUALITY ASSURANCE PROGRAM APPROVALS, AND
CERTIFICATES OF COMPLIANCE (TAC NO. L31120)

Dear Mr. Nardi:

In accordance with your applications dated September 28, 1998; January 18, 1999; and February 22, 1999, and pursuant to Parts 30, 40, 70, 71, and 72 to Title 10 of the Code of Federal Regulations, enclosed are authorizing documents (i.e., materials licenses, quality assurance program approvals, and certificates of compliance) which are hereby amended to accommodate the transfer and/or the change of name of the Licensee from "Westinghouse Electric Company, a division of CBS Corporation" to "Westinghouse Electric Company LLC (WELCO)", or "CBS Corporation", or "Westinghouse Government Services Company LLC (WGS)", as appropriate.

The staff's review of the applications was conducted pursuant to requirements in section 184 of the Atomic Energy Act (AEA) and 10 CFR 30.34(b), 40.46, and 70.36 to ensure there would be no adverse impact on the public health and safety or common defense and security as a result of the change of control of Westinghouse Electric Company's ownership.

NRC will hold the licensees responsible for all requirements and conditions of their respective licenses, including financial responsibility for decommissioning. CBS will continue to be the licensee, to retain responsibility for, and to retain financial responsibility for decommissioning and/or decontaminating activities under materials licenses 37-00497-15 (Forest Hills Site) and SMB-1527 (Former Lamp Manufacturing Facility). Based upon CBS' September 28, 1998, letter from Louis Briskman, Executive Vice President and General Counsel, included as part of the applications, NRC recognizes that CBS has a contractual agreement with the buyers to retain financial responsibility for decommissioning and/or decontaminating certain facilities associated with licenses SNM-770 (Waltz Mill Site) and SNM-1460 (Science and Technology Center). Insofar as CBS will remain, under the contractual agreement with the buyers, an active participant in the decommissioning and/or decontamination activities of those facilities, NRC will notify CBS, as well as the licensee, on matters related to decommissioning and/or decontamination under those two licenses.

Each authorizing document has been revised to include the effective date based upon the closing date of the transfer of assets and amended to accommodate the transfer and/or name change of the licensee from "Westinghouse Electric Company, a division of CBS Corporation" to "Westinghouse Electric Company LLC (WELCO)", or "CBS Corporation", or "Westinghouse Government Services Company LLC (WGS)", as appropriate.

Since the effective date of each authorizing document is the date of closing of the assets transfer, that date must be formally communicated to the Nuclear Regulatory Commission within thirty (30) days of the date of this letter and will be incorporated by reference into each license. While not a condition of the license, it is NRC's expectation that the parties will notify NRC of the closing within 24 hours of its occurrence. Within this same thirty (30)-day period the licensees for the respective licenses will file all required changes to decommissioning financial assurance instruments to reflect the change of control.

All other conditions of these authorizing documents shall remain the same. It should be noted that no physical license amendment was required to effectuate the transfer of license number COC-1001 (Irradiated Fuel Storage Cask - MC-10).

Based on the information provided, the staff has determined that the requested transfer and name change amendments are acceptable, are in accordance with requirements in section 184 of the AEA and 10 CFR 30.34(b), 40.46, and 70.36, and would not be inimical to the common defense and security or to the health and safety of the public. Change of control of the licensee does not change the license. There will be no changes in the current license conditions covering the existing health and safety program; professional qualifications of personnel, including safety personnel; equipment and facilities; or any other existing license requirements and conditions of the license.

Attached are copies of the revised authorizing documents and the Safety Evaluation Report, which includes the Categorical Exclusion determination.

Sincerely,

Original signed by:

Charles W. Emeigh, Acting Chief
Licensing Branch
Division of Fuel Cycle Safety
and Safeguards, NMSS

Enclosures: 1. Listing of Materials Licenses, QA Approvals, and Certificates of Compliance
2. Safety Evaluation Report
3. Revised Materials Licenses, QA approvals, and Certificates of Compliance

Distribution: w/encls. (Control No. 930S) [COMPLETE]

Docket 70-1151	PUBLIC	NRC File Center	Region I	WSchwink
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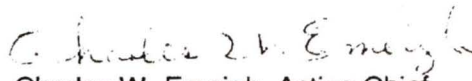
Since the effective date of each authorizing document is the date of closing of the assets transfer, that date must be formally communicated to the Nuclear Regulatory Commission within thirty (30) days of the date of this letter and will be incorporated by reference into each license. While not a condition of the license, it is NRC's expectation that the parties will notify NRC of the closing within 24 hours of its occurrence. Within this same thirty (30)-day period the licensees for the respective licenses will file all required changes to decommissioning financial assurance instruments to reflect the change of control.

All other conditions of these authorizing documents shall remain the same. It should be noted that no physical license amendment was required to effectuate the transfer of license number COC-1001 (Irradiated Fuel Storage Cask - MC-10).

Based on the information provided, the staff has determined that the requested transfer and name change amendments are acceptable, are in accordance with requirements in section 184 of the AEA and 10 CFR 30.34(b), 40.46, and 70.36, and would not be inimical to the common defense and security or to the health and safety of the public. Change of control of the licensee does not change the license. There will be no changes in the current license conditions covering the existing health and safety program; professional qualifications of personnel, including safety personnel; equipment and facilities; or any other existing license requirements and conditions of the license.

Attached are copies of the revised authorizing documents and the Safety Evaluation Report, which includes the Categorical Exclusion determination.

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Charles W. Emeigh, Acting Chief
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Enclosures: 1. Listing of Materials Licenses, QA Approvals, and Certificates of Compliance
2. Safety Evaluation Report
3. Revised Materials Licenses, QA approvals, and Certificates of Compliance

**ENCLOSURE 1. LISTING OF MATERIALS LICENSES, QA APPROVALS,
AND CERTIFICATES OF COMPLIANCE**

- (1) License Number 37-00497-15, Forest Hills Site, will be changed to read "CBS Corporation". In the same application, License Number SMB-1527, Former Lamp Manufacturing Facility, will also be changed to read "CBS Corporation".
- (2) License Number SNM-1151, Columbia Fuel Fabrication Facility in Columbia, SC, will be changed to read, "Westinghouse Electric Company LLC (WELCO)".
- (3) License Number SNM-770, Waltz Mill site, will be changed to read, "Westinghouse Electric Company LLC (WELCO)". In the same application, License Number SNM-1460, Science & Technology Center, will also be changed to read "Westinghouse Electric Company LLC (WELCO)".
- (4) License Number 37-05809-01, Pump Repair Facility, Cheswick, PA, will be changed to read "Westinghouse Government Services Company LLC (WGS)". In the same application, License Number 37-05809-02, Industrial Radiography Facility, Cheswick, PA, will also be changed to read "Westinghouse Government Services Company LLC (WGS)". In the same application, License Number SNM-1120, former Westinghouse Plutonium Operations, will also be changed to read "Westinghouse Government Services Company LLC (WGS)".
- (5) The names of the organizations for the following QA Approvals and Certificates of Compliance will be changed to read "Westinghouse Electric Company LLC (WELCO)":
 - (a) Transportation Quality Assurance Approval, License Number QAA-0638;
 - (b) Transportation Quality Assurance Approval, License Number QAA-0708;
 - (c) Super Tiger, Model No. 6400, License Number COC-6400;
 - (d) New Fuel Shipping Container, MCC Series, License Number COC-9239; and
 - (e) Irradiated Fuel Storage Cask - MC-10, License Number COC-1001.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 10, 1999

ENCLOSURE 2. SAFETY EVALUATION REPORT

DOCKET 70-1151

LICENSEE: Westinghouse Electric Company, a division of CBS Corporation

SUBJECT: SAFETY EVALUATION REPORT: APPLICATION TO TRANSFER AND AMEND
WESTINGHOUSE MATERIALS LICENSES, QUALITY ASSURANCE
PROGRAM APPROVALS AND CERTIFICATES OF COMPLIANCE
(TAC NO. L31120)

BACKGROUND

By letters dated September 28, 1998; January 18, 1999; and February 22, 1999; Westinghouse Electric Company, a division of CBS Corporation, submitted applications for the transfer and/or amendment of each of its authorizing documents (i.e., materials licenses, quality assurance approvals, and certificates of compliance) to change the name of the Licensee from "Westinghouse Electric Company, a division of CBS Corporation" to "Westinghouse Electric Company LLC (WELCO)", or "CBS Corporation", or "Westinghouse Government Services Company LLC (WGS)", as appropriate.

The staff's review of the applications was conducted pursuant to requirements in section 184 of the Atomic Energy Act and 10 CFR 30.34(b), 40.46, and 70.36 to ensure there would be no adverse impact on the public health and safety or common defense and security as a result of the change of control of Westinghouse Electric Company's ownership.

The staff also performed a financial review to determine whether the proposed change in Westinghouse Electric Company's ownership will affect its financial resources for protecting the public health and safety, including financial assurance for decommissioning, and the common defense and security. CBS provided draft instruments in its applications dated September 28, 1998, and January 18, 1999. The review concluded that the proposed change of control will not significantly change the assurance that funds for decommissioning will be available, and protection of public health and safety thereby will not be adversely affected.

DISCUSSION

CBS is selling, with certain exceptions, the assets of its nuclear and government operations business to a consortium of Morrison Knudsen Corp. (MK) and BNFL USA Group, Inc. (BNFL USA). MK is a U.S. corporation; BNFL USA is a wholly-owned subsidiary of British Nuclear Fuels plc (BNFL), which is wholly owned by the government of the United Kingdom. BNFL USA is incorporated under the laws of Delaware. The MK-BNFL USA consortium has formed three limited liability companies to operate the businesses and assets to be transferred as part of the sale. The proposed transaction would transfer authorizing documents to two of the three companies, WGS and WELCO.

WGS will be 100% owned by MK. BNFL Nuclear Services, Inc., a wholly-owned subsidiary of BNFL USA, will have a passive contractual right to 40% of the profits and losses of WGS. WGS is U.S.-owned and -directed. WELCO will be wholly-owned by BNFL Nuclear Services,

Inc. Four of its five directors will be citizens of the United Kingdom. MK has no economic interest in WELCO. In its September 28, 1998, application, CBS stated that BNFL will not have access to Restricted Data, classified information, or sensitive nuclear technology. In addition, the United Kingdom is a signatory to the Nuclear Nonproliferation Treaty and is party to an Agreement for Cooperation with the United States.

The combined requests for transfer and amendment stipulated that the nuclear business of Westinghouse Electric Company, a division of CBS Corporation, that operates under each authorizing document would remain intact and continue with the re-named corporate entity. There is no change in the management, organization, location, facilities, equipment, or procedures related to or personnel responsible for the licensed activities. All existing commitments, obligations and representations remain in effect. Where required, each authorizing document, as appropriate, contains a condition regarding the necessity for financial assurance. NRC will hold the licensee responsible for all requirements and conditions of its license, including financial responsibility for decommissioning. CBS will continue to be the licensee, to retain responsibility for, and to retain financial responsibility for decommissioning and/or decontaminating activities under materials licenses 37-00497-15 (Forest Hills Site) and SMB-1527 (Former Lamp Manufacturing Facility). Based upon CBS' September 28, 1998, letter from Louis Briskman, Executive Vice President and General Counsel, included as part of the applications, NRC recognizes that CBS has a contractual agreement with the buyers to retain financial responsibility for decommissioning and/or decontaminating certain facilities associated with licenses SNM-770 (Waltz Mill Site) and SNM-1460 (Science and Technology Center). Insofar as CBS will remain, under the contractual agreement with the buyers, an active participant in the decommissioning and decontamination activities under these licenses, NRC will notify CBS, as well as the licensee, on matters related to decontamination and decommissioning under those two licenses.

Further, each authorizing document requires an official notification of closing which will be incorporated by reference into each license.

ENVIRONMENTAL REVIEW

These changes are considered administrative in nature. The staff has determined that the proposed changes do not adversely affect public health and safety or the environment and are categorically excluded from the requirement to prepare a site-specific environmental assessment. Therefore, in accordance with 10 CFR 51.22(c)(11), neither an environmental assessment nor an environmental impact statement is warranted for this action.

CONCLUSION

Based on the above, the staff has determined that the requested transfer and name change amendments are acceptable, are in accordance with requirements in section 184 of the Atomic Energy Act and 10 CFR 30.34(b), 40.46, and 70.36, and would not be inimical to the common defense and security or to the health and safety of the public. Change of control of the licensee does not change the license. The proposed change in control is effected by a change in Westinghouse Electric Company's ownership through a transfer of assets. There will be no changes in the current license conditions covering the existing health and safety program; professional qualifications of personnel, including safety personnel; equipment and facilities; or any other existing license requirements and conditions of the license.

The Region I and II project inspectors have no objection to the proposed actions.

Principal Contributors

Charles Gaskin

Harry Felsher

Handwritten signature
1/1/99

**ENCLOSURE 3. REVISED MATERIALS LICENSES, QA APPROVALS,
AND CERTIFICATES OF COMPLIANCE**

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License Number SNM-1151, Columbia Fuel Fabrication Facility	3
License Number SNM-770, Waltz Mill Site	4
License Number SNM-1460, Science & Technology Center	5
License Number 37-05809-01, Pump Repair Facility, Cheswick, PA	6
License Number 37-05809-02, Industrial Radiography Facility, Cheswick, PA	7
License Number SNM-1120, former Westinghouse Plutonium Operations	8
Transportation Quality Assurance Approval, License Number QAA-0638	9
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134

030-06135

030-32414

070-00698

070-01143

070-01503

040-08976

License Nos. 37-05809-01

37-05809-02

37-00497-15

SNM-770

SNM-1120

SNM-1460

SMB-1527

Control Nos. 126122

126123

126133

126125

126124

126126

126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

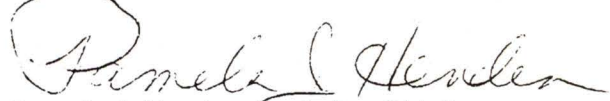
Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely,

A handwritten signature in cursive script, reading "Pamela J. Henderson". The signature is written in dark ink and is positioned above the printed name and title.

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

ATTACHMENT 1

License Number, 37-00497-15

Forest Hills Site



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406 1415

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Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

CORRECTED COPY

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee		In accordance with the letter dated September 28, 1998,
1. CBS Corporation		3. License number 37-00497-15 is amended in its entirety to read as follows:
2. Energy Systems P. O. Box 355 Pittsburgh, Pennsylvania 15230-0355		4. Expiration date November 30, 2001
		5. Docket No. 030-32414 Reference No.
6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A. As specified in Section 33.100, Schedule A, of 10 CFR 33 (Type C Broad License)	A. Any	A. See Condition 12
B. Cesium-137	B. Sealed sources (Amersham Searle Model X-60 or equivalent)	B. Not to exceed [redacted] per source and [redacted] total
C. Cesium-137	C. Sealed source (Nuclear Chicago Model RR-137)	C. [redacted]
D. Plutonium-239	D. Sealed source (U.S. Nuclear Corp. Model 383)	D. [redacted]
E. Americium-241	E. Sealed Neutron sources (Monsanto Research 2700 Series)	E. Not to exceed [redacted] per source and [redacted] total
F. Any byproduct material with Atomic Numbers 1 through 83	F. Contamination	F. [redacted]
G. Any byproduct material with Atomic Numbers 84 through 102	G. Contamination	G. [redacted]
H. Uranium-235	H. Contamination	H. [redacted]
I. Thorium	I. Contamination	I. [redacted]

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
37-00497-15Docket or Reference Number
030-32414

Amendment No. 03

CORRECTED COPY

- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen-3; or
 - (ii) they contain only a radioactive gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or
 - (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source shall be removed from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within five days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Director, Division of Nuclear Materials Safety, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source involved, the test results, and corrective action taken.
- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
15. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders or detector cells by the licensee.
16. The licensee shall conduct a physical inventory every six months to account for all sealed sources and devices containing licensed material received and possessed under the license.
17. The licensee may transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

**MATERIALS LICENSE
SUPPLEMENTARY SHEET****CORRECTED COPY**

License Number

37-00497-15

Docket or Reference Number

030-32414

Amendment No. 03

18. Notwithstanding the provisions of 10 CFR 20.1401 (a), the Forest Hills Site (except for the area in Building B designated as the "pit"), may be decommissioned in accordance with the SDMP Action Plan criteria specified in 10 CFR 20.1401 (b) (3).
19. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
20. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee
21. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
22. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Letter dated July 1, 1991
 - B. Letter dated July 12, 1991
 - C. Letter dated October 2, 1991
 - D. Letter dated April 10, 1995
 - E. Letter dated August 28, 1998
 - F. Letter dated September 28, 1998
 - G. Letter dated November 24, 1998

For the U.S. Nuclear Regulatory Commission

Original signed by Pamela J. HendersonDate March 8, 1999

By

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

82036313

ATTACHMENT 2

License Number, SMB-1527

Former Lamp Manufacturing Facility



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134

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Executive Vice President
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51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

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Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

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Thank you for your cooperation.

Sincerely,

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Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

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2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	In accordance with the application dated September 28, 1998,	
1. CBS Corporation	3. License number SMB-1527 is amended in its entirety to read as follows:	
2. P. O. Box 355 Pittsburgh, Pennsylvania 15230	4. Expiration date February 28, 1994 (Extended)	
	5. Docket No. 040-08976 Reference No.	
6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A. Natural Uranium	A. Residual contamination	A. [REDACTED]
B. Natural Thorium	B. Residual contamination	B. [REDACTED]
9. Authorized use:		
A. through B. Decontamination of equipment, facilities and land and the packaging of licensed material.		

CONDITIONS

10. Licensed material may be used only at the portion of the licensee's facilities at
1 Westinghouse Plaza, Bloomfield, New Jersey 07004, east of Arlington Avenue.
11. A. Licensed material shall be used by, or under the supervision of, C. W. Bickerstaff.
B. The Radiation Safety Officer for this license is C. W. Bickerstaff.
12. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
13. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SMB-1527

Docket or Reference Number

040-08976

Amendment No. 03

14. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
15. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Application dated November 4, 1988
 - B. Letter dated May 25, 1990
 - C. Letter dated September 28, 1998
 - D. Letter dated November 24, 1998

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By



Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81365571

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	In accordance with the application dated September 28, 1998, 3. License number SMB-1527 is amended in its entirety to read as follows:
1. CBS Corporation	
2. P. O. Box 355 Pittsburgh, Pennsylvania 15230	4. Expiration date February 28, 1994 (Extended) 5. Docket No. 040-08976 Reference No.

- | | | |
|---|----------------------------------|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| A. Natural Uranium | A. Residual contamination | A. <input type="text"/> |
| B. Natural Thorium | B. Residual contamination | B. <input type="text"/> |

9. Authorized use:

A. through B. Decontamination of equipment, facilities and land and the packaging of licensed material.

CONDITIONS

10. Licensed material may be used only at the portion of the licensee's facilities at 1 Westinghouse Plaza, Bloomfield, New Jersey 07004, east of Arlington Avenue.
11. A. Licensed material shall be used by, or under the supervision of, C. W. Bickerstaff.
B. The Radiation Safety Officer for this license is C. W. Bickerstaff.
12. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
13. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SMB-1527

Docket or Reference Number

040-08976

Amendment No. 03

14. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
15. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Application dated November 4, 1988
 - B. Letter dated May 25, 1990
 - C. Letter dated September 28, 1998
 - D. Letter dated November 24, 1998

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By

Original signed by Pamela J. Henderson

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81365571

ATTACHMENT 3

License Number, SNM-1151

Columbia Fuel Fabrication Facility

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Westinghouse Electric Company LLC (WELCO)	3. License Number SNM-1107, Amendment 18
2. P.O. Box 355 Pittsburgh, Pennsylvania 15230-0355	4. Expiration Date November 30, 2005
	5. Docket No. 70-1151 Reference No.

6. Byproduct Source, and/or
Special Nuclear Material7. Chemical and/or Physical
Form8. Maximum amount that Licensee
May Possess at Any One Time
Under This License

A. U-235

A. Any

A. [REDACTED]

B. U-235

B. Any, except
metal, enriched
to not more
than 5.0 w/o

B. [REDACTED]

C. U-233

C. Any

C. [REDACTED]

D. Pu-238,
Pu-239

D. Sealed sources

D. [REDACTED]

E. Plutonium

E. Feedstock with
transuranics and
fission products

E. [REDACTED]

9. Authorized place of use: The licensee's existing facilities at Columbia, South Carolina.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1107

Docket or Reference Number

70-1151

Amendment No. 18

10. This license shall be deemed to contain two sections: Safety Conditions and Safeguards Conditions. These sections are part of the license and the licensee is subject to compliance with all listed conditions in each section.
11. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.

FOR THE NUCLEAR REGULATORY COMMISSION

Date: Date of closing*

Charles W. Emeigh
By: Charles W. Emeigh, Acting Branch Chief
Division of Fuel Cycle Safety
and Safeguards
Washington, DC 20555

* To be provided to the NRC Office Director by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
SNM-1107Docket or Reference Number
70-1151

Amendment No. 18

SAFETY CONDITIONS

- S-1. Authorized use: For use in accordance with statements, representations, and conditions in the license application dated April 30, 1995, and supplements dated August 4 and 25, and September 25, 1995; November 8, and August 30, 1996; July 14 and 25, November 17, 1997; and name change amendment December 22, 1997; and April 3, June 30, July 13, July 23, October 21, and October 30, 1998; and February 12, 1999; and name change amendment September 28, 1998; January 18, and February 22, 1999.
- S-2 Criticality Safety Evaluations (CSEs) and Criticality Safety Analyses (CSAs) will define the interim criticality safety bases utilized throughout the CFFF. All CSEs/ CSAs will be upgraded and/or completed in accordance with all applicable commitments in Chapter 6.0 of the License Application and all other regulatory requirements. Summaries of the CSEs/CSAs (in the format of License Annexes) will be submitted to NRC for review and approval. Proprietary versions containing Sections 1 and 5 of the CSEs/CSAs for UN Tanks, ADU Conversion, URRS Dissolver, and Powder Blending (ADU) will also be provided at the same time as the License Annexes for those systems. All completed CSEs/CSAs will be independently peer-reviewed in accordance with all applicable regulatory requirements and related procedures. Configuration control data packages for ongoing changes to facility structures, systems and components, and controls will be filed with their respective CSEs/CSAs to provide a substantially complete "living" framework for system Integrated Safety Assessments (ISAs) that will ultimately become the Final CFFF Design Safety Basis described in Chapter 4.0 of the License Application. CFFF will provide the License Annexes to the NRC using the following schedule:

<u>COMPLETION DATE (Calendar Year)</u>	<u>PROCESS SYSTEM</u>
2Q98	UN Tanks*
3Q98	ADU Conversion* Rods (ADU & IFBA) UF6 Cylinder Washing
4Q98	ADU Pelleting URRS Dissolver* Powder Blending (ADU)*
1Q99	Low-Level Waste Processing Hoods & Containment
2Q99	URRS Scrap Processing Solvent Extraction URRS Waste Treatment
3Q99	Final Assembly Storage IFBA (Excluding Rods) Laboratories

* Will be done as ISAs ("Final Design Safety Basis")

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

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Docket or Reference Number

70-1151

Amendment No. 18

- S-3. The licensee shall maintain and execute the response measures in the Site Emergency Plan, dated April 30, 1990, and revisions dated March 31, and September 30, 1992; March 25, August 15, and September 30, 1994; January 9, February 17, August 17, and October 23, 1995; or as further revised by the licensee consistent with 10 CFR 70.32(i).
- S-4. Deleted by Amendment 12, April 1998.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1107

Docket or Reference Number

70-1151

Amendment No. 18

SAFEGUARDS CONDITIONS**SECTION 1.0 -- MATERIAL CONTROL AND ACCOUNTING**

- SG-1.1 The licensee shall follow pages i through xviii and Chapters 1.0 through 9.0 of its "Fundamental Nuclear Material Control Plan for the Columbia Fuel Fabrication Facility," which has been partially revised as indicated by Revision 28 (dated December 18, 1997). Any further revisions to this Plan shall be made only in accordance with, and pursuant to, either 10 CFR 70.32(c) or 70.34.
- SG-1.2 Operations involving special nuclear material which are not referenced in the Plan identified in Condition SG-1.1 shall not be initiated until an appropriate safeguards plan has been approved by the Nuclear Regulatory Commission.
- SG-1.3 In lieu of the requirements contained in 10 CFR 74.13(a)(1) and (a)(2) to use the Forms DOE/NRC-742 and 742C, the licensee may use computer generated forms provided all information required by the latest printed instructions for completing the particular form is included.
- SG-1.4 In lieu of the requirements contained in CFR 70.54 and 74.15 to use the DOE/NRC Form-741, the licensee may use computer generated forms provided all information required by the latest printed instructions for completing the particular form is included.
- SG-1.5 Deleted Per Amendment 3, August 1996 Commitment now contained in licensee's Fundamental Nuclear Material Control Plan.
- SG-1.6 Notwithstanding the requirements of the FNMC Plan identified in License Condition SG-1.1, the licensee may use (1) a single standard for measurement control (including daily control limit monitoring and bias corrections) for any linear-response tube or rod scales, in any initially demonstrated to be linear over its range of use within the discrimination of the scale by calculating a bias at four levels across the range of use and demonstrating that the four results are not statistically different, and (2) that the continued linearity of response of the scales is verified by monthly calibration against at least four traceable standards covering the range of use.
- SG-1.7 Notwithstanding the requirements contained in Sections 5.2.2 and 5.2.3 of the licensee's Fundamental Nuclear Material Control Plan, the licensee is exempted from physical inventory requirements relative to the material identified in Condition S-4; provided the conditions and commitments contained in the licensee's November 30, 1993, letter (identification # NRC-93-036) are satisfied.
- SG-1.8 Notwithstanding the requirement of Section 6.2.1(a).5 of the licensee's Fundamental Nuclear Material Control Plan to unpackage and perform an item count upon receipt of special nuclear material, the licensee is exempted from such requirement relative to the material identified in Condition S-4; provided the conditions and commitments contained in the licensee's November 30, 1993, letter (identification # NRC-93-036) are satisfied.

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70-1151

Amendment No. 18

SG-1.9 Notwithstanding the requirement of Section II.A.7, block U, of NUREG/BR-0006, which is incorporated via 10 CFR 74.15, to complete receiver's measurements of scrap receipts (following recovery processing) within 60 days of receipt, the licensee shall not be subject to any time limit relative to recovering and measuring received UF₆ heels when the block U action code (of DOE/NRC Form 741) is used to book such receipts.

SG-1.10 With respect to Section 5.1.4 (b) of the Plan identified by Condition SG-1.1, "*allowed number*" within the phrase "*allowed number of defects*" is hereby specified as being:

- (i) up to two defects when each item within a batch of items has an assigned value equal to or less than [REDACTED] U-235;
- (ii) no more than one defect when each item within a batch of items has an assigned value of less than [REDACTED] U-235, but one or more items has an assigned value in excess of 50 grams U-235; and
- (iii) zero defect when any item within a batch of items contains [REDACTED] U-235.

SG-1.11 Notwithstanding the first paragraph of Section 7.1 of the Plan identified by Condition SG-1.1, the licensee shall conduct shipper-receiver comparisons on all SNM materials received (regardless of whether booked on the basis of receiver's or shipper's values), except for those materials identified in Section 7.1 of NUREG-1065 (Rev. 2) as being exempted from shipper-receiver comparisons.

SECTION 2.0 -- PHYSICAL PROTECTION OF SNM OF LOW STRATEGIC SIGNIFICANCE

SG-2.1 The licensee shall follow the "Site Physical Security Plan - Westinghouse Electric Corporation, Columbia, S.C.," dated March 1980; Revision 10.0 dated October 1, 1987; the "Revision Record" contained in the approved Physical Security Plan (current to October 1, 1994); and as may be revised in accordance with the provisions of 10 CFR 70.32(e).

SECTION 3.0 -- INTERNATIONAL SAFEGUARDS

SG-3.1 The licensee shall follow Codes 1 through 6 of Transitional Facility Attachment No. 5A dated August 31, 1988, to the US/IAEA Safeguards Agreement. Such Transitional Facility Attachment shall be interpreted in accordance with Conditions SG-3.1.1 through SG-3.1.7.

SG-3.1.1 With respect to Transitional Facility Attachment Code 2:

The reference design information is that dated by the licensee on October 14, 1985. "Information on the Facility" also includes other facility information submitted via Concise Notes in accordance with 10 CFR 75.11(c).

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SG-3.1.2 With respect to Transitional Facility Attachment Code 2.2:

Substantive changes to the information provided in the Columbia Plant Design Information Questionnaire (DIQ) means those changes requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by letter to the NRC Office of Nuclear Material Safety and Safeguards at least 70-days in advance of implementation.

Non-substantive changes to the information in the DIQ means those changes not requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by Concise Note (From DOE/NRC-740M) within 30 days of receiving notification from the NRC that the facility has been identified under Article 39(b) of the US/IAEA Safeguards Agreement.

The types of modifications with respect to which information is required under 10 CFR 75.11, (to be submitted in advance), are those items stated in Code 2.2, specifically:

- (a) "Any change in the purpose of type of facility" means:

Any deviation from the described activities involving special nuclear material and any change to the maximum enrichment and/or quantities of U-235 currently authorized by License No. SNM-1107, and/or as described in Paragraph 5 of the Design Information Questionnaire (DIQ) dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any deviation from the described special nuclear material (SNM) production activities described in paragraph 6 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

- (b) "Any changes in the layout of the facility which affects safeguards implementation of the provisions of the Protocol" means:

Any change in the existing facility and/or site layout or new addition affecting any activity involving SNM as described in Paragraphs 10 and 11 (per the referenced attachments of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any modification to, or deviation from, the data provided in Paragraphs 13 and 14 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

- (c) "Any change that makes the selected Key Measurement Points (KMPs) (as described in Code 3.1.2) inadequate for the Agency's accounting purpose" means:

Any change to the KMPs as described in Code 3.1.2 of the Westinghouse-Columbia Transitional Facility Attachment to the US/IAEA Safeguards Agreement, or as modified in accordance with 10 CFR 75.11(c), that results in any KMP alteration affecting the purpose of KMPs as stipulated by 10 CFR 75.4(m)

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70-1151

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- (d) "Any change in the physical inventory procedures that would adversely affect the inventory taking for the Agency's accounting purposes" means:

Any change to the description data contained in Paragraph 34 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that would not permit the Agency to conclude an SNM material balance for the Westinghouse-Columbia facility.

- (e) "Introduction of a significantly less accurate analytical method for accounting purposes" means:

Any recalculation of the "Relative Errors-Random and Systematic" as listed in Attachment 36.2 referenced in Paragraph 36 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that results in the estimates of the random and systematic errors being affected by a factor of two or more.

- (f) "Decrease in the frequency of calibrating measuring equipment if it significantly decreases the accuracy of the materials accounting system" means:

Any change that results in the estimates of the systematic error being affected by a factor of two or more.

- (g) "Any change in the statistical procedures used to combine individual measurement error estimates to obtain limits of error for shipper/receiver (S/R) differences and material unaccounted for (MUF)" means:

Any deviation from (or modification of) the equations and/or calculations outlined in Attachments 37.1, 37.2, and 37.3 referenced in Paragraph 37 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

SG-3.1.3 With respect to Transitional Facility Attachment Code 3.1.2:

KMP* -- This is a KMP in which all shipper receiver differences (SRDs) must be recorded and reported even if numerically zero. SRDs are computed and reported by the Nuclear Materials Management and Safeguards System upon receipt of the receiver's measurement results.

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SG-3.1.4 With respect to Transitional Facility Attachment Code 4:

The licensee shall use the material composition codes documented in the DIQ dated October 14, 1985, and as modified by Concise Notes. Further, notwithstanding any other requirements for advance notification and/or reporting, the licensee may add or delete composition codes for nuclear material routinely processed and on inventory at CFFF immediately upon telephone notification to the Office of Nuclear Material Safety and Safeguards. Follow-up documentation, in the form of a Concise Note accompanied by appropriate changes to Table 1 of Attachment 34.8 to the DIQ shall be submitted within three regular workdays of the telephone notification.

SG-3.1.5 With respect to Transitional Facility Attachment Code 4.1:

Measured discards should be reported as an SN (Shipment to non-safeguards facility) when shipped off-site to an authorized burial ground. (The IAEA system will not process measured discards as loss/disposal (LDs) when they are shipped off-site).

SG-3.1.6 With respect to Transitional Facility Attachment Code 5.1.1:

For inventory changes, time of recording, "upon" means: No later than the next regular workday (Monday through Friday).

For those occasions where natural or depleted uranium is inadvertently enriched above 0.711 percent through commingling with residual enriched uranium in process equipment, the resultant product shall be considered as being produced through a blending operation and the material category change shall be recorded upon obtaining measurement confirmation that a material category change has occurred.

SG-3.1.7 With respect to Transitional Facility Attachment Code 6.2.2:

For Concise Notes describing the anticipated operational programme, "anticipated operational programme" means: Anticipated physical inventory schedule.

ATTACHMENT 4

License Number, SNM-770

Waltz Mill Site



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134

030-06135
030-32414
070-00698
070-01143
070-01503
040-08976

License Nos. 37-05809-01

37-05809-02
37-00497-15
SNM-770
SNM-1120
SNM-1460
SMB-1527

Control Nos. 126122

126123
126133
126125
126124
126126
126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely,

A handwritten signature in cursive script, appearing to read "Pamela J. Henderson". The signature is written in dark ink and is positioned above the printed name.

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p>Licensee</p> <p>1. Westinghouse Electric Company LLC (WELCO)</p> <p>2. P. O. Box 355 Pittsburgh, Pennsylvania 15230</p>	<p>In accordance with the letter dated September 28, 1998,</p> <p>3. License number SNM-770 is amended in its entirety to read as follows:</p> <hr/> <p>4. Expiration date February 28, 2002</p> <hr/> <p>5. Docket No. 070-00698 Reference No.</p>
--	---

6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A. Any byproduct material with atomic number 1 through 83	A. Any	A. [REDACTED] except as limited by Condition 12
B. Manganese 54	B. Any	B. [REDACTED]
C. Iron 55	C. Any	C. [REDACTED]
D. Cobalt 58	D. Any	D. [REDACTED]
E. Cobalt 60	E. Any	E. [REDACTED]
F. Nickel 63	F. Any	F. [REDACTED]
G. Enriched uranium (up to fully enriched)	G. Any	G. [REDACTED]
H. Uranium 233	H. Any	H. [REDACTED]
I. Plutonium	I. Any	I. [REDACTED]
J. Any source material	J. Any	J. [REDACTED]
K. Any byproduct material with atomic number 84 through 103	K. Any	K. [REDACTED] except as limited by Condition 12
L. Any byproduct material	L. Any	L. [REDACTED]
M. Any source material	M. Any	M. [REDACTED]

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-770

Docket or Reference Number

070-00698

Amendment No. 19

- | | | |
|---|--|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| N. Enriched uranium (up to fully enriched) | N. Any | N. [REDACTED] |
| O. Any byproduct material | O. Sealed sources | O. Not to exceed [REDACTED] per source and [REDACTED] total |
| P. Cesium 137 | P. Sealed sources (J.L. Shepherd Model 6810) | P. Not to exceed [REDACTED] per source and [REDACTED] total |
| Q. Any byproduct, source, or special nuclear material | Q. Contaminated structures, equipment, soil and debris | Q. See Condition 22 |

9. Authorized use:

- A. through K. Research and development as defined in 10 CFR 30.4; storage, decontamination, testing and maintenance of service equipment and supplies; sample analysis.
- L. through N. Decontamination, testing and maintenance of service equipment at temporary job sites.
- O. and P. Instrument testing and calibration.
- Q. Decommissioning.

CONDITIONS

10. A. Licensed material in items 6.A. through 6.K. and 6.O. through 6.Q. may be used only at the licensee's facilities located at the Waltz Mill site, Interstate 70, Exit 25A and Madison Road, Madison, Pennsylvania.
- B. Licensed material in items 6.L., 6.M., and 6.N. may be used only at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.
11. A. Licensed material shall be used by, or under the supervision of, individuals designated in writing by the Radiation Safety Committee, Bruce W. Bevilacqua, Chairperson.
- B. The Radiation Safety Officer for this license is Wayne Vogel.
12. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material at a single location to quantities below the limits specified in 10 CFR 30.72 which require consideration of the need for an emergency plan for responding to a release of licensed material.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
SNM-770Docket or Reference Number
070-00698

Amendment No. 19

13. Licensed material shall not be used in or on human beings.
14. A. Sealed sources and detector cells containing licensed material shall be tested for leakage and/or contamination at intervals not to exceed six months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed three years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed three months.
- C. In the absence of a certificate from a transferor indicating that a leak test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen-3; or
 - (ii) they contain only a radioactive gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or
 - (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source or detector cell shall be removed immediately from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within five days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source or detector cell involved, the test results, and corrective action taken.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-770

Docket or Reference Number

070-00698

Amendment No. 19

- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
15. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.
16. The licensee shall not acquire licensed material in a sealed source or device unless the source or device has been registered with the U.S. Nuclear Regulatory Commission pursuant to 10 CFR 32.210 or equivalent regulations of an Agreement State.
17. The licensee shall conduct a physical inventory every six months to account for all sealed sources and devices containing licensed material received and possessed under the license.
18. A. Detector cells containing a titanium tritide foil or a scandium tritide foil shall only be used in conjunction with a properly operating temperature control mechanism which prevents the foil temperatures from exceeding that specified in the certificate of registration referred to in 10 CFR 32.210.
- B. When in use, detector cells containing a titanium tritide foil or a scandium tritide foil shall be vented to the outside.
19. The licensee is authorized to hold radioactive material with a physical half-life of less than or equal to 120 days for decay-in-storage before disposal in ordinary trash, provided:
- A. Waste to be disposed of in this manner shall be held for decay a minimum of ten half-lives.
- B. Before disposal as ordinary trash, the waste shall be surveyed at the container surface with the appropriate survey instrument set on its most sensitive scale and with no interposed shielding to determine that its radioactivity cannot be distinguished from background. All radiation labels shall be removed or obliterated.
- C. A record of each such disposal permitted under this License Condition shall be retained for three years. The record must include the date of disposal, the date on which the byproduct material was placed in storage, the radionuclides disposed, the survey instrument used, the background dose rate, the dose rate measured at the surface of each waste container, and the name of the individual who performed the disposal.
20. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
21. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

**MATERIALS LICENSE
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22. The licensee may possess the licensed material as contaminated structures, equipment, soil and debris that existed on December 31, 1996.
23. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
24. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee
25. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
26. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Application dated March 21, 1996
 - B. Letter dated December 3, 1996
 - C. Letter dated November 16, 1998
 - D. Letter dated November 24, 1998
 - E. Letter dated January 18, 1999
 - F. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81364401

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p style="text-align: center;">Licensee</p> <p>1. Westinghouse Electric Company LLC (WELCO)</p> <p>2. P. O. Box 355 Pittsburgh, Pennsylvania 15230</p>	<p>In accordance with the letter dated September 28, 1998,</p> <p>3. License number SNM-770 is amended in its entirety to read as follows:</p> <hr/> <p>4. Expiration date February 28, 2002</p> <hr/> <p>5. Docket No. 070-00698 Reference No.</p>
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6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A. Any byproduct material with atomic number 1 through 83	A. Any	A. [REDACTED] except as limited by Condition 12
B. Manganese 54	B. Any	B. [REDACTED]
C. Iron 55	C. Any	C. [REDACTED]
D. Cobalt 58	D. Any	D. [REDACTED]
E. Cobalt 60	E. Any	E. [REDACTED]
F. Nickel 63	F. Any	F. [REDACTED]
G. Enriched uranium (up to fully enriched)	G. Any	G. [REDACTED]
H. Uranium 233	H. Any	H. [REDACTED]
I. Plutonium	I. Any	I. [REDACTED]
J. Any source material	J. Any	J. [REDACTED]
K. Any byproduct material with atomic number 84 through 103	K. Any	K. [REDACTED] except as limited by Condition 12
L. Any byproduct material	L. Any	L. [REDACTED]
M. Any source material	M. Any	M. [REDACTED]

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070-00698

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- | | | |
|---|--|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| N. Enriched uranium (up to fully enriched) | N. Any | N. [REDACTED] |
| O. Any byproduct material | O. Sealed sources | O. Not to exceed [REDACTED] per source and [REDACTED] total |
| P. Cesium 137 | P. Sealed sources (J.L. Shepherd Model 6810) | P. Not to exceed [REDACTED] per source and [REDACTED] total |
| Q. Any byproduct, source, or special nuclear material | Q. Contaminated structures, equipment, soil and debris | Q. See Condition 22 |

9. Authorized use:

- A. through K. Research and development as defined in 10 CFR 30.4; storage, decontamination, testing and maintenance of service equipment and supplies; sample analysis.
- L. through N. Decontamination, testing and maintenance of service equipment at temporary job sites.
- O. and P. Instrument testing and calibration.
- Q. Decommissioning.

CONDITIONS

10. A. Licensed material in items 6.A. through 6.K. and 6.O. through 6.Q. may be used only at the licensee's facilities located at the Waltz Mill site, Interstate 70, Exit 25A and Madison Road, Madison, Pennsylvania.
- B. Licensed material in items 6.L., 6.M., and 6.N. may be used only at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.
11. A. Licensed material shall be used by, or under the supervision of, individuals designated in writing by the Radiation Safety Committee, Bruce W. Bevilacqua, Chairperson.
- B. The Radiation Safety Officer for this license is Wayne Vogel.
12. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material at a single location to quantities below the limits specified in 10 CFR 30.72 which require consideration of the need for an emergency plan for responding to a release of licensed material.

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13. Licensed material shall not be used in or on human beings.
14. A. Sealed sources and detector cells containing licensed material shall be tested for leakage and/or contamination at intervals not to exceed six months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed three years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed three months.
- C. In the absence of a certificate from a transferor indicating that a leak test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen-3; or
 - (ii) they contain only a radioactive gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or
 - (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source or detector cell shall be removed immediately from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within five days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source or detector cell involved, the test results, and corrective action taken.

**MATERIALS LICENSE
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070-00698

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- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
15. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.
16. The licensee shall not acquire licensed material in a sealed source or device unless the source or device has been registered with the U.S. Nuclear Regulatory Commission pursuant to 10 CFR 32.210 or equivalent regulations of an Agreement State.
17. The licensee shall conduct a physical inventory every six months to account for all sealed sources and devices containing licensed material received and possessed under the license.
18. A. Detector cells containing a titanium tritide foil or a scandium tritide foil shall only be used in conjunction with a properly operating temperature control mechanism which prevents the foil temperatures from exceeding that specified in the certificate of registration referred to in 10 CFR 32.210.
- B. When in use, detector cells containing a titanium tritide foil or a scandium tritide foil shall be vented to the outside.
19. The licensee is authorized to hold radioactive material with a physical half-life of less than or equal to 120 days for decay-in-storage before disposal in ordinary trash, provided:
- A. Waste to be disposed of in this manner shall be held for decay a minimum of ten half-lives.
- B. Before disposal as ordinary trash, the waste shall be surveyed at the container surface with the appropriate survey instrument set on its most sensitive scale and with no interposed shielding to determine that its radioactivity cannot be distinguished from background. All radiation labels shall be removed or obliterated.
- C. A record of each such disposal permitted under this License Condition shall be retained for three years. The record must include the date of disposal, the date on which the byproduct material was placed in storage, the radionuclides disposed, the survey instrument used, the background dose rate, the dose rate measured at the surface of each waste container, and the name of the individual who performed the disposal.
20. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
21. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

ATTACHMENT 5

License Number, SNM-1460

Science & Technology Center



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSS, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134

030-06135
030-32414
070-00698
070-01143
070-01503
040-08976

License Nos. 37-05809-01

37-05809-02
37-00497-15
SNM-770
SNM-1120
SNM-1460
SMB-1527

Control Nos. 126122

126123
126133
126125
126124
126126
126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely,

A handwritten signature in cursive script, appearing to read "Pamela J. Henderson".

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p>Licensee</p> <p>1. Westinghouse Electric Company LLC (WELCO)</p> <p>2. Westinghouse Building Gateway Center Pittsburgh, Pennsylvania 15222</p>	<p>In accordance with the letter dated September 28, 1998,</p> <p>3. License number SNM-1460 is amended in its entirety to read as follows:</p> <p>4. Expiration date February 28, 2008</p> <p>5. Docket No. 070-01503 Reference No.</p>	
<p>6. Byproduct, source, and/or special nuclear material</p> <p>A. Any byproduct material with atomic numbers 1 through 96</p> <p>B. Any byproduct material with atomic numbers 3 through 83</p> <p>C. Any byproduct material with atomic numbers 3 through 83</p> <p>D. Any byproduct material with atomic numbers 84 through 96</p> <p>E. Hydrogen 3</p> <p>F. Cobalt 60</p> <p>G. Cobalt 60</p> <p>H. Xenon 133</p> <p>I. Barium 133</p>	<p>7. Chemical and/or physical form</p> <p>A. Any</p> <p>B. Any</p> <p>C. Sealed sources</p> <p>D. Contamination in various matrices</p> <p>E. Any</p> <p>F. Sealed sources (Gamma Industries Model VD-HP)</p> <p>G. Sealed source (ICN Model 371)</p> <p>H. Any</p> <p>I. Sealed sources (New England Nuclear)</p>	<p>8. Maximum amount that licensee may possess at any one time under this license</p> <p>A. [REDACTED]</p> <p>B. Not to exceed [REDACTED] per radionuclide and [REDACTED] total</p> <p>C. Not to exceed [REDACTED] per source and [REDACTED] total</p> <p>D. [REDACTED]</p> <p>E. [REDACTED]</p> <p>F. [REDACTED] per source and [REDACTED] total</p> <p>G. [REDACTED]</p> <p>H. [REDACTED]</p> <p>I. Not to exceed [REDACTED]s per source and [REDACTED] total</p>

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1460

Docket or Reference Number

070-01503

Amendment No. 08

6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
J. Cesium 137	J. Sealed source (ORNL Drawing No. B-RD-1430)	J. [REDACTED]
K. Cesium 137	K. Sealed source (3M Company Model 4F6D)	K. [REDACTED]
L. Depleted Uranium	L. Metal	L. [REDACTED]
M. Plutonium 239	M. Any	M. [REDACTED]
N. Plutonium	N. Any	N. [REDACTED]
O. Americium 241	O. Sealed neutron source (General Nuclear Inc. GNI- NB-HP.)	O. [REDACTED]
P. Americium 241	P. Sealed neutron source (Monsanto MRC-AmBe-985)	P. [REDACTED]
Q. Americium 241	Q. Electroplated alpha sources	Q. Not to exceed [REDACTED] per source and [REDACTED] total
R. Americium 241	R. Sealed gamma sources	R. Not to exceed [REDACTED] per source and [REDACTED] total
S. Americium 241	S. Sealed neutron sources	S. [REDACTED]
T. Californium 252	T. Sealed sources (Savannah River Model SR-CF-2000)	T. Not to exceed [REDACTED] per source and [REDACTED] total
U. Natural thorium	U. Any	U. [REDACTED]
V. Uranium 235	V. Any	V. [REDACTED]
W. Natural and/or depleted Uranium	W. Any	W. [REDACTED]
X. Uranium 233	X. Any	X. [REDACTED]
Y. Uranium 234	Y. Any	Y. [REDACTED]
Z. Uranium 237	Z. Any	Z. [REDACTED]

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SUPPLEMENTARY SHEET**License Number
SNM-1460Docket or Reference Number
070-01503

Amendment No. 08

6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
AA. Neptunium 237	AA. Any	AA. [REDACTED]
BB. Neptunium 239	BB. Any	BB. [REDACTED]
CC. Plutonium 236	CC. Any	CC. [REDACTED]
DD. Plutonium 238	DD. Any	DD. [REDACTED]
EE. Plutonium 239	EE. Any	EE. [REDACTED]
FF. Plutonium 240	FF. Any	FF. [REDACTED]
GG. Plutonium 241	GG. Any	GG. [REDACTED]
HH. Plutonium 242	HH. Any	HH. [REDACTED]
II. Californium 252	II. Any	II. [REDACTED]

9. Authorized use:

- A. In the Remote Metallurgic Facility for research and development as defined in 10 CFR 30.4, and for instrument calibration.
- B. through W. Research and development as defined in 10 CFR 30.4 or 10 CFR 70.4; instrument calibration.
- U. through II. Fabrication and calibration of in-core neutron dosimeters. Distribution of manufactured in-core neutron dosimeters to persons authorized to receive the licensed material pursuant to the terms and conditions of specific licenses issued by the U.S. Nuclear Regulatory Commission or any Agreement State.

CONDITIONS

10. A. Licensed material may be used at the licensee's Science and Technology Center facilities located at 1310 Beulah Road, Churchill Borough, Pittsburgh, Pennsylvania.
- B. Licensed material in quantities not exceeding [REDACTED] of any byproduct material, [REDACTED] of any source material (Items 6.U. and 6.W.), [REDACTED] of depleted uranium metal encapsulated in stainless steel for use as a flywheel (Item 6.L.), and [REDACTED] of Uranium 235 (Item 6.V.) may be used at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.

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11. A. Licensed material shall be used by, or under the supervision of, individuals designated by the licensee's Radiation Safety Committee, John L. Spicher, Chairperson. The licensee shall maintain records of individuals designated by users.
- B. The Radiation Safety Officer for this license is John R. Lehnhardt.
12. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material at a single location to quantities below the limits specified in 10 CFR 30.72 which require consideration of the need for an emergency plan for responding to a release of licensed material.
13. Licensed material shall not be used in or on human beings.
14. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
15. A. Sealed sources and detector cells containing licensed material shall be tested for leakage and/or contamination at intervals not to exceed six months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed three years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed three months.
- C. In the absence of a certificate from a transferor indicating that a leak test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen-3; or
 - (ii) they contain only a radioactive gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or

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- (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source or detector cell shall be removed immediately from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within five days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source or detector cell involved, the test results, and corrective action taken.
- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
16. The licensee shall conduct a physical inventory every 6 months to account for all sources and/or devices received and possessed under the license. Records of inventories shall be maintained for 5 years from the date of each inventory.
17. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

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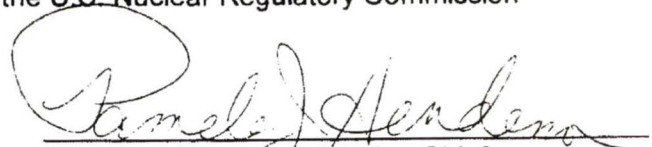
21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application dated January 16, 1998
- B. Letter dated January 16, 1998
- C. Letter dated January 21, 1998
- D. Letter dated November 16, 1998
- E. Letter dated November 24, 1998
- F. Letter dated January 18, 1999
- G. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By



Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81369648

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p style="text-align: center;">Licensee</p> <p>1. Westinghouse Electric Company LLC (WELCO)</p> <p>2. Westinghouse Building Gateway Center Pittsburgh, Pennsylvania 15222</p>	<p>In accordance with the letter dated September 28, 1998,</p> <p>3. License number SNM-1460 is amended in its entirety to read as follows:</p> <hr/> <p>4. Expiration date February 28, 2008</p> <hr/> <p>5. Docket No. 070-01503 Reference No.</p>
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|---|---|--|
| <p>6. Byproduct, source, and/or special nuclear material</p> <p>A. Any byproduct material with atomic numbers 1 through 96</p> <p>B. Any byproduct material with atomic numbers 3 through 83</p> <p>C. Any byproduct material with atomic numbers 3 through 83</p> <p>D. Any byproduct material with atomic numbers 84 through 96</p> <p>E. Hydrogen 3</p> <p>F. Cobalt 60</p> <p>G. Cobalt 60</p> <p>H. Xenon 133</p> <p>I. Barium 133</p> | <p>7. Chemical and/or physical form</p> <p>A. Any</p> <p>B. Any</p> <p>C. Sealed sources</p> <p>D. Contamination in various matrices</p> <p>E. Any</p> <p>F. Sealed sources (Gamma Industries Model VD-HP)</p> <p>G. Sealed source (ICN Model 371)</p> <p>H. Any</p> <p>I. Sealed sources (New England Nuclear)</p> | <p>8. Maximum amount that licensee may possess at any one time under this license</p> <p>A. [REDACTED]</p> <p>B. Not to exceed [REDACTED] per radionuclide and [REDACTED] total</p> <p>C. Not to exceed [REDACTED] per source and [REDACTED]</p> <p>D. [REDACTED]</p> <p>E. [REDACTED]</p> <p>F. [REDACTED] per source and [REDACTED] total</p> <p>G. [REDACTED]</p> <p>H. [REDACTED]</p> <p>I. Not to exceed [REDACTED] per source and [REDACTED] total</p> |
|---|---|--|

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1460

Docket or Reference Number

070-01503

Amendment No. 08

6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
J. Cesium 137	J. Sealed source (ORNL Drawing No. B-RD-1430)	J. [REDACTED]
K. Cesium 137	K. Sealed source (3M Company Model 4F6D)	K. [REDACTED]
L. Depleted Uranium	L. Metal	L. [REDACTED]
M. Plutonium 239	M. Any	M. [REDACTED]
N. Plutonium	N. Any	N. [REDACTED]
O. Americium 241	O. Sealed neutron source (General Nuclear Inc. GNI-NB-HP.)	O. [REDACTED]
P. Americium 241	P. Sealed neutron source (Monsanto MRC-AmBe-985)	P. [REDACTED]
Q. Americium 241	Q. Electroplated alpha sources	Q. Not to exceed [REDACTED] per source and [REDACTED] total
R. Americium 241	R. Sealed gamma sources	R. Not to exceed [REDACTED] per source and [REDACTED] total
S. Americium 241	S. Sealed neutron sources	S. [REDACTED]
T. Californium 252	T. Sealed sources (Savannah River Model SR-CF-2000)	T. Not to exceed [REDACTED] per source and [REDACTED] total
U. Natural thorium	U. Any	U. [REDACTED]
V. Uranium 235	V. Any	V. [REDACTED]
W. Natural and/or depleted Uranium	W. Any	W. [REDACTED]
X. Uranium 233	X. Any	X. [REDACTED]
Y. Uranium 234	Y. Any	Y. [REDACTED]
Z. Uranium 237	Z. Any	Z. [REDACTED]

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1460

Docket or Reference Number

070-01503

Amendment No. 08

6. Byproduct, source, and/or special
nuclear material

7. Chemical and/or physical form

8. Maximum amount that licensee may
possess at any one time under this
license

AA. Neptunium 237

AA. Any

AA. [REDACTED]

BB. Neptunium 239

BB. Any

BB. [REDACTED]

CC. Plutonium 236

CC. Any

CC. [REDACTED]

DD. Plutonium 238

DD. Any

DD. [REDACTED]

EE. Plutonium 239

EE. Any

EE. [REDACTED]

FF. Plutonium 240

FF. Any

FF. [REDACTED]

GG. Plutonium 241

GG. Any

GG. [REDACTED]

HH. Plutonium 242

HH. Any

HH. [REDACTED]

II. Californium 252

II. Any

II. [REDACTED]

9. Authorized use:

A. In the Remote Metallurgic Facility for research and development as defined in 10 CFR 30.4, and for instrument calibration.

B. through W. Research and development as defined in 10 CFR 30.4 or 10 CFR 70.4; instrument calibration.

U. through II. Fabrication and calibration of in-core neutron dosimeters. Distribution of manufactured in-core neutron dosimeters to persons authorized to receive the licensed material pursuant to the terms and conditions of specific licenses issued by the U.S. Nuclear Regulatory Commission or any Agreement State.

CONDITIONS

10. A. Licensed material may be used at the licensee's Science and Technology Center facilities located at 1310 Beulah Road, Churchill Borough, Pittsburgh, Pennsylvania.

B. Licensed material in quantities not exceeding [REDACTED] of any byproduct material, [REDACTED] of any source material (Items 6.U. and 6.W.), [REDACTED] of depleted uranium metal encapsulated in stainless steel for use as a flywheel (Item 6.L.), and [REDACTED] of Uranium 235 (Item 6.V.) may be used at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.

**MATERIALS LICENSE
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Docket or Reference Number

070-01503

Amendment No. 08

11. A. Licensed material shall be used by, or under the supervision of, individuals designated by the licensee's Radiation Safety Committee, John L. Spicher, Chairperson. The licensee shall maintain records of individuals designated by users.
- B. The Radiation Safety Officer for this license is John R. Lehnhardt.
12. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material at a single location to quantities below the limits specified in 10 CFR 30.72 which require consideration of the need for an emergency plan for responding to a release of licensed material.
13. Licensed material shall not be used in or on human beings.
14. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
15. A. Sealed sources and detector cells containing licensed material shall be tested for leakage and/or contamination at intervals not to exceed six months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed three years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed three months.
- C. In the absence of a certificate from a transferor indicating that a leak test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen-3; or
 - (ii) they contain only a radioactive gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or

**MATERIALS LICENSE
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- (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source or detector cell shall be removed immediately from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within five days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source or detector cell involved, the test results, and corrective action taken.
- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
16. The licensee shall conduct a physical inventory every 6 months to account for all sources and/or devices received and possessed under the license. Records of inventories shall be maintained for 5 years from the date of each inventory.
17. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
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License Number

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21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application dated January 16, 1998
- B. Letter dated January 16, 1998
- C. Letter dated January 21, 1998
- D. Letter dated November 16, 1998
- E. Letter dated November 24, 1998
- F. Letter dated January 18, 1999
- G. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By

Original signed by Pamela J. Henderson

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81369648

ATTACHMENT 6

**License Number, 37-05809-01 Number,
Pump Repair Facility, Cheswick, PA**



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134
030-06135
030-32414
070-00698
070-01143
070-01503
040-08976

License Nos. 37-05809-01
37-05809-02
37-00497-15
SNM-770
SNM-1120
SNM-1460
SMB-1527

Control Nos. 126122
126123
126133
126125
126124
126126
126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely,

A handwritten signature in cursive script, appearing to read "Pamela J. Henderson".

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p>Licensee</p> <p>1. Westinghouse Government Services Company LLC (WGS)</p> <p>2. Electro-Mechanical Division P. O. Box 355 Pittsburgh, Pennsylvania 15230-0355</p>	<p>In accordance with the letter dated September 28, 1998</p> <p>3. License number 37-05809-01 is amended in its entirety to read as follows:</p> <p>4. Expiration date December 31, 2002</p> <p>5. Docket No. 030-06134 Reference No.</p>	
<p>6. Byproduct, source, and/or special nuclear material</p> <p>A. Any byproduct material between Atomic Numbers 1 and 83</p> <p>B. Cobalt 60</p> <p>C. Cesium 137</p> <p>D. Iron 55</p> <p>E. Nickel 63</p> <p>F. Any byproduct material between Atomic Numbers 84 and 102 inclusive</p> <p>G. Cobalt 60</p> <p>H. Strontium 90</p>	<p>7. Chemical and/or physical form</p> <p>A. Reactor Component contaminants</p> <p>B. Reactor Component contaminants</p> <p>C. Reactor Component contaminants</p> <p>D. Reactor Component contaminants</p> <p>E. Reactor Component contaminants</p> <p>F. Reactor Component contaminants</p> <p>G. Sealed sources (Westinghouse Drawing No. 2424B18)</p> <p>H. Sealed sources (Westinghouse Drawing No. 2424B18)</p>	<p>8. Maximum amount that licensee may possess at any one time under this license</p> <p>A. [REDACTED] total and except as limited by Condition 16</p> <p>B. [REDACTED]</p> <p>C. [REDACTED]</p> <p>D. [REDACTED]</p> <p>E. [REDACTED]</p> <p>F. [REDACTED] total</p> <p>G. Not to exceed [REDACTED] per source and [REDACTED] total</p> <p>H. Not to exceed [REDACTED] per source and [REDACTED] total</p>

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-01

Docket or Reference Number

030-06134

Amendment No. 23

- | | | |
|--|---|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| I. Cesium 137 | I. Sealed source
(Radiation Materials Corporation Model GK6) | I. [REDACTED] |
| J. Cesium 137 | J. Sealed source
(Atomchem Model CS-2-10) | J. [REDACTED] |
| K. Cesium 137 | K. Sealed sources
(Westinghouse Drawing No. 2424B18) | K. Not to exceed [REDACTED] per source and [REDACTED] total |
| L. Neptunium 237 | L. Encapsulated neutron dosimeter blocks | L. Not to exceed [REDACTED] per source and [REDACTED] total |
| M. Any byproduct material with atomic numbers 1 through 83 | M. Contamination | M. [REDACTED] total and except as limited by Condition 16 |
| N. Any byproduct material with atomic numbers 84 through 102 inclusive | N. Contamination | N. [REDACTED] total and except as limited by Condition 16 |

9. Authorized use:

- A. through F. For possession incident to decontamination, repair, testing, and maintenance of pump and/or reactor components.
- G. through K. For calibration and testing of instruments.
- L. For possession incident to assembly of neutron dosimeter blocks into fixtures and distribution to persons authorized to receive the licensed material pursuant to the terms and conditions of a specific license issued by the Nuclear Regulatory Commission or an Agreement State.
- M. and N. Decontamination and decommissioning of facilities; packaging of stock material and radioactive waste; storage of radioactive material and packaged radioactive waste prior to shipment.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-01

Docket or Reference Number

030-06134

Amendment No. 23

CONDITIONS

10. Licensed material may be used only at the licensee's facilities at Cheswick Avenue, Cheswick, Pennsylvania.
11. A. Licensed material shall be used by, or under the supervision of, individuals approved by the Radiation Safety Committee, C.T. Lim, Chairman.
- B. The Radiation Safety Officer for this license is Lisa A. Lamantia.
12. A. Sealed sources and detector cells shall be tested for leakage and/or contamination at intervals not to exceed 6 months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed 3 years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed 3 months.
- C. In the absence of a certificate from a transferor indicating that a test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen 3; or
 - (ii) they contain only a gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or
 - (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.

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37-05809-01

Docket or Reference Number

030-06134

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- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. Records of leak test results shall be kept in units of microcuries and shall be maintained for inspection by the Commission. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source shall be removed from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within 5 days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source involved, the test results, and corrective action taken.
- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
13. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders or detector cells by the licensee.
14. The licensee shall conduct a physical inventory every six months to account for all sealed sources and devices containing licensed material received and possessed under the license.
15. The licensee may transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material".
16. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material so that at no time is a quantity of radioactive material is possessed in excess of a quantity which requires consideration of the need for an emergency plan for responding to a release of licensed material in accordance with 10 CFR 30.72.
17. Notwithstanding the provisions of 10 CFR 20.1401, building #9 (old and new Naval pump facility) and two (2) associated butler buildings at the site, may be decommissioned in accordance with the SDMP Action Plan criteria specified in 10 CFR 20.1401 (b) (3).
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
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License Number

37-05809-01

Docket or Reference Number

030-06134

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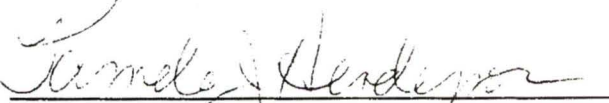
21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application and letter dated September 15, 1992
- B. Letter dated January 31, 1995
- C. Letter dated September 24, 1996
- D. Letter dated August 28, 1998
- E. Letter dated November 16, 1998
- F. Letter dated November 24, 1998
- G. Letter dated January 18, 1999
- H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By



Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81363863

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee		In accordance with the letter dated September 28, 1998
1. Westinghouse Government Services Company LLC (WGS)		3. License number 37-05809-01 is amended in its entirety to read as follows:
2. Electro-Mechanical Division P. O. Box 355 Pittsburgh, Pennsylvania 15230-0355		4. Expiration date December 31, 2002
		5. Docket No. 030-06134 Reference No.
6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A. Any byproduct material between Atomic Numbers 1 and 83	A. Reactor Component contaminants	A. [REDACTED] total and except as limited by Condition 16
B. Cobalt 60	B. Reactor Component contaminants	B. [REDACTED]
C. Cesium 137	C. Reactor Component contaminants	C. [REDACTED]
D. Iron 55	D. Reactor Component contaminants	D. [REDACTED]
E. Nickel 63	E. Reactor Component contaminants	E. [REDACTED]
F. Any byproduct material between Atomic Numbers 84 and 102 inclusive	F. Reactor Component contaminants	F. [REDACTED] total
G. Cobalt 60	G. Sealed sources (Westinghouse Drawing No. 2424B18)	G. Not to exceed [REDACTED] per source and [REDACTED] total
H. Strontium 90	H. Sealed sources (Westinghouse Drawing No. 2424B18)	H. Not to exceed [REDACTED] per source and [REDACTED] total

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-01

Docket or Reference Number

030-06134

Amendment No. 23

- | | | |
|--|---|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| I. Cesium 137 | I. Sealed source
(Radiation Materials Corporation Model GK6) | I. [REDACTED] |
| J. Cesium 137 | J. Sealed source
(Atomchem Model CS-2-10) | J. [REDACTED] |
| K. Cesium 137 | K. Sealed sources
(Westinghouse Drawing No. 2424B18) | K. Not to exceed [REDACTED] per source and [REDACTED] total |
| L. Neptunium 237 | L. Encapsulated neutron dosimeter blocks | L. Not to exceed [REDACTED] per source and [REDACTED] total |
| M. Any byproduct material with atomic numbers 1 through 83 | M. Contamination | M. [REDACTED] total and except as limited by Condition 16 |
| N. Any byproduct material with atomic numbers 84 through 102 inclusive | N. Contamination | N. [REDACTED] total and except as limited by Condition 16 |

9. Authorized use:

- A. through F. For possession incident to decontamination, repair, testing, and maintenance of pump and/or reactor components.
- G. through K. For calibration and testing of instruments.
- L. For possession incident to assembly of neutron dosimeter blocks into fixtures and distribution to persons authorized to receive the licensed material pursuant to the terms and conditions of a specific license issued by the Nuclear Regulatory Commission or an Agreement State.
- M. and N. Decontamination and decommissioning of facilities; packaging of stock material and radioactive waste; storage of radioactive material and packaged radioactive waste prior to shipment.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
37-05809-01Docket or Reference Number
030-06134

Amendment No. 23

CONDITIONS

10. Licensed material may be used only at the licensee's facilities at Cheswick Avenue, Cheswick, Pennsylvania.
11. A. Licensed material shall be used by, or under the supervision of, individuals approved by the Radiation Safety Committee, C.T. Lim, Chairman.
- B. The Radiation Safety Officer for this license is Lisa A. Lamantia.
12. A. Sealed sources and detector cells shall be tested for leakage and/or contamination at intervals not to exceed 6 months or at such other intervals as are specified by the certificate of registration referred to in 10 CFR 32.210, not to exceed 3 years.
- B. Notwithstanding Paragraph A of this Condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed 3 months.
- C. In the absence of a certificate from a transferor indicating that a test has been made within six months prior to the transfer, a sealed source or detector cell received from another person shall not be put into use until tested.
- D. Each sealed source fabricated by the licensee shall be inspected and tested for construction defects, leakage, and contamination prior to any use or transfer as a sealed source.
- E. Sealed sources and detector cells need not be leak tested if:
- (i) they contain only hydrogen 3; or
 - (ii) they contain only a gas; or
 - (iii) the half-life of the isotope is 30 days or less; or
 - (iv) they contain not more than [REDACTED] of beta and/or gamma emitting material or not more than [REDACTED] of alpha emitting material; or
 - (v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use or transfer to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.

**MATERIALS LICENSE
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License Number

37-05809-01

Docket or Reference Number

030-06134

Amendment No. 23

- F. The test shall be capable of detecting the presence of [REDACTED] of radioactive material on the test sample. Records of leak test results shall be kept in units of microcuries and shall be maintained for inspection by the Commission. If the test reveals the presence of [REDACTED] or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source shall be removed from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within 5 days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region I, ATTN: Chief, Nuclear Materials Safety Branch, 475 Allendale Road, King of Prussia, Pennsylvania 19406. The report shall specify the source involved, the test results, and corrective action taken.
- G. The licensee is authorized to collect leak test samples for analysis by the licensee. Alternatively, tests for leakage and/or contamination may be performed by persons specifically licensed by the Commission or an Agreement State to perform such services.
13. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders or detector cells by the licensee.
14. The licensee shall conduct a physical inventory every six months to account for all sealed sources and devices containing licensed material received and possessed under the license.
15. The licensee may transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material".
16. In addition to the possession limits in Item 8, the licensee shall further restrict the possession of licensed material so that at no time is a quantity of radioactive material is possessed in excess of a quantity which requires consideration of the need for an emergency plan for responding to a release of licensed material in accordance with 10 CFR 30.72.
17. Notwithstanding the provisions of 10 CFR 20.1401, building #9 (old and new Naval pump facility) and two (2) associated butler buildings at the site, may be decommissioned in accordance with the SDMP Action Plan criteria specified in 10 CFR 20.1401 (b) (3).
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
37-05809-01Docket or Reference Number
030-06134

Amendment No. 23

21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application and letter dated September 15, 1992
- B. Letter dated January 31, 1995
- C. Letter dated September 24, 1996
- D. Letter dated August 28, 1998
- E. Letter dated November 16, 1998
- F. Letter dated November 24, 1998
- G. Letter dated January 18, 1999
- H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Original signed by Pamela J. HendersonDate March 1, 1999

By

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81363863

ATTACHMENT 7

License Number, 37-05809-02

Industrial Radiography Facility, Cheswick, PA



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

March 1, 1999

Docket Nos. 030-06134

030-06135

030-32414

070-00698

070-01143

070-01503

040-08976

License Nos. 37-05809-01

37-05809-02

37-00497-15

SNM-770

SNM-1120

SNM-1460

SMB-1527

Control Nos. 126122

126123

126133

126125

126124

126126

126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely

A handwritten signature in cursive script, appearing to read "Pamela J. Henderson".

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	In accordance with the letter dated September 28, 1998,
1. Westinghouse Government Services Company LLC (WGS)	3. License number 37-05809-02 is amended in its entirety to read as follows:
2. P. O. Box 355 Pittsburgh, Pennsylvania 15230-0355	4. Expiration date September 30, 2005
	5. Docket No. 030-06135 Reference No.

- | | | |
|---|--|---|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| A. Iridium 192 | A. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation | A. Not to exceed [REDACTED] per source and [REDACTED] total except as specified by Condition 13 |
| B. Cobalt 60 | B. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation | B. Not to exceed [REDACTED] per source and [REDACTED] total except as specified by Condition 13 |
| C. Ytterbium 169 | C. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation | C. Not to exceed [REDACTED] per source and [REDACTED] total except as specified by Condition 13 |
| D. Depleted Uranium | D. Metal | D. [REDACTED] |

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-02

Docket or Reference Number

030-06135

Amendment No. 18

9. Authorized use:

- A. through C. For use in a compatible radiographic exposure device registered pursuant to 10 CFR 32.210 or an equivalent Agreement State regulation for performing industrial radiography and in a compatible source changer registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation for source storage and exchange.
- D. Shielding for radiographic equipment.

CONDITIONS

10. Licensed material may be used only at the licensee's facilities located at Electro-Mechanical Division, Cheswick, Pennsylvania and at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.
11. Licensed material shall be used by, or under the supervision and in the physical presence of individuals who have been designated in writing by the Radiation Safety Officer and have been trained:
- A. As specified in the letters dated February 21, 1995 and August 17, 1995; and
- B. In accordance with the provisions of 10 CFR 34.31.
12. The Radiation Safety Officer for this license is Lisa Lamantia.
13. The licensee is authorized to receive, possess, and use sealed sources of iridium-192, ytterbium-169 or cobalt-60 where the radioactivity exceeds the maximum amount of radioactivity specified in this license provided:
- A. such possession does not exceed the quantity per source specified in Item 8 by more than 20% for iridium-192 or 10% for cobalt-60 or ytterbium-169; and
- B. records of the licensee show that no more than the maximum amount of radioactivity per source specified in this license was ordered from the supplier or transferor of the byproduct material; and
- C. the levels of radiation for radiographic exposure devices and storage containers do not exceed those specified in 10 CFR 34.21.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**License Number
37-05809-02Docket or Reference Number
030-06135

Amendment No. 18

14. A. Notwithstanding the periodic leak test required by 10 CFR 34.25(b), the requirement does not apply to radiography sources that are stored and not being used. The sources excepted from this test shall be tested for leakage before use or transfer to another person. No sealed source shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- B. Sealed sources authorized for a use other than radiography shall be tested for leakage in accordance with 10 CFR 34.25.
15. Notwithstanding the requirements of 10 CFR 34.20(a), and pursuant to 10 CFR 34.51, radiographic equipment authorized for use in radiographic operations under this license need not comply with the torque criteria of Section 8.9.2(c) of American National Standard N432-1980.
16. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.
17. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-02

Docket or Reference Number

030-06135

Amendment No. 18

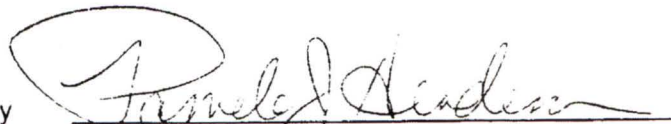
21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application dated February 7, 1992
- B. Letter dated February 21, 1995
- C. Letter dated July 21, 1995
- D. Letter dated August 17, 1995
- E. Letter dated November 16, 1998
- F. Letter dated November 24, 1998
- G. Letter dated January 18, 1999
- H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By



Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81362030

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

<p style="text-align: center;">Licensee</p> <p>1. Westinghouse Government Services Company LLC (WGS)</p> <p>2. P. O. Box 355 Pittsburgh, Pennsylvania 15230-0355</p>	<p>In accordance with the letter dated September 28, 1998,</p> <p>3. License number 37-05809-02 is amended in its entirety to read as follows:</p> <hr/> <p>4. Expiration date September 30, 2005</p> <hr/> <p>5. Docket No. 030-06135 Reference No.</p>	
<p>6. Byproduct, source, and/or special nuclear material</p> <p>A. Iridium 192</p> <p>B. Cobalt 60</p> <p>C. Ytterbium 169</p> <p>D. Depleted Uranium</p>	<p>7. Chemical and/or physical form</p> <p>A. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation</p> <p>B. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation</p> <p>C. Sealed radiography source contained in a source assembly registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation</p> <p>D. Metal</p>	<p>8. Maximum amount that licensee may possess at any one time under this license</p> <p>A. Not to exceed [redacted] per source and [redacted] total except as specified by Condition 13</p> <p>B. Not to exceed [redacted] per source and [redacted] total except as specified by Condition 13</p> <p>C. Not to exceed [redacted] per source and [redacted] total except as specified by Condition 13</p> <p>D. [redacted]</p>

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-02

Docket or Reference Number

030-06135

Amendment No. 18

9. Authorized use:

- A. through C. For use in a compatible radiographic exposure device registered pursuant to 10 CFR 32.210 or an equivalent Agreement State regulation for performing industrial radiography and in a compatible source changer registered pursuant to 10 CFR 32.210 or an equivalent Agreement State Regulation for source storage and exchange.
- D. Shielding for radiographic equipment.

CONDITIONS

10. Licensed material may be used only at the licensee's facilities located at Electro-Mechanical Division, Cheswick, Pennsylvania and at temporary job sites of the licensee anywhere in the United States where the U.S. Nuclear Regulatory Commission maintains jurisdiction for regulating the use of licensed material.
11. Licensed material shall be used by, or under the supervision and in the physical presence of individuals who have been designated in writing by the Radiation Safety Officer and have been trained:
- A. As specified in the letters dated February 21, 1995 and August 17, 1995; and
 - B. In accordance with the provisions of 10 CFR 34.31.
12. The Radiation Safety Officer for this license is Lisa Lamantia.
13. The licensee is authorized to receive, possess, and use sealed sources of iridium-192, ytterbium-169 or cobalt-60 where the radioactivity exceeds the maximum amount of radioactivity specified in this license provided:
- A. such possession does not exceed the quantity per source specified in Item 8 by more than 20% for iridium-192 or 10% for cobalt-60 or ytterbium-169; and
 - B. records of the licensee show that no more than the maximum amount of radioactivity per source specified in this license was ordered from the supplier or transferor of the byproduct material; and
 - C. the levels of radiation for radiographic exposure devices and storage containers do not exceed those specified in 10 CFR 34.21.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-02

Docket or Reference Number

030-06135

Amendment No. 18

14. A. Notwithstanding the periodic leak test required by 10 CFR 34.25(b), the requirement does not apply to radiography sources that are stored and not being used. The sources excepted from this test shall be tested for leakage before use or transfer to another person. No sealed source shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.
- B. Sealed sources authorized for a use other than radiography shall be tested for leakage in accordance with 10 CFR 34.25.
15. Notwithstanding the requirements of 10 CFR 34.20(a), and pursuant to 10 CFR 34.51, radiographic equipment authorized for use in radiographic operations under this license need not comply with the torque criteria of Section 8.9.2(c) of American National Standard N432-1980.
16. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.
17. The licensee is authorized to transport licensed material in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."
18. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
19. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
20. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

37-05809-02

Docket or Reference Number

030-06135

Amendment No. 18

21. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

- A. Application dated February 7, 1992
- B. Letter dated February 21, 1995
- C. Letter dated July 21, 1995
- D. Letter dated August 17, 1995
- E. Letter dated November 16, 1998
- F. Letter dated November 24, 1998
- G. Letter dated January 18, 1999
- H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By

Original signed by Pamela J. Henderson

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81362030

ATTACHMENT 8

License Number, SNM-1120

Former Westinghouse Plutonium Operations



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION
OF ALLENTOWN
KING OF PRUSSIA, PENNSYLVANIA 19381-2198

March 1, 1999

Docket Nos. 030-06134

030-06135

030-32414

070-00698

070-01143

070-01503

040-08976

License Nos. 37-05809-01

37-05809-02

37-00497-15

SNM-770

SNM-1120

SNM-1460

SMB-1527

Control Nos. 126122

126123

126133

126125

126124

126126

126134

Louis J. Brinkman
Executive Vice President
and General Counsel
CBS Corporation
51 West 52 Street
New York, NY 10019-6188

Dear Mr. Brinkman:

This refers to your license amendment requests for License Nos. 37-05809-01, 37-05809-02, 37-00497-15, SNM-770, SNM-1120, SNM-1460 and SMB-1527. Enclosed with this letter are the amended licenses.

Please review the enclosed documents carefully and be sure that you understand and fully implement all the conditions incorporated into the amended license. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region I Office, Licensing Assistance Team, (610) 337-5093 or 5239, so that we can provide appropriate corrections and answers.

L. Brinkman

2

Thank you for your cooperation.

Sincerely,

A handwritten signature in cursive script, appearing to read "Pamela J. Henderson", is written over a circular stamp.

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety

Enclosures:

1. Amendment No. 23 for License No. 37-05809-01
2. Amendment No. 18 for License No. 37-05809-02
3. Amendment No. 3 for License No. 37-00497-15
4. Amendment No. 19 for License No. SNM-770
5. Amendment No. 14 for License No. SNM-1120
6. Amendment No. 8 for License No. SNM-1460
7. Amendment No. 3 for License No. SMB-1527

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	In accordance with the letter dated September 28, 1998,
1. Westinghouse Government Services Company LLC (WGS)	3. License number SNM-1120 is amended in its entirety to read as follows:
2. Power Systems Division P. O. Box 355 Pittsburgh, Pennsylvania 15230	4. Expiration date January 31, 2004
	5. Docket No. 070-01143 Reference No.

- | | | |
|---|----------------------------------|--|
| 6. Byproduct, source, and/or special nuclear material | 7. Chemical and/or physical form | 8. Maximum amount that licensee may possess at any one time under this license |
| A. Uranium-235 | A. Contamination | A. [REDACTED] |

9. Authorized use:

- A. Decontamination and decommissioning of facilities; packaging of stock material and radioactive waste; storage of radioactive material and packaged waste prior to shipment.

CONDITIONS

10. Licensed material may be used only at the licensee's facilities located at the Westinghouse Electric Company Cheswick site, Cheswick, Pennsylvania.
11. A. Licensed material shall be used by, or under the supervision of, individuals approved by the Radiation Safety Committee, C. T. Lim, Chairman.
- B. The Radiation Safety Officer for this license is Lisa A. Lamantia.
12. Licensed material shall not be used in or on human beings.
13. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
14. The licensee is authorized to transport licensed material only in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1120

Docket or Reference Number

070-01143

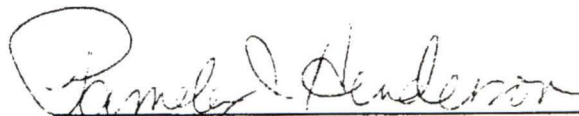
Amendment No. 14

15. Training of personnel for work under this license shall be conducted in accordance with the Cheswick Site Health Physics Manual which is part of NRC License No. 37-05809-01. This training shall be administered at least once each 12 months by the Cheswick site Radiation Safety Office.
16. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
17. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
18. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
19. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Letter dated July 25, 1978
 - B. Application dated July 18, 1991
 - C. Letter dated May 30, 1997
 - D. Letter dated December 22, 1998
 - E. Letter dated November 16, 1998
 - F. Letter dated November 24, 1998
 - G. Letter dated January 18, 1999
 - H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Date March 1, 1999

By



Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81366520

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee

1. Westinghouse Government Services Company LLC
(WGS)

2. Power Systems Division
P. O. Box 355
Pittsburgh, Pennsylvania 15230

In accordance with the letter dated
September 28, 1998,

3. License number SNM-1120 is amended in
its entirety to read as follows:

4. Expiration date January 31, 2004

5. Docket No. 070-01143
Reference No.

6. Byproduct, source, and/or special
nuclear material

7. Chemical and/or physical form

8. Maximum amount that licensee may
possess at any one time under this
license

A. Uranium-235

A. Contamination

A. XXXXXXXXXX

9. Authorized use:

A. Decontamination and decommissioning of facilities; packaging of stock material and radioactive waste; storage of radioactive material and packaged waste prior to shipment.

CONDITIONS

10. Licensed material may be used only at the licensee's facilities located at the Westinghouse Electric Company Cheswick site, Cheswick, Pennsylvania.
11. A. Licensed material shall be used by, or under the supervision of, individuals approved by the Radiation Safety Committee, C. T. Lim, Chairman.
- B. The Radiation Safety Officer for this license is Lisa A. Lamantia.
12. Licensed material shall not be used in or on human beings.
13. The licensee shall not use licensed material in field applications where activity is released except as provided otherwise by specific condition of this license.
14. The licensee is authorized to transport licensed material only in accordance with the provisions of 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

**MATERIALS LICENSE
SUPPLEMENTARY SHEET**

License Number

SNM-1120

Docket or Reference Number

070-01143

Amendment No. 14

15. Training of personnel for work under this license shall be conducted in accordance with the Cheswick Site Health Physics Manual which is part of NRC License No. 37-05809-01. This training shall be administered at least once each 12 months by the Cheswick site Radiation Safety Office.
16. The licensee shall meet all financial assurance requirements within 30 days of the closing of ownership transfer of assets.
17. Notwithstanding the date this amendment was signed, this amendment becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this license to the former licensee.
18. The licensee shall notify the Administrator of the appropriate NRC Regional Office listed in Appendix D to Part 20 by letter or facsimile not later than 30 days after the date of the letter transmitting this license to the former licensee of the date the transfer occurred.
19. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.
- A. Letter dated July 25, 1978
 - B. Application dated July 18, 1991
 - C. Letter dated May 30, 1997
 - D. Letter dated December 22, 1998
 - E. Letter dated November 16, 1998
 - F. Letter dated November 24, 1998
 - G. Letter dated January 18, 1999
 - H. Letter dated February 22, 1999

For the U.S. Nuclear Regulatory Commission

Original signed by Pamela J. HendersonDate March 1, 1999

By

Pamela J. Henderson, Acting Chief
Nuclear Materials Safety Branch 2
Division of Nuclear Materials Safety
Region I
King of Prussia, Pennsylvania 19406

81366520

ATTACHMENT 9

License Number, QAA-0638

Transportation Quality Assurance Approval



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 1, 1999

Mr. A. Joseph Nardi
Supervisory Engineer
EHS Regulatory Affairs
Westinghouse Electric Company
A Division of CBS
P. O. Box 355
Pittsburgh, PA 15230-0355

Dear Mr. Nardi:

Enclosed is Quality Assurance Program Approval for Radioactive Material Packages No. 0638, Revision No. 5, as amended. This Approval satisfies the requirements of 10 CFR § 71.12(b) for a Quality Assurance Program approved by the Commission.

Please note the conditions in the Approval.

This Approval is to be effective as of the closing date of the transfer of assets from "Westinghouse Electric Company A Division of CBS" to "Westinghouse Electric Company LLC (WELCO)". Such date is to be formally communicated to the NRC and incorporated as part of this Approval.

This Approval will remain in effect until the expiration date, indicated in Block No. 3. Termination of your materials license does not cause this Approval to be automatically terminated. If you wish to renew, amend, or terminate this Approval, please request it in writing.

Your request for amendment of your Certificates of Compliance for radioactive materials packages for transportation will be addressed in separate correspondence from NRC.

This letter also serves as a reminder that if you are using or planning to use an NRC-approved packaging, you must be registered for use of that packaging with NRC. Registration for use of NRC-approved packagings should be made pursuant to 10 CFR § 71.12(c)(3).

Sincerely,

A handwritten signature in dark ink, appearing to read "Patricia L. Eng", is written over the typed name.

Patricia L. Eng, Section Chief
Transportation and Storage Inspection Section
Spent Fuel Project Office, NMSS

Docket No. 71-0638

Enclosure: As stated

ATTACHMENT 10

License Number, QAA-0708

Transportation Quality Assurance Approval



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 1, 1999

Mr. A. Joseph Nardi
Supervisory Engineer
EHS Regulatory Affairs
Westinghouse Electric Company
A Division of CBS
P. O. Box 355
Pittsburgh, PA 15230-0355

Dear Mr. Nardi:

Enclosed is Quality Assurance Program Approval for Radioactive Material Packages No. 0708, Revision No. 4, as amended. This Approval satisfies the requirements of 10 CFR § 71.12(b) for a Quality Assurance Program approved by the Commission.

Please note the conditions in the Approval.

This Approval is to be effective as of the closing date of the transfer of assets from "Westinghouse Electric Company A Division of CBS" to "Westinghouse Electric Company LLC (WELCO)". Such date is to be formally communicated to the NRC and incorporated as part of this Approval.

This Approval will remain in effect until the expiration date, indicated in Block No. 3. Termination of your materials license does not cause this Approval to be automatically terminated. If you wish to renew, amend, or terminate this Approval, please request it in writing.

This letter also serves as a reminder that if you are using or planning to use an NRC-approved packaging, you must be registered for use of that packaging with NRC. Registration for use of NRC-approved packagings should be made pursuant to 10 CFR § 71.12(c)(3).

Sincerely,

A handwritten signature in dark ink, appearing to read "Patricia L. Eng", is written over the typed name.

Patricia L. Eng, Section Chief
Transportation and Storage Inspection Section
Spent Fuel Project Office, NMSS

Docket No. 71-0708

Enclosure: As stated

0708

4

**QUALITY ASSURANCE PROGRAM APPROVAL
FOR RADIOACTIVE MATERIAL PACKAGES**

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, and Title 10, Code of Federal Regulations, Chapter 1, Part 71, and in reliance on statements and representations heretofore made in Item 5 by the person named in Item 2, the Quality Assurance Program identified in Item 5 is hereby approved. This approval is issued to satisfy the requirements of Section 71.101 of 10 CFR Part 71. This approval is subject to all applicable rules, regulations, and orders of the U.S. Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

2. NAME

Westinghouse Electric Company LLC (WELCO)

3. EXPIRATION DATE

March 31, 2001

STREET ADDRESS

P.O. Box 355

4. DOCKET NUMBER

CITY

Pittsburgh

STATE

ZIP CODE

PA**15230-0355****71-0708**

5. QUALITY ASSURANCE PROGRAM APPLICATION DATE(S)

September 19, 1995, December 22, 1997 and January 18, 1999

6. CONDITIONS

1. Activities conducted under applicable criteria of Subpart H of 10 CFR Part 71 to be executed with regard to transportation packagings. Authorized activities include: design, procurement, fabrication, assembly, testing, modification, maintenance, repair, and use of transportation packagings.
2. Records shall be maintained in accordance with the provisions of 10 CFR Part 71. Specifically:
 - a. Records of each shipment of licensed material shall be maintained for three years after that shipment [10 CFR § 71.91(a)].
 - b. Records providing evidence of packaging quality shall be maintained for three years after the life of the packaging [10 CFR § 71.91(c)].
 - c. Records describing activities affecting packaging quality shall be maintained for three years after this Quality Assurance Program Approval is terminated [10 CFR § 71.135].

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

SIGNATURE

DATE

Patricia L. Eng
PATRICIA L. ENG, CHIEF
TRANSPORTATION AND STORAGE INSPECTION SECTION
SPENT FUEL PROJECT OFFICE
OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS

3/1/99

ATTACHMENT 11

License Number, COC-6400

Super Tiger, Model No. 6400



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 3, 1999

Mr. A. Joseph Nardi
License Administrator
Westinghouse Electric Company LLC
P.O. Box 355
Pittsburgh, PA 15230-0355

SUBJECT: MODEL NO. 6400 PACKAGE

Dear Mr. Nardi:

As requested by your application dated September 28, 1998, as supplemented on February 22, 1999, enclosed is Certificate of Compliance No. 6400, Revision No. 25, for the Model No. 6400 package. This certificate supersedes, in its entirety, Certificate of Compliance No. 6400, Revision No. 24, dated February 17, 1998.

Changes made to the enclosed certificate are indicated by vertical lines in the margin.

Those on the attached list have been registered as users of the package under the general license provisions of 10 CFR §71.12 or 49 CFR §173.471.

Notwithstanding the date the enclosed Certificate of Compliance was signed, the change in Certificate holder becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this Certificate of Compliance to the former Certificate holder. The Certificate holder shall notify the Director of the Spent Fuel Project Office, by letter or facsimile, not later than 30 days after the date of this letter, of the date the transfer occurred.

The approval constitutes authority to use the package for shipment of radioactive material and for the package to be shipped in accordance with the provisions of 49 CFR §173.471.

Sincerely,

A handwritten signature in cursive script, appearing to read "Cass R. Chappell", is written above the typed name.

Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Docket No.: 71-6400

Enclosures: 1. Certificate of Compliance
No. 6400, Rev. No. 25
2. Approval Record

cc w/encl: J. K. O'Steen, Department of Transportation
Registered Users

**CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIALS PACKAGES**

1. a. CERTIFICATE NUMBER 6400	b. REVISION NUMBER 25	c. PACKAGE IDENTIFICATION NUMBER USA/6400/B()F	d. PAGE NUMBER 1	e. TOTAL NUMBER PAGES 9
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2. PREAMBLE

- a. This certificate is issued to certify that the packaging and contents described in Item 5 below, meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

a. ISSUED TO (Name and Address)

Westinghouse Electric Company
LLC (WELCO)
P.O. Box 355
Pittsburgh, PA 15230-0355

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION:

Westinghouse Electric Corporation application
dated August 7, 1981, as supplemented.

71-6400

c. DOCKET NUMBER

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model No.: 6400
- (2) Description

A protective overpack which provides impact and thermal protection for its contents. The inner shell (cavity) is approximately 76" x 76" x 172" constructed of 3/16" thick and 10-gauge mild steel. Closure of the cavity is by a 1/4" thick aluminum plate with silicone rubber gasket which is bolted to the main inner shell. The cavity is centered and supported in an outer 3/16" thick steel jacket by approximately 32" of polyurethane foam insulation at the end and 10" on the sides. A removable section or cap consisting of approximately 34" of polyurethane foam insulation encased in steel with a silicone rubber gasket is bolted to the main outer steel jacket. The overall dimensions of the package are approximately 8' x 8' x 20'. Vent holes are provided on the sides and ends of the container. Set into each corner of the outer container are standard I.S.O. steel castings. The total weight including weight of the contents is [REDACTED].

(3) Drawings

Packaging is constructed in accordance with one of the following sets of drawings: (1) Protective Packaging, Inc, Drawing Nos. 32106, Sheet 1, Rev. F and 32106, Sheet 2, Rev. 0; or (2) Westinghouse Electric Corporation Drawing No. 2020D08, Sheet 1 and 2, Rev. 0; or (3) Babcock and Wilcox Company Drawing No. 11-D-2130, Rev. 0; or (4) Protective Packaging, Inc., Drawing Nos. 32106-1, Sheet 1, Rev. F and 32106, Sheet 2, Rev. 0, as modified by Nuclear Packaging Inc. Drawing No. E.G.-60-01D, Sheets 1 and 2, Rev. 0; or (5) Protective Packaging, Inc. Drawing No. 32395, Sheets 1 through 9, Rev. B, as modified by Sandia Laboratories letter dated May 8, 1980; or (6) Lawrence Livermore National Laboratory Drawing Nos. AAA81-108683-00, Rev. 0 and AAA81-110194-00, Rev. 0.

Page 2 - Certificate No. 6400 - Revision No. 25 - Docket No. 71-6400

5.(b) Contents

- (1) Large, decontaminated equipment waste of such size as not to fit into a 55-gallon drum (with legs or other readily removable appendages removed). Not to exceed [REDACTED] plutonium within the package.

Equipment waste surfaces containing more than [REDACTED] must be decontaminated to a smearable level of no more than [REDACTED] dpm/100 cm² prior to fixation or until successive decontamination cleaning operations do not reduce the smearable contamination levels by more than ten percent. After fixation, equipment waste surfaces must have a smearable level of contamination of no greater than [REDACTED] dpm/100 cm². Outer surfaces must have a smearable level of contamination of no greater than [REDACTED] dpm/100 cm². Prior to fixing of contamination, large equipment waste must be inspected to insure that: (a) all sharp or protruding objects have been removed, blunted or protected with packaging material, and (b) pipe caps, gasketed blind flanges, covers, etc., have been installed wherever possible. Following such inspection, the inner surfaces containing more than [REDACTED] must be fixed with "strip" or "clear" coating. The inner surface(s) may alternatively be fixed with a polyurethane foam.

The large equipment waste must be enclosed in a tight-fitting, 1-inch thick plywood box constructed in accordance with Westinghouse Electric Corporation's Drawing No. 1620E43, Sheets 1, 2, 3, and 4, Rev. 3; a tight fitting 3/16" thick corrugated steel box constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified); or enclosed in a tight fitting box constructed in accordance with General Electric Company Drawing Nos. 908E614, Rev. 1, and 908E619, Rev. 2 or 908E648, Rev. 0 or 908E649, Rev. 0; or enclosed in a tight fitting box constructed in accordance with Babcock and Wilcox Company Drawing No. LRC-70019 H, Rev. 2. The space between the equipment and the box must be filled with foam (1" minimum foam thickness) and between equipment (1/2" minimum foam thickness). Alternatively, gloveboxes contaminated and fixed as described above may be broken down as follows:

Glovebox windows are removed and separately packaged in 12-mil thick PVC bags and sealed. The inner bag is tape sealed and the outer bag is heat sealed.

Glovebox panels are cut to dimensions to fit inside the 3/16" thick corrugated steel burial crates constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified). All sharp or protruding objects are removed, blunted, or protected with packaging material. The glovebox panels are bundled such that internal box surfaces are facing inward. Cut glovebox panels from not more than one glovebox are banded with metal strap banding such that two metal strap bands in each direction are placed around the length and width of the glovebox sections. The glovebox window and cut panel packages are enclosed and foamed in place within the box.

Blocking or dunnage is placed within the box to ensure a one inch foam barrier on the sides and bottom of the box. Likewise, dunnage is provided between the banded glovebox sections to maintain a 1/2" thick foam barrier between banded packages.

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5.(b) Contents (continued)

- (2) Decontaminated hard waste items, such as equipment, metal cans, tools, etc., must be double bagged within 12-mil thick PVC with each bag heat sealed. The total fissile quantity of all the sealed packages in one container must not exceed [REDACTED].

Hard waste surfaces must be decontaminated to a smearable level of no more than [REDACTED] dpm/100 cm² prior to fixation or until successive decontamination cleaning operations do not reduce the smearable contamination levels by more than 10 percent. After fixation, hard waste surfaces must have a smearable level of contamination of no greater than [REDACTED] dpm/100 cm². Prior to fixing of contamination, hard waste must be inspected to insure that sharp or protruding objects have been removed, blunted, or protected with packaging material. Following such inspection, the outer surfaces must be fixed with "strip" or "clear" coating. Hard waste items such as furnace shells, muffles, or other items with large cavities not accessible for decontamination must be filled with foam within the cavities. Surfaces that are not easily accessible, e.g., interiors of small diameter tubing and piping which were in contact with process materials, must have been swabbed or immersed in cleaning solution to insure removal of residual material. Open ends of the tubing and piping must be sealed using mechanical fittings.

Alternately, large heavy walled process glassware must be painted inside and outside to fix contamination and double bagged in 12-mil thick PVC with each bag heat sealed. The glassware must be secured in a box constructed in accordance with General Electric Company Drawing No. 272E81-4, Rev. 0. The box must be filled with foam and total activity limited to less than [REDACTED] in a box.

Alternately, stainless steel transfer tubes and HEPA filters must be double bagged in 12-mil thick PVC with each bag heat sealed. The tubes/filters must be secured in a box constructed in accordance with General Electric Company Drawing No. 272E81-28, Rev. 0. The box must be filled with foam and total activity limited to less than [REDACTED] in a box.

Alternately, round steel ducting must be capped and secured in a box constructed in accordance with General Electric Company Drawing No. 272E81-29, Rev. 0; 272E81-30, Rev. 0; or 272E81-31, Rev. 0. Outer surfaces ducting will have a smearable level of contamination no greater than 20 d/m/100 cm². The box must be filled with foam and total activity limited to less than [REDACTED] in a box.

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5.(b) Contents (continued)

Sealed packages and boxes of hard waste must be enclosed in a tight-fitting, 1-inch thick plywood box constructed in accordance with Westinghouse Electric Corporation's Drawing No. 1620E43, Sheets 1, 2, 3, and 4, Rev. 3; a tight-fitting 3/16" thick corrugated steel box constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified); enclosed in a tight fitting box constructed in accordance with General Electric Company Drawing Nos. 908E614, Rev. 1 and 908E619, Rev. 2 or 908E648, Rev. 0 or 908E649, Rev. 0; or enclosed in a tight fitting box constructed in accordance with Babcock and Wilcox Company Drawing No. LRC-70019 H, Rev. 2. The space between the packages and the box must be filled with foam to a minimum thickness of 1 inch. Void spaces between the sealed packages must be filled with foam (1/2" minimum foam thickness).

- (3) Glove box absolute (HEPA) filters must be double bagged within 12-mil thick PVC, with each bag heat sealed and packaged within DOT Specification 17H or 17C steel drums (maximum size of 55 gallons). Each drum must be lined with a sealed plastic liner and equipped with a standard drum closure. Each drum must not exceed a fissile quantity of [REDACTED]. Sealed drums must be enclosed in a tight-fitting 1-inch thick plywood box constructed in accordance with Westinghouse Electric Corporation's Drawing No. 1620E43, Sheets 1, 2, 3, and 4, Rev. 3; a tight-fitting 3/16" thick corrugated steel box constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified); enclosed in a tight fitting box constructed in accordance with General Electric Company Drawing Nos. 908E614, Rev. 1 and 908E619, Rev. 2, or 908E648, Rev. 0, or 908E649, Rev. 0; or enclosed in a tight fitting box constructed in accordance with Babcock and Wilcox Company Drawing No. LRC-70019 H, Rev. 2. The space between the drums and the box must be filled with foam to a minimum thickness of 1 inch. Void spaces between drums must be filled with foam (1/2" minimum foam thickness).
- (4) Soft waste items such as sheeting, gloves, paper, prefilter media, polyethylene bottles, shoe covers, etc., must be double bagged in 12-mil thick PVC, with each bag heat sealed (bag size must not exceed 22" x 16" x 10") and packaged within DOT Specification 17H or 17C steel drums (maximum size of 55 gallons). Each drum must be lined with a sealed plastic liner and equipped with a standard drum closure. Each drum must not exceed a fissile quantity of [REDACTED]. Sealed drums must be enclosed in a tight-fitting 1-inch thick plywood box constructed in accordance with Westinghouse Electric Corporation's Drawing No. 1620E43, Sheets 1, 2, 3, and 4, Rev. 3; a tight-fitting 3/16" thick corrugated steel box constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified); or enclosed in a tight fitting box constructed in accordance with Babcock and Wilcox Company Drawing No. LRC-70019 H, Rev. 2. The space between the drums and the box must be filled with foam to a minimum thickness of 1 inch. Void spaces between drums must be filled with foam (1/2" minimum foam thickness).

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5.(b) Contents (continued)

- (5) Liquid waste (decontamination solutions only) must be solidified in concrete in a 30-gallon drum which must be sealed in a plastic bag and centered and supported in a DOT Specification 17H or 17C 55-gallon steel drum by absorbent material. The 55-gallon drum must be lined with a sealed plastic liner and equipped with a standard drum closure. Each drum must not exceed a fissile quantity of [REDACTED].

Alternatively, liquid waste is solidified in concrete in maximum size one (1) gallon packages which are double bagged and heat sealed in 12-mil thick PVC and placed with a DOT Specification 17H or 17C steel drum (maximum size of 55 gallons). The drum is lined with a sealed plastic liner and equipped with a standard drum closure. Each 55-gallon drum must not exceed a fissile quantity of [REDACTED]. For drums smaller than 55 gallons, the total fissile quantity of all the sealed packages (drums) in one container must not exceed [REDACTED]. Sealed drums must be enclosed in a tight-fitting 1-inch thick plywood box constructed in accordance with Westinghouse Electric Corporation's Drawing No. 1620E43, Sheets 1, 2, 3, and 4, Rev. 3; or a tight-fitting 3/16" thick corrugated steel box constructed in accordance with Rockwell Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified); enclosed in a tight-fitting box constructed in accordance with General Electric Company Drawing Nos. 908E614, Rev. 1 and 908E619, Rev. 2 or 908E648, Rev. 0 or 908E649, Rev. 0; or enclosed in a tight fitting box constructed in accordance with Babcock and Wilcox Company Drawing No. LRC-70019 H, Rev. 2. The space between the drums and the box must be filled with foam to a minimum thickness of 1 inch. Void spaces between drums must be filled with foam (1/2" minimum foam thickness).

- (6) Uranium 233 oxide and thorium oxide in the form of intact LWBR-type fuel rods with the following limitations:
- (i) Rods must be packaged within the Model No. 6400 packaging as described in Section 1 of WAPD-LP(FE)-220, Rev. 3 (February 1983);
 - (ii) The fuel content must not exceed [REDACTED] U-233 per shipment;
 - (iii) All rod storage containers must be filled to capacity (at least 70% of cross-sectional area) with rods or aluminum shim stock;
 - (iv) Each rod storage container must contain not more than one sub-container of 5/9 or 12 w/o BMU seed rods;
 - (v) Each rod storage container must weigh not more than [REDACTED];
 - (vi) The fuel rod heat generation must not exceed [REDACTED]; and
 - (vii) Operating Procedures and Acceptance Tests and Maintenance Program must be modified to meet the requirement of Item 11 of this approval.

Page 6 - Certificate No. 6400 - Revision No. 25 - Docket No. 71-6400

5.(b) Contents (continued)

- (7) Liquid analytical residues from the dissolution of spent reactor fuel rods, solidified in cement (see table, p. 3 of application*). The cement is contained in 1.5-gal steel can closed with a slip cover lid. The two primary cans are packed in a secondary steel can sealed with a press fit lid (see Figure 2 of application*). The secondary containment package contents are placed within a radiation shield (lid secured with six (6), 1/2"-13UNC bolts with welds in accordance with application*) centered in a DOT Specification 17-C 55-gal steel drum (see Figure 1 of application*). The drums are sealed with styrene-butadiene rubber gasket contained with a standard drum closer. Total weight of the drum will be less than 1,450 lb, and each drum will not exceed a fissile quantity of [REDACTED] and [REDACTED] of fission products.

Six (6), 55-gal sealed drum assemblies will be enclosed in a tight-fitting 3/16-in thick corrugated steel box constructed in accordance with Rockwell-Hanford Operations' Drawing No. H-2-91888, Sheet 1, Rev. 0 (modified or unmodified). The space between the drums and the box must be filled with foam to a minimum thickness of 1 inch. Void spaces between drums must be fitted with foam to a minimum thickness of 1/2 inch. Two (2) corrugated steel box assemblies may be transported in the packaging.

* U.S. Department of Energy letter dated April 15, 1983.

- (8) Uranium 233 oxide and thorium oxide in the form of intact LWBR-type fuel rods with the following limitations:
- (i) Rods must be packaged as shown in Figure 4, Application dated July 8, 1983, and contained within the Model No. NNFD-SA-2 packaging (Certificate of Compliance No. 5910);
 - (ii) The fuel content must not exceed [REDACTED] U-233 per shipment;
 - (iii) Each loaded LWBR Rod Transport Box must weigh not more than [REDACTED];
 - (iv) The fuel rod heat generation rate must not exceed [REDACTED] and
 - (v) Operating Procedures and Acceptance Tests and Maintenance Program must be modified to meet the requirement of Item 11 of this approval.

Page 7 - Certificate No. 6400 - Revision No. 25 - Docket No. 71-6400

5.(b) Contents (continued)

- (9) Maximum of four (4) Cf-252 sources with the following limitations:
- (i) Each source must be doubly encapsulated with the inner capsule meeting the requirements for special form radioactive material;
 - (ii) The total Cf-252 content must not exceed [REDACTED];
 - (iii) The sources must be packaged in a shielded container as described in Chapter 1 of WAPD-LP(CE)POB-591 (January 1984); and
 - (iv) The decay heat generation from the source material must not exceed [REDACTED].
- (10) Compressed krypton-85 gas in mixture with other non-radioactive gases that are chemically compatible with the 3AA2015 cylinder. No fissile material (Requirement of 5.(c) does not apply). Shipment of krypton-85 gas is subject to the following limitations:
- (i) Radioactivity not to exceed [REDACTED]. Maximum internal decay heat not to exceed [REDACTED]. Maximum volume of krypton-85 and other non-radioactive gases shall not exceed 1480 liters at STP (1 atm, 25°C);
 - (ii) The maximum initial fill pressure shall not exceed 500 psig at 25°C;
 - (iii) The DOT Specification 3AA2015 gas cylinder shall be certified for an operating load of 2,015 psig, at least once every 5 years by testing to 3,360 psig;
 - (iv) A minimum of 24 hours after loading with krypton-85 gas the krypton packaging primary containment shall have a leak rate of less than [REDACTED] per second. The leak test shall be performed with the containment vessel within the lead shield container prior to placement within its thermal overpack;
 - (v) Content of the package shall be verified by mass spec analysis;
 - (vi) Acceptance, maintenance and use of the krypton package shall be in accordance with the procedures and requirements of Chapter 7 and 8 of Westinghouse Idaho Nuclear Company, Inc. Report No. WIN-236, Revision 1, March 1988. The retaining ring shall be tightened around the gas cylinder to a 40 to 50 inch-pound torque;
 - (vii) The position and securement of the krypton package within the Model No. 6400 is as specified in Westinghouse Idaho Nuclear Company, Inc. Drawing No. 059888;
 - (viii) Krypton package must be enclosed within a tight fitting plywood box constructed in accordance with Westinghouse Idaho Nuclear Company, Inc. Drawing No. 059886.

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5.(c) Transport Index for Criticality Control

Minimum transport index to be shown on
label for nuclear criticality control: 100

6. The polyurethane foam must be Instapak 200, or equivalent.
7. The maximum weight of the contents including secondary packaging, dunnage, shoring and bracing must not exceed [REDACTED].
8. Sufficient dunnage, shoring and/or bracing must be utilized to minimize secondary impact of the secondary packaging within the cavity under accident conditions.
9. Protrusions from secondary packaging such as lifting eyes, etc., must be positioned such that they will not contact the cavity walls, or shoring must be provided to prevent puncture of the cavity walls by the protrusions under the accident conditions.
10. Contents must be positioned in the cavity such that the center of gravity of the loaded package is substantially the same as the center of gravity of an empty package.
11. The cavity of the overpack must be vented through an absolute filter to equalize pressure between the outside and inside of the overpack.
12. Contents packaged under the conditions of this certificate of compliance are exempt from the requirements of 10 CFR §71.63. Condition 5(c) of this certificate of compliance is not applicable where the fissile material is excluded as provided by 10 CFR §71.53.
13. In addition to the requirements of Subpart G of 10 CFR Part 71, the package must be prepared for shipment, operated, and maintained in accordance with "Operating Inspection and Maintenance Procedure No. CSK-003, Rev. 0," included in the Westinghouse Electric Corporation supplement dated April 14, 1992.
14. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR §71.12.
15. Expiration date: July 31, 2002.

Page 9 - Certificate No. 6400 - Revision No. 25 - Docket No. 71-6400

REFERENCES

Westinghouse Electric Corporation application dated August 7, 1981.

General Electric Company supplement dated: October 1, 1981.

Babcock and Wilcox Company supplements dated: March 8, 1982; and January 10, 1985.

Department of Energy, Division of Naval Reactors, supplements dated: April 22, and July 8, 1983; and March 5, 1984.


Department of Energy, Chicago Operations Office, supplement dated: April 15, 1983.

Department of Energy, Washington, DC, supplement dated: June 6, 1988.

Westinghouse Electric Corporation supplements dated: April 14, 1992; and April 14, 1997.

Westinghouse Electric Company, Division of CBS Corporation supplement dated:
December 22, 1997, September 28, 1998 and February 22, 1999.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION



Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards


Date: 3/3/99



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

APPROVAL RECORD
Model No. 6400 Package
Certificate of Compliance No. 6400
Revision No. 25

By application dated September 28, 1998, as supplemented on February 22, 1999, CBS Corporation, requested a name change to Certificate of Compliance No. 6400 for the Model No. 6400 Package. The applicant requested that the name be changed from Westinghouse Electric Company, a Division of CBS Corporation, to Westinghouse Electric Company LLC (WELCO). No design changes were requested to the package.


Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Date: 11-17-99

ATTACHMENT 12

License Number, COC-9239

New Fuel Shipping Container, MCC Series



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 3, 1999

Mr. A. Joseph Nardi
License Administrator
Westinghouse Electric Company LLC
P.O. Box 355
Pittsburgh, PA 15230-0355

SUBJECT: MODEL NOS. MCC-3, MCC-4 and MCC-5 PACKAGES

Dear Mr. Nardi:

As requested by your application dated September 28, 1998, as supplemented on February 22, 1999, enclosed is Certificate of Compliance No. 9239, Revision No. 8, for the Model Nos. MCC-3, MCC-4, and MCC-5 packages. This certificate supersedes, in its entirety, Certificate of Compliance No. 9239, Revision No. 7, dated February 22, 1999.

Changes made to the enclosed certificate are indicated by vertical lines in the margin.

Those on the attached list have been registered as users of the package under the general license provisions of 10 CFR §71.12 or 49 CFR §173.471.

Notwithstanding the date the enclosed Certificate of Compliance was signed, the change in Certificate holder becomes effective on the date of the closing of ownership transfer of assets, so long as that transfer occurs not later than 30 days from the date of the letter transmitting this Certificate of Compliance to the former Certificate holder. The Certificate holder shall notify the Director of the Spent Fuel Project Office, by letter or facsimile, not later than 30 days after the date of this letter, of the date the transfer occurred.

The approval constitutes authority to use the package for shipment of radioactive material and for the package to be shipped in accordance with the provisions of 49 CFR §173.471.

Sincerely,

A handwritten signature in cursive script, reading "Cass R. Chappell", is written above the typed name.

Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Docket No.: 71-9239

Enclosures: 1. Certificate of Compliance
No. 9239, Rev. No. 8
2. Approval Record

cc w/encl: J. K. O'Steen, Department of Transportation
Registered Users

**CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIALS PACKAGES**

U.S. NUCLEAR REGULATORY COMMISSION

1. a. CERTIFICATE NUMBER 9239	b. REVISION NUMBER 8	c. PACKAGE IDENTIFICATION NUMBER USA/9239/AF	d. PAGE NUMBER 1	e. TOTAL NUMBER PAGES 4
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2. PREAMBLE

- a. This certificate is issued to certify that the packaging and contents described in Item 5 below, meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

- a. ISSUED TO (Name and Address)
- b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION:

Westinghouse Electric Company
LLC (WELCO)
P.O. Box 355
Pittsburgh, PA 15230

Westinghouse Electric Corporation application
dated January 31, 1991, as supplemented.

c. DOCKET NUMBER **71-9239**

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model Nos.: MCC-3, MCC-4, and MCC-5
- (2) Description

The MCC packages are shipping containers for unirradiated uranium oxide fuel assemblies. The packagings consist of a steel fuel element cradle assembly equipped with a strongback and an adjustable fuel element clamping assembly. The cradle assembly is shock mounted to a 13-gauge carbon steel outer container by shear mounts. The MCC-3 container is closed with thirty 1/2-inch T-bolts. The MCC-4 and MCC-5 containers are closed with fifty 1/2-inch T-bolts.

The MCC-3 and MCC-4 containers are permanently equipped with vertical Gd_2O_3 neutron absorber plates that are mounted on the center wall of the strongback. Additional horizontal Gd_2O_3 neutron absorber plates, mounted on the underside of the strongback, are required for the contents as specified.

The MCC-5 container is permanently equipped with both the vertical and horizontal Gd_2O_3 neutron absorber plates. Additional vee-shaped, guided Gd_2O_3 neutron absorber plates are required for the contents as specified.

Approximate dimensions of the MCC-3 packaging are 44-1/2 inches O.D. by 194-1/2 inches long. The gross weight of the packaging and contents is [REDACTED]. The maximum weight of the contents is [REDACTED].

Approximate dimensions of the MCC-4 packaging are 44-1/2 inches O.D. by 226 inches long. The gross weight of the packaging and contents is [REDACTED]. The maximum weight of the contents is [REDACTED] s.

Page 2 - Certificate No. 9239 - Revision No. 8 - Docket No. 71-9239

5. (a) Packaging (continued)

Approximate dimensions of the MCC-5 packaging are 44-1/2 inches O.D. by 226 inches long. The gross weight of the packaging and contents is [REDACTED]. The maximum weight of the contents is [REDACTED].

(3) Drawings

The MCC-3 packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL301, Sheets 1, 2 and 3, Rev. 4.

The MCC-4 packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL401, Sheets 1, 2, 3, and 4, Rev. 6.

The MCC-5 packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL501, Sheets 1 through 9, Rev. 3.

(b) Contents

(1) Type and form of material

Unirradiated PWR uranium dioxide fuel assemblies with a maximum uranium-235 enrichment of 5.0 weight percent.

The fuel assemblies shall meet the specifications given in Westinghouse Drawing No. 6481E15, Rev. 3, and in the following tables of Appendix 1-4 of the application, as supplemented:

Table 1-4.1, Rev. 6, dated July 26, 1994	Fuel Assembly Parameters 14x14 Type Fuel Assemblies
Table 1-4.2, Rev. 6, dated July 26, 1994	Fuel Assembly Parameters 15x15 Type Fuel Assemblies
Table 1-4.3, Rev. 6, dated July 26, 1994	Fuel Assembly Parameters 16x16 Type Fuel Assemblies*
Table 1-4.4, Rev. 7, dated February 19, 1999	Fuel Assembly Parameters 17x17 Type Fuel Assemblies*
Table 1-4.5, Rev. 4, dated January 14, 1994	Fuel Assembly Parameters VVER-1000 Type Fuel Assembly**

* 16x16 CE fuel assemblies and the 17x17 W-STD/XL fuel assemblies may be shipped only in the Model No. MCC-4 package.

** VVER-1000 fuel assemblies may be shipped only in the Model No. MCC-5 package.

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5. (b) Contents (continued)

(2) Maximum quantity of material per package

Two (2) fuel assemblies

(c) Transport Index for Criticality Control

Minimum transport index to be shown on
label for nuclear criticality control: 0.4

6. For shipments of 14x14, 15x15, 16x16, and 17x17 fuel assemblies with U-235 enrichments of over 4.65 wt% and up to 5.0 wt%, horizontal Gd_2O_3 neutron absorber plates shall be positioned underneath each assembly. The horizontal absorber plates shall be placed horizontally on the underside of the strongback, as shown on Westinghouse Electric Corporation Drawing No. MCCL301, Sheet 1, Rev. 4, or Westinghouse Electric Corporation Drawing No. MCCL401, Sheet 1, Rev. 6.
7. For shipments of VVER-1000 fuel assemblies with U-235 enrichments of over 4.80 wt% and up to 5.0 wt%, a guided Gd_2O_3 neutron absorber plate shall be positioned underneath each assembly. The guided absorber plates shall be placed horizontally on the topside of the strongback, as shown on Westinghouse Electric Corporation Drawing No. MCCL501, Sheet 5, Rev. 3.
8. Each fuel assembly must be unsheathed or must be enclosed in an unsealed plastic sheath which may not extend beyond the ends of the fuel assembly. The ends of the sheath may not be folded or taped in any manner that would prevent flow of liquids into or out of the sheathed fuel assembly.
9. The dimensions, minimum Gd_2O_3 loading and coating specifications, and acceptance testing of the neutron absorber plates shall be in accordance with the " Gd_2O_3 Neutron Absorber Plates Specifications," Appendix 1-6, Rev. 2, dated January 14, 1994, of the application. The minimum Gd_2O_3 coating areal density on the vertical and horizontal neutron absorber plates shall be 0.054 g- Gd_2O_3 /cm². The minimum Gd_2O_3 coating areal density on guided neutron absorber plates shall be 0.027 g- Gd_2O_3 /cm².
10. In addition to the requirements of Subpart G of 10 CFR Part 71:
- (a) The MCC-3 packaging shall be acceptance tested in accordance with Notes 3, 4, and 5 of Westinghouse Electric Corporation Drawing No. MCCL301, Sheet 1, Rev. 4, and with the Acceptance Tests in supplement dated March 24, 1997.
- (b) The MCC-4 packaging shall be acceptance tested in accordance with Notes 4, 5, and 6 of Westinghouse Electric Corporation Drawing No. MCCL401, Sheet 2, Rev. 6, and with the Acceptance Tests in supplement dated March 24, 1997.
- (c) The MCC-5 packaging shall be acceptance tested in accordance with the Acceptance Tests in supplement dated March 24, 1997.

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- (d) The packages shall be maintained in accordance with the Maintenance Program in supplement dated March 24, 1997.
 - (e) The packages shall be operated and prepared for shipment in accordance with the Operating Procedures in supplement dated January 14, 1994, as revised in supplement dated August 2, 1994.
11. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR §71.12.
12. Expiration date: March 31, 2002.

REFERENCES

Westinghouse Electric Corporation application dated January 31, 1991.

Supplements dated: October 2, October 9, November 1, and November 13, 1991; January 27, March 30, May 12, and June 18, 1992; August 18, 1993; January 14, April 22, May 24, July 26, and August 2, 1994; October 1, 1996; March 24 and December 22, 1997; September 28, 1998, February 19 and February 22, 1999.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Cass R. Chappell

Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards


Date: 3/3/99



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

APPROVAL RECORD
Model Nos. MCC-3, MCC-4, and MCC-5 Packages
Certificate of Compliance No. 9239
Revision No. 8

By application dated September 28, 1998, as supplemented on February 22, 1999, CBS Corporation, requested a name change to Certificate of Compliance No. 9239 for the Model Nos. MCC-3, MCC-4 and MCC-5 Packages. The applicant requested that the name be changed from Westinghouse Electric Company, a Division of CBS Corporation, to Westinghouse Electric Company LLC (WELCO). No design changes were requested to the package.


Cass R. Chappell, Chief
Package Certification Section
Spent Fuel Project Office
Office of Nuclear Material Safety
and Safeguards

Date: 9/3/99

ATTACHMENT 13

License Number, COC-1001

Irradiated Fuel Storage Cask - MC-10

Docket No. 72-1001 (Project No. M-41)
Package Identification . . . USA/72-1001

Westinghouse Electric Corp.
ATTN: William J. Johnson, Manager
Nuclear Safety Department
P. O. Box 355
Pittsburgh, Pennsylvania 15230-0355

AUG 17 1990

Gentlemen:

Pursuant to Title 10, Code of Federal Regulations, Part 72 (55 FR 29181), we are enclosing is Certificate of Compliance No. 1001, issued on the basis of the safety analysis report of the cask design, Model No. MC-10, identified in "Topical Safety Analysis Report for the Westinghouse MC-10 Cask for an Independent Spent Fuel Storage Installation (Dry Storage) (TSAR)."

This Certificate of Compliance constitutes authorization for a twenty-year term. Casks of the Model No. MC-10 are approved for general use by holders of 10 CFR Part 50 licenses for use at civilian power reactor sites under the general license issued pursuant to §72.210, 10 CFR Part 72, subject to the conditions specified by §72.212 and Conditions for Cask Use.

If you have any questions regarding this issuance of Certificate of Compliance No. 1001, please contact me or John P. Roberts of my staff (301-492-0608).

Sincerely,

Original Signed by

Charles J. Haughney, Chief
Fuel Cycle Safety Branch
Division of Industrial and
Medical Nuclear Safety
Office of Nuclear Material Safety
and Safeguards

Enclosures:
Certificate of Compliance
No. 1001

cc: Mr. W. L. Stewart, VEPCO
Mr. M. Smith, VEPCO

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Certificate of Compliance

FOR DRY SPENT FUEL STORAGE CASKS

10 CFR 72

AUG 17 1990

1. a. **CERTIFICATE NUMBER:** 1001

b. **REVISION NUMBER:** 0

c. **PACKAGE IDENTIFICATION NUMBER:** USA/72-1001

d. **PAGE NUMBER:** 1

e. **TOTAL NUMBER OF PAGES:** 3

2. **Preamble** This certificate is issued to certify that the cask and contents, described in item 5 below, meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 72, "Licensing Requirements for the Independent Storage of Spent Nuclear Fuel and High-Level Radioactive Waste."

3. **THIS CERTIFICATE** is issued on the basis of a safety analysis report of the cask design.

a. **PREPARED BY (Name and Address)**

Westinghouse Electric Corp.
Power Systems
P.O. Box 355
Pittsburgh, PA 15230-0355 (USA)

b. **TITLE AND IDENTIFICATION
OF REPORT OR APPLICATION**

Topical Safety Analysis Report
for the Westinghouse MC-10 Cask
for an Independent Spent Fuel
Storage Installation (Dry Storage)
(TSAR)

c. **DOCKET NUMBER** 72-1001

4. **CONDITIONS** This certificate is conditional upon fulfilling the requirements of 10 CFR 72, as applicable, and the conditions specified below.

5. Cask

a. **Model No:** MC-10

b. **Description**

The MC-10 cask is designed for the storage and shipment of irradiated spent fuel assemblies. This certificate addresses spent fuel handling, transfer, and storage on an NRC-licensed nuclear reactor site but does not address any use or certification of this cask design for offsite transport of spent fuel.

AUG 17 1990

The MC-10 cask consists of a thick-walled forged steel cylinder and weighs approximately 85.2 tonne (94 ton). The cask has a cylindrical cask cavity which holds a fuel basket and is designed to accommodate PWR fuel assemblies. The loaded weight of the cask is about tonne (ton).

The overall length is 4775 mm (188 in), and the side wall thickness including neutron absorber and without fins is 333.8 mm (13.1 in). The cross-sectional diameter of the cask including neutron absorber is 2394 mm (94.3 in). The overall diameter including fins is 2725 mm (107.28 in). The cask cavity has a diameter of 1727 mm (68 in) and a length of 4130 mm (162.6 in). The cask body is low alloy steel approximately 2235 mm (88 in) in diameter and 4699 mm (185 in) long. The forged steel walls and bottom are approximately 254 mm (10 in) thick to provide radiation (gamma) shielding and structural integrity. Three covers seal the top end of the cask cylinder. A low alloy steel cover, approximately 127 mm (5 in) thick with metallic O-rings provides initial seal and shielding following fuel loading. A carbon steel cover approximately 89 mm (3.5 in) thick with a metallic O-ring provides the primary seal.

The cask contains a basket assembly which consists of 24 storage locations utilizing a honeycomb-type basket structure. The stainless steel basket structure maintains the subcritical array of storage locations, provides lateral structural integrity, and conducts fuel assembly decay heat to the cask wall.

Each of the 24 removable cell storage locations consists of an enclosure, neutron poison material, and wrappers. The enclosure is a stainless steel sheet, 2 mm (.75 in) thick by 890 mm (35.06 in) basket structure. The upper ends of the enclosure walls are flared to facilitate fuel loading. Neutron absorbing material is attached to the enclosure walls and held in place with a stainless steel wrapper welded to the panel.

c. Drawing

The Model No. MC-10 dry spent fuel storage cask is described by drawings in Figures 4.2-1 thru 4.2-10 of the TSAR.

d. Basic Components

The Basic Components of the Model No. MC-10 storage cask that are important to safety are listed on page 3.4-1 of the TSAR.

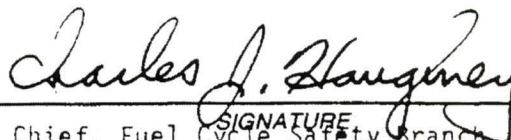
6. Cask fabrication activities shall be conducted in accordance with the reviewed and approved quality assurance program submitted with the TSAR.
7. Notification of cask fabrication schedules shall be made in accordance with the requirements of §72.232(c), 10 CFR Part 72.

8. Casks of the Model No. MC-10 authorized by this certificate are hereby approved ^{AUG 17 1999} for general use by holders of 10 CFR Part 50 licenses for nuclear reactors at reactor sites under the general license issued pursuant to §72.210, 10 CFR Part 72, subject to the conditions specified by §72.212 and the attached Conditions for Cask Use.

9. Expiration Date:

August 31, 2010

FOR THE NUCLEAR REGULATORY COMMISSION



SIGNATURE
Chief, Fuel Cycle Safety Branch
Division of Industrial and
Medical Nuclear Safety
Office of Nuclear Material Safety
and Safeguards

CONDITIONS FOR CASK USE
CERTIFICATE OF COMPLIANCE

72-1001

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1.0 INTRODUCTION

These Conditions for Cask Use govern the safety of the receipt, possession, and storage of irradiated nuclear fuel at an Independent Spent Fuel Storage Installation (ISFSI) and the transfer of such irradiated nuclear fuel to and from a Nuclear Power Station and its ISFSI.

1.1 GENERAL CONDITIONS

1.1.1 Operating Procedures

Written operating procedures shall be prepared for cask handling, movement, emplacement, surveillance, and maintenance.

1.1.2 Quality Assurance

Activities at the ISFSI shall be conducted in accordance with the requirements of Appendix B, 10 CFR Part 50.

1.2 PREOPERATIONAL CONDITIONS

The user shall not allow the initial loading of spent nuclear fuel in the Model No. MC-10 cask until such time as the following preoperational license conditions are satisfied:

- (1) A training module shall be developed for the Station Training Program establishing an ISFSI Training and Certification Program which will cover the following:
 - a. Cask Design (overview)
 - b. ISFSI Facility Design (overview)
 - c. ISFSI Safety Analysis (overview)
 - d. Fuel loading and cask handling procedures and abnormal procedures

(2) A training exercise (Dry Run) of cask loading and handling activities shall be held which shall include but not be limited to:

- a. Moving cask in and out of spent fuel pool area.
- b. Loading a fuel assembly (using dummy assembly).
- c. Cask sealing and cover gas backfilling operations.
- d. Moving cask to and placing it on the storage pad.
- e. Returning the cask to the reactor.
- f. Unloading the cask assuming fuel cladding failure.
- g. Cask decontamination.

2.0 FUNCTIONAL AND OPERATING LIMITS

2.1 FUEL TO BE STORED AT ISFSI

2.1.1 Specification

The spent nuclear fuel to be received and stored at the ISFSI in MC-10 casks shall meet the following requirements:

- (1) Only irradiated 14 x 14, 15 x 15, and 17 x 17 PWR fuel assemblies with Zircaloy fuel rod cladding may be used. Total assemblies per cask ≤ 24 .
- (2) Maximum initial enrichment shall not exceed 3.7 weight percent U-235 for fuel stored in the stainless steel basket (with boral plates attached to each of the 24 cell enclosure walls) reviewed and found acceptable.

- (3) Maximum assembly average burnup shall not exceed [REDACTED] per metric ton uranium and specific power shall not exceed [REDACTED].
- (4) Maximum heat generation rate shall not exceed [REDACTED] per fuel assembly.
- (5) Fuel shall be intact unconsolidated fuel. Partial fuel assemblies, that is, fuel assemblies from which fuel pins are missing must not be stored unless dummy fuel pins are used to displace an amount of water equal to that displaced by the original pins.
- (6) Fuel assemblies known or suspected to have structural defects sufficiently severe to adversely affect fuel handling and transfer capability unless canned shall not be loaded into the cask for storage.
- (7) A procedure shall be developed for the documentation of the characterizations performed to select spent fuel to be stored in the casks. Such procedure shall include independent verification of fuel assembly selection by an individual other than the original individual making the selection.
- (8) Immediately prior to insertion of a spent fuel assembly into a cask, the identity of the assembly shall be independently verified by two individuals.

2.1.2 Basis

The design criteria and subsequent safety analysis assumed certain characteristics and limitations for the fuels that are to be received and stored. Specification 2.1.1 assures that these bases remain valid by defining the type of spent fuel, maximum initial enrichment, irradiation history, and maximum thermal heat generation.

2.2 MC-10 DRY STORAGE CASK

2.2.1 Specification

The MC-10 Dry Storage Casks used to store spent nuclear fuel at an ISFSI shall have the operating limits shown in Table 2-1.

2.2.2 Basis

The design criteria and subsequent safety analysis of the MC-10 assumed certain characteristics and operating limits for the use of the casks. This specification assures that those design criteria are not exceeded.

Table 2-1 MC-10 OPERATING LIMITS

	<u>Operating Limit</u>
Max. Lifting Height with Non-Redundant Lifting Device	5 feet
Dose Rate	
. 2 m Distance	\leq [REDACTED] hr
. Surface	\leq [REDACTED] hr
Cask Tightness (at closure):	
(Standard He-Leak Rate)	
. Primary Cover Seal	$\leq 10^{-6}$ std cc/s
. Primary Cover, Vent, Drain and Pressure Sensing Element Penetrations	$\leq 10^{-6}$ std cc/s
Optional Seal Cover Weld	$\leq 2 \times 10^{-4}$ std cc/s
Max. Specific Power of One Fuel Assembly	[REDACTED] kW
Initial Helium Pressure Limit (Cask Cavity)	≤ 1.5 atmospheres

2.3 LIMITING CONDITION - HANDLING HEIGHT

2.3.1. Specification

This specification applies to handling of a cask being used for spent fuel storage outside of the Fuel Building and Crane Enclosure Building.

The MC-10 dry storage cask shall not be handled at a height of greater than 5 feet.

2.3.2 Basis

The drop analysis performed for the MC-10 dry storage casks for postulated cask drop incidents on the ISFSI storage pad indicates that the material of the fuel basket and cask body has sufficient ductility and toughness to sustain a drop of 5 feet or less without sustaining unacceptable damage to the casks and fuel basket. This limiting condition ensures that the handling height limits will not be exceeded at the storage pad or in transit to and from the reactor.

2.4 DRY STORAGE CASK SURFACE CONTAMINATION

2.4.1 Specification

Initial removable contamination on the dry storage cask shall not exceed 2200 dis/min/100 cm² from beta-gamma sources, and 220 dis/min/100 cm² from alpha sources.

2.4.2 Basis

Compliance with this limit ensures that the decontamination requirements of 49 CFR 173.443, will be met over the lifetime of the cask in storage.

2.5 DRY STORAGE CASK INTERNAL COVER GAS

2.5.1 Specification

The dry storage casks shall be backfilled with helium.

2.5.2 Basis

The thermal analysis performed for the dry storage casks assumes the use of helium as a cover gas. On addition, the use of an inert gas (helium) is to ensure long-term maintenance of fuel clad integrity.

2.6 LIMITING CONDITION - PRESSURE MONITORING DEVICE

2.6.1 Specification

The pressure monitoring device used to monitor the leak tightness of MC-10 dry storage cask or fuel rod integrity shall have the performance characteristics shown in Figure 5.1-1 of the TSAR.

3.0 SURVEILLANCE REQUIREMENTS

Requirements for surveillance of various radiation levels, cask internal pressure, contamination levels, cask seal leak rates, and fuel related parameters are contained in this section. These requirements are summarized in Table 3-1 from details contained in Section 3.1 through 3.6. Specified time intervals may be adjusted plus or minus 25 percent to accommodate normal test schedules.

Table 3-1 SURVEILLANCE REQUIREMENTS SUMMARY

<u>Section</u>	<u>Quantity or Item</u>	<u>Period</u>
3.1.1	Cask Loading Measurements	P
3.2.1	Cask Seal Testing	L
3.3.1	Cask Contamination	L
3.4.1	Dose Rates (Cask surface or up to 2 meters from cask surface)	L
	Dose Rates (Fence)	Q
3.5.1	Safety Status Surveillance	Q
3.6.1	Pressure Monitoring Device Parameters	P & L
3.7.1	Alarm System	A

P - Prior to cask loading

L - During loading operations

Q - Quarterly

A - Annually

3.1 CASK LOADING MEASUREMENTS

3.1.1 Specification

For the first loading of a cask model, cask side-wall surface dose rate shall be measured upon cask draining. Prior to moving the cask to the storage pad, cask surface temperature shall be measured after the cask has been sealed for an appropriate period, which should not be less than that expected for the cask surface temperature to come into approximate equilibrium. These dose rate and temperature measurements shall be made at the cask side-wall mid-line at three locations 120° apart around the cask circumference and shall be recorded to establish a baseline of comparison for all subsequent loadings of this model of cask.

For all subsequent loadings of casks of this model, measure and record cask side-wall surface dose rates and temperatures at the cask side-wall mid-line at three locations 120° apart and compare these to the baseline established during first cask use. Do not transfer the cask to the storage pad if unexplained variations (which can not be resolved through known differences in spent fuel assemblies loaded) are found.

3.1.2 Basis

These measurements are to assure that casks have been properly loaded.

3.2 CASK SEAL TESTING

3.2.1 Specification

Prior to storage, the cask must be properly sealed by testing as specified in Section 10.2.6 of the TSAR to an initial leak rate of 10^{-6} std cc/sec.

3.2.2 Basis

The safety analysis of leak tightness of the cask as discussed in the topical report is based on the seals after 20 years being leak tight to 10^{-4} std cc/s. This check is done to ensure compliance with this design criteria.

3.3 CASK CONTAMINATION

3.3.1 Specification

After cask loading and prior to moving the cask to the storage pad, the cask shall be swiped to ensure that removable surface contamination levels are less than dis/min/100 cm² from beta-gamma emitting sources, and 220 dis/min/100 cm² from alpha emitting sources.

3.3.2 Basis

This surveillance requirement will ensure compliance with the decontamination requirements of 49 CFR 173.443 during storage in the ISFSI.

3.4 DOSE RATE5

3.4.1 Specification

The following dose rate measurements shall be made for the ISFSI:

- (1) Cask Surface Gamma and Neutron Dose Rates: After completion of cask loading, gamma and neutron measurements shall be taken on the outside surface (or within 2 meters of the cask surface). The combined gamma and neutron dose rates shall be less than the surface dose rate stated in Table 2-1 (or the specified rate at a distance of up to 2 meters from the cask surface).
- (2) Dry Cask ISFSI Boundary: Doses shall be determined by measurement at the Dry Cask ISFSI site fence and shall be evaluated on a quarterly basis to demonstrate compliance with §20.105(b)(2), 10 CFR Part 20.

3.4.2 Basis

These measurements are necessary to assure compliance with the cask specifications and that the dose rates at the security fence meet Part 20 limits as additional casks are placed in storage.

3.5 SAFETY STATUS SURVEILLANCE

3.5.1 Specification

A visual surveillance shall be performed on a quarterly basis of the ISFSI to determine that no significant damage or deterioration of the exterior of the exploded casks has occurred. Surveillance shall also include observation to determine that no significant accumulation of debris on cask surfaces has occurred.

3.5.2 Basis

The surveillance requirements shall ensure cask maintenance.

3.6 CASK CONFINEMENT INTEGRITY (MC-10)

3.6.1 Specification

The cask confinement integrity shall be monitored by use of a pressure monitoring device to verify the leak tightness of the cask. A functional test shall be performed during cask preparation.

3.6.2 Basis

This specification requires the cask cavity atmosphere be maintained and monitored to detect any possible leakage of cask seals.

3.7 ALARM SYSTEM

3.7.1 Specification

An alarm system to which all of the pressure monitoring devices are connected shall be installed at the storage site and functionally tested annually to ensure proper operation of the system.

3.7.2 Basis

The alarm system must be capable of alerting surveillance personnel of possible cask seal failure and must permit identification of the specific cask indicating a seal failure.

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