



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

October 28, 2005

Virginia Electric and Power Company
ATTN: Mr. David A. Christian
Senior Vice President and
Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060

SUBJECT: NOTIFICATION OF SURRY NUCLEAR STATION INTEGRATED DESIGN AND
COMPONENT CAPABILITY INSPECTION - NRC INSPECTION REPORT
05000280/2005005 AND 05000281/2005005

Dear Mr. Christian:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct an integrated design and component capability inspection at your Surry Nuclear Station during the weeks of January 9-13, 2006, January 23-27, 2006, and February 6-10, 2006. The inspection team will be led by Mr. Caswell Smith, a Senior Reactor Inspector from the NRC's Region II Office. This inspection will be conducted in accordance with the soon to be issued baseline inspection Procedure 71111.21, Integrated Design and Component Capability Inspection. (This inspection procedure has not been issued yet.)

As currently planned, the inspection will evaluate the capability of risk significant / low margin components to function as designed and support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on October 18, 2005, Mr. Smith of my staff, and Mr. Barry Garber of your staff, confirmed arrangements for an information gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of December 12, 2005
- Onsite weeks: January 9, 2006; January 23, 2006; and February 6, 2006

The purpose of the information gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. Walter Rogers, a Region II Senior Reactor Analyst, will accompany Mr. Smith and the inspection team during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection. Please contact Mr. Smith prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space and prior to the inspection preparation week of January 2, 2006; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Smith at (404) 562-4630 or me at (404) 562-4605.

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-280, and 50-281
License Nos.: DPR - 32 and DPR - 37

cc: w/o encl: (See page 3)

VEPCO

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DATE	10/28/2005	10/27/2005	10/19/2005	10/19/2005	10/ /2005	10/ /2005	10/ /2005
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INFORMATION REQUEST FOR SURRY NUCLEAR STATION -- INTEGRATED DESIGN AND COMPONENT CAPABILITY INSPECTION

(Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Information in “lists” should contain enough information to be easily understood by someone who has a knowledge of pressurized water reactor technology.)

1. Risk ranking of components from your site specific probabilistic safety analysis (PSA) sorted by Risk Achievement Worth (RAW) and sorted separately by Birnbaum Importance.
2. Provide a list of the top 500 cutsets from your PSA.
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide copies of your human reliability worksheets for these items.
4. If you have an External Events or Fire PSA Model, provide the information requested in Items 1 and 2 for external events and fire.
5. Any pre-existing evaluation or list of components and calculations with low design margins, (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal etc.)
6. The last two years of operating experience evaluations, modifications, and corrective actions sorted by component or system.
7. Information of any common cause failure of components experienced in the last 5 years at your facility.

Enclosure