

5.6 Reporting Requirements (continued)

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5.6.5 Core Operating Limits Report (COLR) (continued)

- 10) TXX-88306, "Steam Generator Tube Rupture Analysis," March 15, 1988.
  - 11) RXE-91-005-A, "Methodology for Reactor Core Response to Steamline Break Events," February 1994.
  - 12) RXE-94-001-A, "Safety Analysis of Postulated Inadvertent Boron Dilution Event in Modes 3, 4, and 5," February 1994.
  - 13) RXE-95-001-P-A, "Small Break Loss of Coolant Accident Analysis Methodology," September 1996.
  - 14) Caldon, Inc. Engineering Report-80P, "Improving Thermal Power Accuracy and Plant Safety While Increasing Operating Power level Using the LEFM<sup>√</sup> System," Revision 0, March 1997 and Caldon Engineering Report – 160P, "Supplement to Topical Report ER-80P; Basis for a Power Uprate With the LEFM<sup>√</sup> System," Revision 0, May 2000.
  - 15) ERX-2001-005-P, "ZIRLO<sup>™</sup> Cladding and Boron Coating Models for TXU Electric's Loss of Coolant Accident Analysis Methodologies," October 2001.
  - 16) WCAP-10444-P-A, "Reference Core Report VANTAGE 5 Fuel Assembly," September 1985.
  - 17) WCAP-15025-P-A, "Modified WRB-2 Correlation, WRB-2M, for Predicting Critical Heat Flux in 17x17 Rod Bundles for Modified LPD Mixing Vane Grids," April 1999.
  - 18) WCAP-13060-P-A, "Westinghouse Fuel Assembly Reconstitution Evaluation Methodology," July, 1993.
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

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